



Transport & Turbulence and Boundary Physics Five Year Plan (FY '04-'08)

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NSTX Program Advisory Committee Meeting

Princeton Plasma Physics Laboratory

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NSTX parameters challenge existing theory framework



- Local $\beta_t \approx 100\%$
- Trapped particle fraction ≈ 1
- $\beta_i/L \sim 0.2$ (near edge); $\beta_i \sim 1 - 3$ cm
- High β_B , $E \times B$ flow (>200 km/sec) \approx flow shear (10^5 to 10^6 /sec)
- $V_{\text{fast}}/v_{\text{Alfven}} \sim 3$ to 4
- $\beta_{\text{fast}}/a \sim 1/5 - 1/3$

Validity of present
gyrokinetic treatment

NSTX transport goals aim to determine attractiveness of the ST and contribute to toroidal transport physics

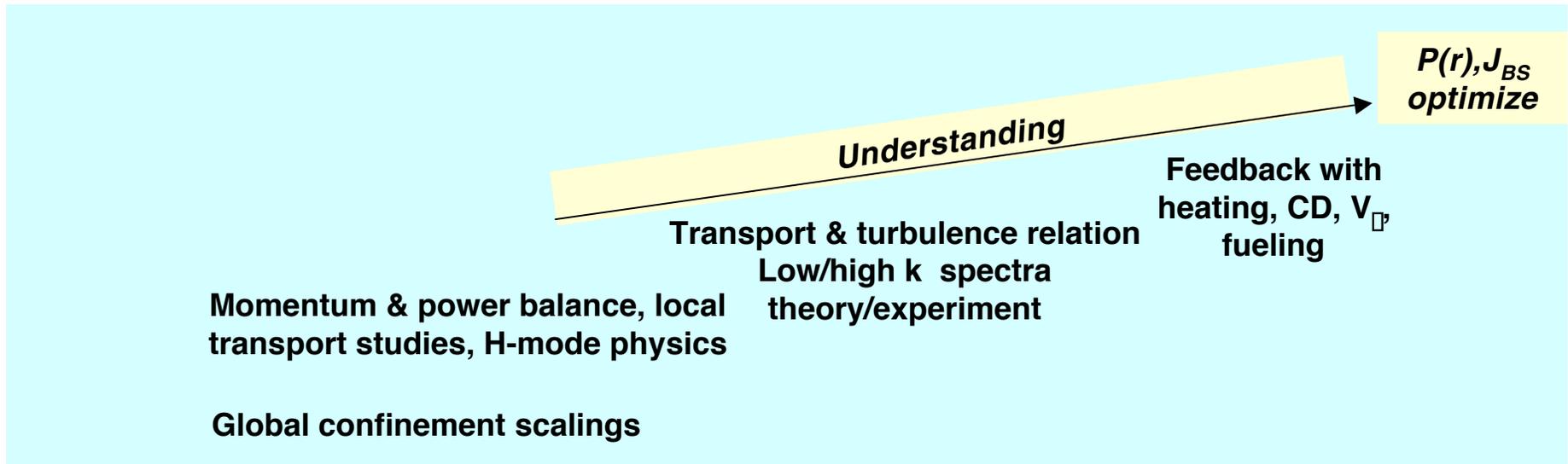
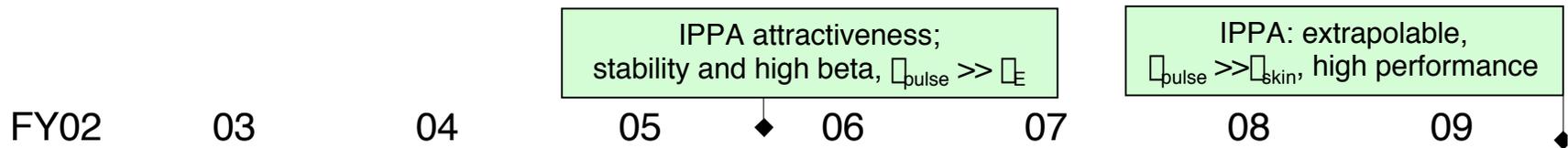


- Establish key global (Γ_E) and local (Γ) transport scalings
 - e^- vs i^+ transport, dependence on β , β_T , β_{ExB}
- Assess roles of low- and high-k turbulence in heating and transport
- Assess fast ion confinement
 - Influence on neoclassical, turbulent heating and transport
- Determine influence of E_r (β_{ExB}) on turbulence, L-H

Use knowledge gained to control plasma transport

Produce $p(r)$, $j(r)$, for high β_T , non-inductive current

Transport & Turbulence contributes to integration goals through H-factor and bootstrap current control



Integration

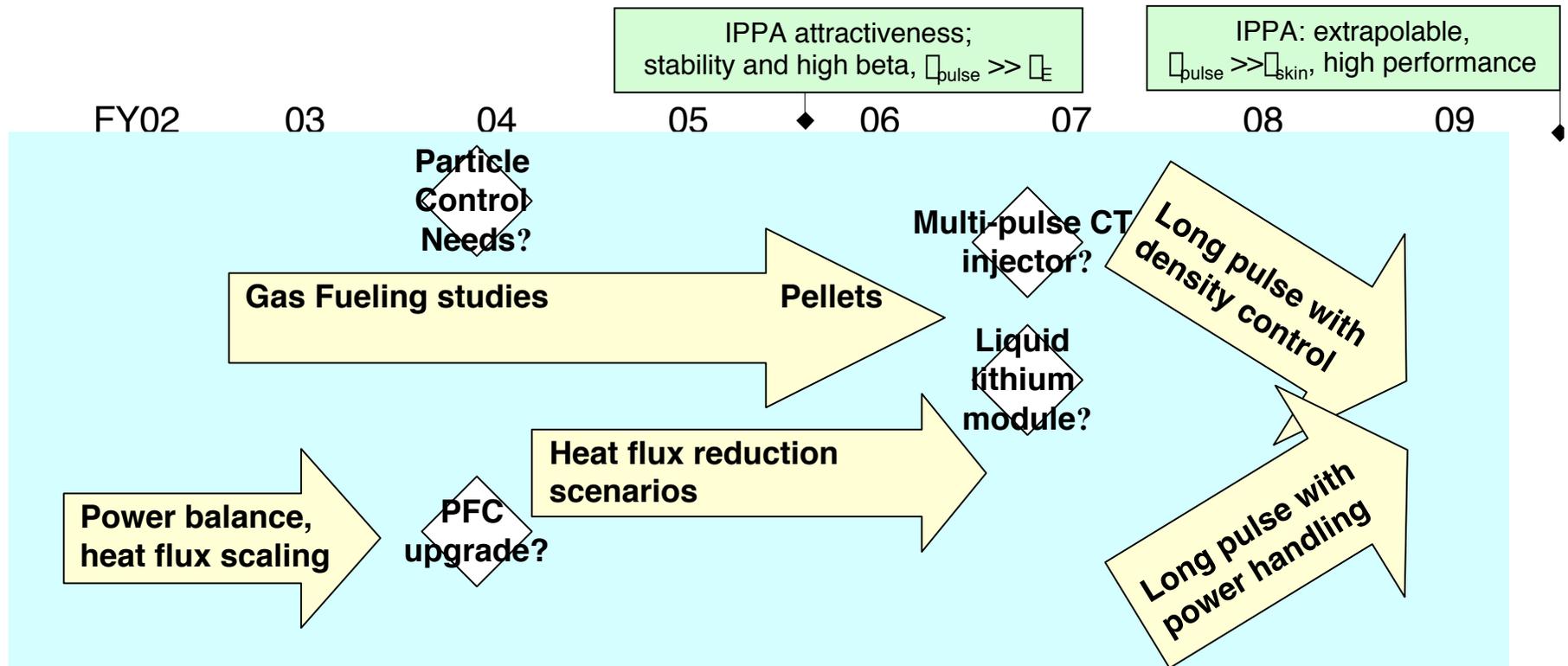


Boundary physics activities have both science goals and enabling technology responsibility



- Science goals
 - characterize edge power and particle transport regimes
 - *parallel vs. cross-field*
 - *do conventional aspect ratio models fit?*
 - understand effect of ST features on boundary physics, e.g.
 - *large in/out B_t ratio*
 - *large SOL mirror ratio and short connection length*
- Enabling technology - tailor edge plasma to optimize discharges
 - evaluate power handling needs and solutions
 - assess fueling and particle pumping needs
 - develop and evaluate wall conditioning techniques

Boundary physics must provide particle and power handling to achieve long pulse integration goals



Integration

• $\tau_T = 30\%$, $\Delta t > \tau_E$
 $\tau_N = 5$: > no wall limit
 HH = 1

Highest performance

• $\tau_T = 40\%$,
 • $\tau_N = 8$
 $t \gg \tau_E$, HH = 1.4

• $J_{NI} > 60\%$, $\Delta t \sim \tau_{skin}$
 • Non-solenoid startup demo

Solenoid-free

• $J_{NI} \sim 100\%$, $\Delta t > \tau_{skin}$
 • Solenoid-free up to hi τ_p

40% τ_T
 $t \gg \tau_{skin}$
 $\tau_N = 8$
 HH = 1.4
 70% J_{BS}

Outline



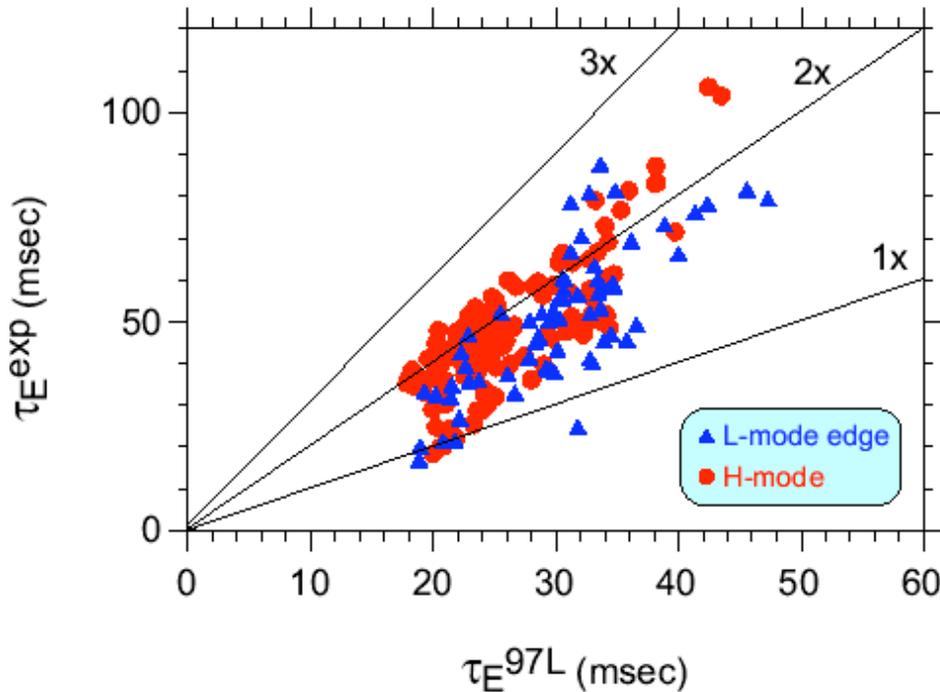
- Core Transport
 - Global energy confinement and scalings
 - Local transport studies
 - Ion and electron transport
 - Momentum and particle/impurity transport
 - Fast Ion Transport

- Boundary transport and characterization
 - Edge and SOL turbulence and transport studies
 - H-mode power threshold, pedestal and ELM studies
 - Particle and power control
 - Fueling, sources and sinks
 - Power handling and mitigation

Global confinement scalings of NSTX data commencing

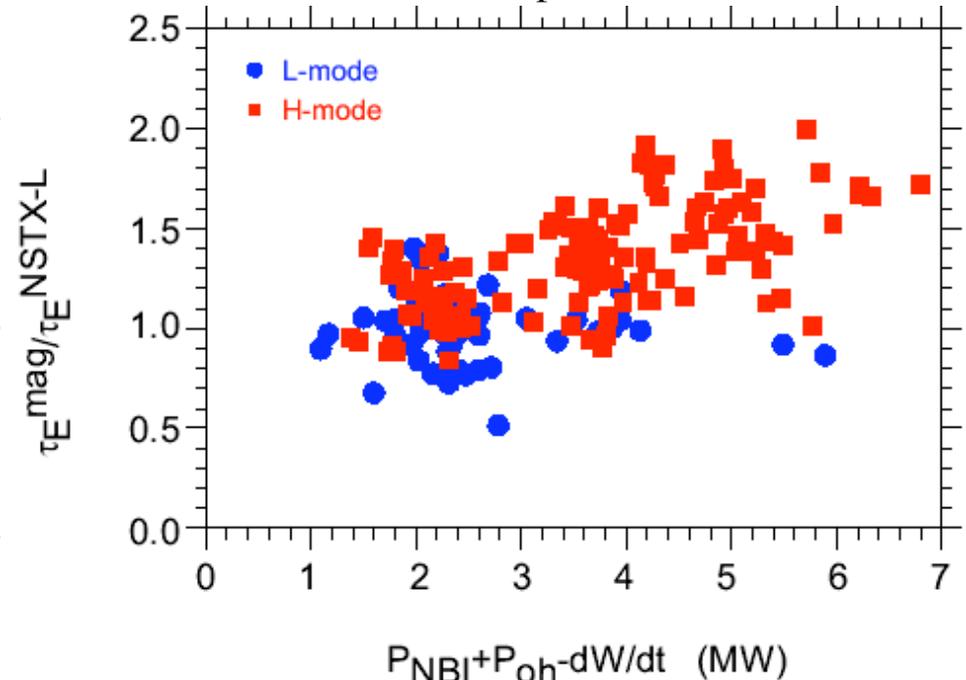


$$\tau_E^{97L} \sim I_p^{0.74} B_T^{0.20} P_L^{-0.57}$$



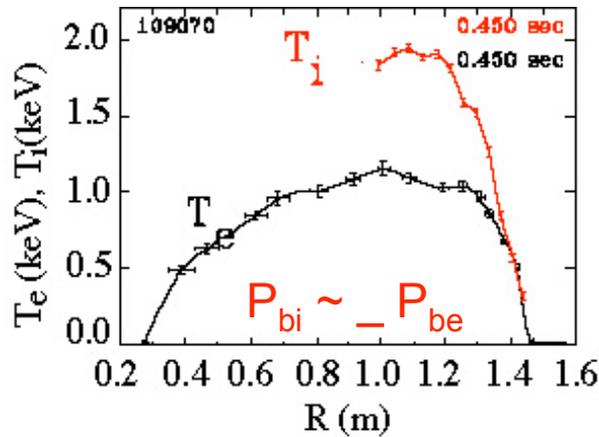
- τ_E^{exp} from EFIT reconstruction
- Includes fast ion component
- Quasi-steady conditions

$$\tau_E^{\text{NSTX-L}} \sim I_p^{0.76} B_T^{0.27} P_L^{-0.76}$$



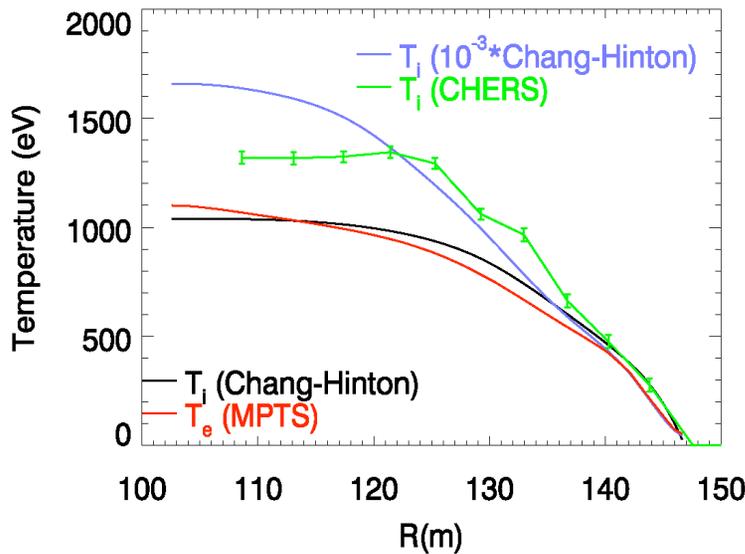
- Less severe power degradation in H-mode $\tau_E \sim P^{-0.50}$
- MHD vs confinement limit?
- Different parametric dependencies for more transient L-mode plasmas
- Role of rotation?

$T_i > T_e$ during NBI indicates relatively good ion confinement



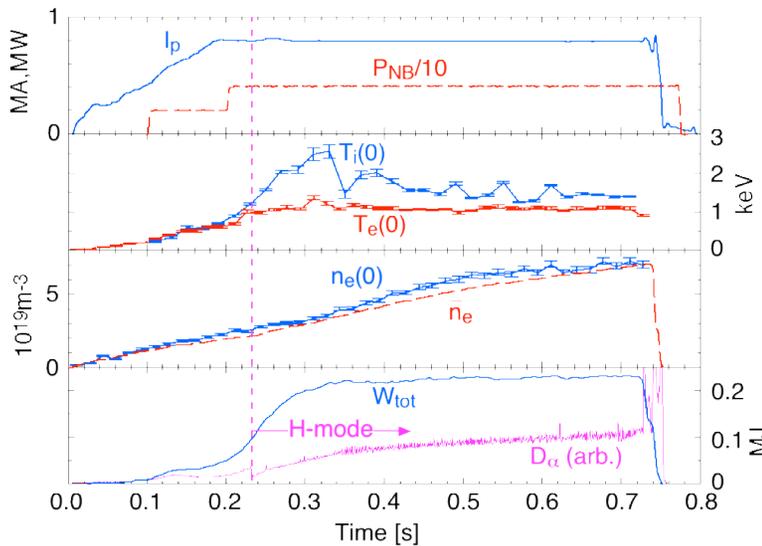
ITG suppressed (GK calculations)

Ion confinement sometimes “too good”

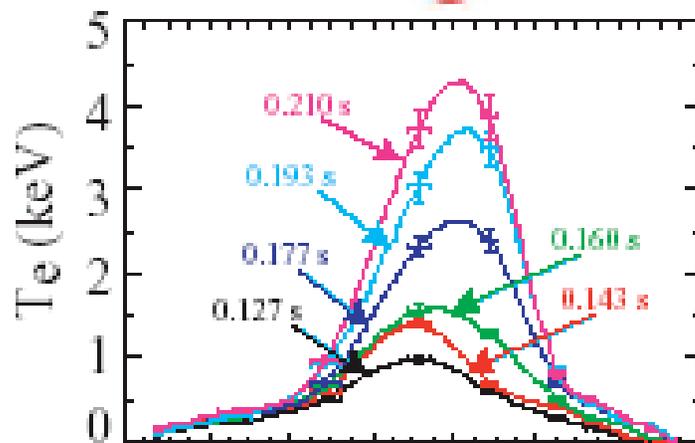
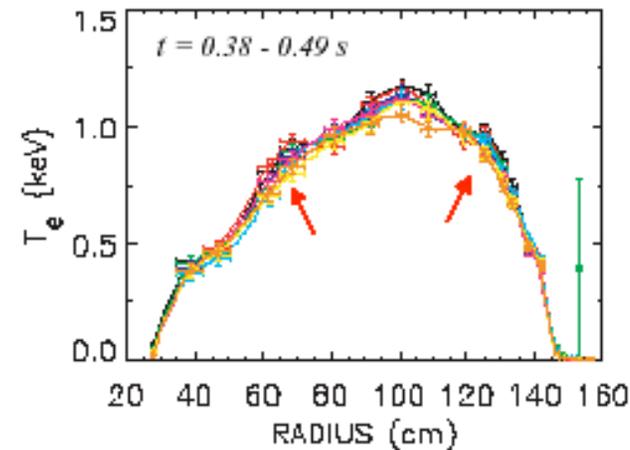


Independent validation of T_i profile needed (2-D X-ray crystal diagnostic)

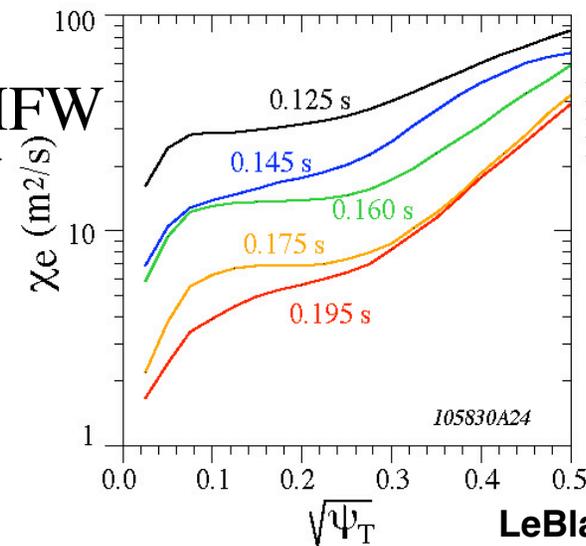
“Stiff” T_e Profiles during flattop period with NBI but sometimes peaked with RF heating



Profile Resilience?



ITB w/ HHFW

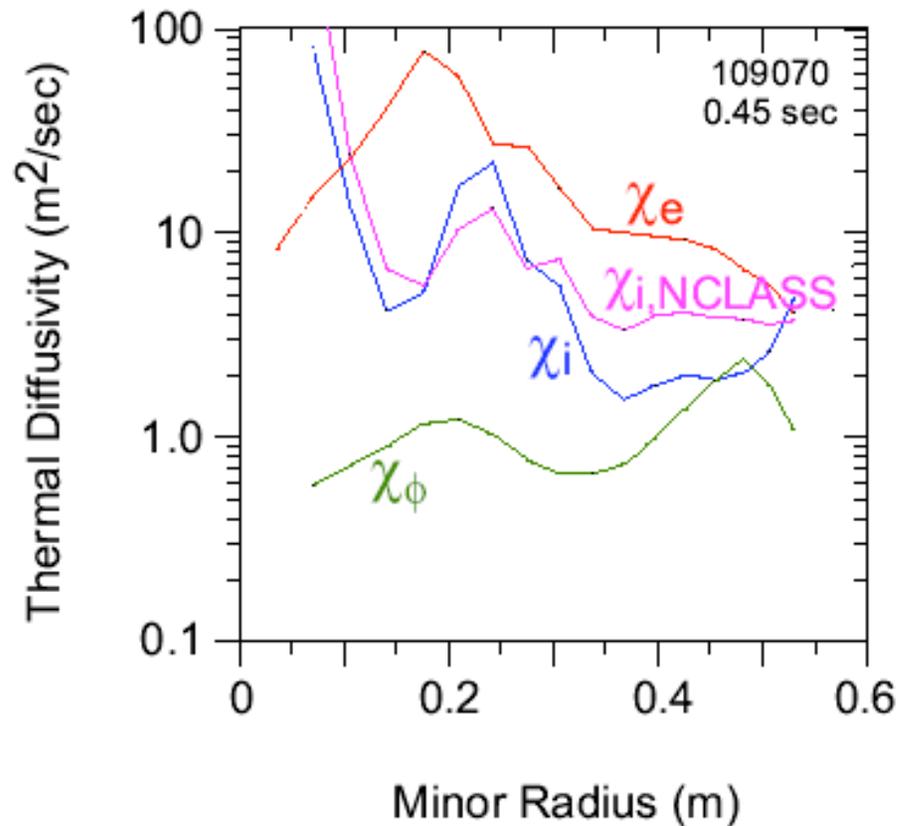


LeBlanc, PPPL

Momentum transport slower than thermal transport



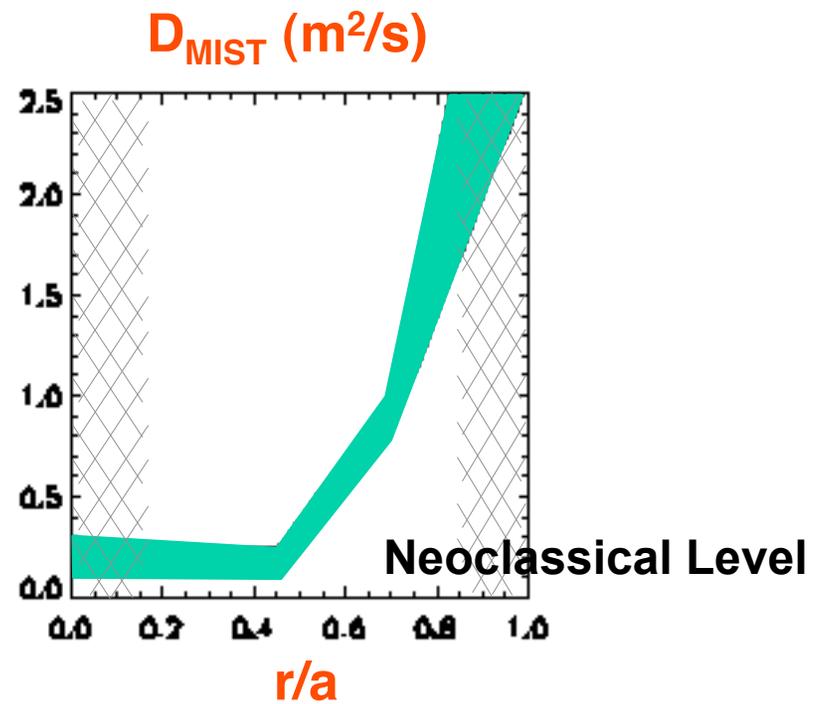
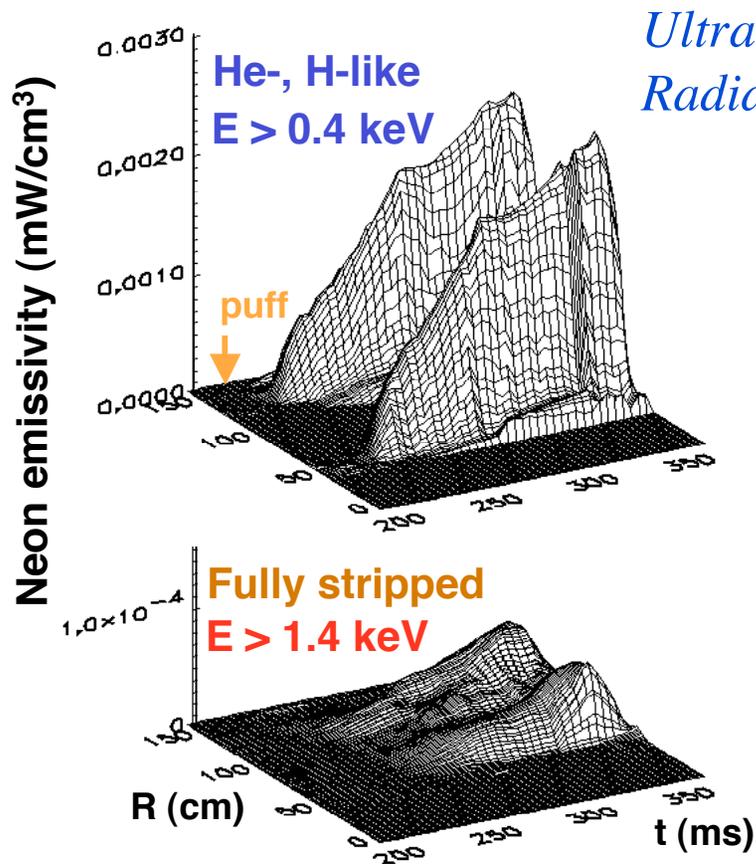
- Inferred transport coefficient ordering: $\chi_\phi < \chi_i < \chi_{neo} < \chi_e$



Core neon transport near neoclassical



Neon injection: Line radiation in He, H-like and continuum in fully stripped



Local transport and global scaling studies

FY 03-04 Plans



FY 03

- Establish χ_e , χ_i , χ_\perp baseline with NBI and/or RF
- Determine if and when χ_i anomalous
- Start to test role of ITG
- Measure D_\perp at higher χ_T with impurity gas injection
- Similarity experiments with MAST/DIII-D
- R/a dependence in ohmic (**rtEFIT control**)

FY 04

- Ion transport relation to rotation
- Assess modes for possible stochastic heating (**calibrated reflectometry**)
- Test T_e resiliency and assess role of ETG
- Relate transport and ITB formation to $q(r)$ (**MSE**)
- Transient vs steady-state confinement dependencies/role of rotation

*** New capabilities**

Local transport and global scaling studies

FY 05-08 Plans



----- FY 05 -----

- Co vs counter NBI to study effect of flow shear (**poloidal CHERs**)
- Role of $n_e(r)$ for ITG, ETG study (**pellets**)
- Assess low-k and high-k fluctuations (**advanced turbulence diagnostics**)
- Deuterium pellet injection for perturbative particle transport

----- FY 06-08 -----

- Detailed study of ion and electron transport to full-k turbulence
- Relate transport fluxes to changes in $q(r)$, $E_r(r)$ (**LIF MSE**)
- Relate E_r (**LIF MSE**) to flows (CHERS)
- Relate E_r , v_{\square} and $q(r)$ with ITB generation
- Determine rotation/confinement dynamics causality (**1 ms CHERs**)
- Impurity pellet injection for perturbative particle transport

* **New capabilities**

Fast ion confinement studies just starting



Results

- Decay of neutrons consistent with classical slowing down
- Loss rate measurements lower than modeling
- Variations in neutron rate for nominally similar discharges

Plans

- **FY03:** Overall confinement trends with parameter, L vs H (**sFLIP**)
- **FY04:** Control loss fraction (vary gap)
- **FY05:** Non-ambipolar losses (co vs ctr); power deposition profile (**neutron collimators**)
- **FY06-08:** Extend studies (**array of solid state detectors**)

* **New capabilities**

Outline

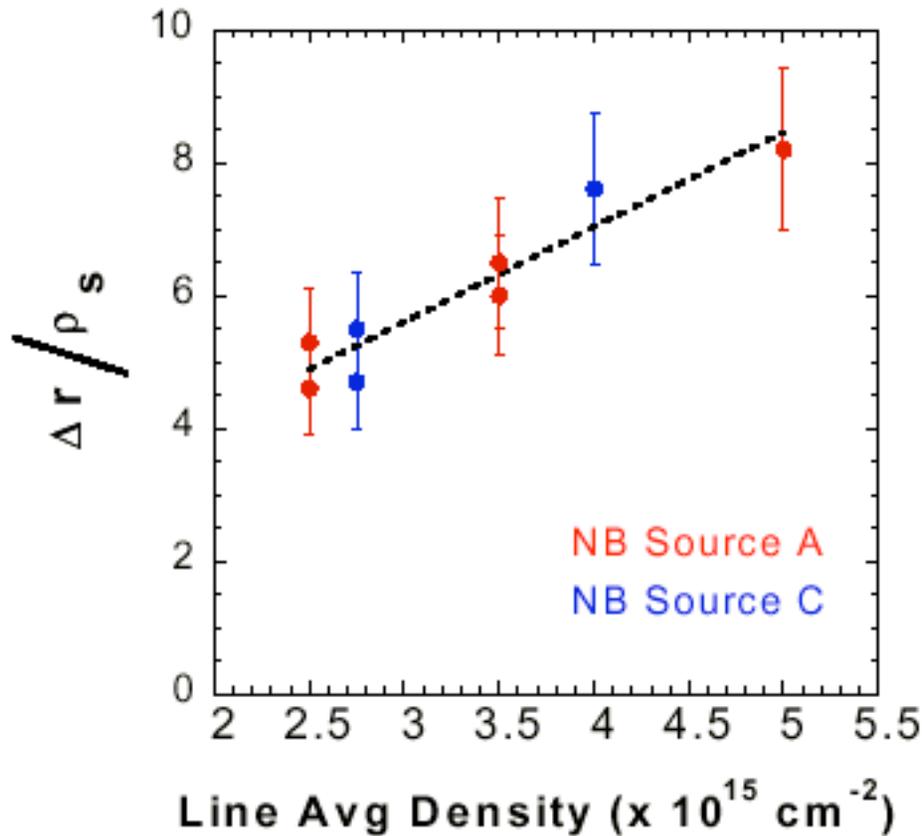


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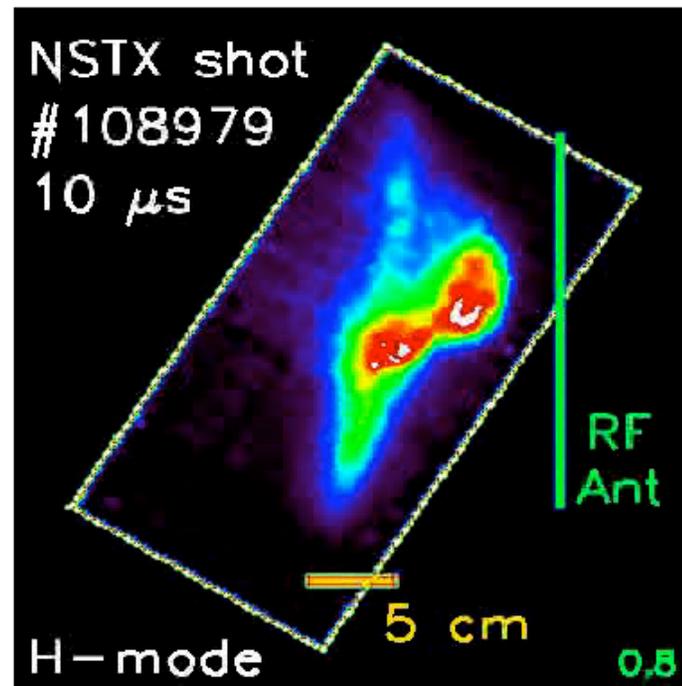
Fluctuation characteristics measured vs. plasma parameters and confinement modes



Δr normalized to ρ_s vs. \bar{nL}



\tilde{n}/n goes down from L to H

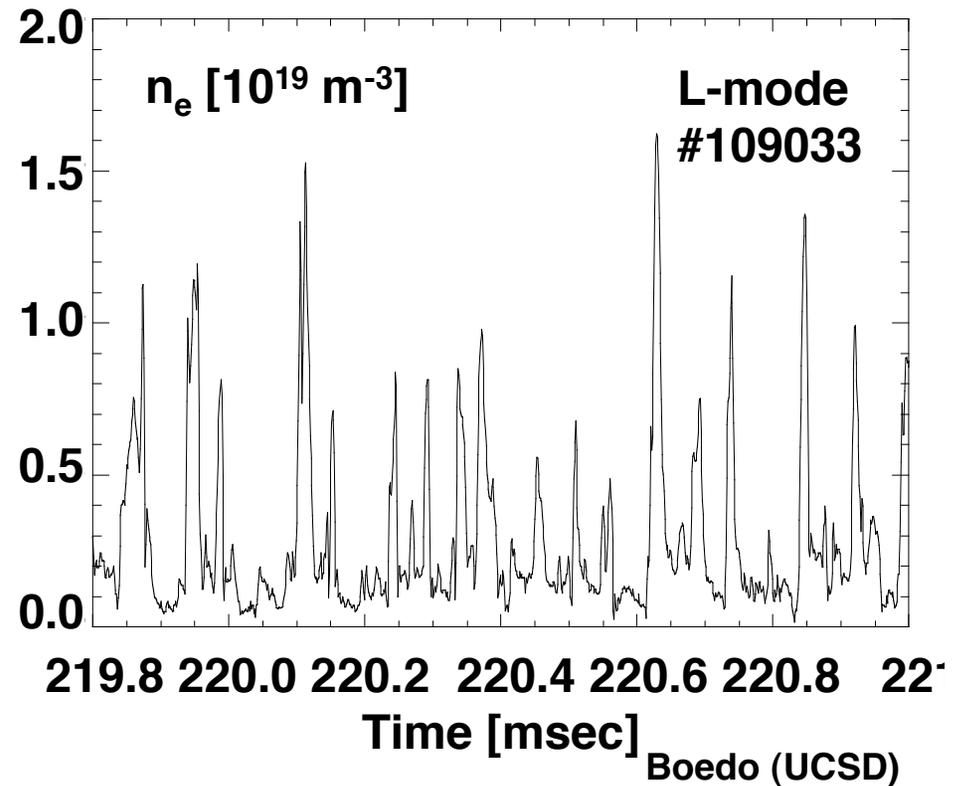
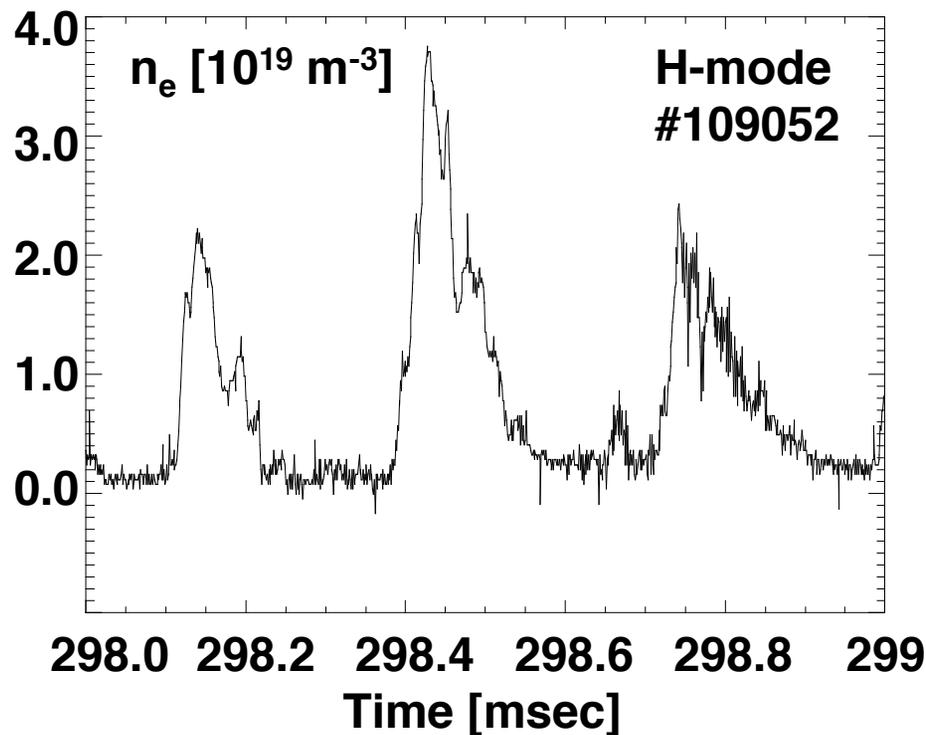


Convective-cell ("blob") transport potentially significant

Intermittent behavior observed in L and H-mode density



- Rate of bursts is ~ 2 kHz in H-mode and ~ 20 kHz in L-mode
- Poloidal field is much less intermittent
- Instantaneous particle flux $\sim 10^{19} \text{ m}^{-2}\text{s}^{-1}$

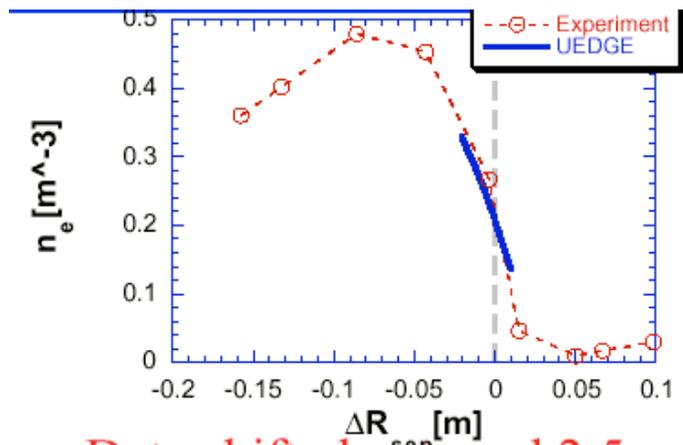


UEDGE modeling needed to estimate transport rates

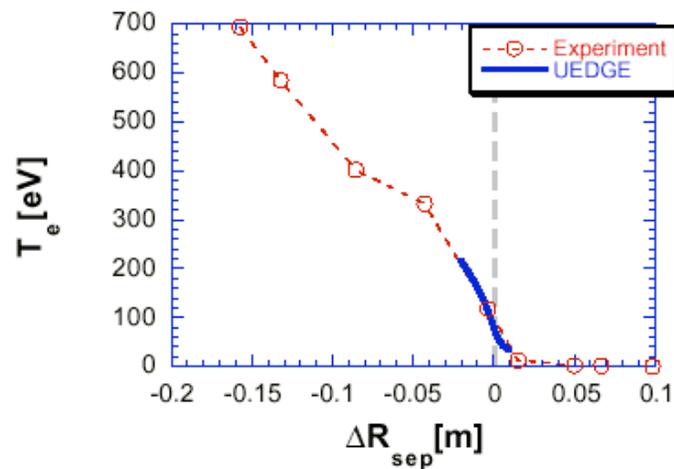


$$D_{\square} = 1 \text{ m}^2/\text{sec}, \square_{\square} = 3 \text{ m}^2/\text{s}$$

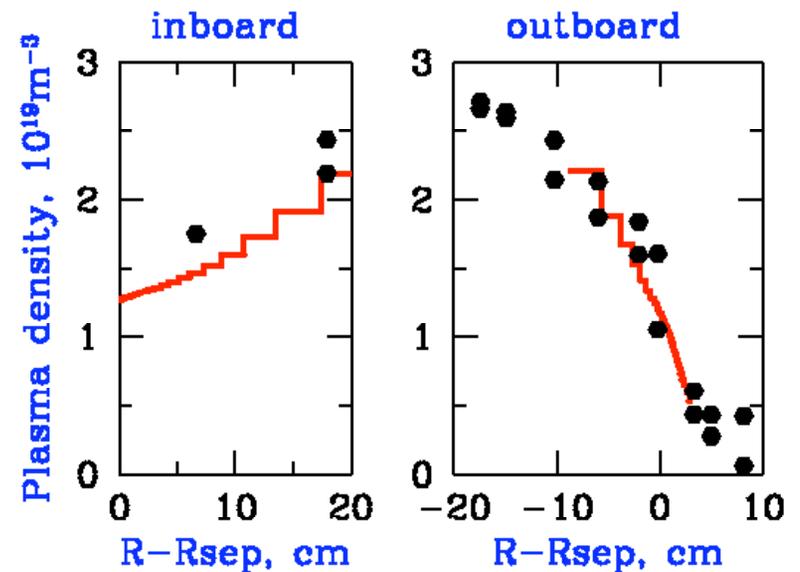
- LLNL using diffusive cross-field transport model for H-mode discharges
- UCSD examining convective cross-field transport model for L-mode discharges



Data shifted outward 2.5 cm



L-mode NSTX shot 109033, 259–276ms



Pigarov (UCSD)

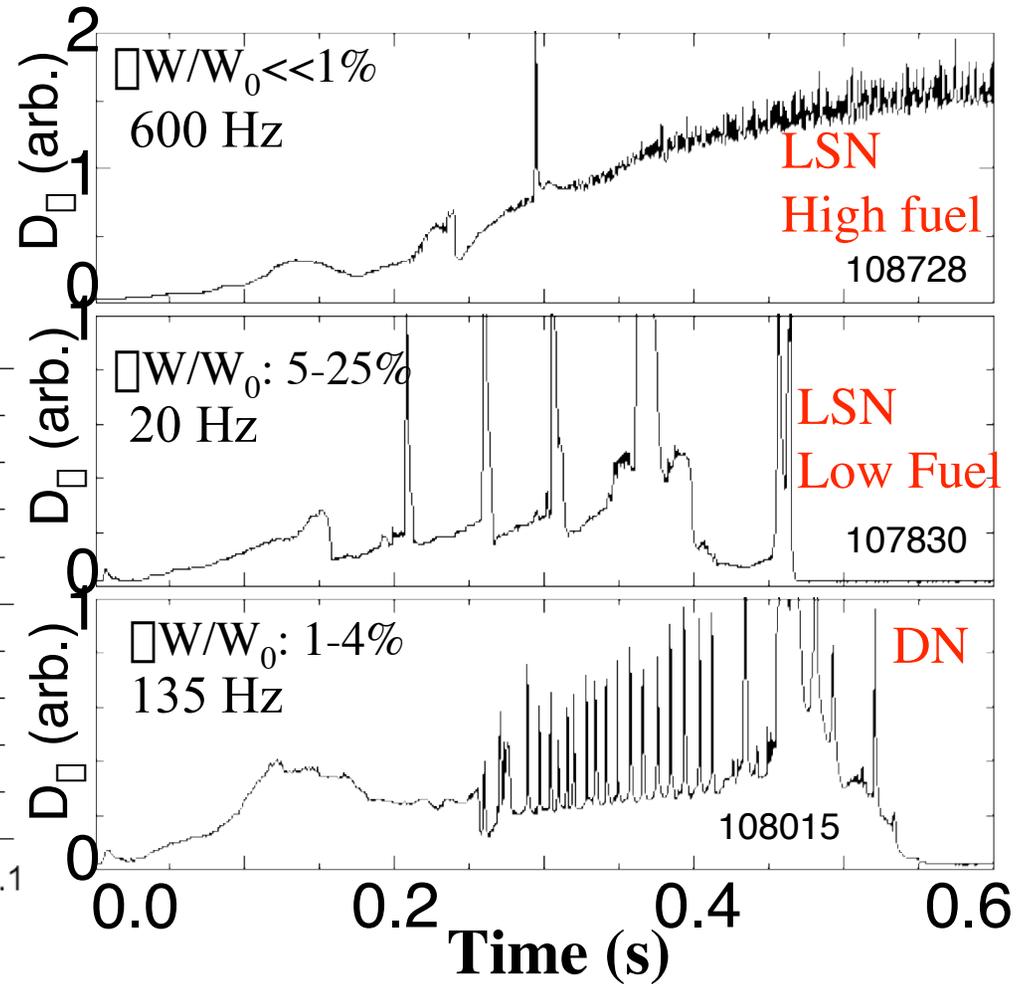
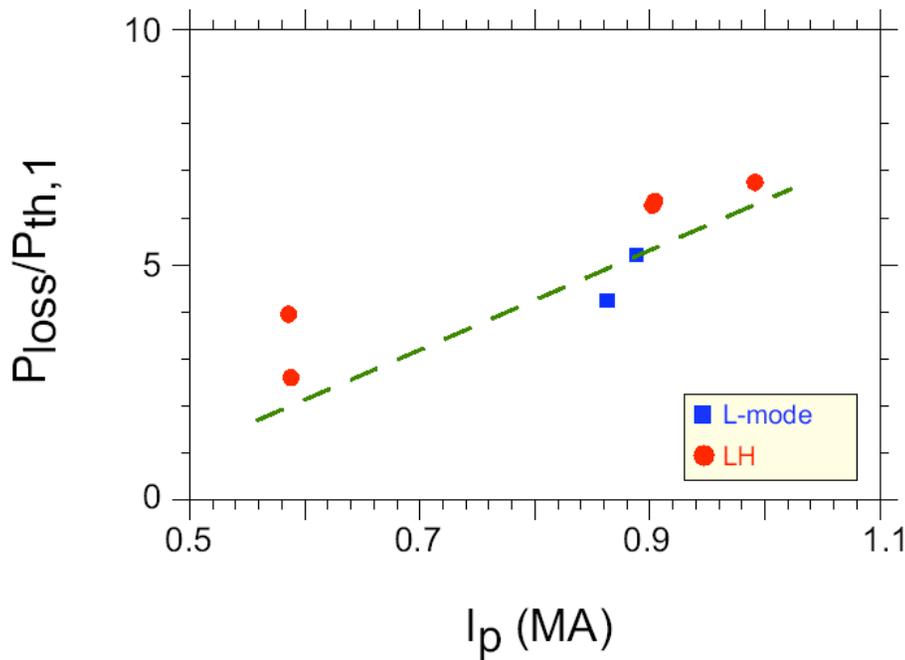
Porter (LLNL)

L-H power threshold and ELM studies reveal differences with conventional aspect ratio tokamaks

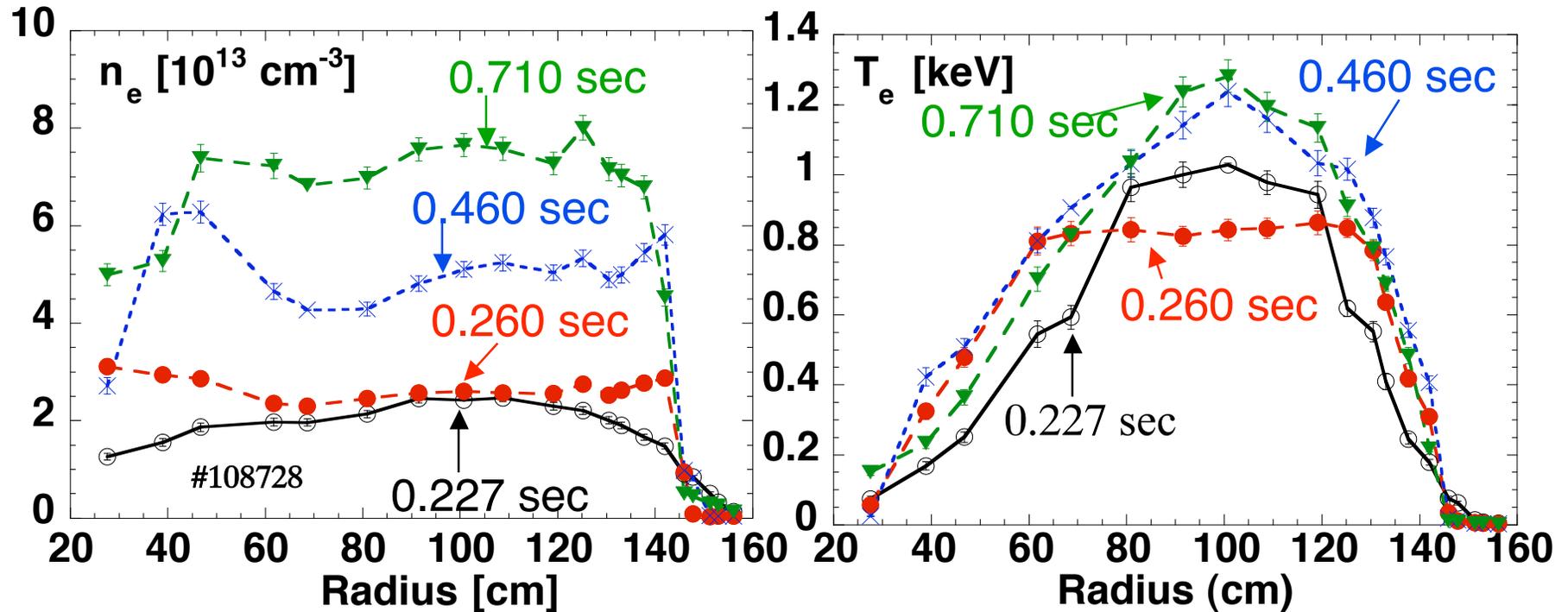


- L-H transition: $P_{L-H}^{NSTX} > P_{th,1}$
- I_p dependence
- related to E_r through fast ion loss?

$$P_{th,1} \sim n_e^{0.61} B_T^{0.78} a^{0.89} R^{0.94}$$



High n_e and relatively low T_e pedestal observed



- n_e profile hollow after transition and fills in 300-500 ms
- Moderate in/out n_e asymmetry usually observed
- T_e profile flattens initially and peaks later in time

Boundary transport and H-mode characterization

FY 03-04 Plans



- Boundary Turbulence and transport:
 - Extend intermittent transport studies (L/H, density limit)
 - Extend radial correlation length studies
 - Determine if classic divertor heat flow models apply
 - Measure SOL midplane width scaling
- L-H transition, ELMS and pedestal physics:
 - Continue power threshold scaling; submit to ITER database
 - Identify dimensionless variables controlling L-H; test theories
 - Study role of E_r on transitions with gap, I_p variations, NBI vs RF
 - Continue studies on role of fueling in H-mode access (**supersonic and new gas injectors**)
 - Characterize edge pedestal; compare w/MAST, DIII-D
 - Characterize ELM sizes and dependencies

*** New capabilities**

Boundary transport and H-mode characterization

FY 05-08 Plans



- Boundary Turbulence and transport:
 - extend studies of low- and high-k turbulence, comparison with non-linear gyrokinetic results
 - extend intermittent transport studies
 - measure density dependence of boundary transport (**pellets, pump**)
 - Measure SOL kinetic effects on SOL transport
- L-H transition, ELMS and pedestal physics:
 - Co vs ctr NBI for assessment of E_r on L-H (**poloidal CHERS**)
 - height, width, and maximum gradient scalings of pedestal (**more edge TS channels**)
 - role of fueling profile in setting pedestal density width (**pellets, pump**)
 - role of shape and fueling in ELMs; conductive vs. convective loss
 - optimization of ELMs for density and impurity control
 - edge transport barriers with **CT injection**

* **New capabilities**

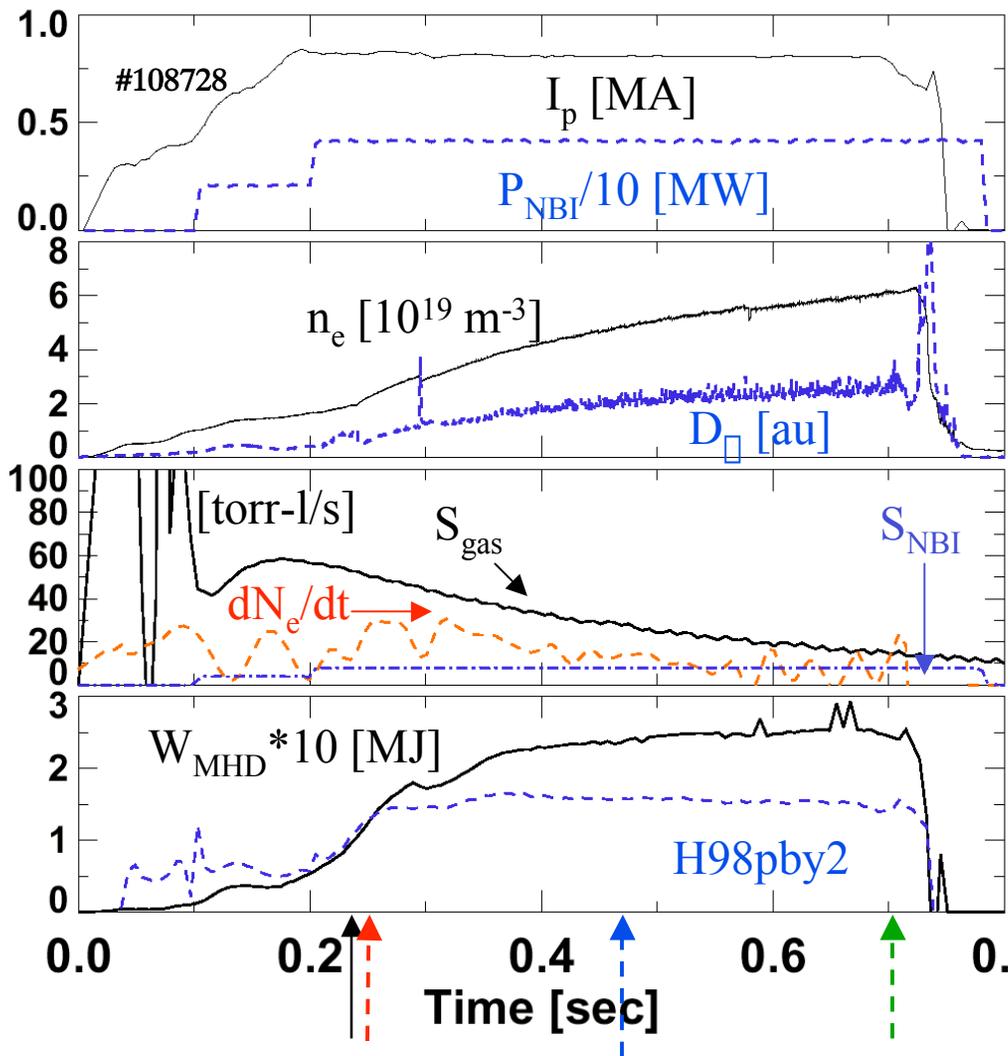
Outline



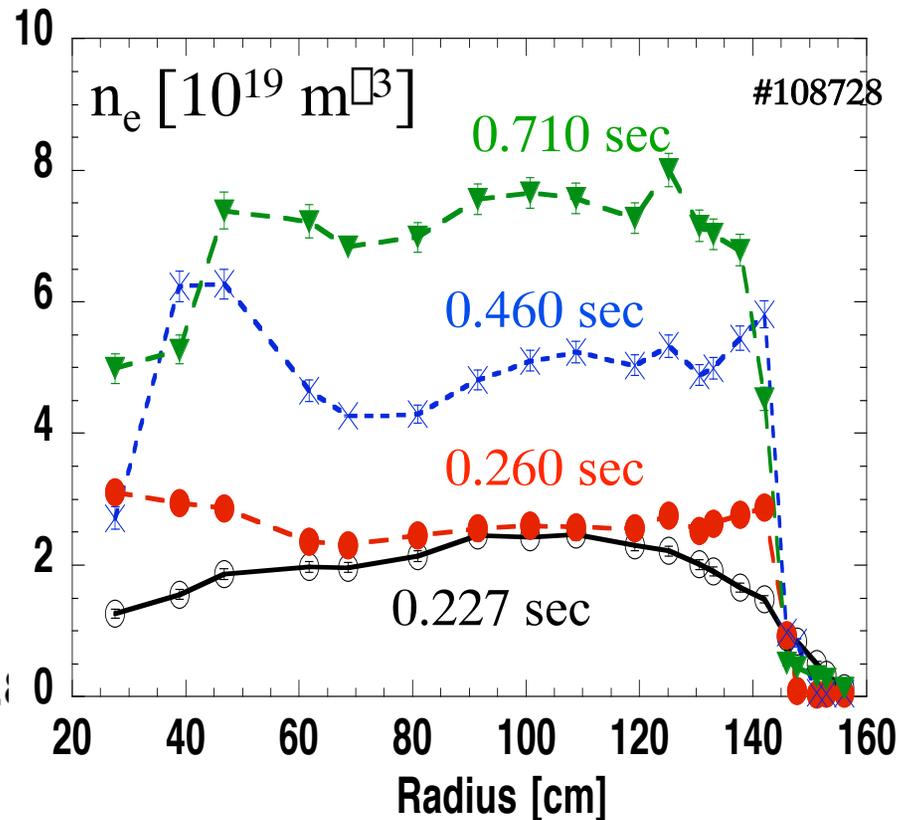
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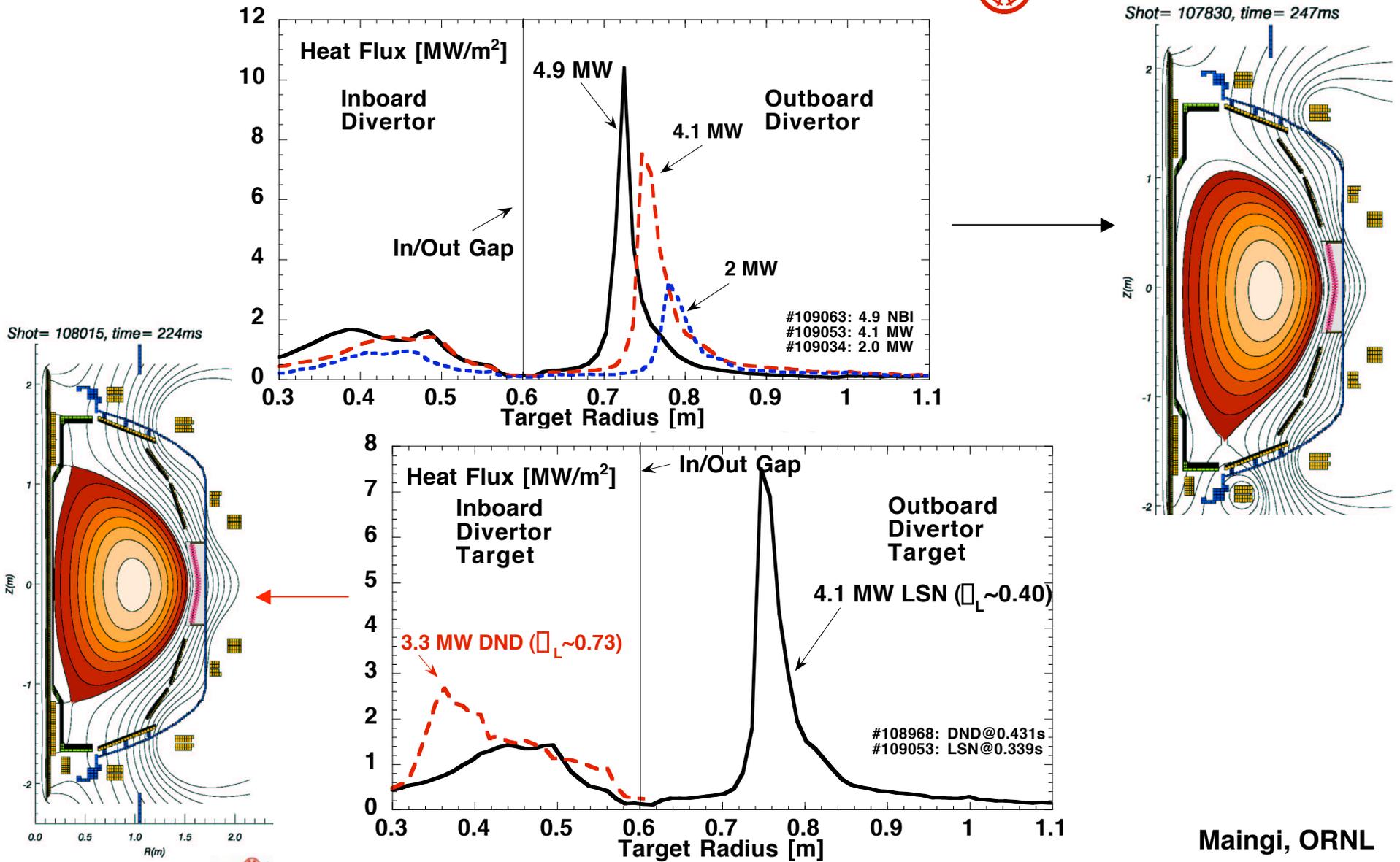
Uncontrolled (non-disruptive) density rise in long pulse H-modes



- Density control needed for improved current drive efficiency, transport studies, and power and particle handling research



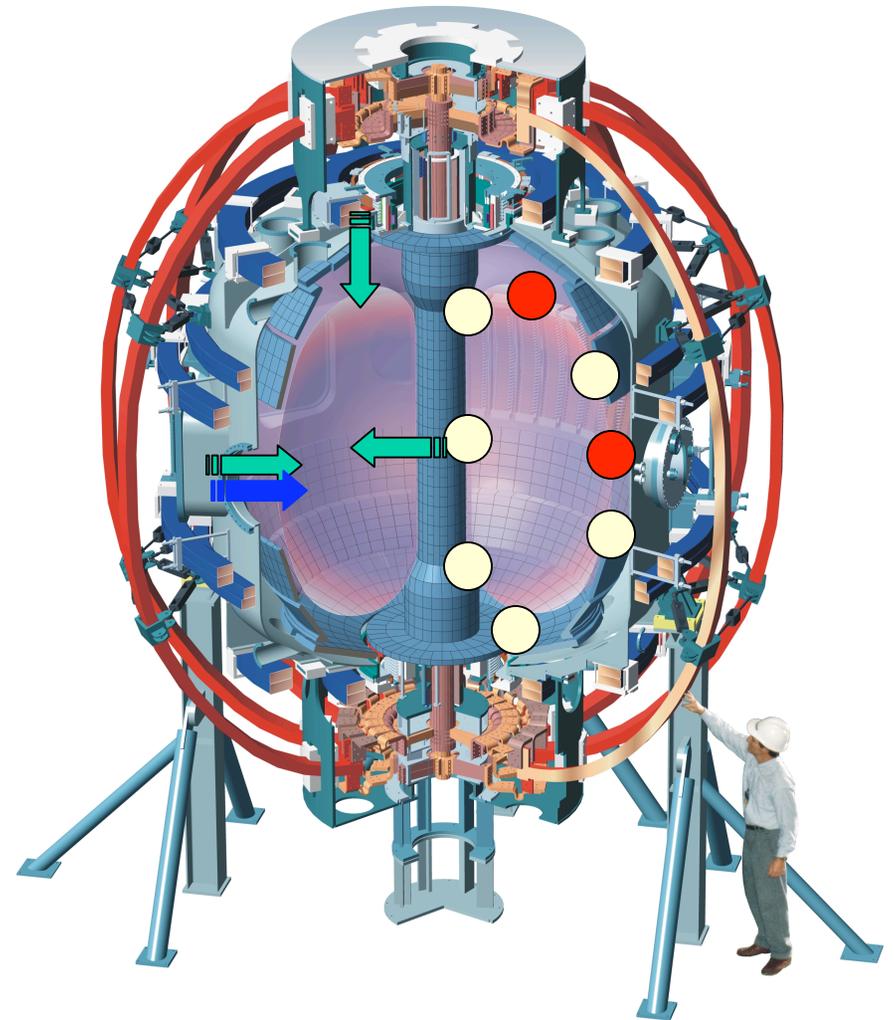
Peak heat flux increased with NBI power in lower-single null, but lower in double-null



Fuel and impurity source control plan



- Improved source control
 - more poloidal locations for gas (FY03-04) ●
 - supersonic gas nozzle (FY03-04) ●
 - Lithium pellets (FY03) →
 - D₂ pellet injection (FY05) →
 - CT injection? (FY07) ◊?
- New diagnostics
 - D_α cameras for core fueling
 - pellet diagnostics
 - more edge Thomson channels
 - divertor Langmuir probe upgrade



* New capabilities

Fuel and impurity sink control plan

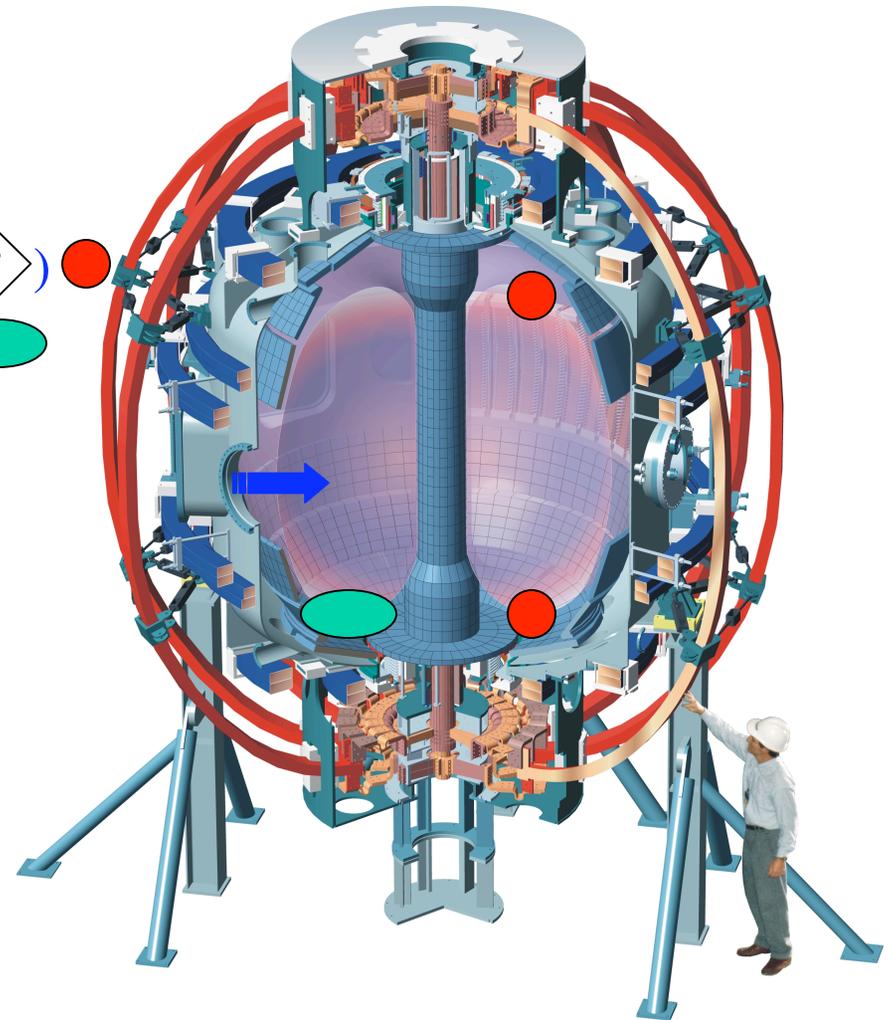


- Improved sink (density) control

- improved boronization (FY03)
- lithium pellets (FY03) 
- in-vessel cryopumps (FY06, FY04 ) 
- lithium module (FY08, FY06 ) 

- New diagnostics

- $n_C(r)$ f/CHERS, Z_{eff} f/MPTS (FY03)
- D_{\square} cameras for core fueling
- divertor SPRED
- upgraded Langmuir probe array
- fast pressure gauges



* New capabilities

Power handling and mitigation plan



- Experimental plan
 - Power balance studies (FY02+)
 - divertor heat flux vs core and divertor radiation
 - parallel vs perpendicular transport
 - Detailed comparison between single-nulls and double-nulls (FY03+)
 - Heat flux reduction studies (FY03+)
 - Impact of fast events, e.g. ELMs, IREs, and disruptions (FY05)
- New Diagnostics
 - Cross-calibrate core bolometry with platinum-foils (FY03)
 - Add and optimize divertor bolometer channels (FY03-04)
 - Add two slow and one fast infrared cameras (FY04, FY05, FY07)

* **New capabilities**

FY03

04

05

06

07

08

09

IPPA: 5

IPPA: 10

Confinement scalings

Momentum & power balance, strong electron & ion heating, low and high beta

Boundary transport & turbulence

Pedestal characterize

Local transport & turbulence
Low/high k spectra theory/experiment, low/high beta

Poloidal flows

V_{ϕ} shear & w/error field

α & pellets

Feedback with heating, CD, V_{ϕ} , fueling

Understanding

$P(r), J_{BS}$
optimize

MSE CIF

CHERS 51 ch

Imaging X-ray crystal

Edge v_{ϕ}

Li pellets

MPTS upgrades

Turbulence diagnostics
Initial

Error field coils

Poloidal
CHERS

He beam spectroscopy

Advanced

Neutron collimator

D pellets

MSE LIF (J, E_r, P);
polarimetry

Fast CHERS (edge)

Liquid Li?
CT injection?

Feedback with MSE, heating, CD, rtEFIT

1 – 3 MW EBW

Cryopump

Impurity injector

Solid state neutral particle detectors

Transport tools

Predictive Transp (GLF23, Multi-Mode)

GS2 linear, non-linear

GTC trapped e⁻ Finite α High α

Gyro non-adiabatic e⁻

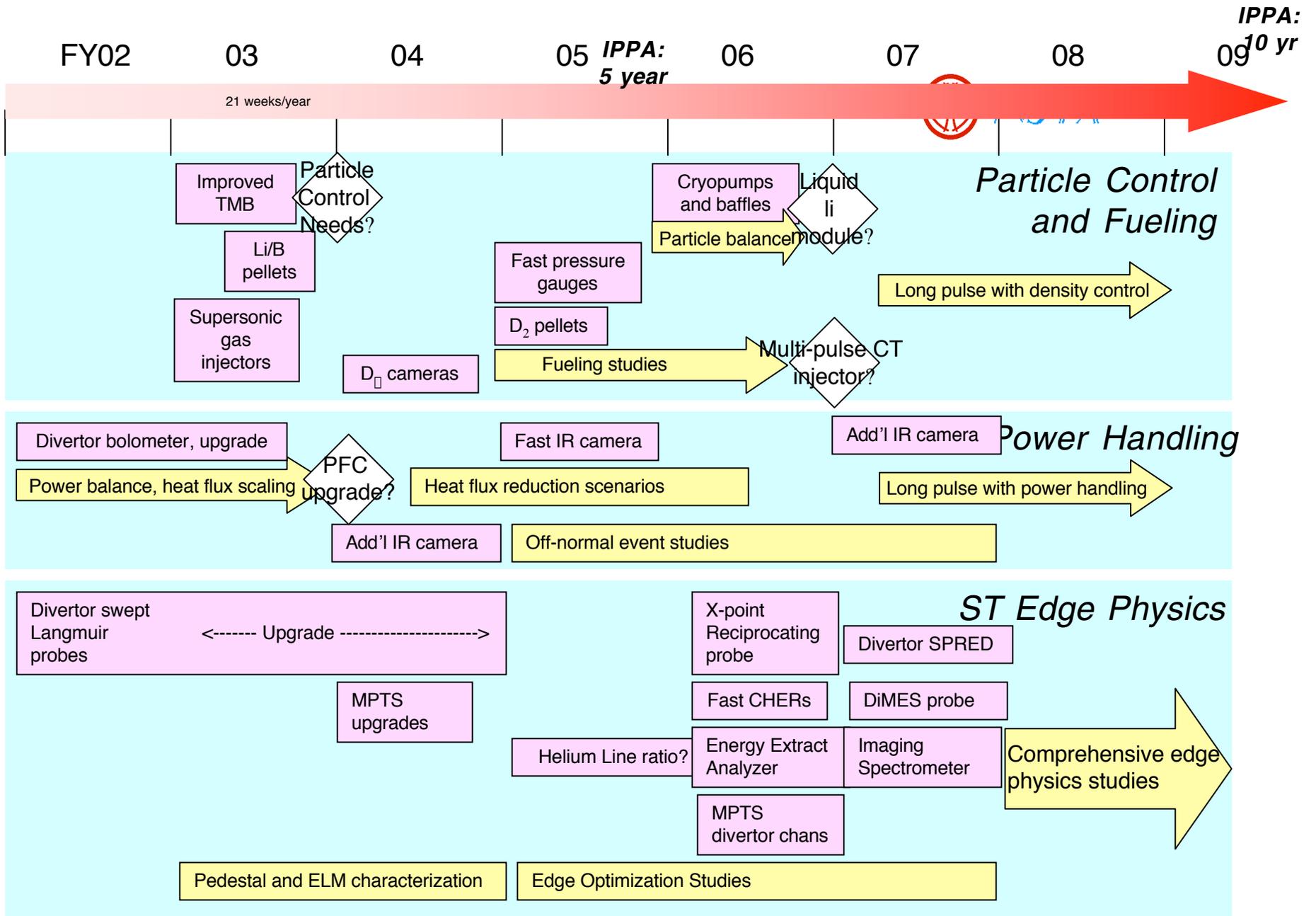
Neoclassical: beam-thermal friction, potato orbit, high α/L , B_{pol}/B_{tor}

Non-linear CAE

Anomalous heating models

Transport
theory

Full predictive
transport simulations



Summary

FY02

03

04

05

06

07

08

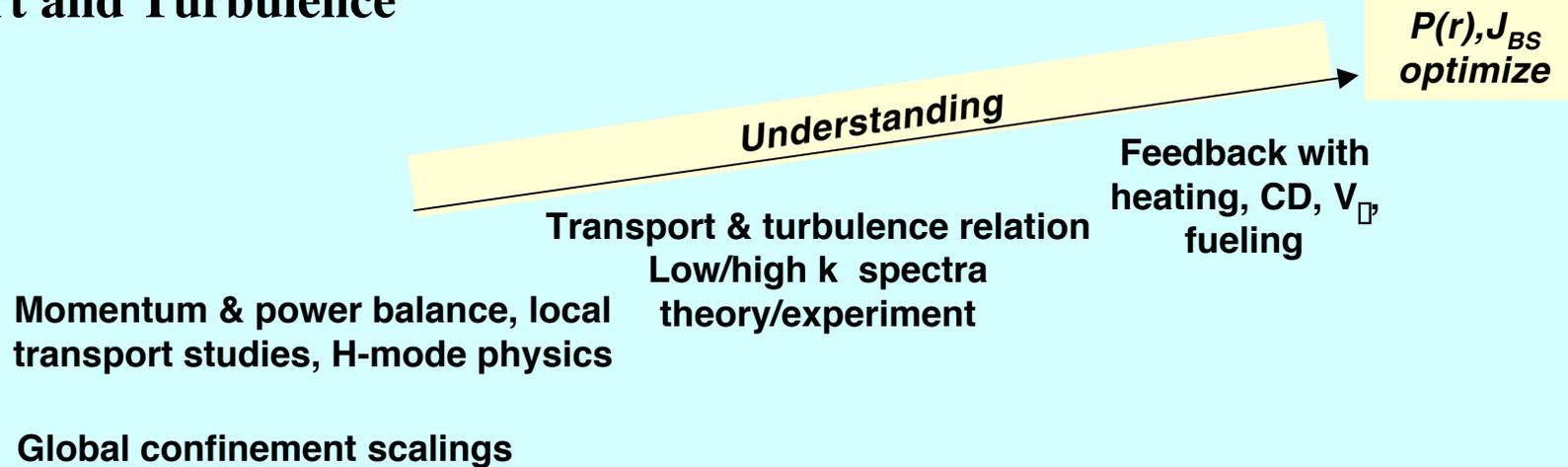
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IPPA attractiveness;
stability and high beta, $\tau_{pulse} \gg \tau_E$

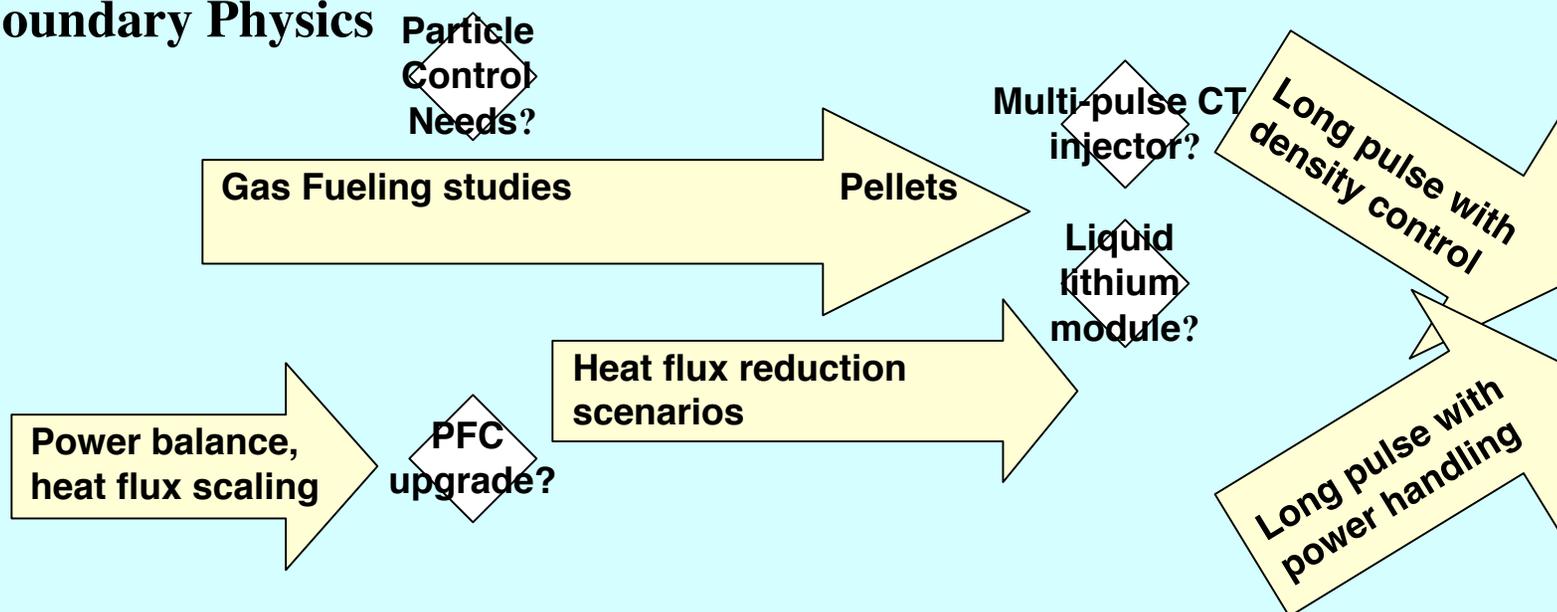
IPPA: extrapolable,
 $\tau_{pulse} \gg \tau_{skin}$, high performance



Transport and Turbulence



Boundary Physics



Backup



Transport Goals are Related to IPPA Goals



- **Five year goal (IPPA 3.1.1)**
 - Advance the scientific understanding of turbulent transport, forming the basis for a reliable predictive capability in externally controlled systems
- **Ten year goal**
 - Develop fully integrated capability for predicting the performance of externally controlled systems including turbulent transport, macroscopic stability, wave particle physics and multi-phase interfaces

Boundary goals are linked to IPPA goals

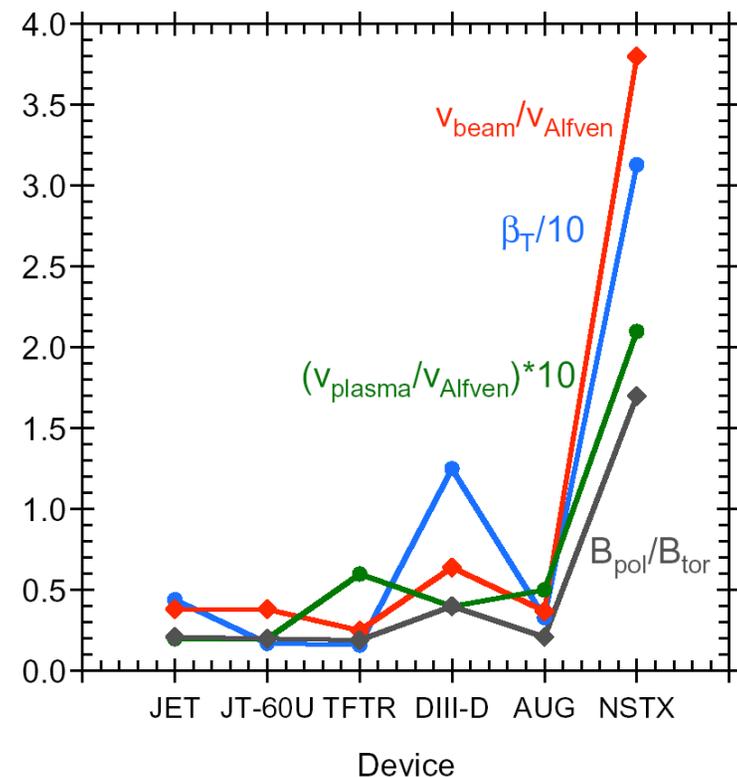
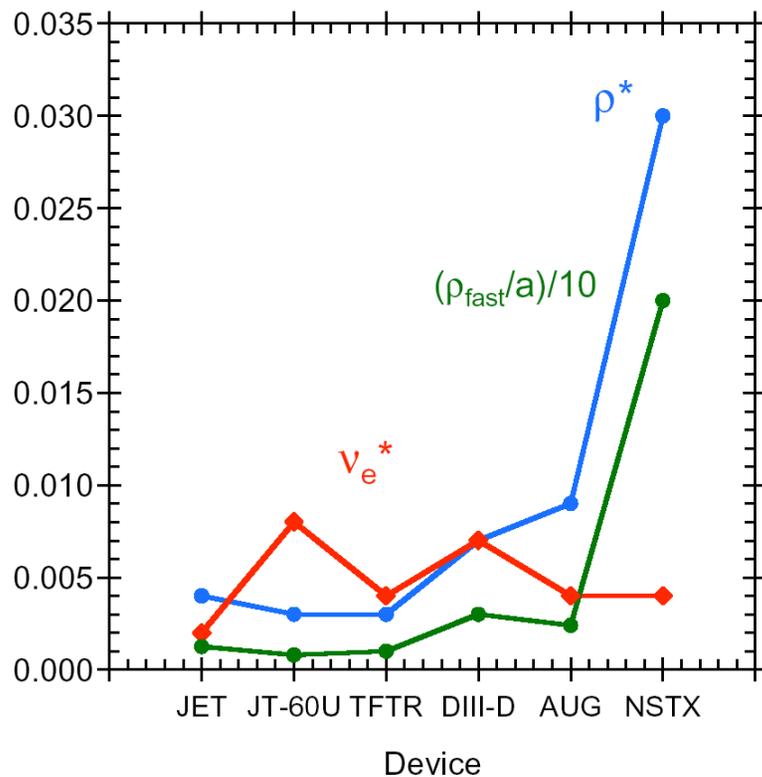


- *5-Year Objective*: Make preliminary determination of the attractiveness of the spherical torus (ST), by assessing high- β_T stability, confinement, self-consistent high bootstrap operation, and acceptable divertor heat flux, for pulse lengths much greater than energy confinement times.
- Implementing Approach 3.2.1.5:
 - Disperse Edge Heat Flux at Acceptable Levels
 - Study ST specific effects
 - high mirror ratio
 - high flux expansion

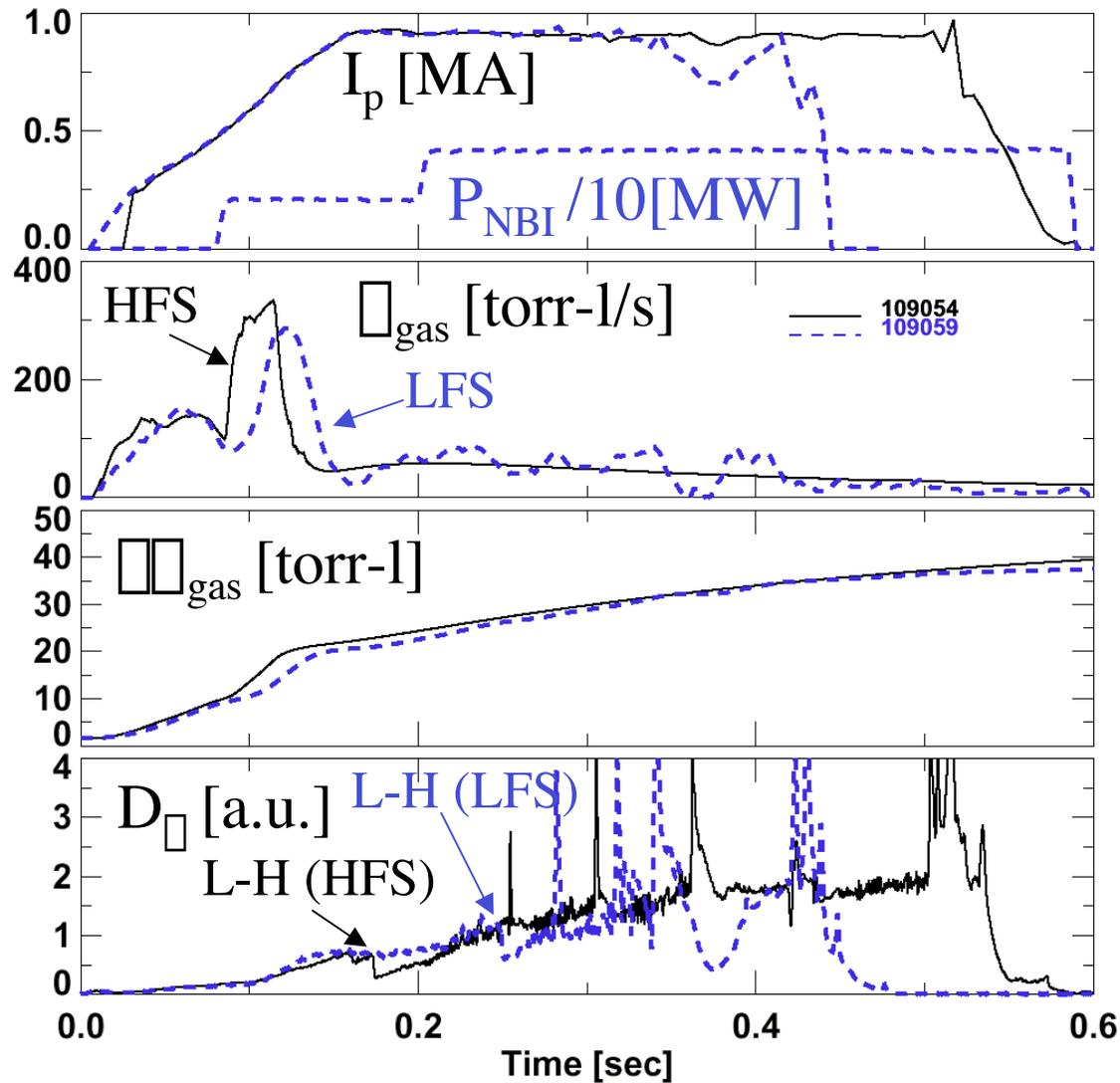
NSTX Offers Unique Opportunities



Low B_T \square Large gyro-scale lengths, high β_T
 Low R/a \square Large field line pitch on LFS
 Relatively high beam energy \square High relative velocity



High-Field Side Gas Injector Fueling Allows Early L-H Transition and Longer H-mode Duration



Theory and Modeling - Plans



- **FY03:** Predictive TRANSP using analytic estimates of χ 's; validate GK codes in low R/a regime (benchmark); update neoclassical (beam-thermal friction, large χ/L); non-adiabatic fast ions; edge transport modeling
- **FY04:** Predictive TRANSP analysis with χ 's from non-linear GK runs; incorporate large χ^* , χ_T , f_T , shaping effects into GK codes; non-linear calcs of CAE mode amplitudes; start to compare turbulence measurements to theory
- **FY05:** Incorporate ExB, non-local effects in GK; extend theory/exp't comparison of turbulence; start developing anomalous heating models
- **FY06-08:** Start to develop high-confidence predictive capability; combine with MHD stability to form self-consistent package for integrated scenario development

Edge, SOL, divertor and wall conditioning physics background



- Classical divertor heat flow regimes applicable?
 - high divertor T_e , sheath-limited heat flow
 - low divertor T_e , conduction-limited heat flow
 - detachment
- Impact of unique ST features
 - high SOL mirror ratio, short connection length, high flux expansion
 - in/out B_t ratio and target; E X B flows
- Cross-field transport: diffusive vs. intermittent
 - intermittent transport appears strong in both L and H-modes
 - any cross-correlation with ELMs?
- Wall conditioning technique assessment
- Plan: near-term focus on nature of cross-field transport, and longer term on ST kinetic effects

H-mode, pedestal and ELM physics background



- L-H transition physics (common with transport group)
 - P_{LH} higher than scalings - trapped particles, poloidal damping?
 - I_p and maybe B_t dependence of P_{LH} appears different
 - role of $E \times B$ shear?
- Pedestal: profile shapes, gradients and widths
 - density pedestal high and in/out asymmetry
 - pedestal $T_e \leq 400$ eV, below predictions
- ELMs
 - Tokamak-like ELMs in double-null
 - Usually ELM-free or very large events in single-null
 - Fueling affects ELM type strongly
- Plan: characterization studies over next few years and optimization studies over longer term

Fuel and impurity particle control background



- Center stack gas injection enabled routine H-mode access
 - higher β due to lower pressure peaking factor
 - long pulse due to low volt-second consumption rate, owing to high bootstrap fraction from edge pressure gradient
- Limited control leads to uncontrolled density rise
- Location of gas fueling source affects H-mode access
- Goal: make density independent of time
- Staged plan with parallel components
 - improve control of sources, aiming toward higher efficiency
 - improve control of sinks via wall conditioning and active pumping

Power handling and mitigation background

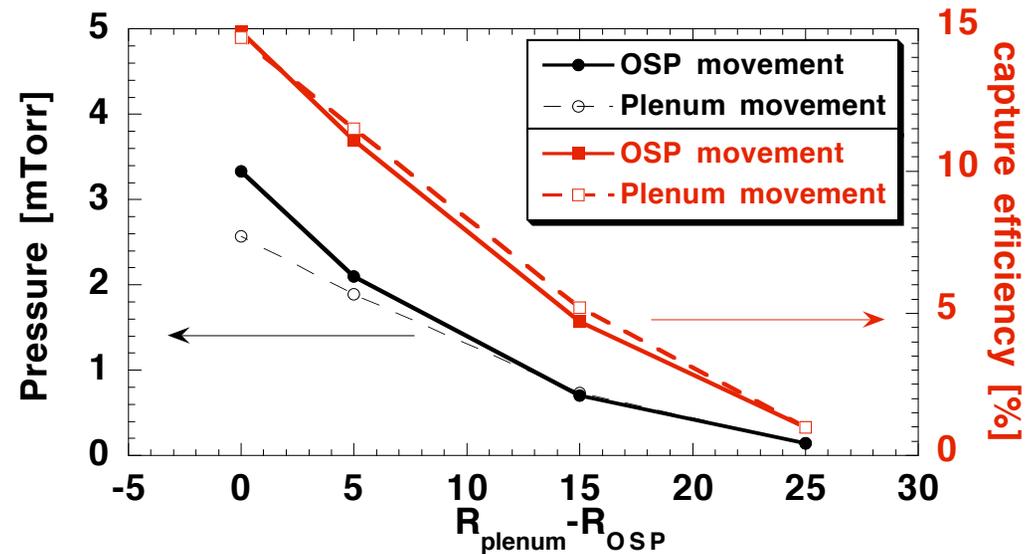
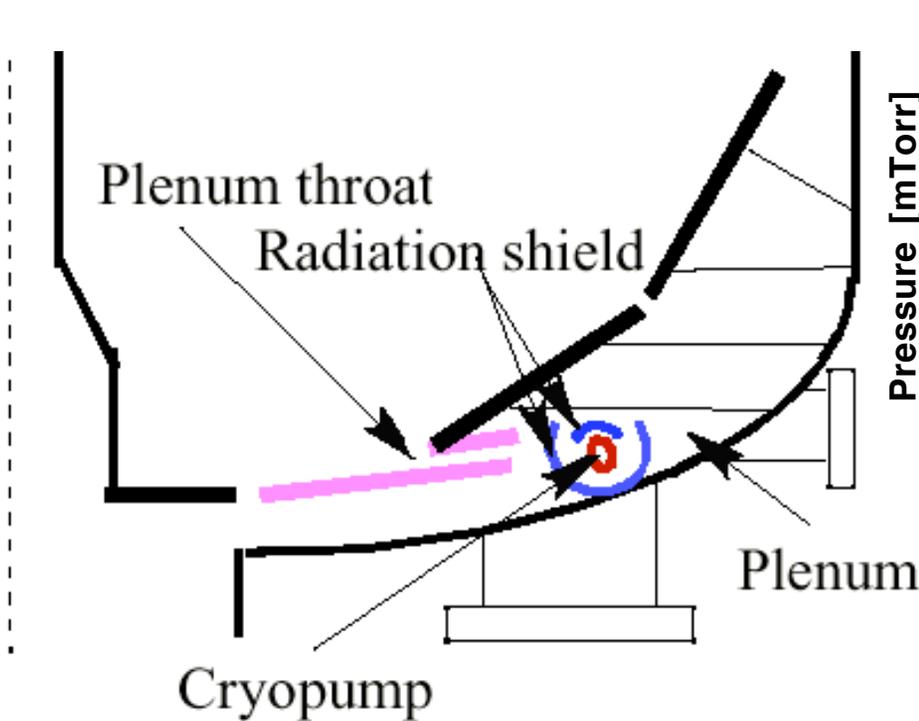


- ST's can have high heat flux because of high P_{heat}/R
 - NSTX: $P_{\text{NBI}} \sim 7 \text{ MW}$, $P_{\text{RF}} \sim 6 \text{ MW}$, $P_{\text{heat}}/R \sim 15.3$
- Highest q_{peak} in NSTX $\sim 10 \text{ MW/m}^2$
 - $\Delta T_{\text{div}} \sim 300 \text{ }^\circ\text{C}$ in LSN
 - extrapolates to $\sim 3 \text{ sec.}$ pulse length limit ($\Delta T_{\text{div}} \sim 1200 \text{ }^\circ\text{C}$)
- For $T_e \leq 2 \text{ keV}$, current diffusion time $\ll 3 \text{ sec.}$
 - If T_e increases, current diffusion time could approach few seconds
- Goal: assess power balance and survey heat flux in many scenarios (vs. shape, input parameters, etc.)
- Staged plan
 - quasi-steady power balance over next few years
 - transient events in future

Preliminary neutral transport studies show possible plenum pressure sensitivity to strike point



- Same model and n_e , T_e , P_e profiles for DIII-D as in Maingi, et. al., *Nucl. Fusion* **39** (1999) 1190
- Ion current = 5kA, used for normalization
- Leakage conductance = 20 m³/sec



Menon, ORNL

Wall conditioning physics plan



- Wall conditioning technique assessment (FY03+)
 - how does boronization change wall physics?
 - how do those changes improve the core plasma?

- New Diagnostics
 - quartz crystal deposition monitors for real time flux
 - divertor SPRED for impurities
 - divertor materials probes (like DiMES on DIII-D)
 - plus all other divertor diagnostics...