

National Spherical Torus Program

Spherical Torus Coordinating Committee

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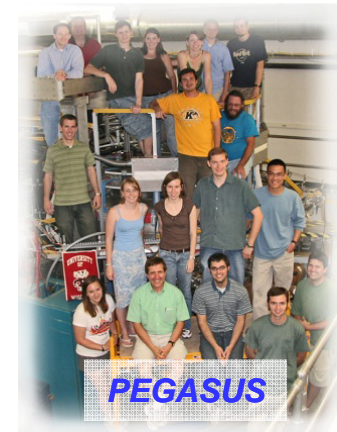
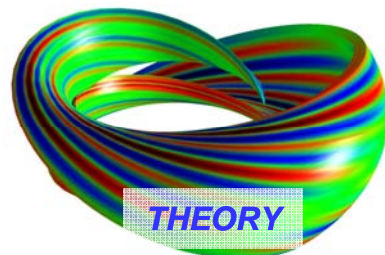
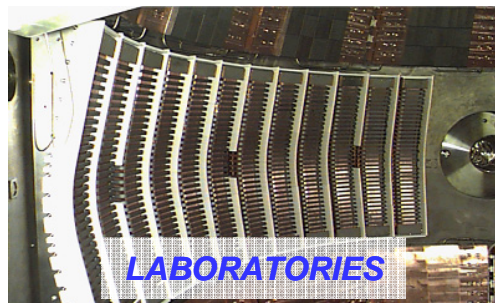
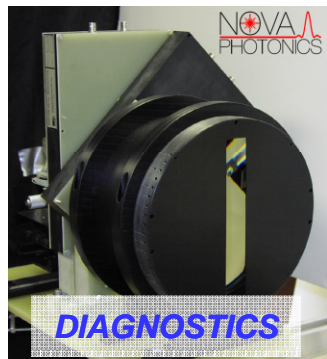
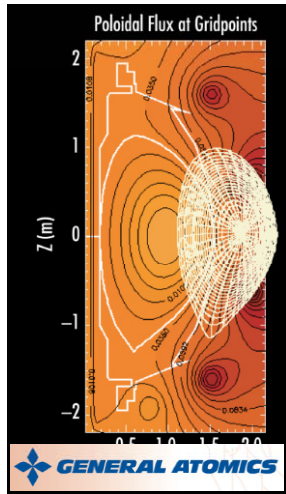
Steve Sabbagh (Columbia U)

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OFES FY10 Budget Planning Meeting

Gaithersburg, Maryland

March 11-12, 2008



U.S. ST Coordinating Committee (STCC) charge



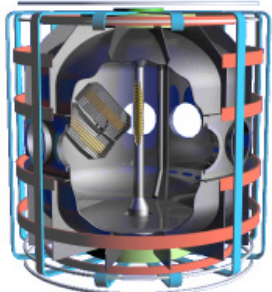
- Organized by OFES and report to ST Program Manager (Steve Eckstrand)
- Develop, support and promote the evolving roles of ST in the U.S. fusion program
- Coordinate milestones, plans, and longer term goals
- Review and report progress relative to funded R&D
- Represent and advocate ST Program nationally, and internationally through the IEA ST Executive Committee
- Membership selected to represent major R&D components

Content of talk

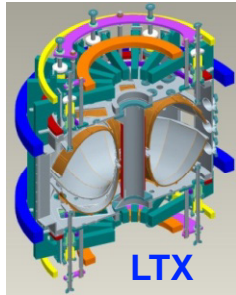


- ST R&D Mission in ITER era
- Pegasus opportunities and proposal
- LTX opportunities and proposal
- Critical research on NSTX resolvable in 3 years
- International collaboration
- Proposed STCC effort for FY09-FY10
- ST R&D Advocacy

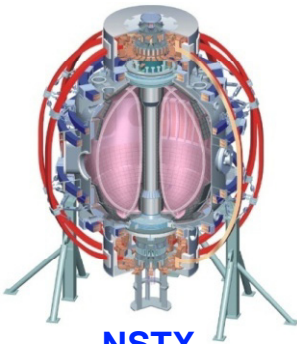
Mission: Develop compact, high beta ST burning plasma capability for use-inspired research and development



PEGASUS



LTX

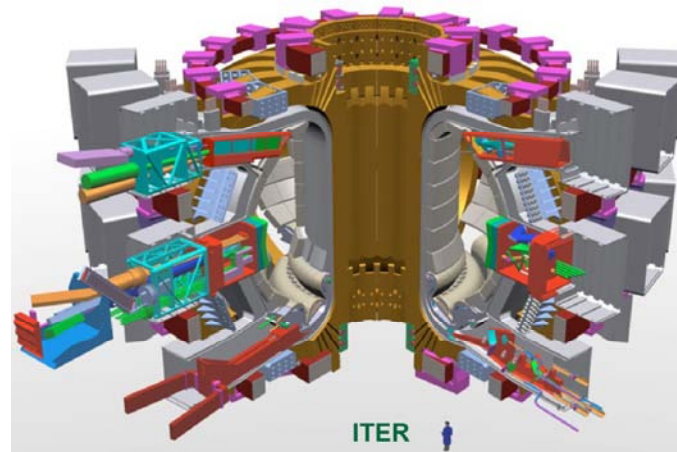


NSTX

International
Research
Collaboration

The ITER Era

ITER for Burning Plasma Science



ITER

ST burning plasma
capability
for
use-inspired R&D

*Work to develop the supporting
ST strategy has just begun*

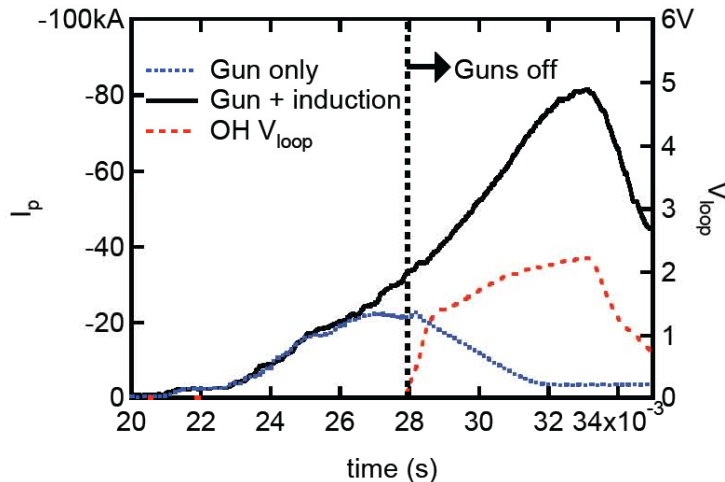
ST & Other
Fusion
Energy
Source(s)



Pegasus Research Program has 3 Thrusts Addressing Critical Issues for the ST & AT

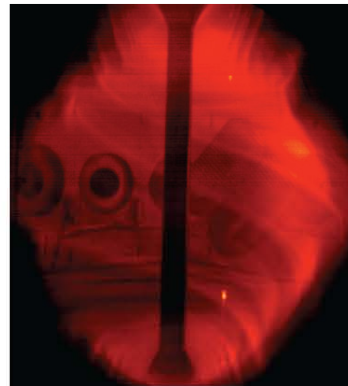
Non-solenoidal startup, ramp-up (Theme C)

- Develop scalable plasma gun arrays
 - demonstrate high I_p with high TF
 - 0.1 - 0.3 MA in FY08-10
- Ramp-up via HHFW
 - FY 09-11
- Ramp-up via EBW
 - collab. w/PPPL & ORNL?



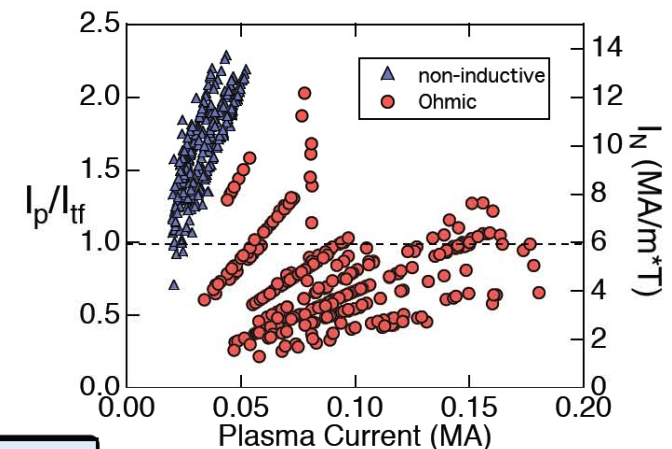
Edge stability at high $(j_{||}/B)_{edge}$ (Theme B)

- High density magnetic probe arrays
 - determine m & n in FY08-09
- Edge probes: directly measure $j_{||}/B$, $p(R)$
 - FY 08-10
- Divertor coils for edge stability mod.



$j(R)$ mod. for high- I_N , β_t (Theme C)

- Edge current drive via guns
 - broad $j(R)$
 - 0.1 - 0.3 MA in FY08-10
- New centerstack
 - access high I_p , β_t
 - FY 10-12



Experimental Parameters

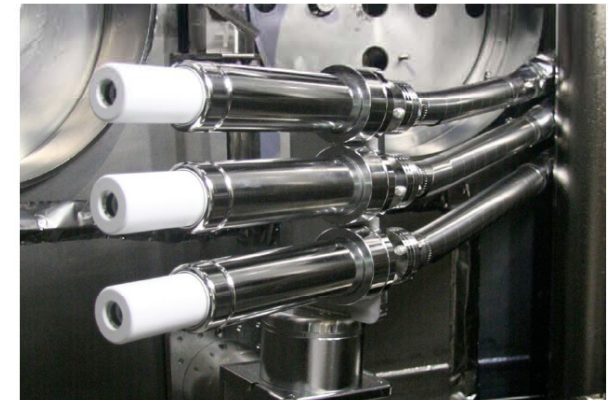
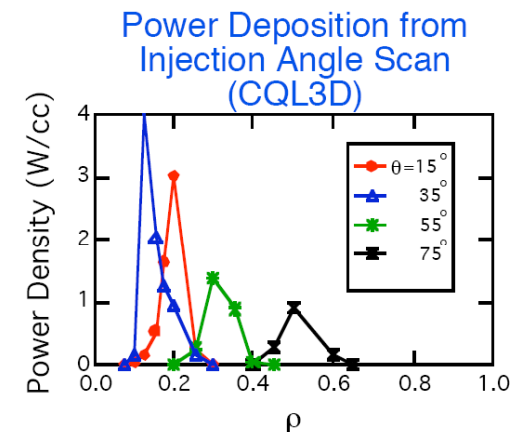
Parameter	Achieved	Goals
A	1.15-1.3	1.12-1.3
I_p (MA)	≤ 0.18	≤ 0.30
I_N (MA/m-T)	6-12	6-20
$I_{non-inductive}$ (MA)	0.05	0.3
β_t (%)	≤ 25	> 40
τ_{shot}	≤ 0.02	≤ 0.1





Full-Use Budget Supports Upgrades for Addressing 3 Main Thrusts of Program

- Full-Use: +~\$500K/yr
 - Optimize centerstack upgrade (+\$75 K/year for FY09-10)
 - Refurbish HHFW to 1 MW (+\$25 K/year for FY09-10)
 - Post-doc for Thomson Scattering (+\$100 K/year)
 - **EBW: 1 MW current ramp-up tests in an ST (+\$300 K/year)**
 - Collaboration with PPPL/ORNL?
- FY08 baseline = \$954K
 - Support staff and students
 - Scalable, high-power gun array development
- Reduced case: -10%
 - Personnel reduction: staff or grad students
 - Downgrade or eliminate one of the High- I_p test gun arrays
- Major Facility Activities - baseline funding
 - **Plasma gun injector systems development and testing**
 - Power supplies; Scalable high-current midplane arrays
 - Divertor coil upgrade for edge stability
 - SXR imaging for $j(r,t)$
 - Upgrade centerstack for high TF and $I_{\text{non-inductive}} \geq 0.2$ MA



LTX focus in FY08-10: Electron transport

LTX

- ◆ FY08 - First plasma
- ◆ FY09 - Lithium wall operations
 - ∇T_e as a function of recycling
 - » Thomson scattering + Lyman-alpha + DEGAS 2
 - Effect of lowered ∇T_e on confinement
- ◆ FY10 - Operation with long current flattop
 - $> \tau_E$, $>$ current diffusion time
 - $T_e(r)$ with relaxed current profile

Design parameters for LTX:

$$R_0 = 0.4 \text{ m}$$

$$a = 0.26 \text{ m}$$

$$\kappa < 1.6$$

$$B_T < 3.4 \text{ kG}$$

$$I_p < 400 \text{ kA}$$

$$\tau_{\text{discharge}} < 0.25 \text{ s}$$

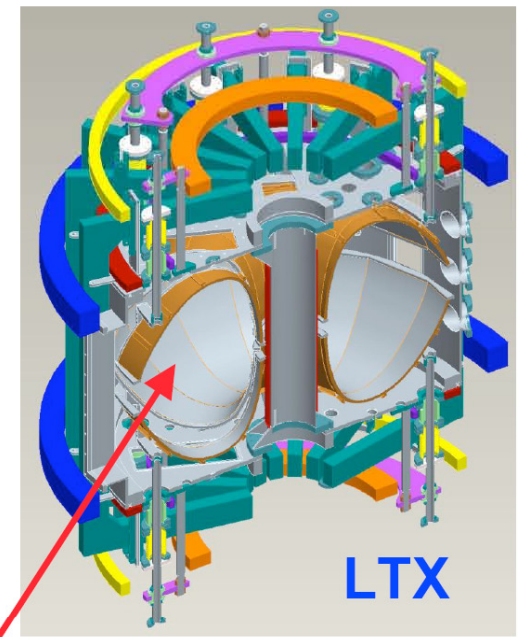
LTX research fits within theme B of the Greenwald panel:

⇒ Understand *and control* plasma material interactions (GW-8)

⇒ Design *replaceable* components without degrading performance (GW-9)

**LTX liquid lithium wall development supports
LLD implementation in NSTX**

3 graduate students, 1.5 post-docs



Liquid lithium coated shell

Full use of LTX: Core fueling via NBI



- ◆ Full use case: baseline +700k/year incremental
 - FY09: OH supply upgrade to design capability.
 - » Add edge Thomson channels
 - FY10: Install short-pulse NBI for tests of core fueling in very low recycling regime (UW-Pegasus collaboration)
 - » Installation of lithium coated porous molybdenum shell
 - » 3.4 kG operation
 - » Begin CHERS install
 - FY11: Full NB fueling, CHERS for T_i
- ◆ Baseline funding: 970k, flat FY08-10
 - First plasma in FY08 (but OH power supply at 1/4 design, no feedback)
 - First lithium wall experiments in FY09
 - Confinement results with long flattops in FY10
- ◆ Reduced case: baseline -10%
 - Abandon Thomson scattering system in mid FY09
 - » Cannot replace postdoc presently working on system
 - No OH upgrade, no long flattops in FY10

STCC asked to assess 3-year critical research on NSTX



- Identified and organized 39 research topics
- Developed and used Measures of Criticality (MOCs) to down-select
 - Physics Regime?
 - Scientific Gap?
 - Future Design Benefit?
 - World-leading?
- PoP Research Maturity?
 - 3-Year Progress?

3-Year, NSTX Only
- Benefit to ITER R&D?
 - Benefit from ITER R&D?
 - Unique Contributions to Toroidal Plasma Science?

Broader, Longer Term
- Results submitted to OFES and shared with NSTX leadership

The most critical research topics identified by STCC



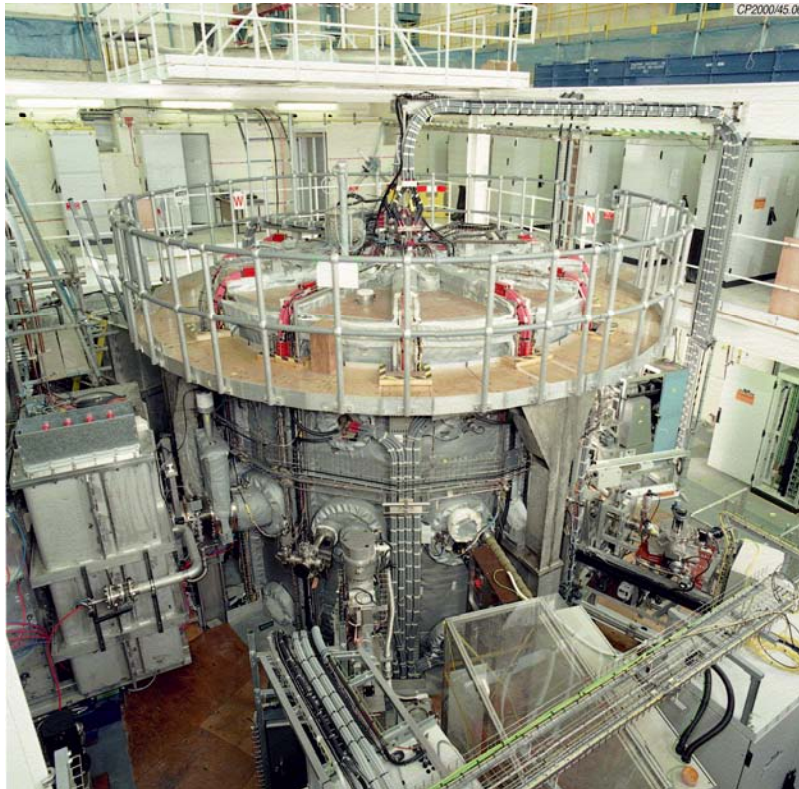
1. Impact of reduced collisionality on high-performance integrated scenarios (Themes A,B,C)
 2. JNBI predictability in sustained beta \sim no-wall, fBS \sim 0.5, high-confinement plasma (Themes B,C)
 3. Predictive capability in electron turbulence and transport (Themes A,B,C)
 4. Multiple harmonic fast wave for ramp-up and sustainment assist (Theme C)
- **With focus and adequate theory support, these 4 critical topics can be resolved in 3 years on NSTX.**
 - **Additional critical topics can be identified if the 3-year constraint is removed.**

MAST/MAST-U offers opportunities for collaboration



Two equal goals:

- Explore the potential of ST as a Component Test Facility (CTF) and/or ST Power Plant
- Advance key tokamak physics issues for ITER and DEMO



MAST-U received provisional commitment for 2/3 funding (of 36M BP total)

- Focus first on 5-s magnets, 12.5MW NBI, divertor, ELM control

Collaboration opportunities:

- PoP (1-MW) level EBW startup tests
 - 350-kW test to start in May-June 2008
- Microwave high-k scattering
- MSE upgrade
- Pellet fueling and ELM pacing upgrade
- SXR tomography
- Long-pulse rotating plasma stability control near no-wall limit

IEA ST Agreement ready to provide international framework

Present work assignment for STCC



Support FES Strategic Planning, working broadly within the community,

- Four “Approaches, Options, Initiatives” workshops for Tokamak/AT
- FESAC Panel on Magnetic Alternates (Stellarator, ST, RFP, CT)
- Assess leverage on “Strengths, Weaknesses, Opportunities, Threats” of initiatives (i.e., Future Design Benefit) to guide ST strategy and research priorities

Substantial efforts and travel will be required

- Meetings: ~30 person-trips/year, 15 for members funded by grants (\$75k/year)
- Dedicated efforts (from 10% to 50% time) to be arranged directly with DOE
- Apply community (including ORNL) expertise in engineering, nuclear, material sciences to assess high-leverage impacts across major candidate initiatives (1.3 FTE/year)
- Joint work with fusion plasma science experts in such assessments



The spherical torus (ST) is an experimentally-proven magnetic fusion configuration that

- (1) has advanced, and is poised to continue to advance, critical fusion energy science knowledge by leveraging its unique geometry and high beta operational space, and
- (2) is envisioned to evolve into potentially cost-effective fusion energy systems.

Rapid progress has been made in devices worldwide with currents up to mega-Ampere, with the U.S. being a leader.

The DOE-formed ST Coordinating Committee (STCC) is committed to advancing world-leading ST research in support of the Fusion Energy Sciences Program, and advocates that this research path be continued during the ITER era by establishing facilities/upgrades (*in U.S. and abroad*) that will take it to the next-step. The committee suggests that the importance of continued U.S. leadership in this research surpasses specific institutional considerations, and calls for a national approach to research planning and management to energize community participation and support.