

# Plans / collaboration discussion – stability/ control theory/modeling (Columbia U. group)

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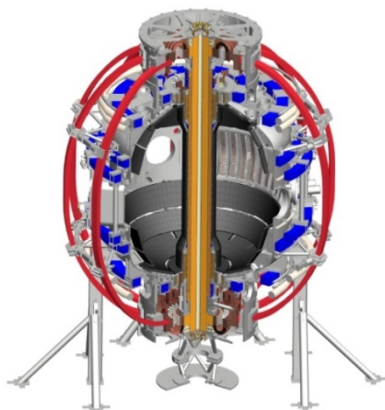
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**NSTX-U / DIII-D Collaboration meeting**

**December 10<sup>th</sup>, 2013**

Coll of Wm & Mary  
Columbia U  
CompX  
General Atomics  
FIU  
INL  
Johns Hopkins U  
LANL  
LLNL  
Lodestar  
MIT  
Lehigh U  
Nova Photonics  
ORNL  
PPPL  
Princeton U  
Purdue U  
SNL  
Think Tank, Inc.  
UC Davis  
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UCLA  
UCSD  
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U Maryland  
U Rochester  
U Tennessee  
U Tulsa  
U Washington  
U Wisconsin  
X Science LLC

Culham Sci Ctr  
York U  
Chubu U  
Fukui U  
Hiroshima U  
Hyogo U  
Kyoto U  
Kyushu U  
Kyushu Tokai U  
NIFS  
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Chonbuk Natl U  
NFRI  
KAIST  
POSTECH  
Seoul Natl U  
ASIPP  
CIEMAT  
FOM Inst DIFFER  
ENEA, Frascati  
CEA, Cadarache  
IPP, Jülich  
IPP, Garching  
ASCR, Czech Rep



**PPPL**



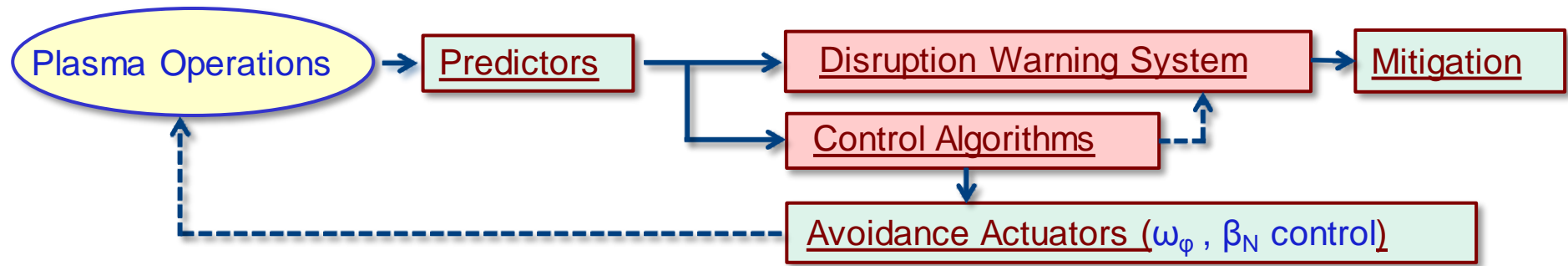
# The Columbia U. group plans to contribute to JRT-14 (which is presently just being defined)

- JRT-14
  - Conduct experiments and analysis to investigate and quantify plasma response to non-axisymmetric (3D) magnetic fields in tokamaks
  - Initial discussion focused on  $n = 1$ , considering both linear and non-linear response
- Present planned Columbia contributions
  - Focused task defined with Todd Evans, Nate Ferraro on determining realistic plasma response magnitudes/profiles in modeling when compared to experiment (w/M3D-C<sup>1</sup>) – Steve at GA this week for this
  - Use data from several NSTX experiments on NTV to determine bounds for the plasma response to different applied 3D field spectra: “ $n = 1$ ”, “ $n = 2$ ”, “ $n = 3$ ” configurations
  - “Bounds” on the RFA would be determined by NTV profile calculations in NSTX experiments using the NTVTOK code
    - Note: NSTX NTV publications (~ 2006) have shown “ $n = 3$ ” vacuum field is sufficient to calculate NTV, but plasma response needed for “ $n = 1$ ”

# The Columbia U. group plans to contribute to JRT-15 (which is also presently being defined)

- JRT-15
  - Experiments and analysis to quantify the impact of broad current and pressure profiles on tokamak plasma confinement and stability
  - Initial discussion focused/defined JRT-15 along the lines of determining the effect of new actuators (NBI, NTV, etc.) in NSTX-U
- Present planned Columbia contributions
  - Kinetic RWM stability physics: impact of pressure, rotation profile changes from the new beam and also EP anisotropic distribution.
  - Focus on NTV and NBI actuator results in pre-programmed plasma rotation control experiments on NSTX-U
  - Compare experimental results with our present rotation control work (about 6 months work already) with I. Goumiri
    - A significant part will be accuracy of NTV model developed in FY14 comparison to NTV theory in the NTVTOK code
  - Also evaluate equilibrium reconstruction variation with NBI modification of  $J$ ,  $q$  profile with varied NBI

# NSTX-U is planning a disruption avoidance system, in which real-time MHD spectroscopy or kinetic physics can be used

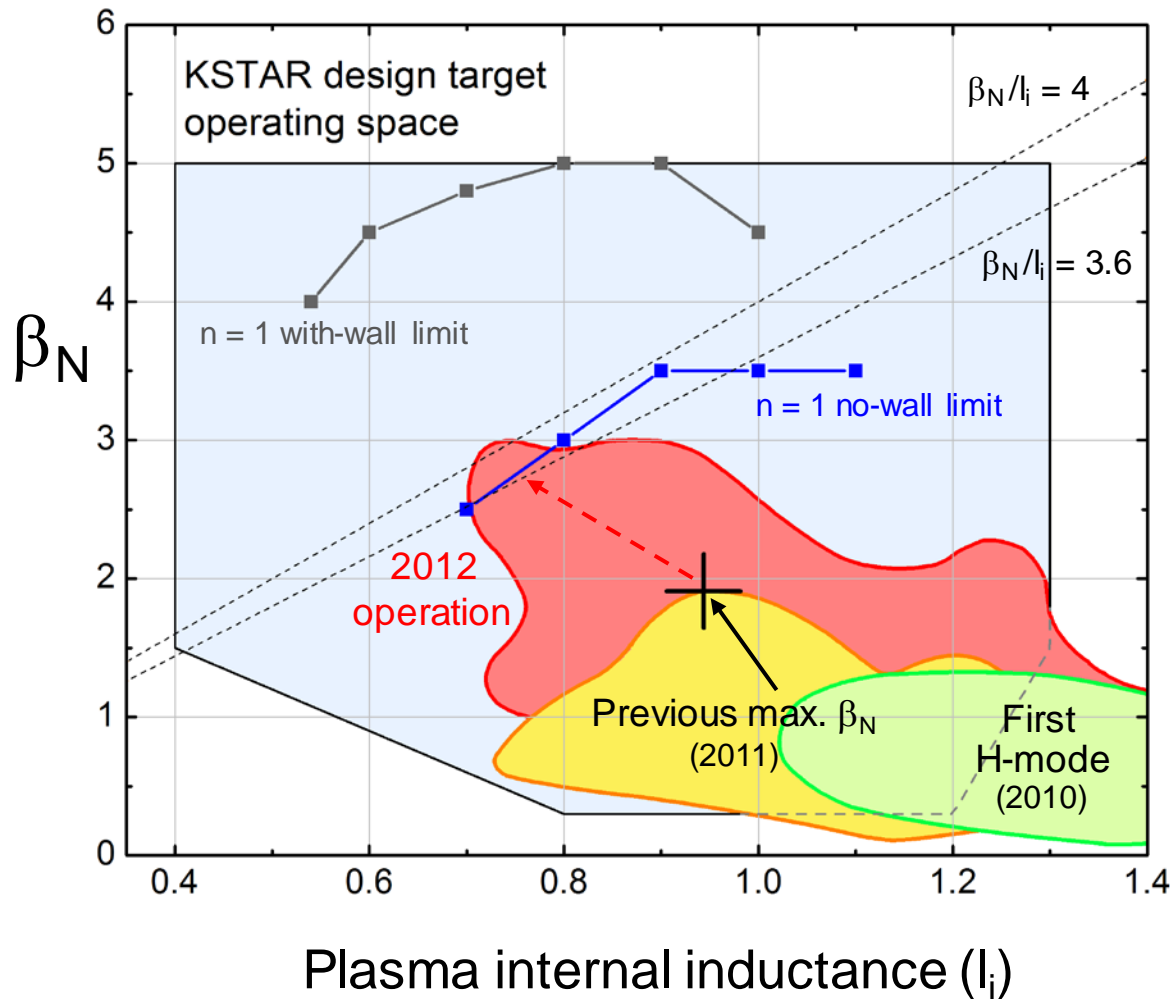


- Both NSTX and DIII-D have a PCS based architecture
  - Exception handling (Egemon)
  - Active mode control
  - Active profile control (rotation control via beams, NTV braking) informed by kinetic physics
  - Real-time MHD spectroscopy

# Columbia U. NSTX-U grant proposal research plans – drive collaborative research

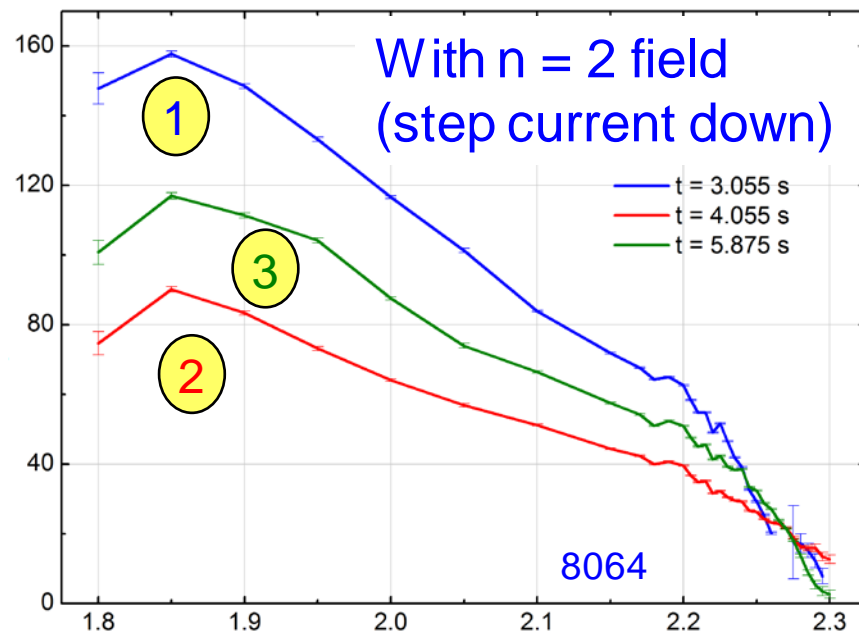
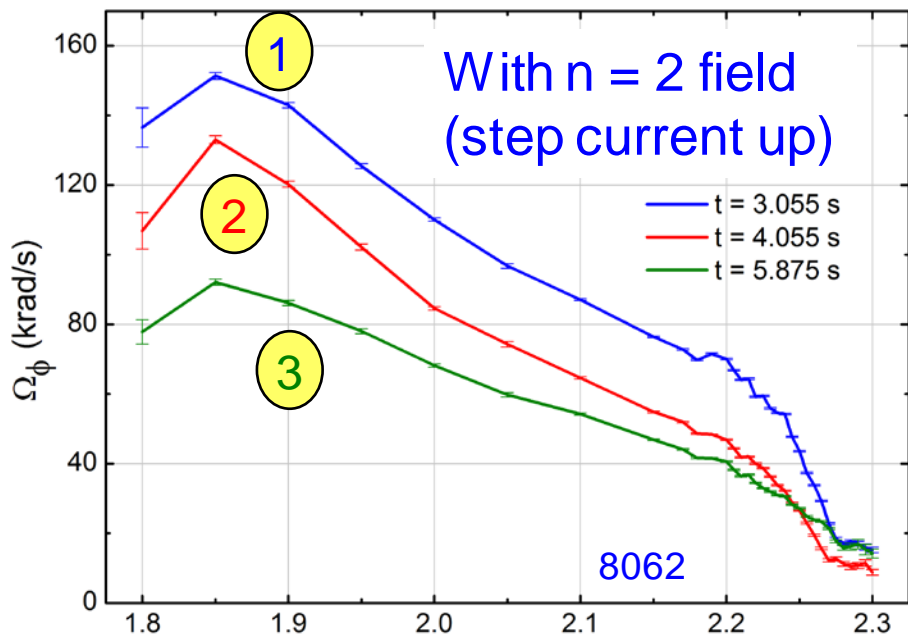
- ❑ **Physics research areas on NSTX-U**
  - ❑ Global MHD mode stabilization physics (incl. kinetic RWM physics)
  - ❑ Global MHD mode active control
  - ❑ Non-resonant plasma rotation alteration / physics / control (NTV)
  - ❑ NCC coil design
- ❑ **Related/coordinated research on DIII-D and KSTAR**
  - ❑ Aimed at verifying kinetic stabilization; limiting modes with TM stable
  - ❑ Aimed at long-pulse, high beta; higher aspect ratio of KSTAR provides opportunity for comparison to NSTX-U to determine role of A
- ❑ **Quantitative analysis for ITER cases, future devices**
  - ❑ New ITPA MDC-21: global mode stabilization / disruption avoidance)
- ❑ **Near-term analysis: continue analysis / publication of NSTX results, with related device/code benchmarking**

# Plasmas have reached and exceeded the predicted “closest approach” to the $n = 1$ ideal no-wall stability limit

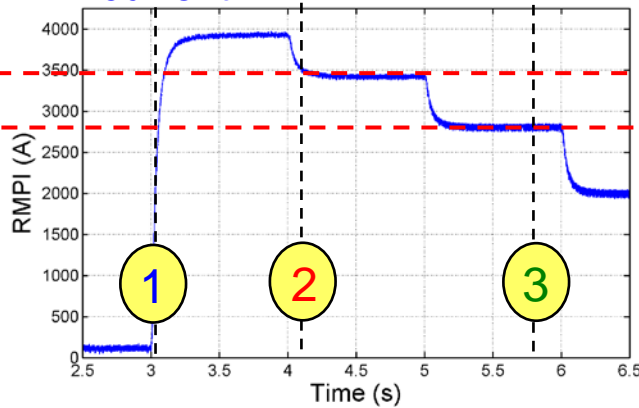
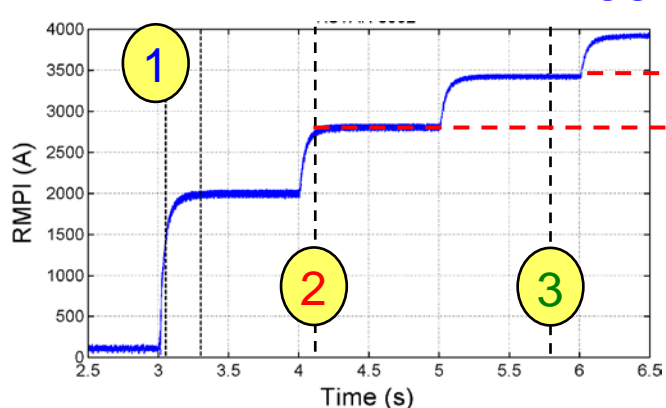


- $I_p$  scan performed to determine “optimal”  $\beta_N$  vs.  $I_p$ 
  - $B_T$  in range 1.3 - 1.5T
  - $\beta_N$  up to 3.0
- $\beta_N/I_i > 3$ . (80% increase from 2011)
  - a high value for advanced tokamaks, e.g. for DIII-D
- Mode stability
  - Target plasma is at published computed ideal  $n = 1$  no-wall stability limit (DCON)
  - Plasma is subject to RWM instability, depending on plasma rotation profile
  - Rotating  $n = 1, 2$  mode activity observed in core during H-mode

# Measured toroidal plasma rotation profile shows non-resonant NTV rotation control using $n = 2$ field



IVCC  $n = 2$  current



At same IVCC current, rotation profile shows **no hysteresis** – important for control

# Discussion topics related to kinetic RWM stabilization

- ❑ Stabilization physics due to fast particles should be further addressed
  - ❑ Effects of anisotropic (e.g. NBI, RF) particle populations still not fully explored
  - ❑ What are destabilization mechanisms (linear, or non-linear) that can be caused by fast particles?
  - ❑ Is stabilization accounted due to Maxwellian distributions complete?
  
- ❑ Long-standing issue – effect of key rational surfaces
  - ❑ Generally, numerical integration of ideal eigenfunction very close to rational surfaces does not yield agreement with experiment
  - ❑ In fact, omitting regions very close to key rationals yields results that compare better in quantitative comparison to experiment
  - ❑ Major task with theory: develop an improved model of the plasma near key rationals – **JRT14 should contribute to this**



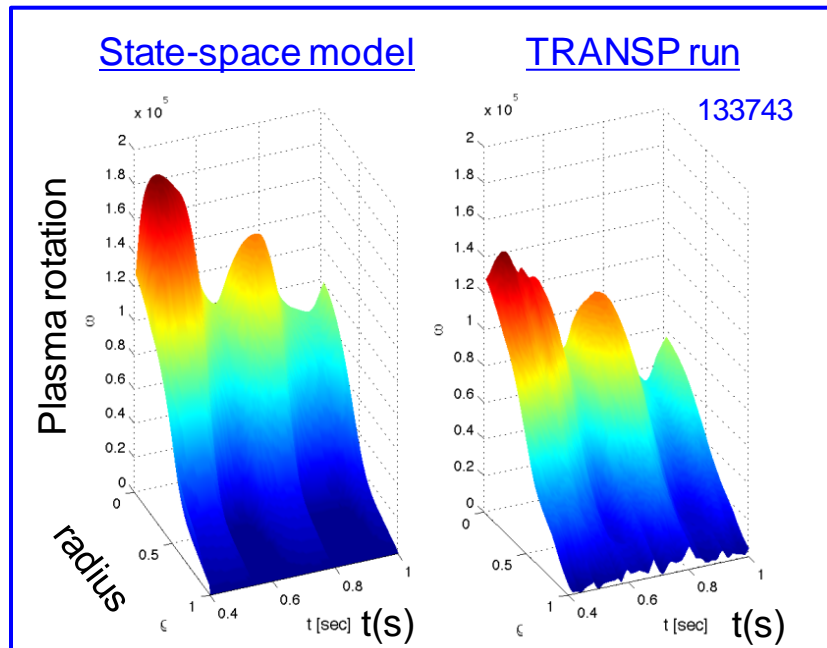
# Model-based, state-space rotation controller designed to use Neoclassical Toroidal Viscosity (NTV) profile as an actuator

- Momentum force balance –  $\omega_\phi$  decomposed into Bessel function states

$$\sum_i n_i m_i \langle R^2 \rangle \frac{\partial \omega}{\partial t} = \left( \frac{\partial V}{\partial \rho} \right)^{-1} \frac{\partial}{\partial \rho} \left[ \frac{\partial V}{\partial \rho} \sum_i n_i m_i \chi_\phi \langle (R \nabla \rho)^2 \rangle \frac{\partial \omega}{\partial \rho} \right] + T_{NBI} + T_{NTV}$$

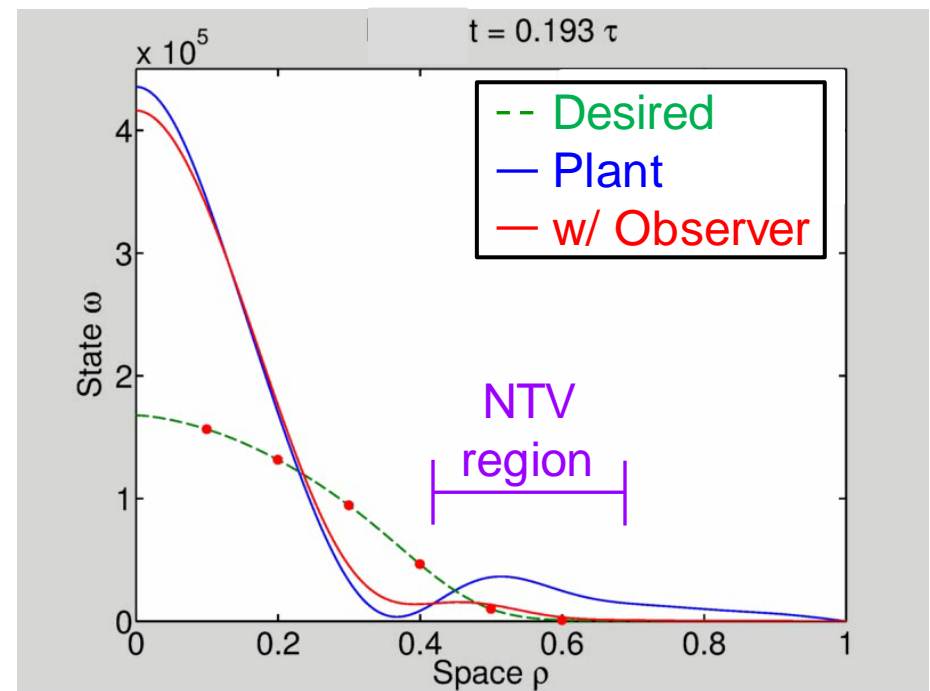
- NTV torque:

$$T_{NTV} \propto K \times f(n_{e,i}^{K1} T_{e,i}^{K2}) g(\delta B(\rho)) [I_{coil}^2 \omega] \quad \text{(non-linear)}$$



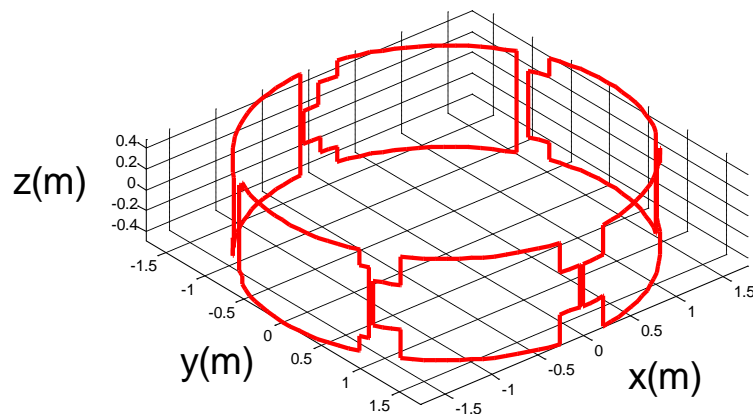
I. Goumiri

## Feedback using NTV: “n=3” $\delta B(\rho)$ spectrum



# Expanded NTV torque profile model for control being developed from theory/comparison to experimental data

## NSTX 3D coils used for rotation control

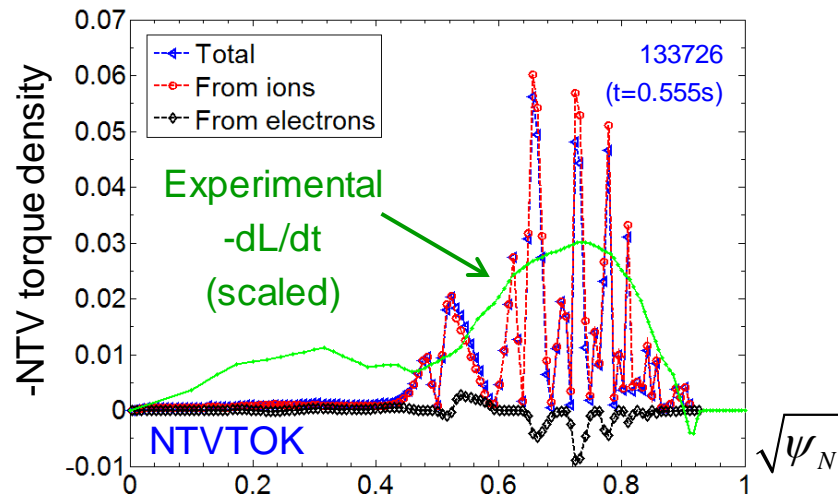


## □ New analysis: NTVTOK code

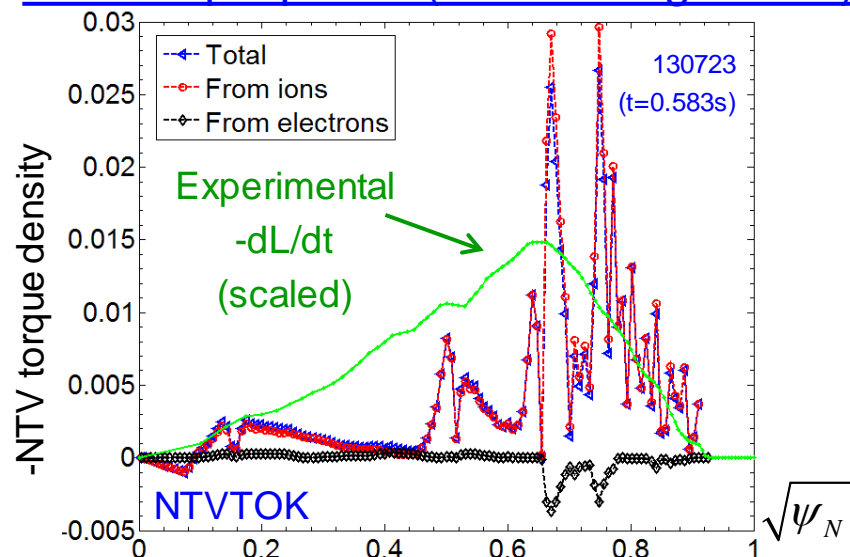
(Sun, Liang, Shaing, et al., NF 51 (2011) 053015)

- Shaing's connected NTV model, covers all  $\nu$ , and superbanana plateau regimes (Shaing, Sabbagh, Chu, NF 50 (2010) 025022)
- Past quantitative agreement with theory found in NSTX for plateau, "1/ $\nu$ " regimes (Zhu, Sabbagh, Bell, et al., PRL 96 (2006) 225002)
- Full 3D coil specification, ion and electron components considered, no A assumptions

## NTV torque profile (n = 3 configuration)



## NTV torque profile (n = 2 configuration)



# Discussion topics related to model-based RWM state-space control (RWMSC)

- ❑ Present NSTX RWMSC will be upgraded by the Columbia U. group for NSTX-U
  - ❑ Upgrade includes independent control of present 6 RWM coils on NSTX-U, multi-mode control capability ( $n = 1 - 3$ ), upgrade path to NCC, etc.
- ❑ Can real-time plasma response model be expanded?
  - ❑ Present model is the Boozer ( $s, \alpha$ ) model
    - E.g. kinetic stabilization effects can be included directly through this model
    - However, basic eigenfunctions are chosen a priori and are then altered by this response model. Can this specification be made more generally?
  - ❑ Present modes and plasma response model are for ideal linear eigenfunctions. Can more general eigenfunctions be specified? (e.g. non-linear saturated?)
    - What would be the appropriate plasma response model in this case?

# Discussion topics related to non-linear MHD code analysis

- ❑ Stability in the presence of a toroidal resistive wall
  - ❑ Much experimental experience in NSTX – can test code using existing data
  - ❑ M3D-C<sup>1</sup>: resistive wall model is close (Steve meeting w/ Nate this week, beta test soon)
  - ❑ NIMROD: collaboration with S. Kruger and student
    - Model recently fixed, NSTX beta scan equilibria sent to Andi Becerra (met at APS 2013, plan defined with Hegna, Kruger, King)
- ❑ Differential rotation between wall and mode is highly desired
  - ❑ Physics studies of a fully locked mode are important, but differential rotation is needed to attempt to mimic kink stabilization dynamics
- ❑ Kinetic stabilization physics is highly desired
  - ❑ For comparison to present tested linear codes (implementation is obviously different!), as well as direct comparison to experiment
  - ❑ M3D-C<sup>1</sup>, NIMROD in different stages of development in this regard

# Testing kinetic RWM stabilization theory at marginal stability in high $\beta$ , TM stabilized plasmas (Idea #549)

## • Goals

1. Test present kinetic RWM stability theory on plasmas very near to, or at RWM marginal stability in DIII-D for confident extrapolation to ITER, DEMO
2. Determine RWM stability at high beta when TM is stable, or controlled

## • Deliverables

1. Direct verification of DIII-D RWM marginal stability point vs. theory, testing kinetic stabilization theory that changes stability dependence on  $\omega_\phi$ ,  $v$  compared to earlier theory (changes extrapolation to future devices). Success would warrant a PRL-level publication.
2. Improvement of DIII-D high  $\beta_N$ , high  $(\beta_N, q_{min})$  steady-state plasmas, and best understanding/extrapolation to analogous ITER and DEMO scenarios
3. Direct input for new ITPA joint experiment MDC-21 and ITER Organization urgent need - disruption avoidance (combined DIII-D, NSTX, theory effort)

## • General Approach

1. Leverage plasmas at/near RWM marginal stability created in past MPS (150312, 149782)
2. With plasma at RWM marginal stability, vary kinetic stabilization by varying plasma rotation speed and profile, and collisionality; Use targets to minimize  $(\delta W_{kin}/\delta W_{ideal})$  (e.g. reduce  $\delta$ )
3. Apply ECCD TM stabilization to plasmas approaching RWM marginal stability that transition to TM instability instead of RWM instability

(run time request: 1 day)

# DIII-D High $\beta_N$ , $q_{\min}$ shots show reproducible rotating RWM dynamics (target plasma #1)

- **Apparent unstable RWMs**

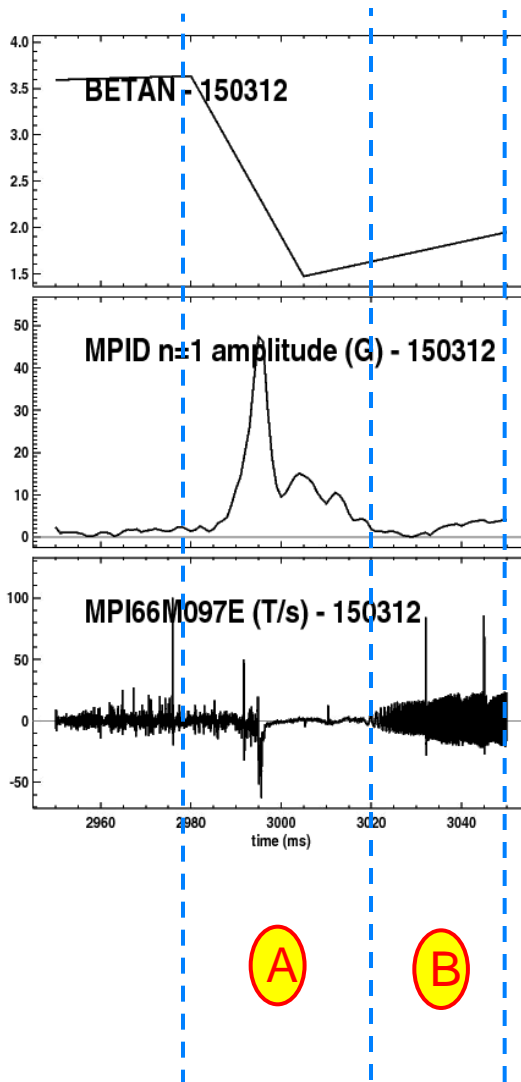
**(A)** – Mode growth time and phase evolution consistent with RWM dynamics

**(B)** – Evolution to saturated mode activity (reaches 5 kHz at end of this time period)

- **ECEI data** indicates that a TM forms at  $t = 3.015s$

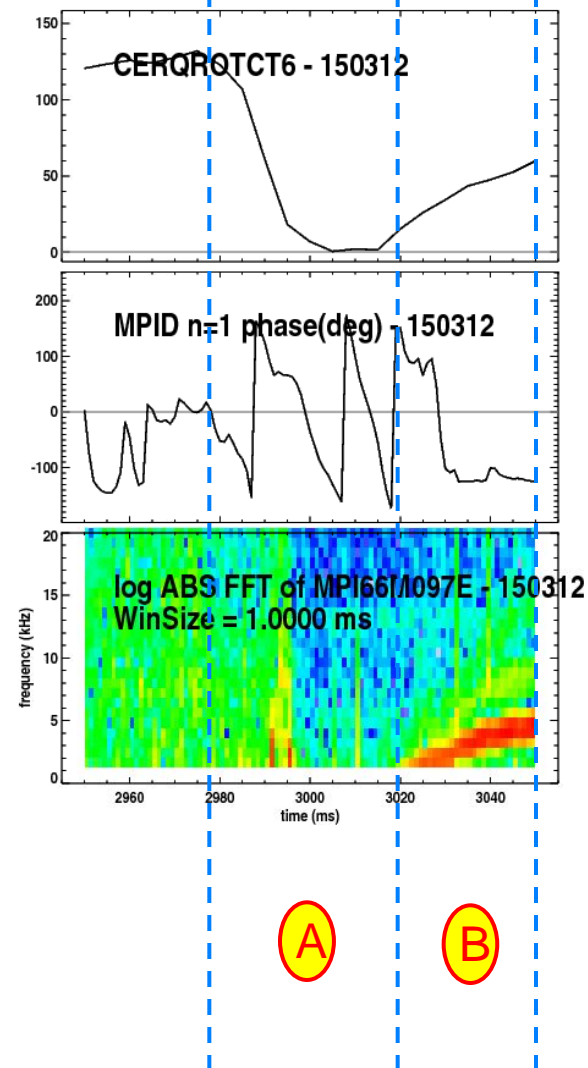
- Significantly after apparent RWM destabilization
- Consistent with magnetics

- Request made for SXR data



**(A)**

**(B)**



**(A)**

**(B)**



# Next steps for analysis / preparation for experiment

- **Determine key physical cause of difference in RWM stability of high  $\beta_N$ , high  $q_{\min}$  plasmas vs. more stable high  $\beta_N$  plasmas**
  - Stability analysis of unstable shot (e.g. 150312) completed
  - Compare to high beta, stable shots 133103 ( $I_i = 0.73$ ,  $\beta_N = 3.9$ ) and (RWM marginally stable?) shot 147634 (equilibria being prepared by J. Hanson)
- **Determine best high  $\beta_N$  target plasmas to test TM control**
  - Started discussion with Ege, Rob, Richard, Ted, Michio
    - Minimum  $B_T$  may be  $\sim 1.5$  T as in past high  $q_{\min}$  targets, can raise  $B_T$  from here
  - TM stabilization has been demonstrated in high  $\beta_N$  plasmas at 20% over the no-wall limit (e.g. M. Okabayashi, et al. NF **49** (2009) 125003 ); further development will be important part of this MP.
- **Follow general guidance to use mainline DIII-D operational scenarios if possible**
  - High  $\beta_N$ ,  $q_{\min}$  scenario is a main scenario for steady-state high  $\beta_N$  goals in 5 year plan



# Grant Research

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# List of physics topics / codes in use / planned

## ❑ Kinetic RWM stabilization

- ❑ MISK (linear kinetic RWM stability code)
- ❑ M3D-C<sup>1</sup>, NIMROD: further linear benchmarking, and non-linear runs for NSTX-U (e.g. NIMROD: toroidal resistive wall tests)

## ❑ Active RWM control

- ❑ RWM state-space controller development (incl. multi-mode)
- ❑ VALEN / mmVALEN (multi-mode)

## ❑ Non-resonant NTV physics (focus on active rotation control)

- ❑ NTVTOK (Shaing et al. formulation, connecting collisionality regimes)

## ❑ Equilibrium development

- ❑ NSTX EFIT – further development for NSTX-U

## ❑ DIII-D/KSTAR data/analysis – closely coupled to NSTX-U

- ❑ Same analysis tools applied on related physics topics to investigate (i) aspect ratio dependence, (ii) long-pulse aspects

# Planned analysis builds from present capabilities and collaborative work

## □ Equilibrium

- Free-boundary: NSTX EFIT
- Fixed boundary: CHEASE (w/Liu), JSOLVER, etc.

## □ Stability

- DCON, PEST: ideal linear stability analysis
- MISK (w/R. Betti): kinetic RWM stability analysis
- M3D-C<sup>1</sup> (w/S. Jardin, N. Ferraro): linear/non-linear stability
- NIMROD (w/S. Kruger): recent collaboration started - NSTX cases being run

## □ 3D Physics

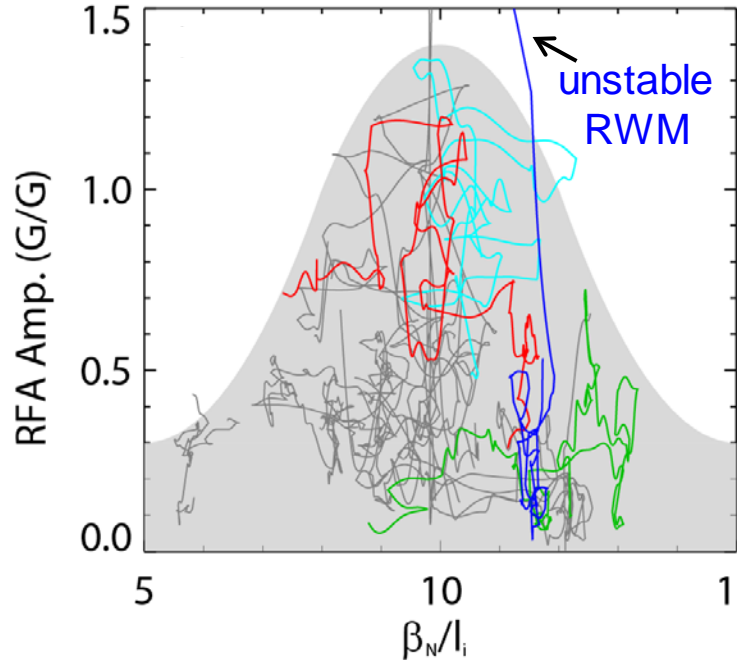
- NTVTOK: NTV code on CU computer, used present NSTX data analysis
- TRIP3D (w/T. Evans): ELM mitigation – used for KSTAR, etc.
- M3D-C<sup>1</sup> (w/S. Jardin): global mode stability, effect of 3D field on stability, (w/ T. Evans, N. Ferraro, S. Jardin): plasma response

## □ Control

- VALEN: RWM / dynamic error field control analysis
- Multi-mode VALEN: Unstable MHD mode spectrum and control
- RWMSC: State-space RWM analysis / feedback control

# Experiments directly measuring global stability using MHD spectroscopy (RFA) support kinetic RWM stability theory

Resonant Field Amplification vs.  $\beta_N/I_i$

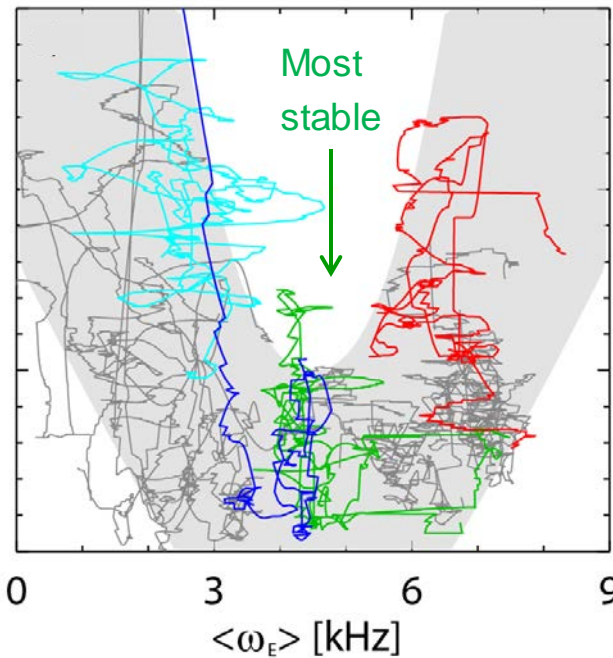


(trajectories of 20 experimental plasmas)

## Stability vs. $\beta_N/I_i$

- ▣ **decreases** up to  $\beta_N/I_i = 10$ , **increases** at higher  $\beta_N/I_i$
- ▣ Consistent with kinetic resonance stabilization

RFA vs. rotation ( $\omega_E$ )



## Stability vs. rotation

- ▣ Largest stabilizing effect from ion precession drift resonance with  $\omega_\phi$

Minimize  $|\langle \omega_D \rangle + \omega_E|$

$$\delta W_k \sim \frac{1}{\langle \omega_D \rangle + \omega_E - i\nu_{eff}}$$

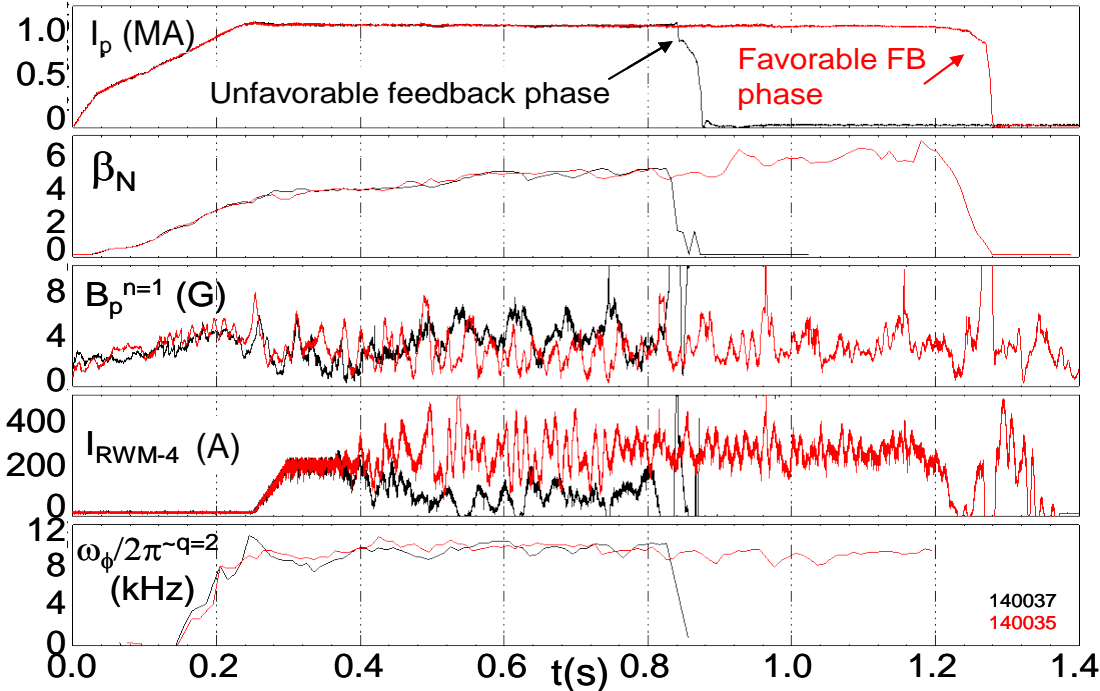
## Stability at lower $\nu$

- ▣ Collisional dissipation is reduced
- ▣ Stabilizing resonant kinetic effects are **enhanced**
- ▣ **Stabilization** when near broad  $\omega_\phi$  resonances; almost no effect off-resonance

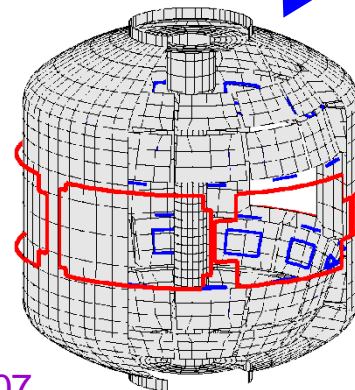
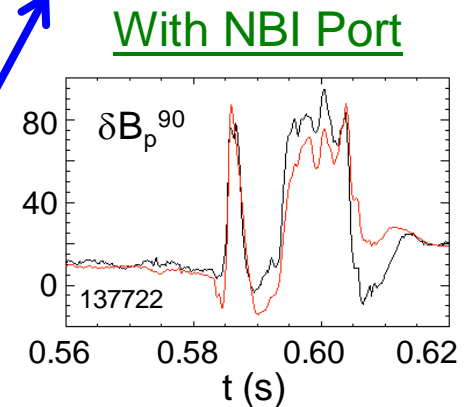
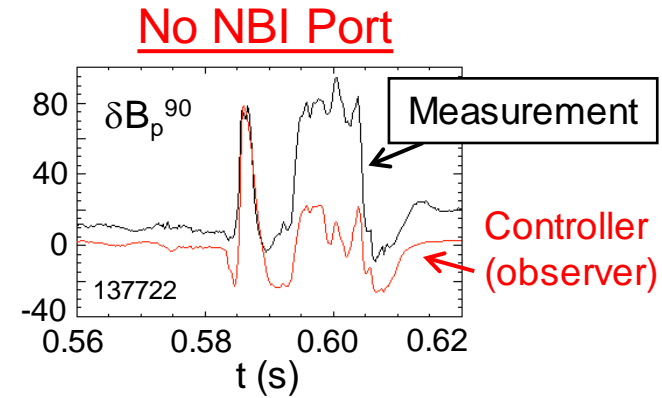
S. Sabbaghet al., NF 53 (2013) 104007

# Model-based RWM state space controller including 3D plasma response and wall currents used at high $\beta_N$ in NSTX

## RWM state space controller in NSTX at high $\beta_N$



## Effect of 3D Model Used



- Potential to allow more flexible control coil positioning

- May allow control coils to be moved further from plasma, and be shielded (e.g. for ITER)

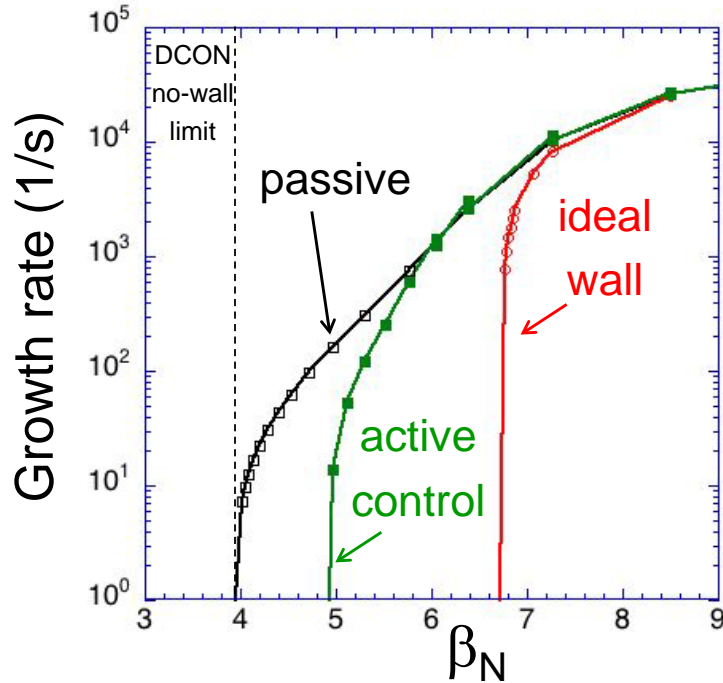
Katsuro-Hopkins, et al., NF 47 (2007) 1157

S.A. Sabbagh, et al., Nucl. Fusion 53 (2013) 104007

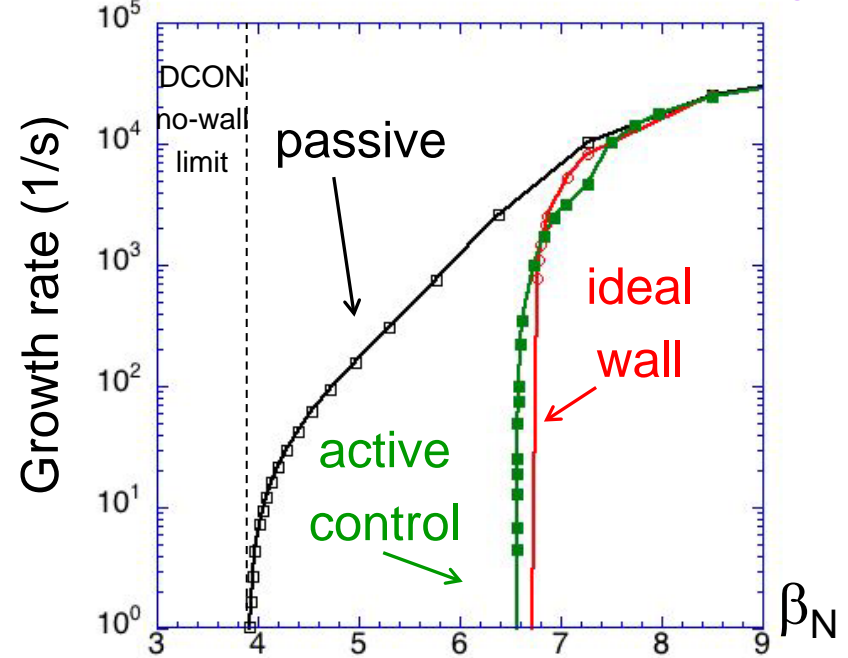
- 3D detail of model is important to improve sensor agreement

# RWM active control capability will increase significantly when Non-axisymmetric Control Coils (NCC) are added to NSTX-U

Using present midplane RWM coils

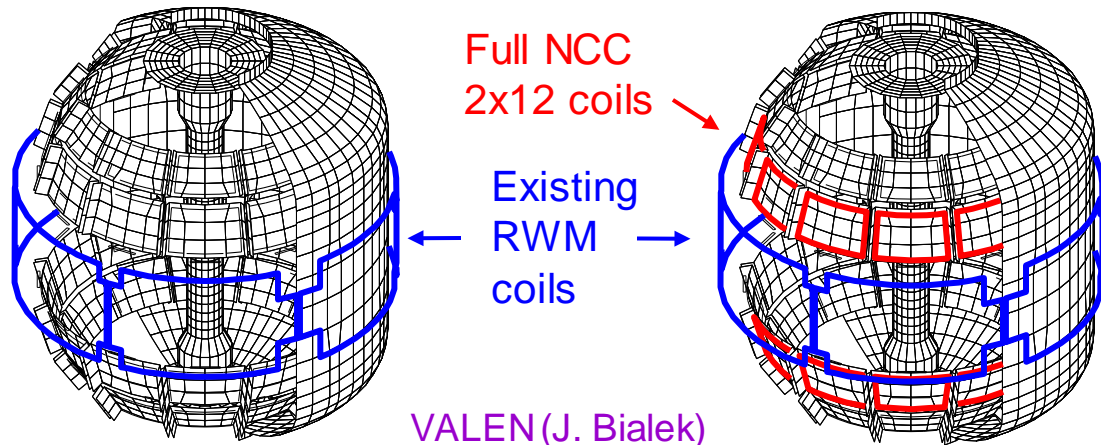


NCC 2x12 with favorable sensors, optimal gain



## Performance enhancement

- Present RWM coils: active control to  $\beta_N/\beta_N^{\text{no-wall}} = 1.25$
- Add NCC 2x12 coils, optimal sensors: active control to  $\beta_N/\beta_N^{\text{no-wall}} = 1.67$
- Partial NCC options also viable



VALEN(J. Bialek)

# Comments and discussion topics related to NCC design analysis for NSTX-U

- ❑ The present approach of combining key figures of merit to produce a multi-use coil system is the correct one
- ❑ We need to be careful that further analysis / results in the coming year that influences the NCC design doesn't greatly decrease multi-use flexibility
  - ❑ E.g. recent DIII-D ELM suppression results (Orlov APS '13) indicate that a subset of I-coils are adequate for ELM control – NCC coil design should (and can) expand to test this, rather than downsize
  - ❑ Need to avoid similar potential issue based on unproven theory that might restrict physics studies rather than provide a coil to prove them
- ❑ Next steps for NCC design for RWM active control (CU plan)
  - ❑ Realistic sensors that minimize coupling to passive plates need to be designed and implemented in calculations of present NCC design
  - ❑ NSTX-U should have sensors with both weak and strong coupling to passive plates for RWM state-space controller physics studies

# Multi-year ITPA MDC-2 benchmarking of kinetic RWM codes reached the group's goals

	$r_{\text{wall}}/a$	Ideal $\delta W / -\delta W_{\infty}$	$\text{Re}(\delta W_k) / \delta W_{\infty}$	$\text{Im}(\delta W_k) / (\delta W_{\infty})$	$\gamma \tau_{\text{wall}}$	$\omega \tau_{\text{wall}}$	$\delta W_k / -\delta W_{\infty}$ ( $\omega_E = \infty$ )
<u>Solov'ev 1</u>	1.15						
(MARS-K)		1.187	0.0256	-0.0121	0.803	0.0180	0.157
(MISK)		1.122	0.0179	-0.0117	0.861	0.0189	0.160
<u>Solov'ev 3</u>	1.10						
(MARS-K)		1.830	0.0919	-0.169	0.471	0.114	1.98
(MISK)		2.337	0.0879	-0.090	0.374	0.051	1.10
<u>ITER</u>	1.50						
(MARS-K)		0.682	0.241	-0.046	0.817	0.090	6.11
(MISK)		0.677	0.367	-0.133	0.581	0.202	6.67

- Recent success in producing agreement between MISK and MARS-K
  - PENT code development added in 2013
- MISK code has been extensively used to quantitatively compare theory/experiment in NSTX
  - 6 publications from NSTX (first is from 2008)
- MDC-2 process has altered both MISK and MARS-K a bit
  - MISK comparison to NSTX RWM stability experiments continues to evaluate changes

# Analysis / code expansion driven by proposed research, NSTX-U device needs

## □ Equilibrium

- NSTX-U EFIT: expand diagnostics/model, increase (R,Z,t) resolution, speed
- CHEASE: (w/Liu), etc.: equilibrium refinement / exchange

## □ Stability

- DCON, PEST: ideal linear stability analysis (resistive DCON very close)
- MISK: continued quantitative development, driven by ITPA MDC-2 NSTX XP data
- M3D-C<sup>1</sup>: resistive wall available soon / desire for kinetic effects (compare to MISK)
- NIMROD: resistive wall / kinetic effects available – collaborative initial tests on NSTX cases with resistive wall underway with S. Kruger and UW student.

## □ 3D Physics

- NTVTOK: once NSTX analysis completed, will compare with IPEC and POCA codes
- TRIP3D: ELM mitigation – use for NSTX-U as desired
- M3D-C<sup>1</sup> (Jardin, Ferraro): desire resistive wall, and kinetic stabilization effects

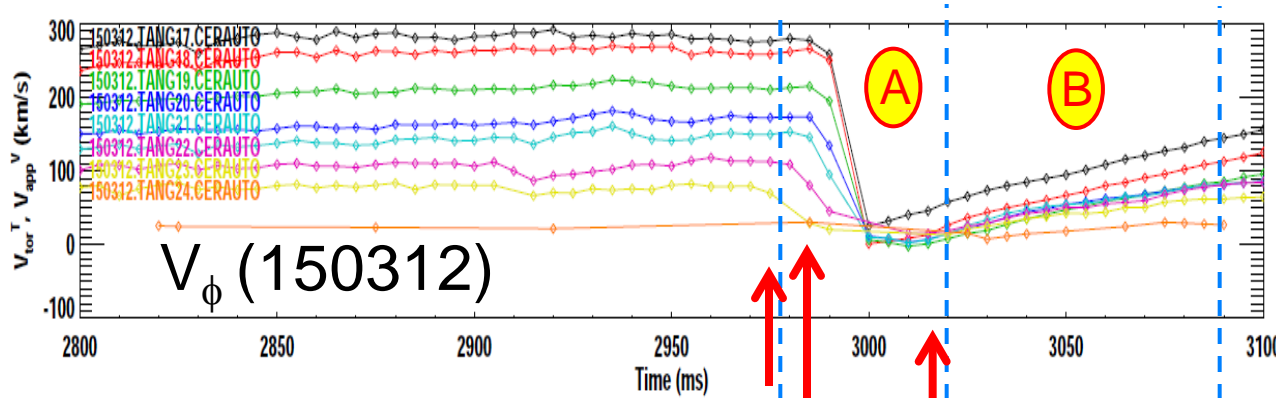
## □ Control

- VALEN: continue NSTX-U RWM control analysis (that has already begun)
- Multi-mode VALEN: multi-mode spectrum NSTX-U, active control w/RWMSC
- RWMSC:  $n > 1$  modeling + upgrades, control simulator w/expanded inputs
  - Inputs: Device data, vacuum field, code results (VALEN, M3D-C<sup>1</sup>, etc. )



# DIII-D experiments

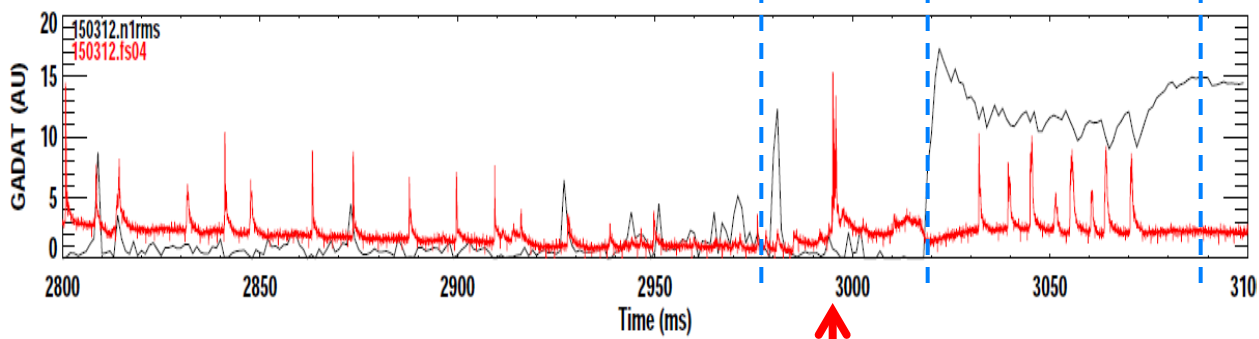
# High $\beta_N$ , $q_{\min}$ shots show reproducible minor disruptions, apparently caused by RWM activity (target plasma #1)



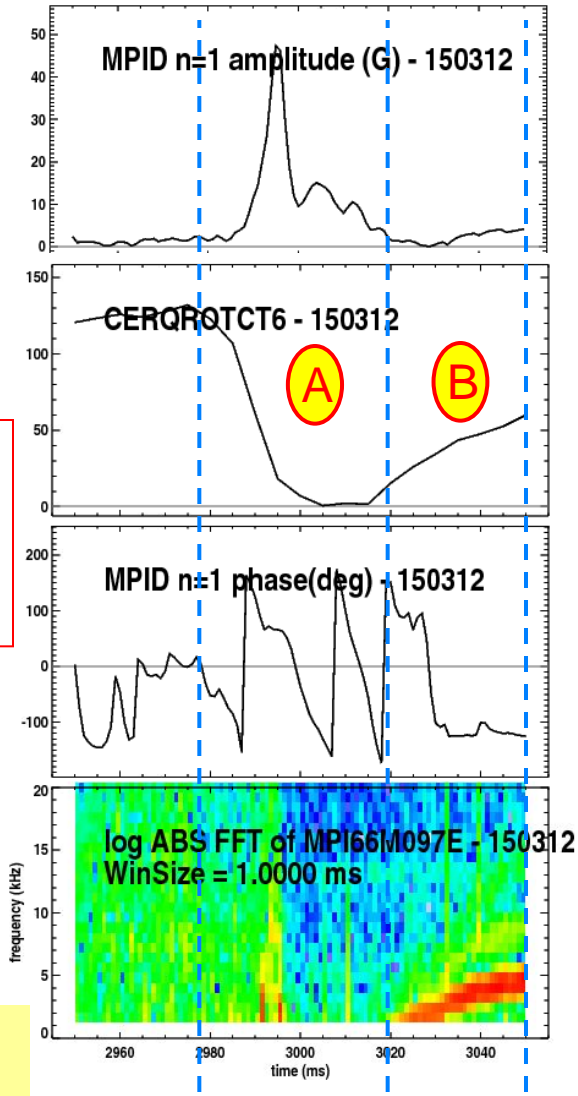
1) Drop in  $V_{\phi}$  corresponding RWM rotation, amplitude increase

2)  $V_{\phi}$  collapse is global (consistent with non-res NTV, no TM at this time)

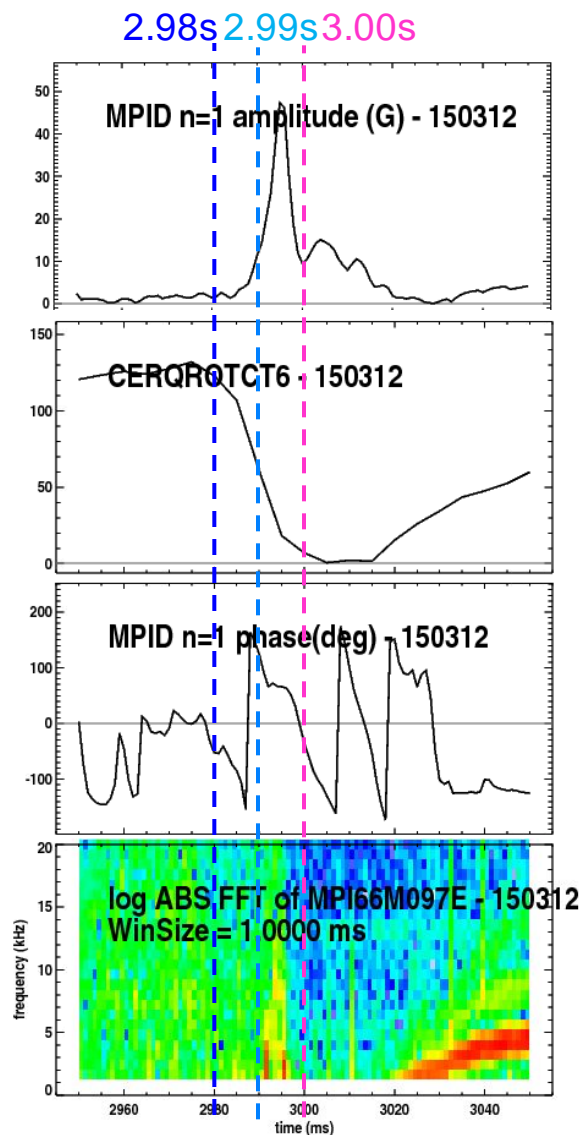
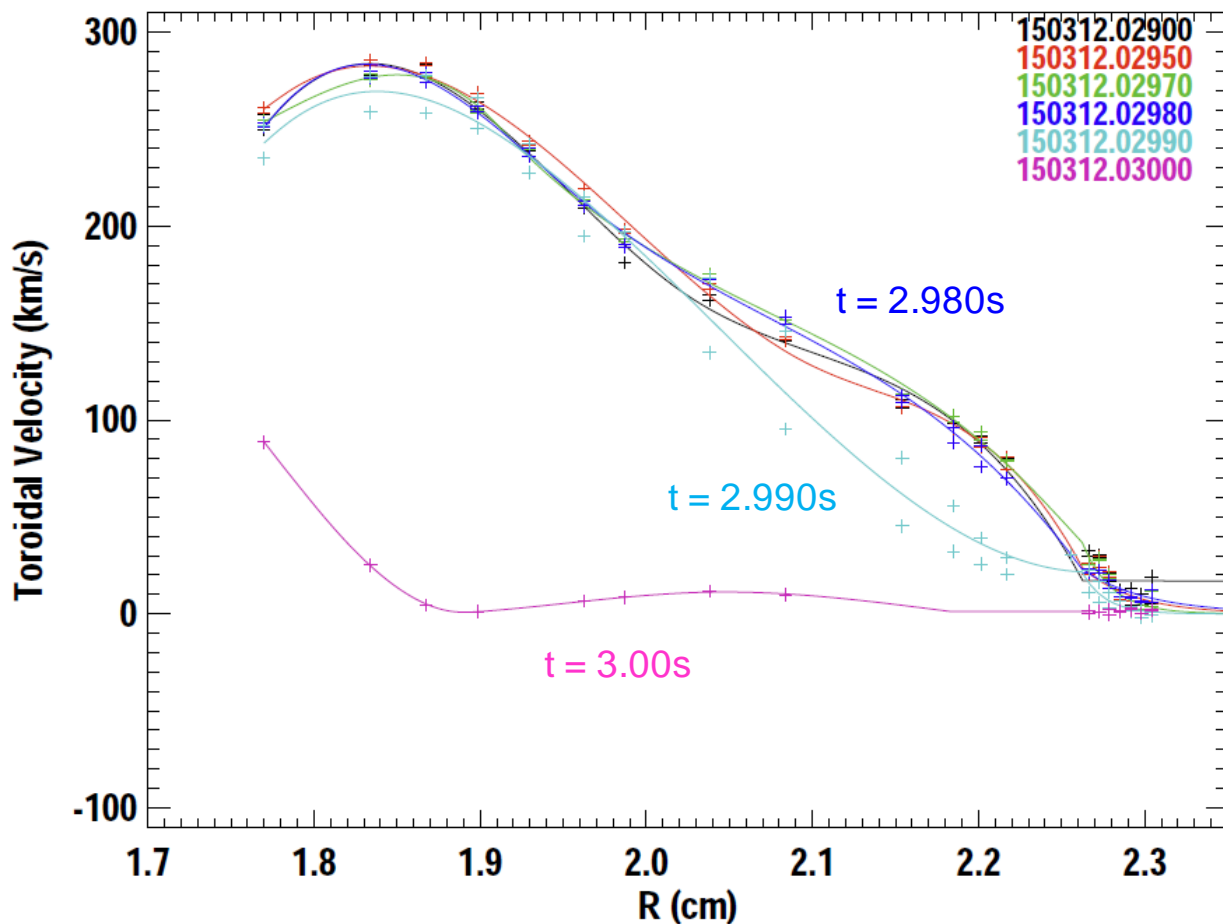
4) ECEI shows TM phase inversion here, spins up as  $V_{\phi}$  increases



3) "ELM" appears AFTER  $V_{\phi}$  collapse (apparently not causal)



# $V_\phi$ profile collapse at the time RWM growth appears to be a rapid, global collapse, rather than a resonant lock

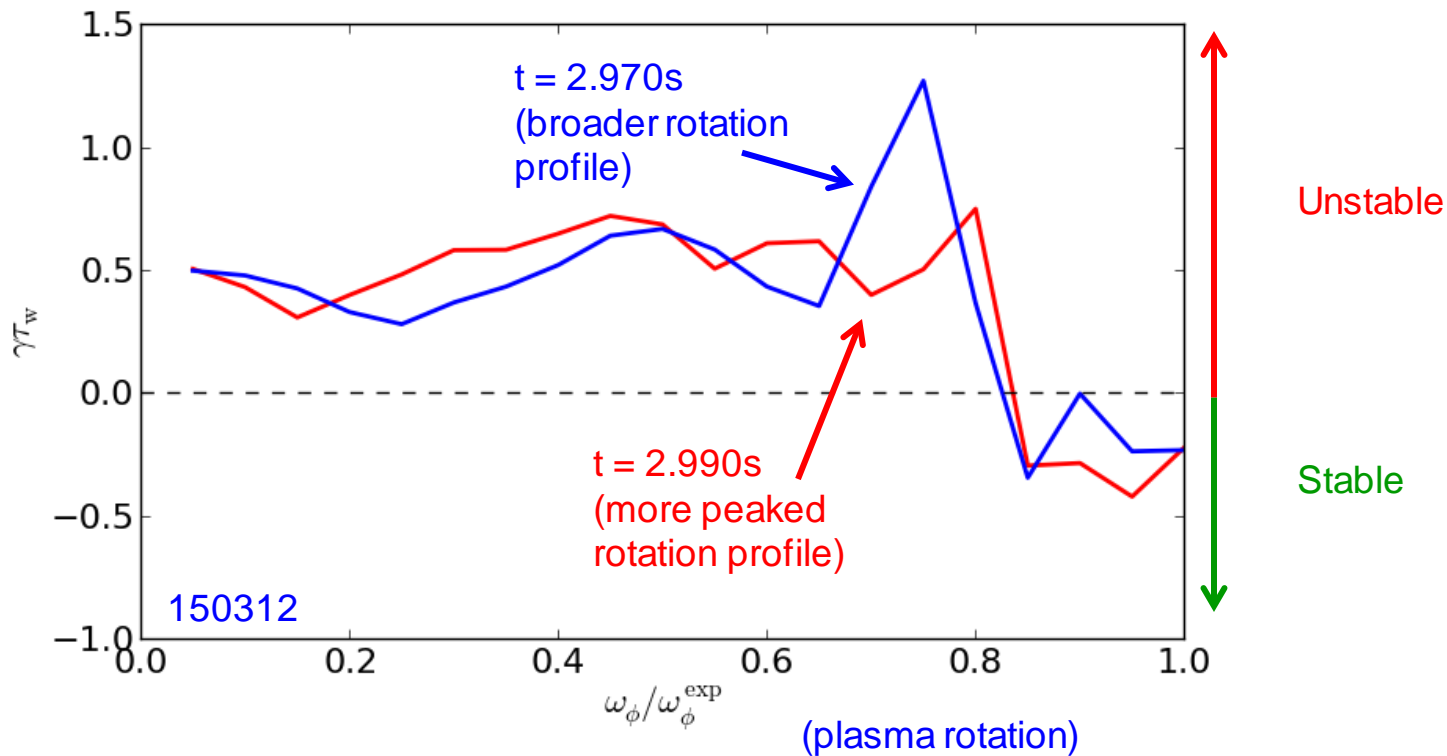


- Rotation damping characteristic of global RWM rather than a TM which would initially damp near a key rational surface



(B. Grierson)

# MISK kinetic RWM stability analysis of shot 150312 shows the plasma to be near marginal stability



- Rotation profile scaled from the experimental profile for the scan
- MISK analysis shows that these equilibria are near marginal stability (RWM unstable when rotation is reduced by  $\sim 10 - 20\%$ )

# High $\beta_N$ targets generated in past kinetic RWM run may become RWM unstable if TM is stabilized (Target plasma #2)

## 1 RFA of $\sim 20\text{Hz}$ $n = 1$ increase to very high levels

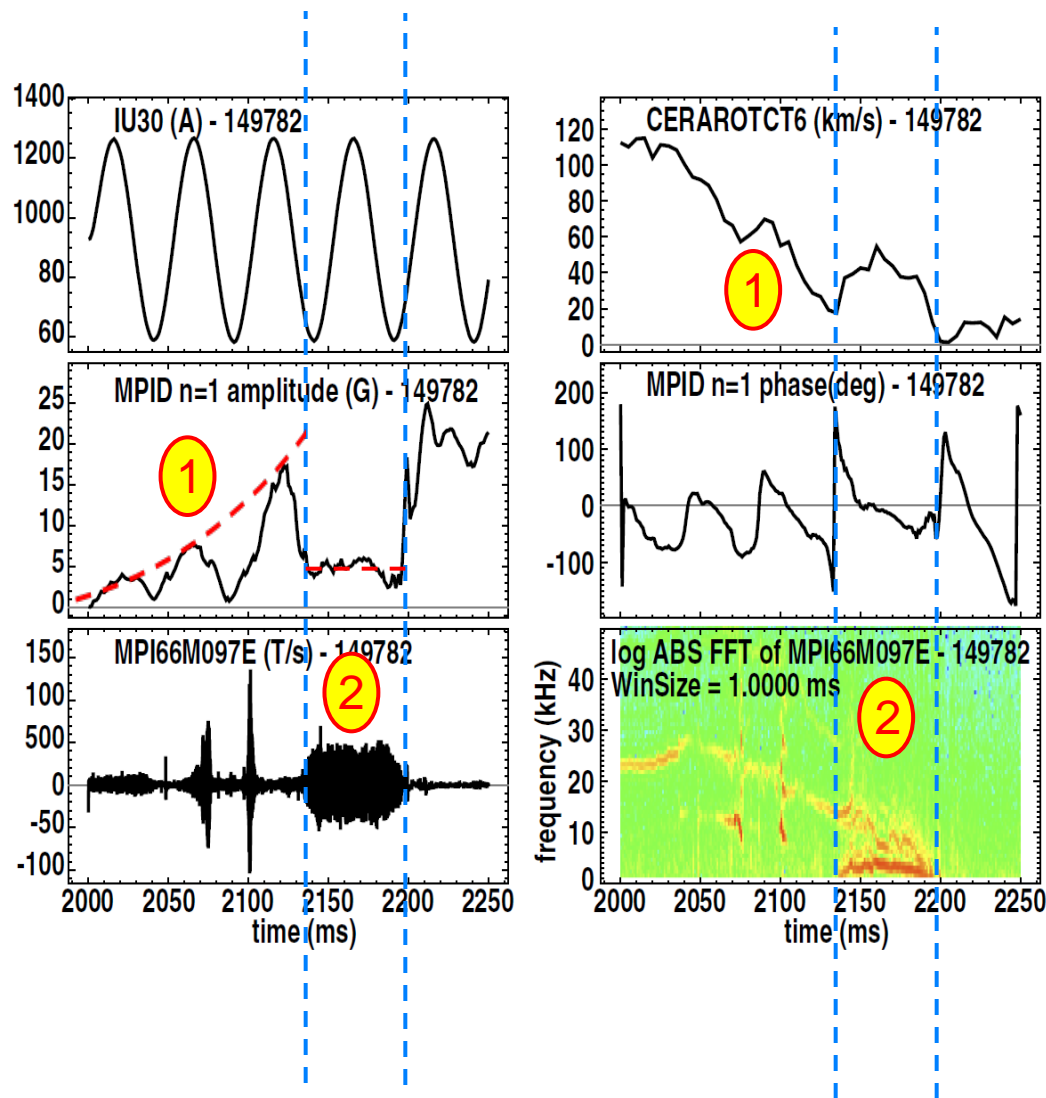
- Greater than **30G/kA** (past high values typically  $\sim 10\text{-}15\text{ G/kA}$ )
- Occurs at high  $\beta_N$  – pressure-driven
- Added counter-NBI power leads to slowing of plasma rotation (but no TM locking evident in  $V_\phi(R)$ )

## 2 Rapidly rotating $n = 1$ appears (TM?), clamps RFA amplitude

- Such activity precludes RWM instability in NSTX

## • Experimental plan

- Stabilize TM plasmas like this via ECCD – run at higher  $B_T \geq 1.5\text{ T}$
- Vary NBI source balance to change RWM marginal point (proximity to broad kinetic resonances) by changing rotation speed, and rotation profile



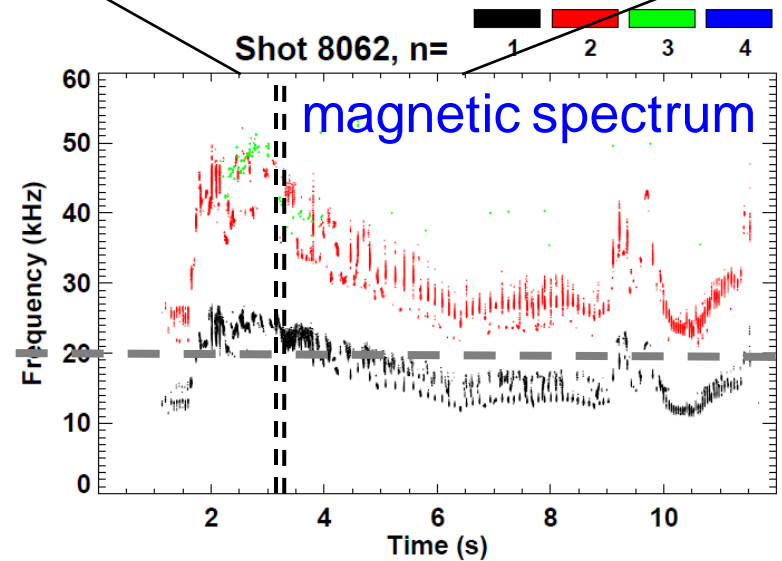
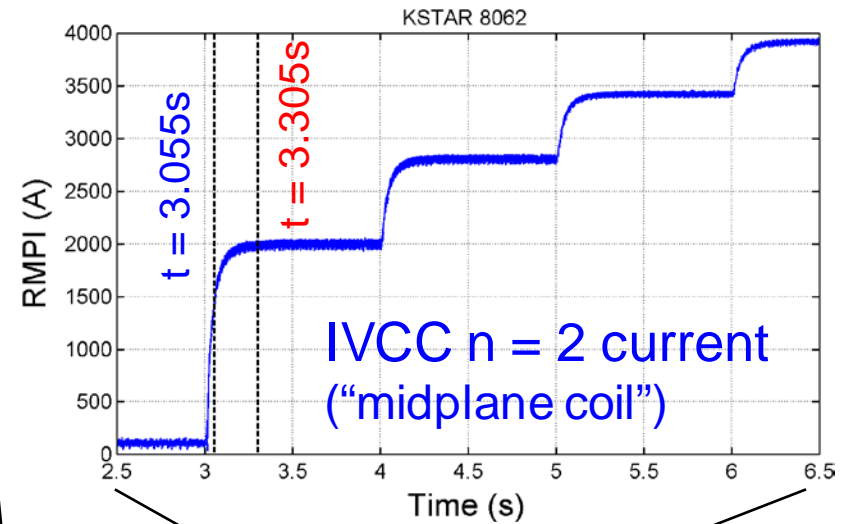
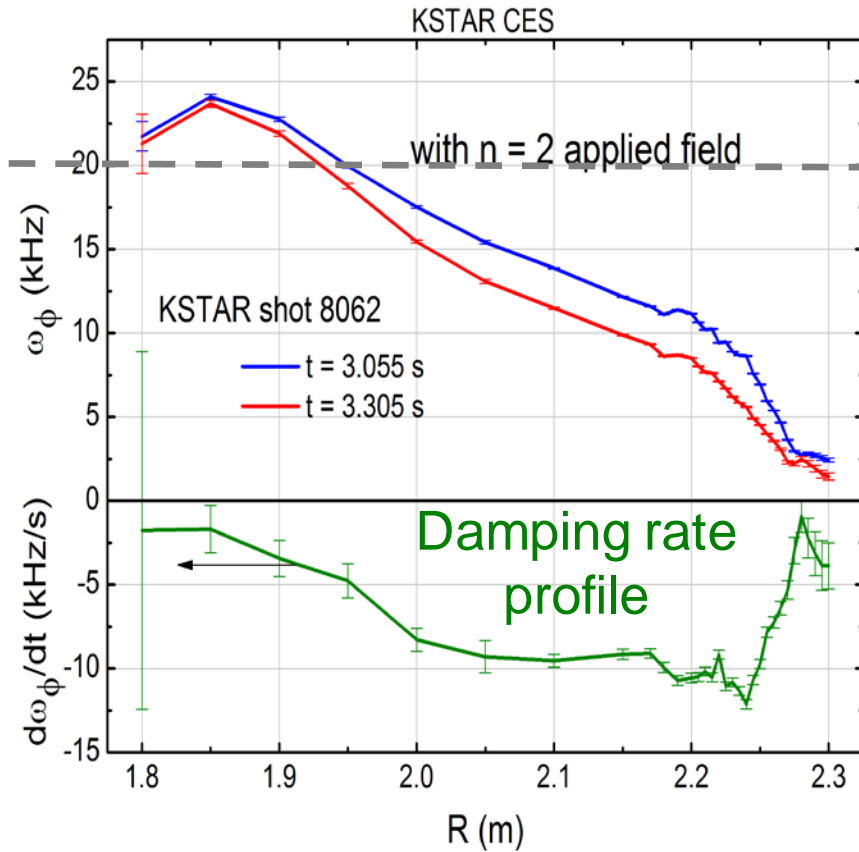
# Unstable modes with RWM characteristics often lead to strong thermal collapses in high $\beta_N$ , $q_{\min}$ plasmas

- **High impact:**
  - Cause large, rapid stored energy collapse  $\Delta W_{\text{tot}} \sim 60\%$  (200 MJ in ITER)
    - For comparison, large ELMs have  $\Delta W_{\text{tot}}$  up to 6% (20 MJ in ITER)
- **High probability:**
  - RWMs and TMs cause these collapses in 82% of the plasmas examined, with an average of 3 collapses every 2 shots (50+ shots examined)
  - RWMs cause collapse 60% of the time, TMs 40% of the time
- **The RWMs also destabilize TMs**
  - RWMs lead to large, rapid collapse of rotation, allowing EF penetration – TMs can be destabilized, typically spin up, but can then lock
  - Plasmas are favorable targets for this kinetic RWM stabilization study

# KSTAR experiments

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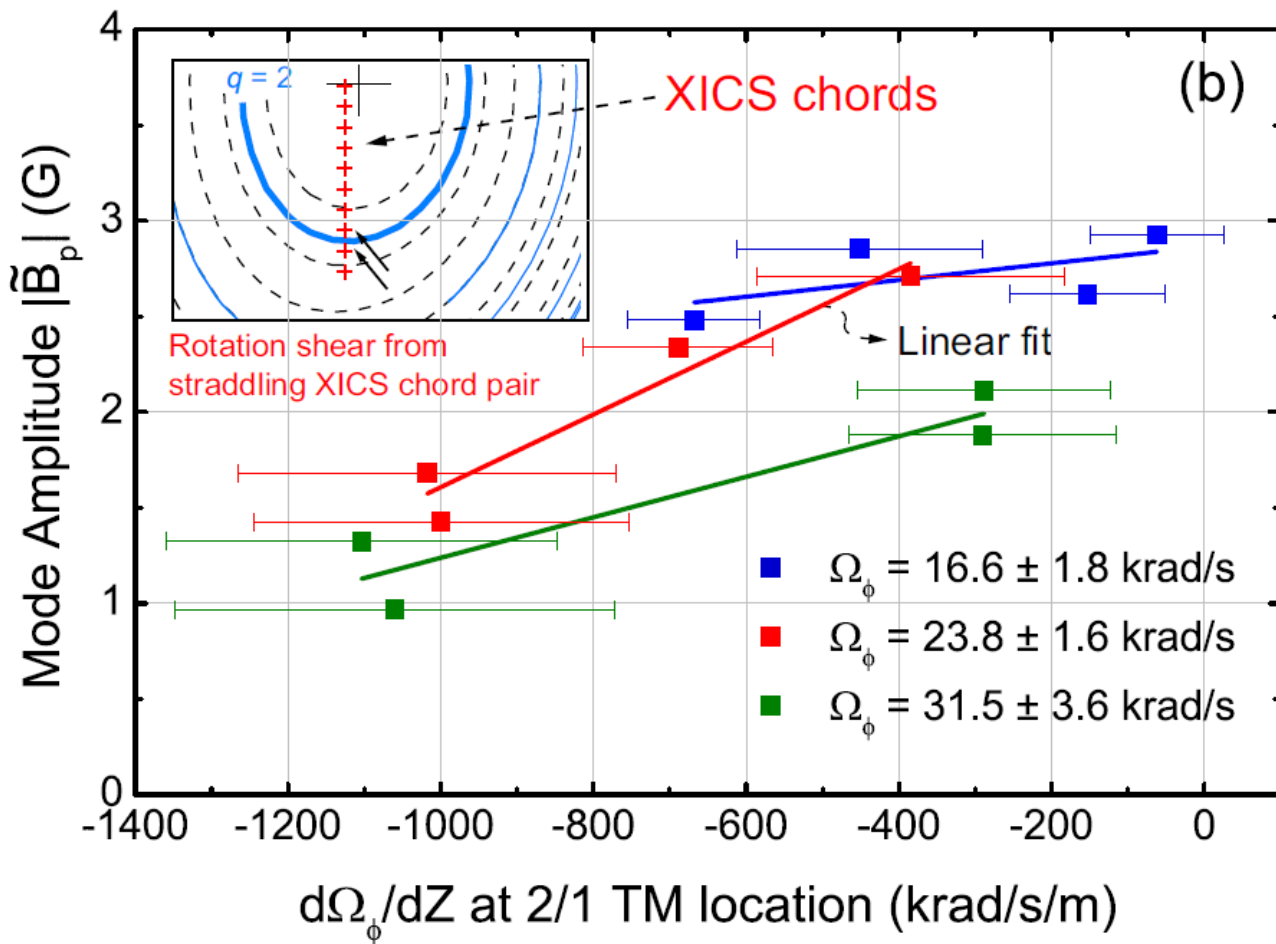
# Rotation reduction by $n = 2$ applied field is global and appears non-resonant (NTV); no mode locking



- Rotating  $n = 1, 2$  modes observed in core
  - would not produce the observed rotation profile change (no change in core)



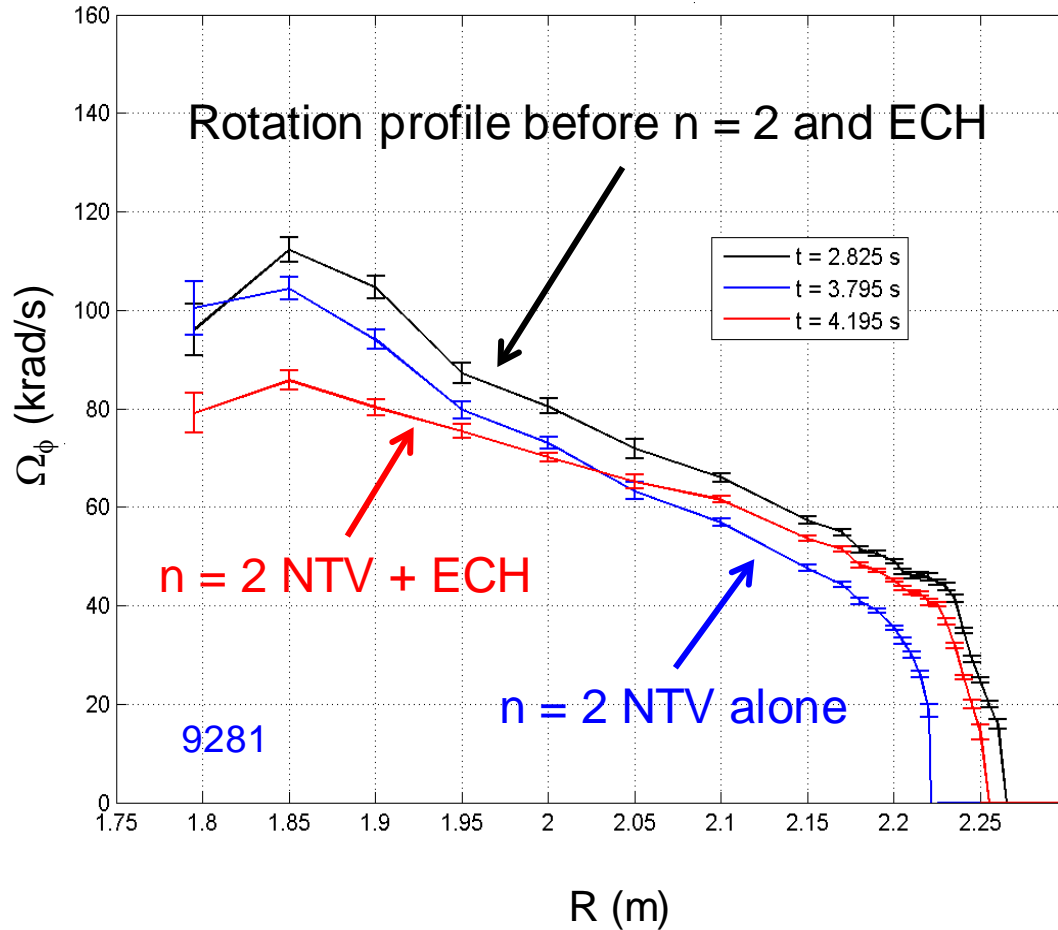
# Correlation between plasma velocity shear and 2/1 TM amplitude found



- Mode identification from ECE, ECEI systems
- Observed increase of TM amplitude vs.  $\Omega_\phi$  shear decreases at reduced  $\Omega_\phi$

Y.S. Park, S.A. Sabbagh, J.M. Bialek, et al. Nucl. Fusion **53** (2013) 083029

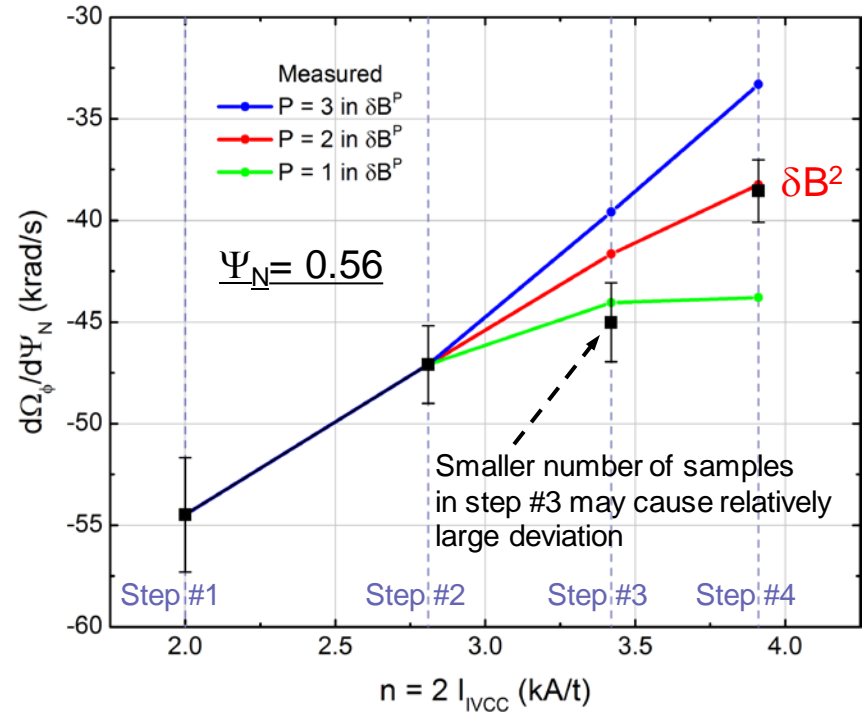
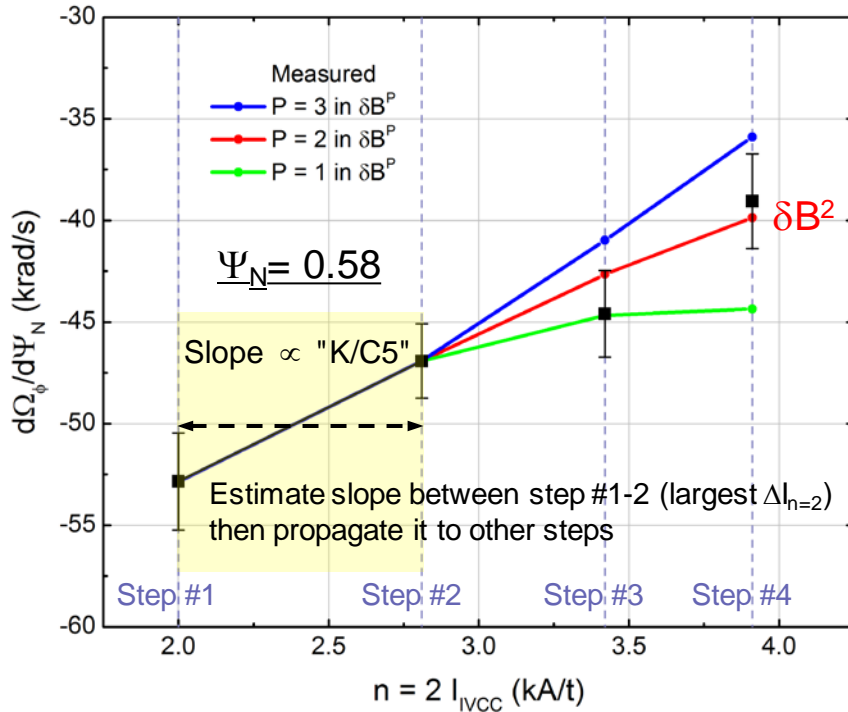
# Rotation profile alteration observed with combined IVCC $n = 2$ NTV + ECH (110 + 170 GHz)



- Combination of effects / timing can change  $\Omega_\phi$  shear
- Note:  $n = 2$  NTV current was run at only  $\frac{1}{2}$  maximum (not max. effect)
- $n = 2$  NTV shows global rotation damping
- Addition of ECH drops core rotation, increases edge  $\Omega_\phi$ , decreases shear
- Outward momentum transfer

# Steady-state profile analysis to examine NTV dependence on $\delta B$

- Resulting NTV correlation with different power in  $\delta B^P$



- For the different normalized flux surfaces,  $T_{NTV}$  scales well with  $\delta B^2$  as similar to collisionless "1/v" regime in NTV theory

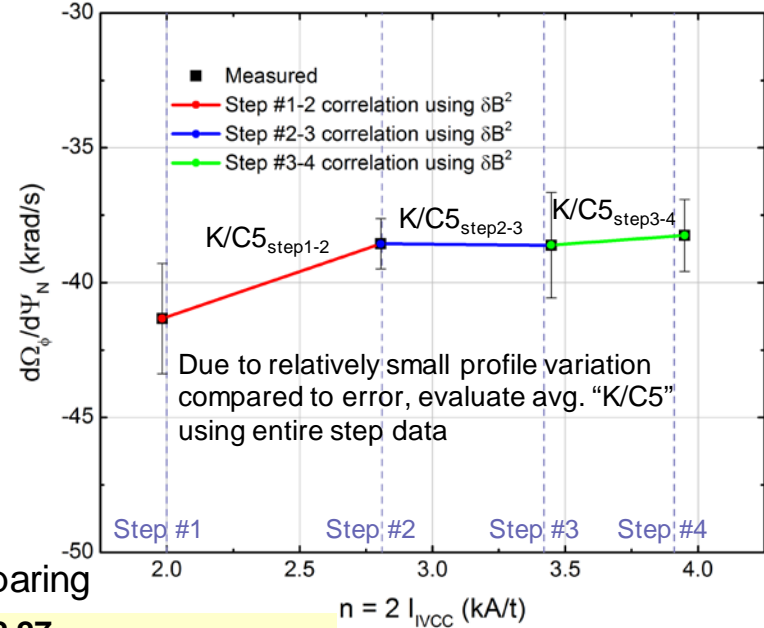
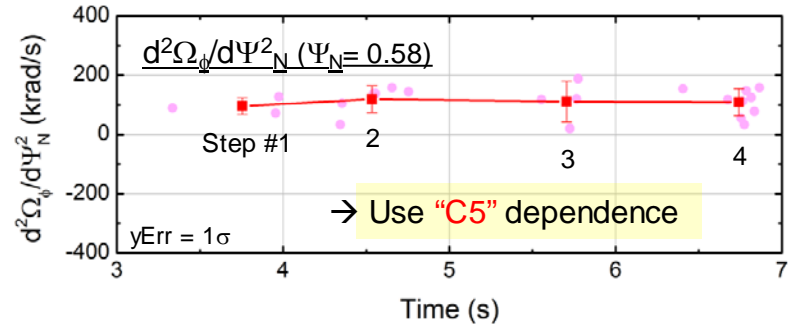
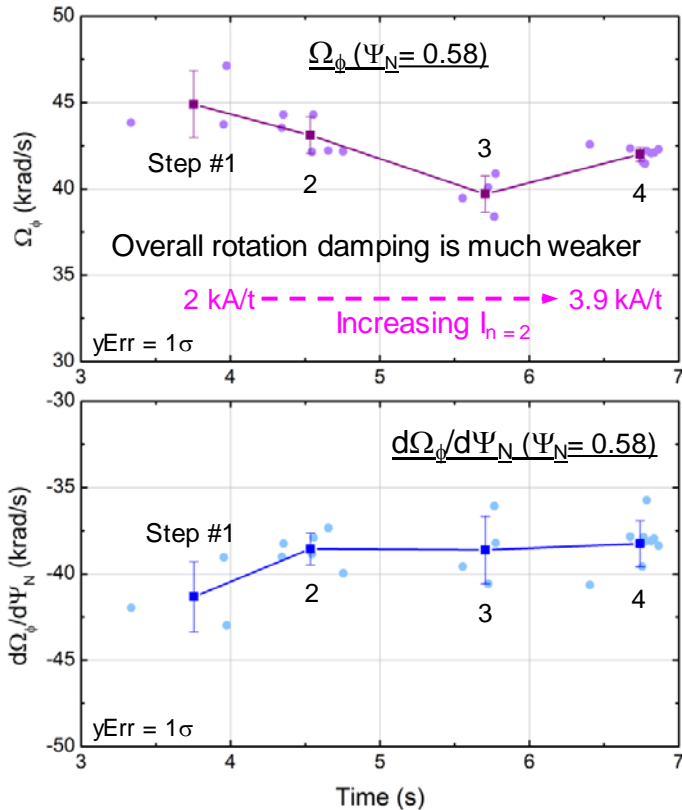
$$T_{NTV-(1/v)} \propto \delta B^2 T_i^{5/2}$$

Y.S. Park, APS DPP 2013

# Reduced rotation braking correlates with lower $T_i$ when NTV scales as $\delta B^2$

## Analysis of increasing $n = 2$ current steps with lower $T_i$ (shot 9199)

Chosen profiles have  $|\Delta T_i| < 50$  eV



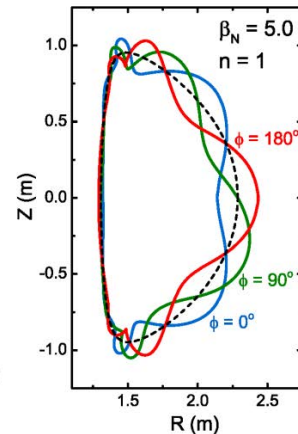
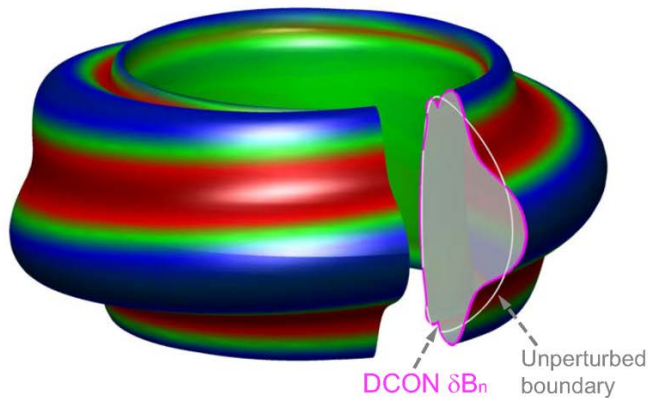
By assuming the same  $C5_{\Psi_N=0.58}$  between two comparing shots,

$$\frac{K_{\Psi_N=0.58}^{8062}}{K_{\Psi_N=0.58}^{9199}} = 6.02 = \left( \frac{T_{i, \Psi_N=0.58}^{8062}}{T_{i, \Psi_N=0.58}^{9199}} = \frac{1262 \text{ eV}}{573 \text{ eV}} \right)^{2.27} \approx T_{NTV-(1/\nu)} \propto T_i^{2.5}$$

Y.S. Park, APS DPP 2013

# M3D-C<sup>1</sup> code example for KSTAR, comparison to DCON

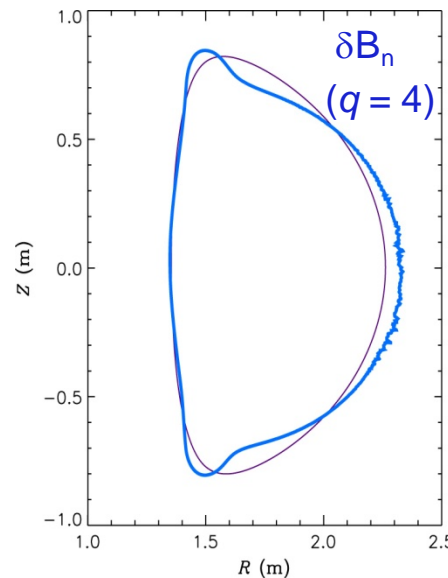
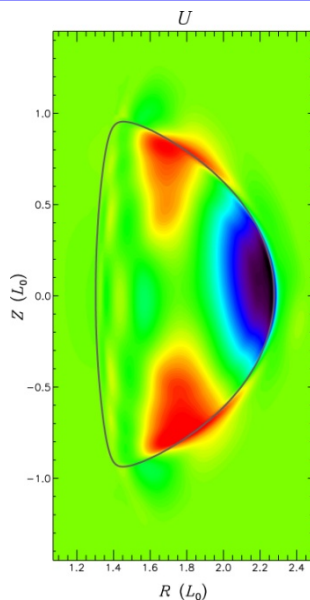
## KSTAR DCON $n = 1$ unstable mode eigenfunction



Linear stability analysis using M3D-C<sup>1</sup> code (collaboration with S. Jardin)

- Extended MHD code solving full two-fluid MHD equations in 3D geometry
- Non-linear code, presently being used in linear mode for initial runs

## M3D-C<sup>1</sup> unstable mode velocity stream function and $\delta B_n$



Ideal  $n = 1$  stability limit from DCON and M3D-C<sup>1</sup> compare well

- For the same input equilibria, “equivalent” wall configurations compared
- With-wall  $n = 1$  stability limit computed as  $\beta_N \sim 5.0$  in both calculations
- Further M3D-C<sup>1</sup> calculations for KSTAR will include improved wall configurations (3D, resistive wall) and analysis for resistive instabilities