

# XP1029: $P_{LH}$ dependence on $R_x$

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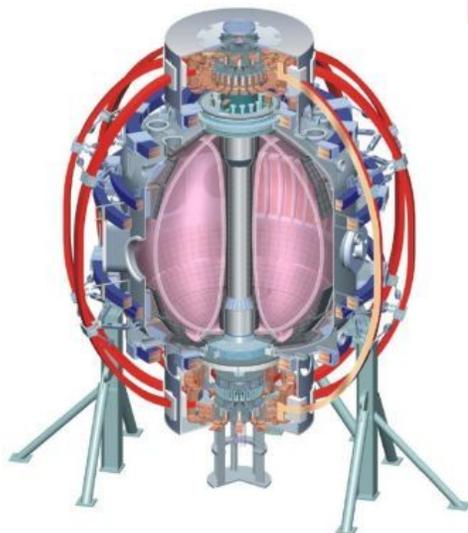
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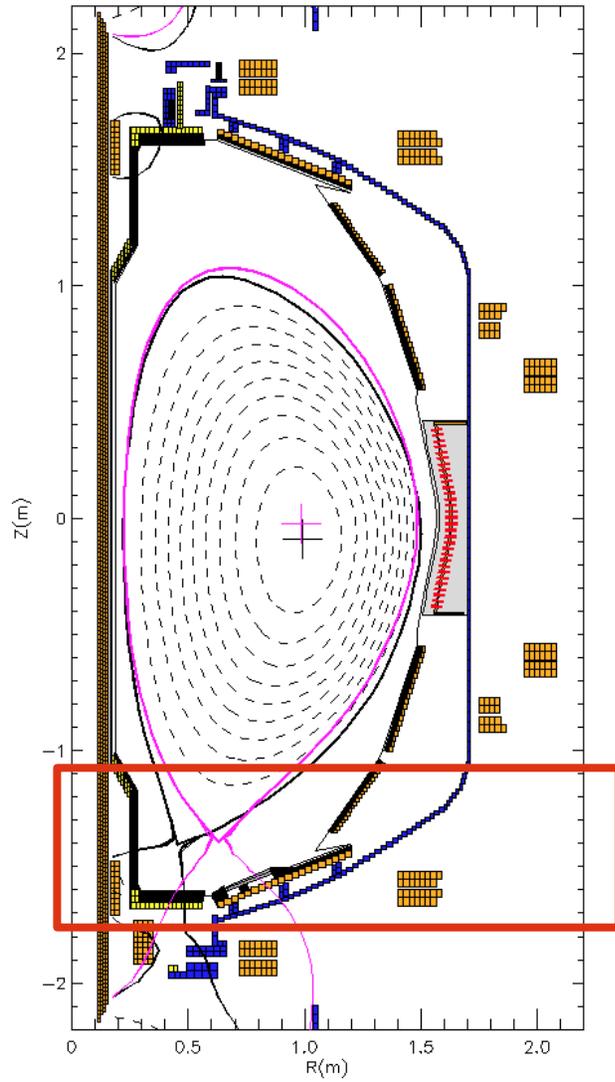
**2010 NSTX Results and Theory Review  
November 30, 2010**

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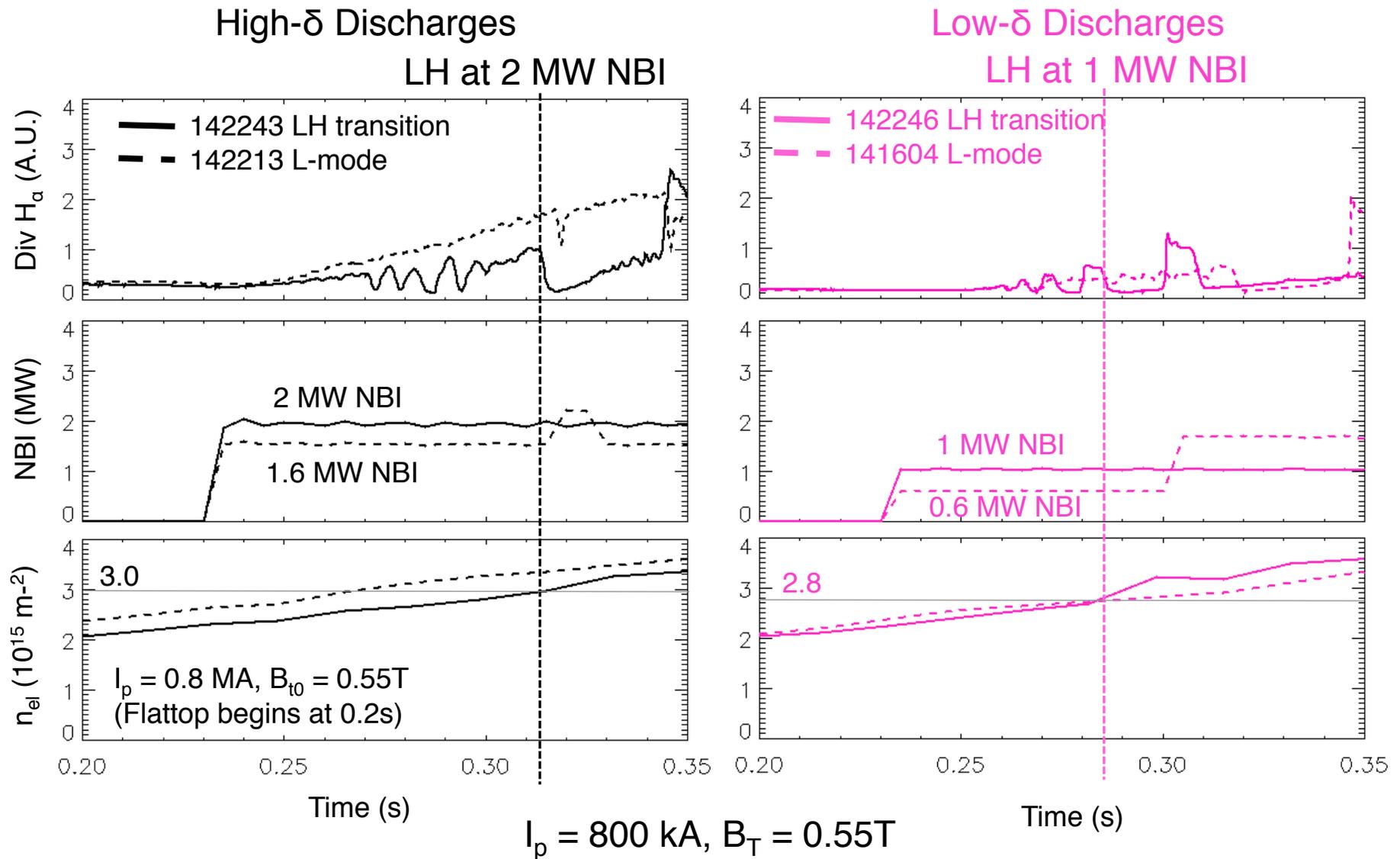
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# XP1029: Examine the dependence of $P_{LH}$ on $R_x$



- With the same TF coil current ...
  - Match inboard  $B_t$  (i.e., inner gap)
  - Match outboard  $|B|$  (i.e., outer gap and  $I_p$ )
  - Match  $B_{t0}$  (same  $R_0$ )
- Try to match other  $P_{LH}$  variables ...
  - Line-averaged density
  - X-point height
  - Plasma surface area
  - HFS and LFS neutral fueling
- Transition with  $P_{OH}$  &  $dW/dt$  nearly constant
- Scan  $R_x$ 
  - Low triangularity:  $R_x = 0.64$  ( $\delta_L = 0.36$ )
  - High triangularity:  $R_x = 0.47$  ( $\delta_L = 0.64$ )
  - $B_{tX}$  ratio (low- $\delta_L$  / high- $\delta_L$ ) = 0.73

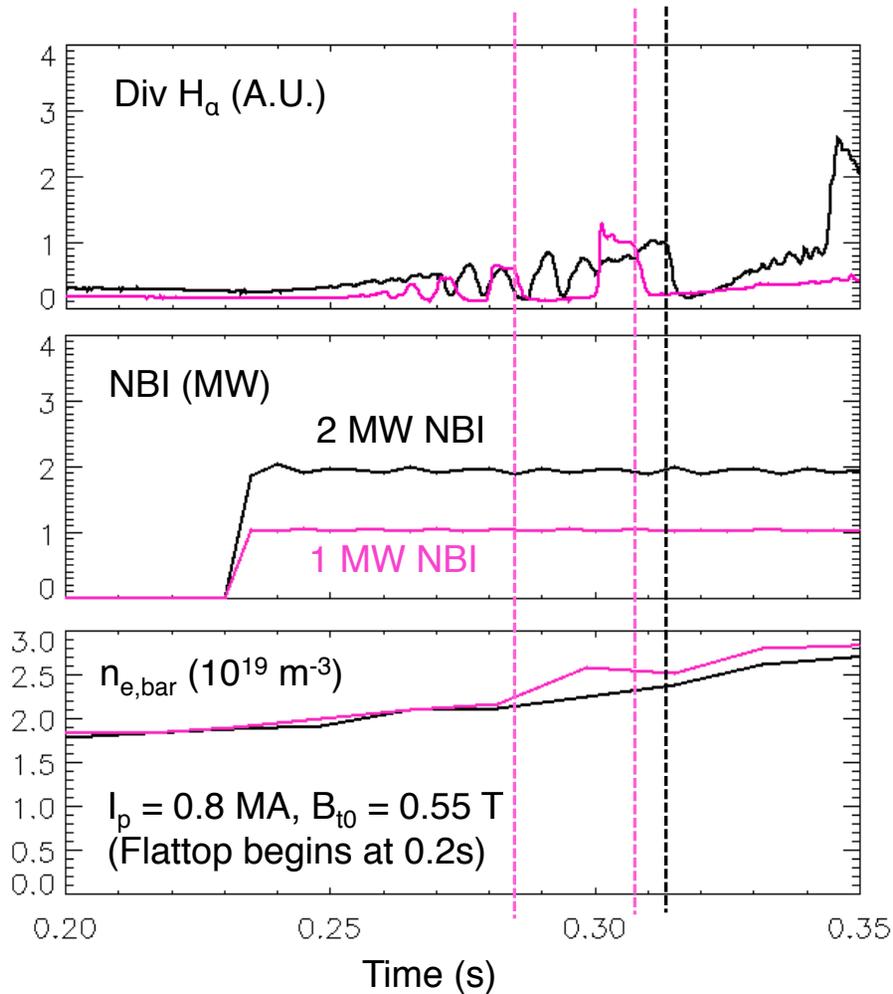
# High- $\delta$ shape requires more NBI power than low- $\delta$ shape to achieve H-mode



# LH transitions occur during periods of steady $P_{OH}$ and $dW/dt$

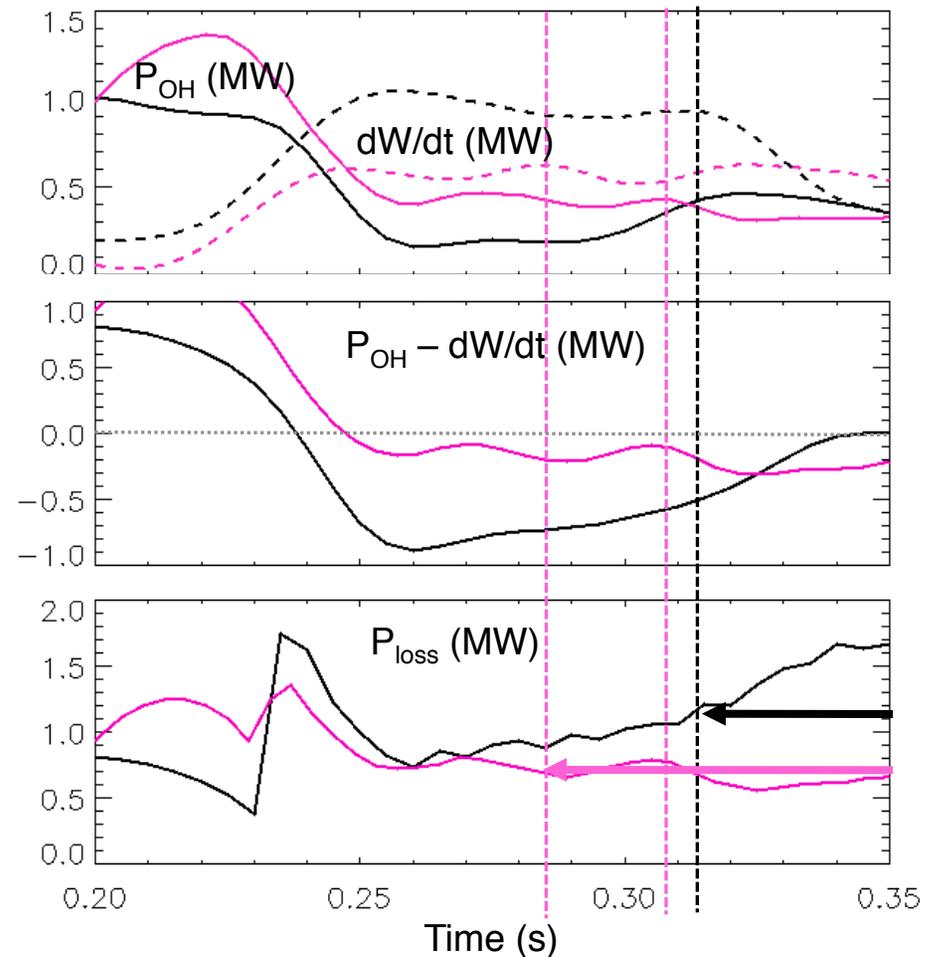
142243  $R_x = 0.47$  (High  $\delta$ )

142246  $R_x = 0.64$  (Low  $\delta$ )

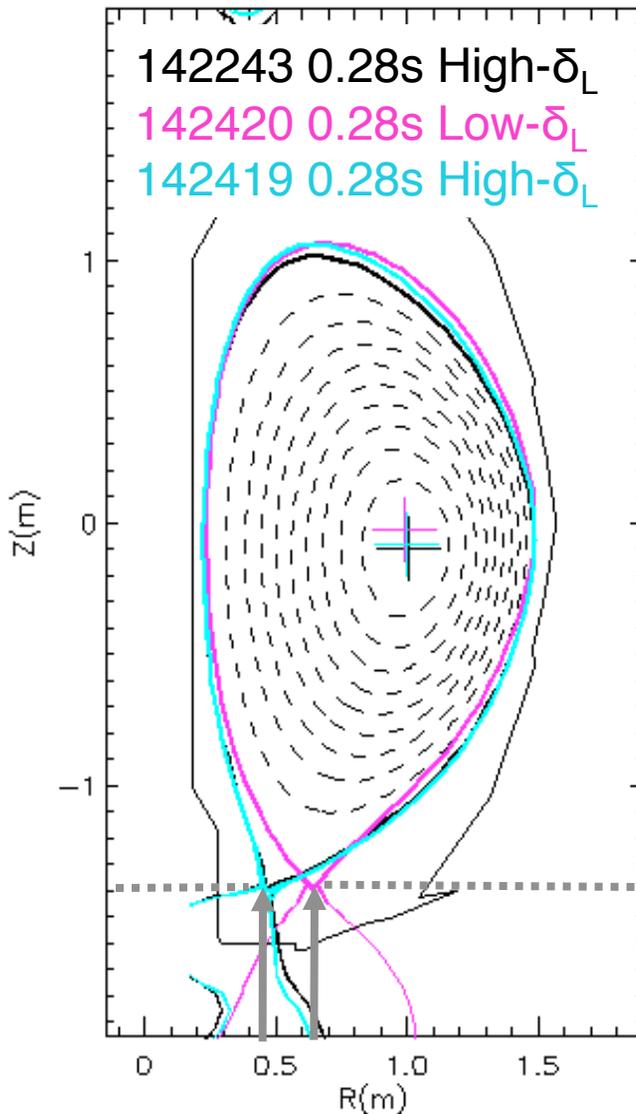


$$P_{\text{loss}} = 0.85 P_{\text{NBI}} + P_{\text{OH}} - dW/dt$$

↑ Average efficiency for similar discharges analyzed using TRANSP



# $P_{LH}$ is similar for both shapes when $B_{tX}$ is matched

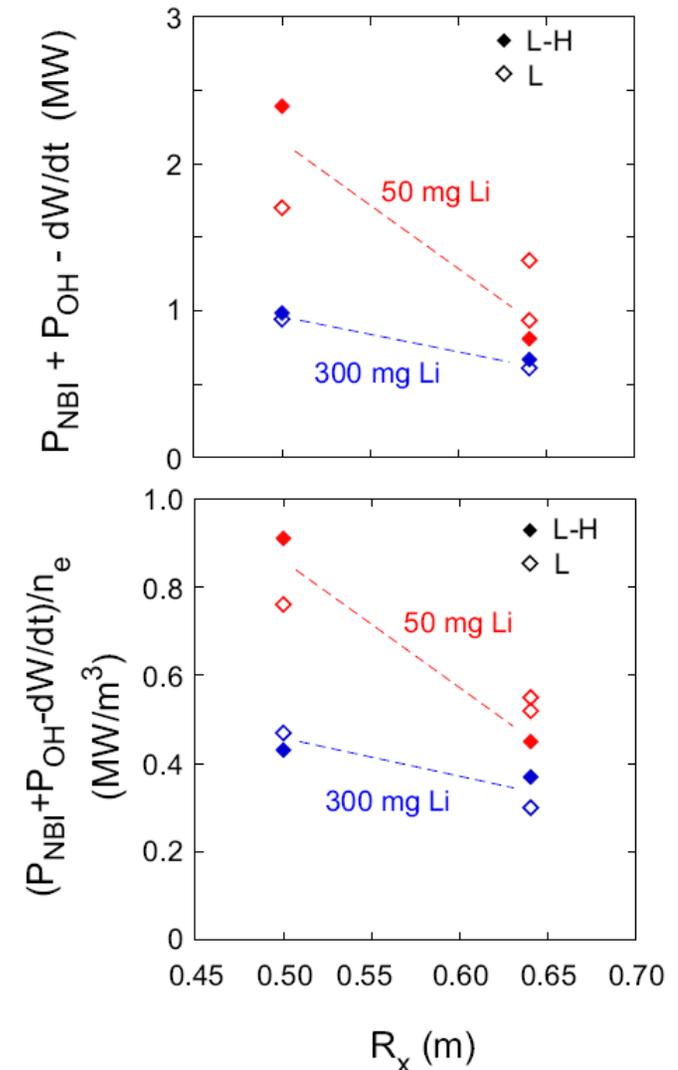


- TF current reduced for high- $\delta$  shape to match  $B_{tX}$ 
  - $P_{LH}$  very similar to low- $\delta$  shape
- Dataset implies  $P_{LH} \sim B_{tX}^{1.0-2.0}$ 
  - 22% - 27% reduction in  $B_{tX}$  gives ...
  - 22% - 45% reduction in  $P_{LH}$

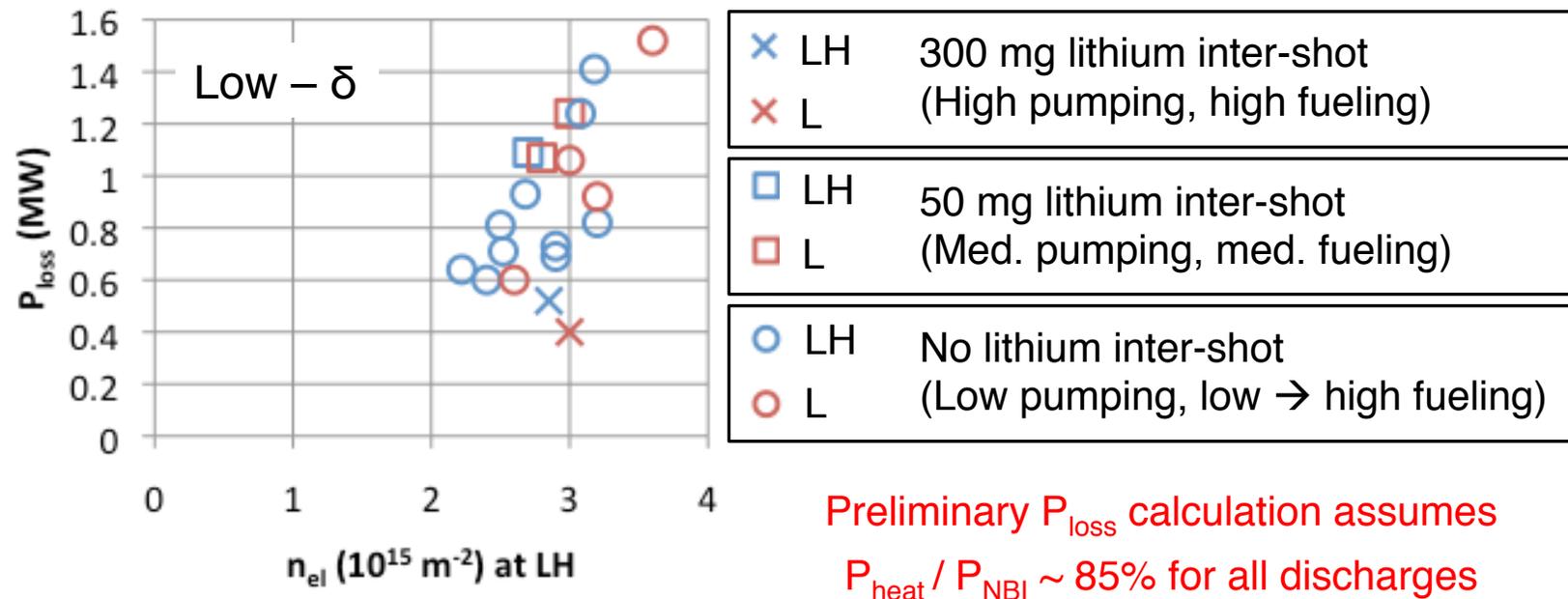
$B_{t0}$ (T)	$R_X$ (m)	$B_{tX}$ (T)	$P_{NBI}$ (MW)	$P_{LH}$ (MW)
0.55	0.47	0.86	1.9	1.1
0.55	0.64	0.63	1.0	0.7
0.40	0.47	0.63	1.0	0.6

# $P_{LH}$ for both shapes strongly influenced by edge fueling and pumping

- Full TRANSP analysis of six discharges
  - Matched  $B_t$ ,  $I_p$ ,  $Z_X$
  - Nearly matched  $n_{el}$ 
    - Required more fueling for high Li shots
- Change in edge fueling has a large effect on  $P_{LH}$ 
  - 20% - 40% change with geometry vs
  - 30% - 40% change with edge fueling

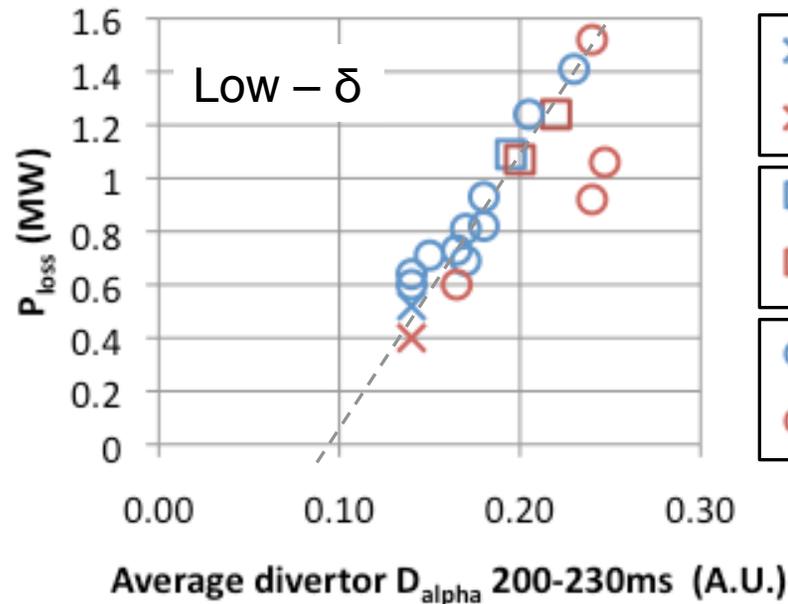


# $P_{LH}$ vs $B_{tX}$ experiment ran both shapes over a wide range of fueling and pumping conditions



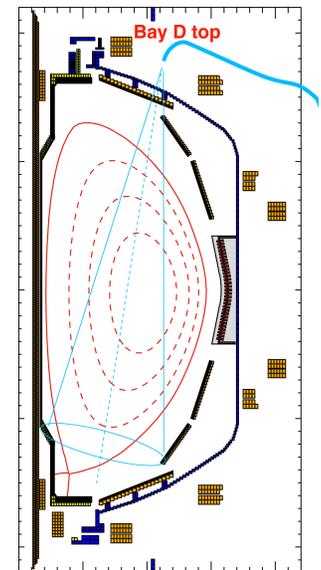
- $P_{LH}$  trends higher with line-integrated density
  - Qualitatively agrees with ITPA scaling
  - Yet,  $P_{loss}$  varies over a factor 3 at  $n_{el} \sim 3 \times 10^{15} \text{ m}^{-2}$
- Future analysis: focus on scaling with edge parameters

# Preliminary observation: $P_{LH}$ scales linearly with initial divertor $D_\alpha$ intensity



×	LH	300 mg lithium inter-shot (High pumping, high fueling)
×	L	300 mg lithium inter-shot (High pumping, high fueling)
□	LH	50 mg lithium inter-shot (Med. pumping, med. fueling)
□	L	50 mg lithium inter-shot (Med. pumping, med. fueling)
○	LH	No lithium inter-shot (Low pumping, low $\rightarrow$ med. fueling)
○	L	No lithium inter-shot (Low pumping, low $\rightarrow$ med. fueling)

- $P_{LH}$  increases with pre-NBI divertor  $D_\alpha$  signal
  - May be proportional to initial edge neutral density ( $n_N$ )
- Indicates  $n_N$  is important in  $P_{LH}$  calculations
  - NBI heating efficiency depends on  $n_N$
  - Neutrals impact LH trigger (ion – neutral collisions)



# XP1029 Summary

- $P_{LH}$  observed to scale with  $B_{tX}$ 
  - Consistent with XGC0 calculations that link the ion loss at the X-point to the  $E_r$  well depth
  - May contribute to the scaling of  $P_{LH}$  with  $B_t$  seen in all toroidal devices
    - ST geometry can decouple  $B_{tX}$  from  $B_{t0}$  through small changes in  $R_X$
- $P_{LH}$  measured over a range of fueling and pumping conditions
  - Initial observation:  $P_{LH}$  scales linearly with the pre-beam divertor  $D_\alpha$
  - Lithium coatings are a powerful tool for altering edge fueling &  $P_{LH}$
- Future work
  - XGC0 calculations of  $E_r$  in L-mode for both shapes
  - TRANSP calculations of full dataset
  - Use dataset to consider critical local variables for LH transition

## XP1036: $P_{LH}$ vs ionic species

- Deuterium:  $P_{RF} \sim 600$  kW for LH transition with  $180^\circ$  phasing
  - 3 L-mode shots at 450 and 560 kW
  - 5 LH transitions at 625 and 700 kW
  - Locked mode about 50ms after flattop
  - Used combo of HFS and LFS fueling
  - $P_{LH}$  higher with only LFS fueling
- Helium: No LH transition observed
  - Started with discharge that transitioned at 1.5 MW early in run
  - Tried up to 2 MW of RF with only LFS fueling
  - Tried up to 1.2 MW of RF with HFS  $D_2$  and LFS He
  - Tried a number of tricks to lower  $P_{LH}$  to no avail

# Backup

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# Ion loss mechanisms at the X-point depend on $R_x$

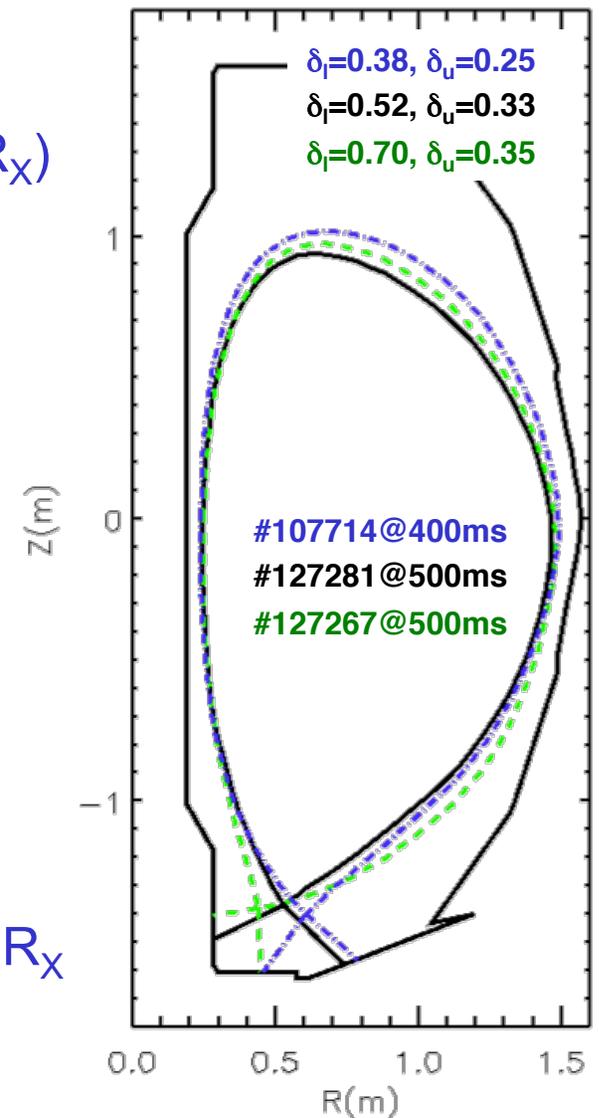
- Orbit losses
  - Ion gyro-orbit (ion loss decreases with  $B_T \sim 1/R_x$ )

- Grad-B drift

- Low  $v_{\parallel}$  ions have slow poloidal procession
  - Ions can grad-B drift out of plasma

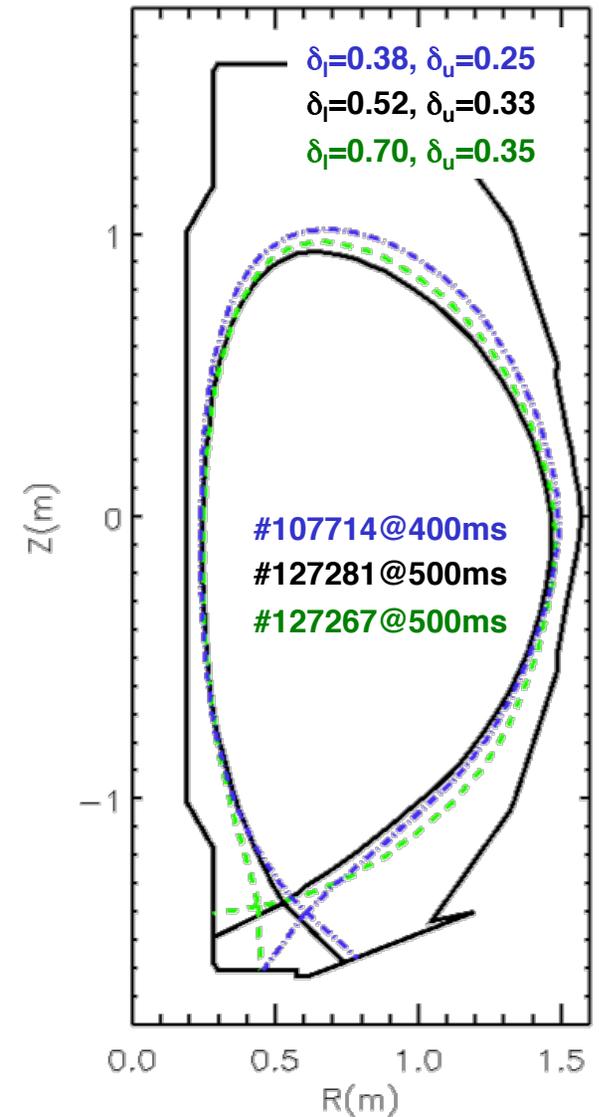
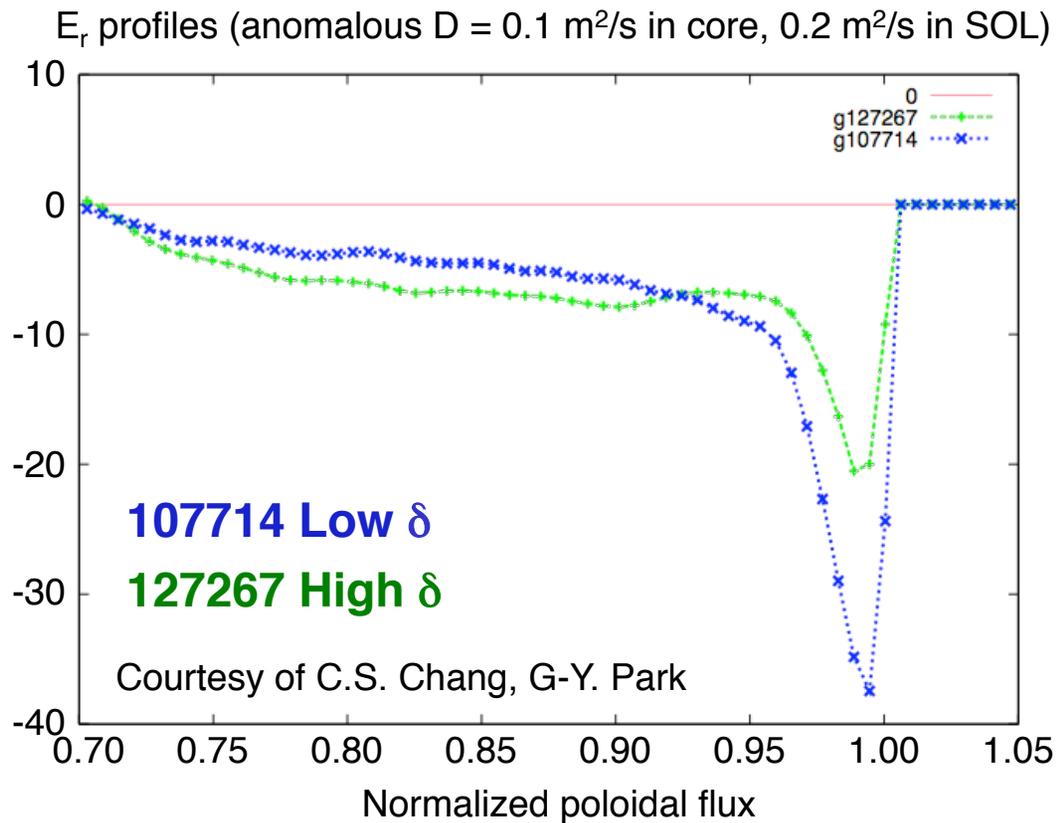
$$v_{\parallel} B_p / B_T < v_{\nabla B}$$

- More complex relation to  $R_x$ 
  - Smaller  $R_x \rightarrow$  smaller  $B_p/B_T$
  - But fewer low energy ions due to banana orbits
- Used to describe DIII-D dependence of  $P_{LH}$  on  $R_x$ 
  - C.S. Chang, S. Kue, H. Weitzner, PoP **9** (2002)



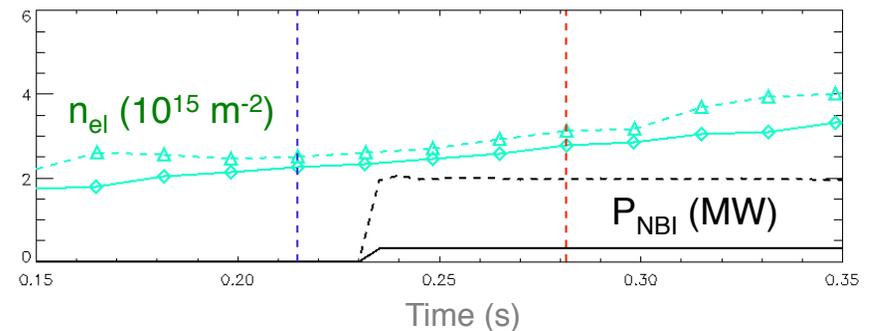
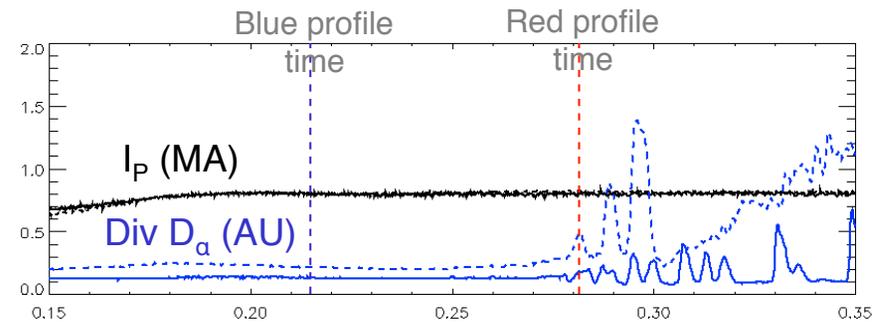
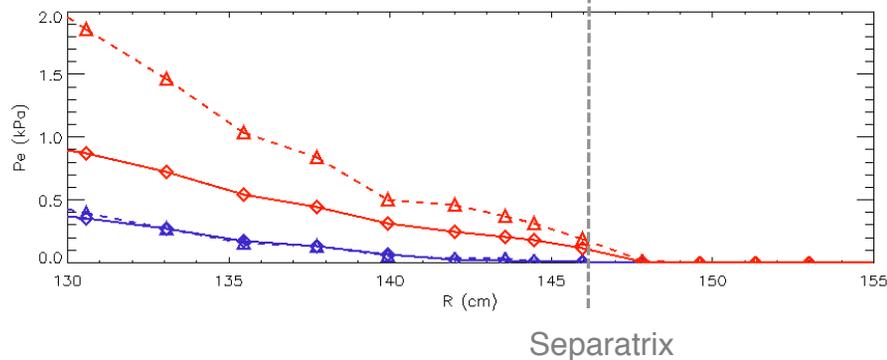
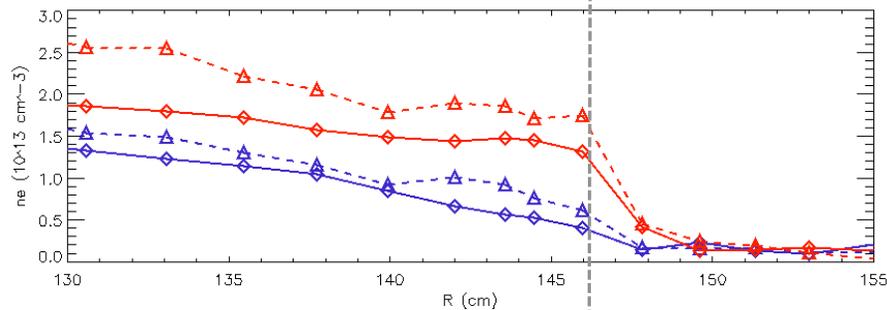
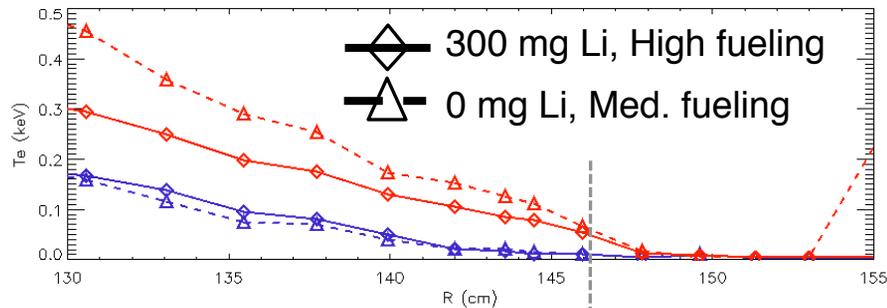
# Edge $E_r$ develops in response to ion losses at X-point

## XGC0: NSTX H-mode profiles



R. Maingi et. al., *Nucl. Fusion*, **50** (2010) 064010

# Profiles indicate that the scaling with divertor $D_\alpha$ is not solely an effect of changes to the NBI efficiency



- L-mode profiles matched before NBI, but not during
  - Expect profiles to be similar if  $P_{\text{abs}}$  was the same despite different  $P_{\text{NBI}}$

# Dependence of $P_{LH}$ with $I_p$ and $B_t$ is well documented for toroidal devices

- ITER-like ITPA database scaling

- Near-linear scaling with  $B_t$
- No significant scaling with  $I_p$

$$P_{LH} = 0.0488 n_{e20}^{0.717} B_t^{0.803} S^{0.941} \quad [1]$$

- Low-A devices exhibit  $I_p$  scaling

- $B_\theta \sim B_t$  at outboard midplane at low-A
- May imply  $P_{LH}$  scales with  $|B|$  at outboard midplane

$$P_{LH} = 0.072 n_{e20}^{0.7} B_{out}^{0.7} S^{0.9} \quad [2]$$

$$P_{LH} = P_{OH} + P_{abs} - dW/dt - P_{floss}$$

$P_{OH}$ : Ohmic heating power

$P_{abs}$ : Absorbed heating power

$dW/dt$ : Change in plasma stored energy

$P_{floss}$ : Power lost by fast ions

$P_{LH}$ : Minimum loss power needed for LH transition

$n_{e20}$ : Line-averaged density ( $10^{20} \text{ m}^{-3}$ )

$B_t$ : On-axis toroidal magnetic field (T)

$S$ : Plasma surface area ( $\text{m}^2$ )

$B_{out}$ : Mag field at outboard midplane

1 Y.R. Martin et. al., *J. Phys.: Conf. Ser.* **123** (2008) 012033

2 T. Takizuka et. al., *PPCF* **46** (2004) A227

# $I_p$ and $B_t$ scaling of $P_{LH}$ may be described by their relationship to the edge shearing rate and $E_r$

- Hypothesis: LH transition at a critical  $E_r \times B$  shearing rate
  - $E_r \times B$  shear rate increases prior to the formation of a pedestal
  - Shearing exceeds a critical value
    - suppresses turbulence → triggers a positive feedback loop

$$\omega_{ExB} = \frac{(RB_\theta)^2}{B} \left( \frac{\partial}{\partial \psi} \right) \frac{E_r}{RB_\theta}$$

- From force balance:  $E_r = v_\theta B_\phi + v_\phi B_\theta - \nabla(n_i T_i) / Z_i e n_i$ 
  - NSTX measurements: core  $E_r \sim v_\phi B_\theta$ , edge  $E_r \sim \nabla P_i / Z_i n_i$
  - Edge pressure gradient related to power lost across separatrix
  - XGC-0 calculations: edge  $n_i$  and  $\nabla n_i$  influenced by ion orbit losses
    - Result: magnetic geometries that enhance ion orbit losses require smaller edge pressure gradients to trigger LH transition

Equations from: K.H. Burrell, *Phys. Plasmas*, 4 (1997) 1499

# Acknowledgement

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