



Issues for Fueling and Particle Control

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Goal is to formulate plan for FY06 decision point



- **What are NSTX particle control requirements?**
 - **Experimental evidence to date**
 - **Near-term and long-term needs**
 - **Adequacy of present and planned capabilities**
 - **Surface conditioning (bakeout, boronization, helium GDC, pellet injection, etc.)**
- **What are NSTX power handling needs?**
 - **Near-term and long-term needs**
 - **“Realistic” schedule for long-pulse (≈ 5 s) operations**
 - **Adequacy of existing PFC's and planned upgrades**

Common understanding of results to date needed



- **What are implications of past lithium experiments?**
 - **“Uniqueness” of TFTR results**
 - **Research on other magnetic confinement devices**
 - **Experience from PFC community**
- **Is cryopump experience sufficient for predicting future performance?**
 - **Past efforts appear to be based on installation of cryopump, collection of comprehensive data set, and adjustment model for “best fit”**
 - **Extensive modifications required for implementation on NSTX may preclude such “cut and try” approaches**

Possible conclusion may be that neither cryopumping nor flowing liquid lithium divertor module may be necessary



- **Evaporative coating may be sufficient for particle control**
 - **Primary means of particle control in LTX**
 - **Implementation already planned for NSTX Experience from PFC community**
 - **UEDGE-2-D code combined with 3-D REDEP/WBC code by Brooks at ANL predict lithium to be confined to divertor region**
- **Cryopump requires extensive and expensive modifications to passive plate geometry**
 - **Little flexibility for testing various “throat” configurations in absence of reliable predictive modeling**
- **Flowing liquid lithium divertor module loses primary**
 - **Five-second pulses may not be realistic for NSTX or necessary for its mission**
 - **Argument for prototyping reactor chamber technology becoming less compelling with growing focus on ITER needs**

Urgent immediate issue concerns installation of new divertor PFC's this opening



- **Evaporative lithium coatings require surfaces other than carbon to prevent intercalation**
 - **High velocity oxygen fueled (HVOF) spraying of molybdenum on carbon tiles may be possible**
- **Need to determine how much of divertor surface can be covered by evaporator(s) planned for upcoming run**
 - **Will coverage have measurable effect on recycling even with “perfect” substrate?**
- **“Technology testing” rather than consequences for plasma performance may be more realistic goal for upcoming run**
 - **Reliability of evaporator probe drive operation and performance of lithium oven or e-beam on lithium sample can be checked**
 - **NSTX run time *not* required to confirm well-established intercalation results**

Arguments could be made for *not* replacing any tiles this opening



- **“Technology testing” rather than consequences for plasma performance may be more realistic goal for upcoming run**
 - **Reliability of evaporator probe drive operation and performance of lithium oven or e-beam on lithium sample can be checked**
 - **NSTX run time *not* required to confirm well-established intercalation results**
- **Useful results could still be obtained with carbon tiles**
 - **Might reported upper limit of $\approx 30\%$ lithium at “saturation” of carbon still have measurable effect on recycling?**

Question for upcoming run concerns usefulness of devoting some of limited run time to “duplicating” TFTR lithium results



- Will extensive conditioning and pellet injection to coat center stack for limiter plasmas be sufficient?
- Will developing such scenarios be informative and useful if NSTX program emphasizes divertor plasmas?

Need to include preparation for FY06 decision point on particle control options in FY05 run planning



<u>NSTX Issue</u>	<u>Cryopump Assessment Status</u>	<u>Cryo Assessment Basis or Schedule for Achievement</u>	<u>Liquid Lithium Module Assessment Status</u>	<u>Liquid Lithium Module Assessment Basis or Schedule for Achievement</u>
Capability for Particle Control	Ability established with operational caveats (see comments under "Operation" in next table)	DIII-D (GA) results on edge plasma modification available - although <i>predictive</i> capability for "first principles" NSTX design needs more work	Ability established	UCSD (PISCES) and UIUC results on hydrogen retention and PPPL (CDX-U) results on recycling reduction
Capability for Power Handling	Not applicable (must be protected from high heat flux)	Not applicable	Required flow rate is 7-12 m/s from analysis	PISCES results on temperature dependence of lithium evaporation confirm temperature limits; power handling tests at LIMITS facility (Sandia) planned but not started
Safety	Cryogenics handled routinely	Experience on DIII-D, NSTX, and elsewhere	1) Static "pools" of liquid lithium handled safely 2) Circulating Li system assessment required	1) CDX-U has safe handling experience with static fully-toroidal liquid lithium limiter 2) Tests of flowing lithium hardware in progress at LIMITS facility; assessment to be completed in mid-FY05

NSTX experiments in FY05 can investigate operational issues that need to be addressed for FY06 decision point



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Installation	In-vessel modifications substantial but potentially straightforward: <ul style="list-style-type: none"> • Close passive plate gaps and redesign secondary passive plate supports 	Preliminary assessment completed – Menon (ORNL - ret.)	In-vessel modifications could be limited but have special requirements: <ul style="list-style-type: none"> • Permit lithium flow into, through, and out of NSTX • Accommodate CHI “gap” and diagnostic penetrations 	Preliminary assessment completed – Nelson (ORNL); prototype flowing liquid metal systems being tested at MTOR (UCLA) and LIMITS; conceptual design for NSTX flow configuration requires experimental data and MHD modeling results for NSTX divertor geometry (UCLA/Hypercomp)
Operation	Pumping dependence of separatrix distance to plenum limits achievable plasma geometries	Preliminary assessment completed – Menon (ORNL - ret.)	MHD effects on liquid lithium may limit permissible magnetic field ramp rate; assess ELM, thermoelectric current, and plasma wind effects; control external current loops	Experimental and computational assessment of MHD effects on liquid lithium flow in NSTX fields in progress at SNL and UCLA; ELM effects and other issues related to NSTX plasmas require further investigation on large MFE facilities