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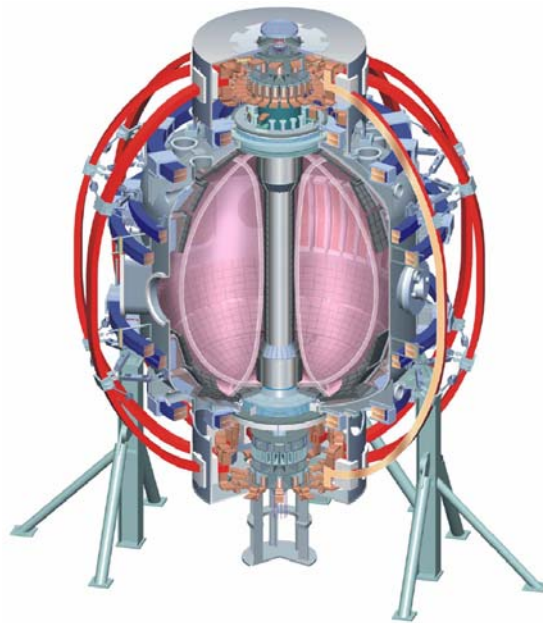


NSTX

NSTX contributions to new ITPA/ITER disruption database

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For the NSTX Team

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Present knowledge indicates risk from electromagnetic impulse of current quench barely acceptable for ITER



USBPO

- 3D simulations of ITER VDE performed

- Consistent with ITER disruption database

- Halo fraction $I_{\text{halo-max}} / I_{p0} \sim 0.3$
- Toroidal peaking factor (TPF) $\sim 1-2.5$

- Toroidal peaking of halo current result of current channel kinking during quench

- More simulations needed to explain present data and extrapolate to BP

- Understand I_p quench rate scaling

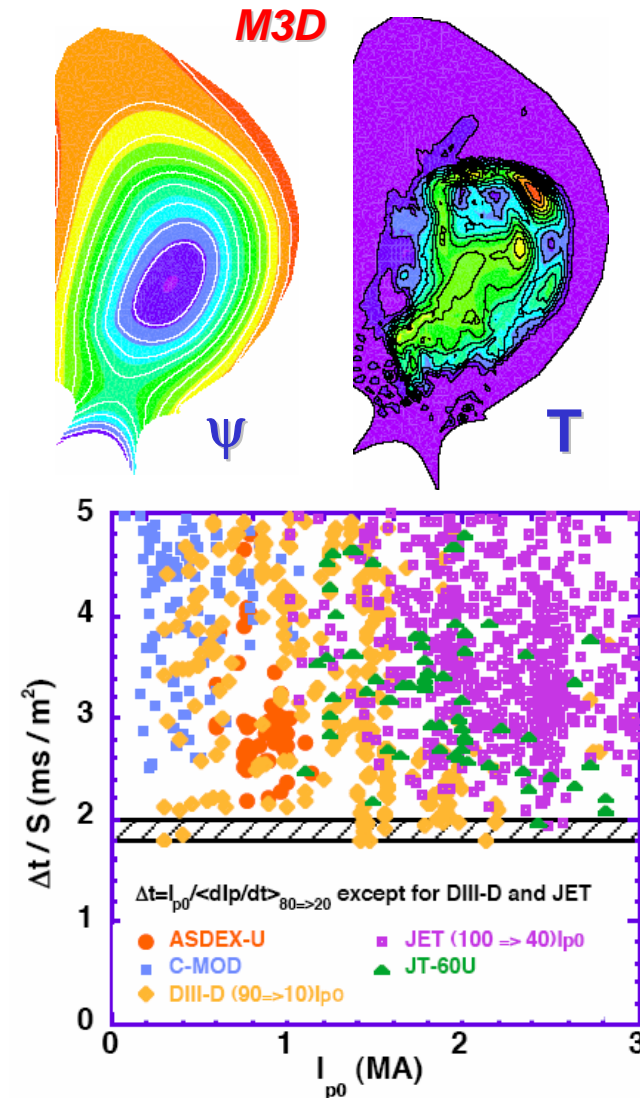
- Disruption database indicates $\Delta t / S \approx 2 \text{ ms/m}^2 \rightarrow$

- Understand scaling of TPF and f_{halo}

- Experiments observe $\text{TPF} \times f_{\text{halo}} \approx 0.7$

- Accurately compute EM forces on blanket

- Need 3D plasma **and** 3D structure models



Status of disruption analysis for NSTX

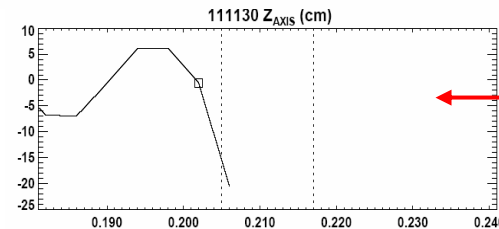
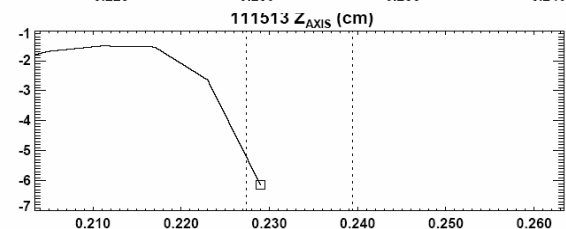
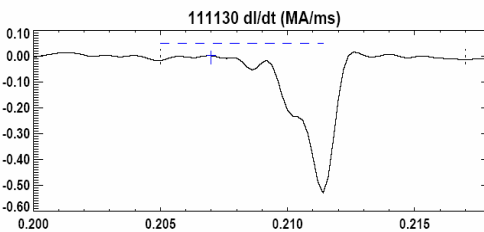
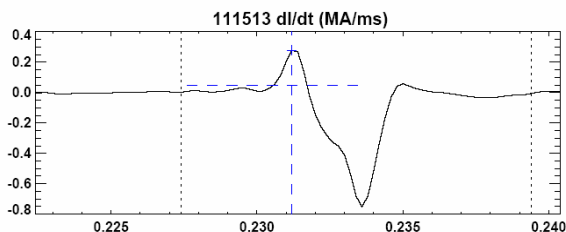
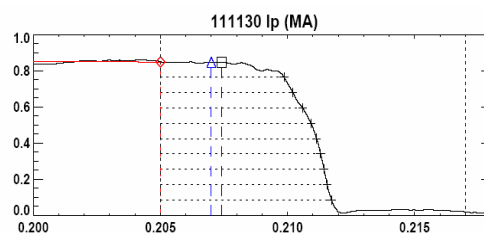
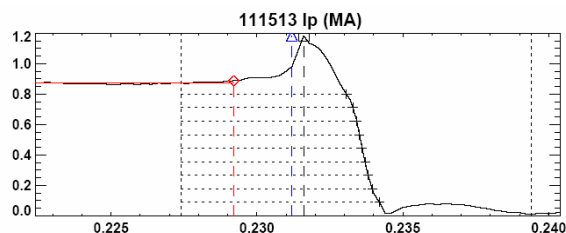


- Scanned > 15000 NSTX shots (P. Roney)
 - Compute times at 90% and 10% of max I_p
 - Shot list for 500 fastest disrupting shots created
- 200 fastest validated for subsequent analysis
 - Automated IDL routine → pre-disruption current, current spike parameters, decay evolution
 - Require good EFIT01 χ^2 just prior to disruption
 - Compute 59 scalar parameters for DDB →

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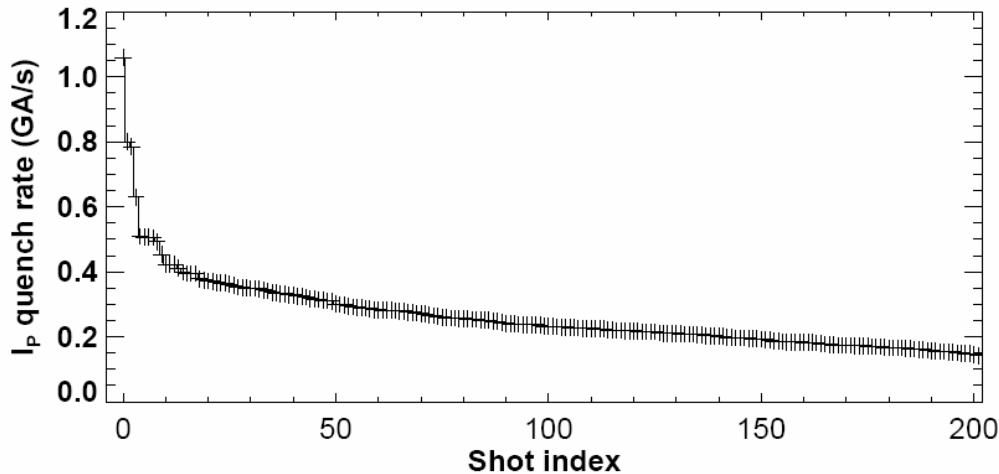
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BETMHD_D = 9.40708
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BTD = -0.290775
CAUSED =
CHISQD = 128.806
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CONFIGD = DND
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DELTALD = 0.389872
DELTAUD = 0.357423
DIDTMAX = 2.93594e+06
DIVNAME = Open
DRSEPD = -0.00319415
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EVIDRAE_E =
INDENTD = 0.00000
INTLID = 1.06075
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IPEQD = 855434.
IPPHASED = flattop
IPSPK = 846804.
IPSPK_E = N
KAPPAD = 2.00466
NINDXD = 0.00000
PHASED =
Q95D = 3.71889

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RMAGD = 0.966210
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TIME2 = 0.211559
TIME3 = 0.211435
TIME4 = 0.211301
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TIME6 = 0.210917
TIME7 = 0.210598
TIME8 = 0.210207
TIME9 = 0.209859
TIME95MAX = 0.192629
TIMED = 0.205000
TIMEDIDTMAX = 0.2070
TIMEQD = 0.202000
TIMERMAX = 0.00000
TIMESPK = 0.207400
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TQ_E =
VDE_E =
VDEDRIFT = DN
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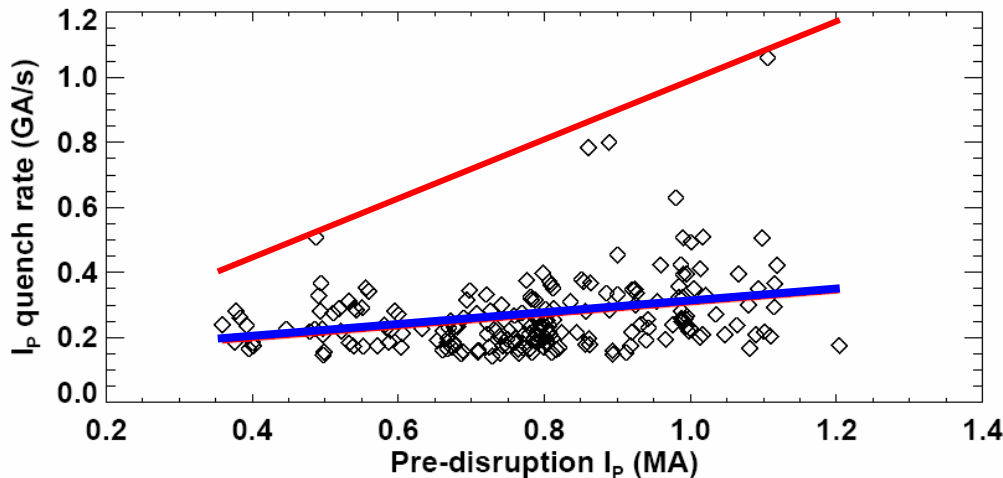


- Many disruptions correlated with loss of vertical control

Absolute quench rates increase with plasma current



- Very few shots (<10) exceed quench rates of 0.4GA/s



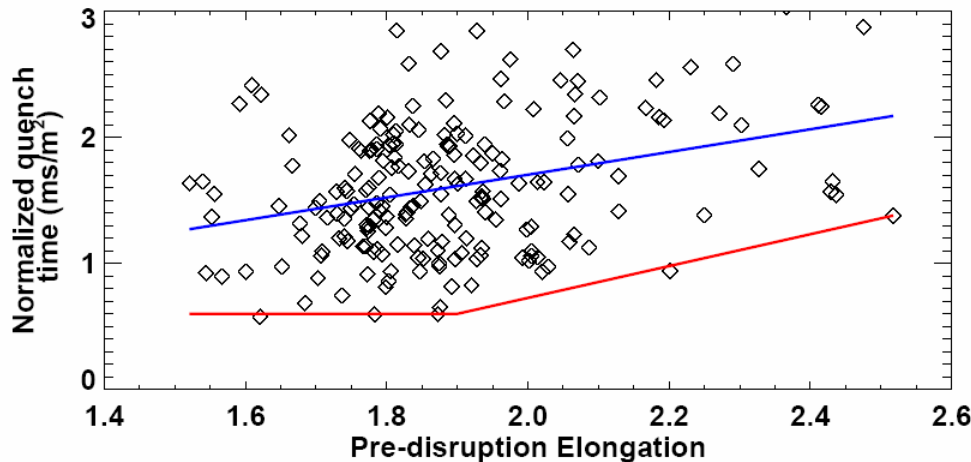
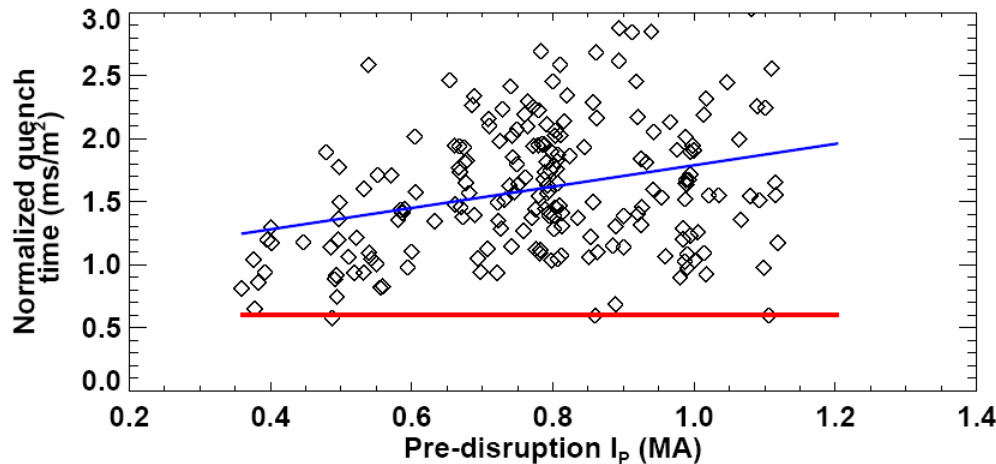
- The scaling with current is relatively weak on average
- However, the maximum quench rate does scale strongly with current...
 - Determined by very few shots

Minimum normalized quench time is independent of I_p



Expected normalized
L/R decay time:

$$\frac{\tau}{S} = \frac{L_p^{eff} / 2\pi R_0}{\eta_p}$$



- The average normalized quench time increases with I_p
 - Hotter initial plasma?
- The minimum normalized quench time $\approx 0.6 \text{ ms/m}^2$
 - Like tokamaks, no I_p dependence
 - Absolute value is lower however
 - Colder plasma? Lower I_i ?
- The average normalized quench time also increases with kappa
- The minimum normalized quench time also increases with kappa
 - Favorable scaling with kappa?

Status of ITPA DDB data submission



- DDB software “new” as of July 2005
 - In process of debugging DB submissions
- 200 shots loaded to ITPA DDB at GA for testing
 - Data written to MDS+
 - Tree data read and transferred to SQL DB nightly
- Web interface to data now available
 - Can plot pre-disruption current and decay values
 - Available for any shot from any machine
 - DIII-D, JET, C-MOD, NSTX (JT-60U, MAST)

• Shot	106939
• Tokamak	NSTX
• IPD	782253
• TIMED	0.389

