

# NSTX-U/Theory Partnership

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NSTX-U PAC  
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## *Charge to the NSTX-U PAC*

**Charge:** Please comment on the plans for the NSTX-U/PPPL Theory Partnership, and how well this Partnership and the broader NSTX-U research activities support “integrated predictive capability.”

**Response:** To help answer this charge, we will provide a discussion of NSTX-U Theory Partnership status and future plans, and how the plans are aligned with the recent FES Workshop on Integrated Simulations for MFE Sciences.

# ***NSTX-U/Theory Partnership well-integrated into the NSTX-U Science and Topical Group structures***

- Theory Department leadership in SGs/TSGs
- \$1M funding, which was incremental in  $\leq 2014$ , is now a permanent part of NSTX-U budget
- NSTX-U funding of Theory Department researchers  $\sim 4.7$  FTEs
- Synergistic support through SCIDAC/Base Theory funding/Collaborators/NSTX-U project theorists (e.g., W. Guttenfelder, J.-K. Park, Z. Wang)
- Partnership facilitates collaboration over a wide range of topics

# PPPL Flagship Codes

*“The committee considers the three “Flagship codes” to be a very positive step.”  
(Advisory Board, November 2014)*

## Extended MHD

**M3D-C<sup>1</sup>** (supported by SciDAC project CEMM, S. Jardin, *Lead*): Extended MHD, fully implicit algorithm, enabling long runs over transport time scales.

## Gyrokinetics

**XGC** (supported by SciDAC project EPSI---C.-S. Chang, *Lead*): full-*f* code, focused on edge physics, including X-point geometry, electromagnetic effects (implemented).

**GTS** (W. Wang, *Lead*):  $\delta f$  code focused on core physics with options for shaped boundary, electromagnetic effects (in progress).

(DOE Notable Outcome on Electromagnetic Effects in Gyrokinetic PIC codes, 2015)

## Other major codes

**TRANSP** (Report of a new PPPL Committee, R. Hawryluk, *Chair*)

In addition to PPPL codes, we use **GYRO**, **GENE**, **GS2**, **NIMROD**, **AORSA**, and **TORIC** for design, analysis, and interpretation of experiments.



REPORT OF THE WORKSHOP ON INTEGRATED  
 SIMULATIONS FOR MAGNETIC FUSION  
 ENERGY SCIENCES  
 JUNE 2 – 4, 2015



Sponsored by the Office of Fusion Energy Sciences and the Office of Advanced Scientific Computing Research

## ***FES Workshop Report, June 2-4, 2015***

Assessed recent progress and gaps and challenges in fusion theory and computation directly related to theory and computation directly related to integrated simulations. The *science applications* were focused on:

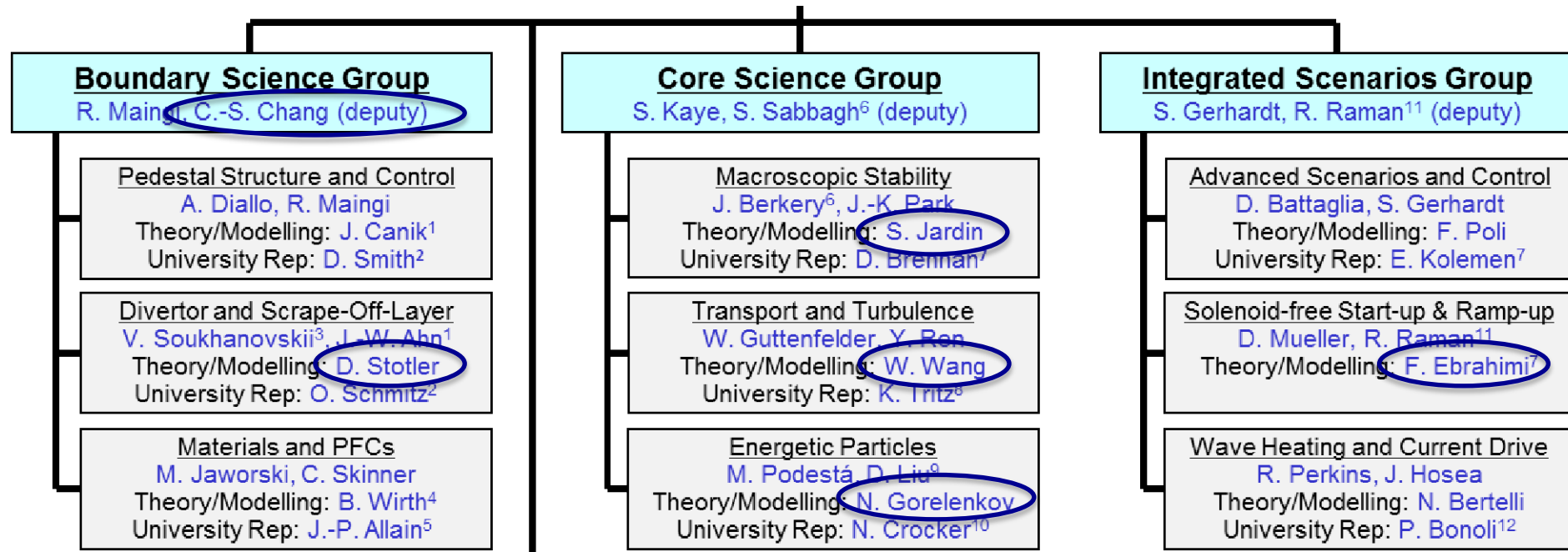
- Disruption physics (aligned with goals of M3DC<sup>1</sup>/M3DC<sup>1</sup>-K) (*PPPL participants*: Brennan, Ferraro, Fu, Gerhardt, Jardin)
- Plasma boundary physics (aligned with goals of XGC1) (*PPPL participants*: Chang, Hammett)
- Whole device modeling (aligned with goals of TRANSP modernized) (*PPPL participants*: Grierson, Kaye, Poli)

The crosscutting ASCR areas in *mathematical and computational enabling technologies* were:

- Multiphysics and multiscale coupling
- Numerical optimization and uncertainty quantification
- Data analysis, management, and assimilation
- Software integration and performance

*PPPL observers*: Bhattacharjee, Zarnstorff

# *NSTX-U/Theory Partnership well-integrated into the NSTX-U Science and Topical Group structures*



# ***NSTX-U Mission Elements***

## ***5 Highest Research Priorities***

- Explore unique ST parameter regimes to advance predictive capability - for ITER and beyond
  1. Understand confinement and stability at high beta and low collisionality
  2. Study energetic particle physics prototypical of burning plasmas
- Develop solutions for PMI challenge
  3. Dissipate high edge heat loads using expanded magnetic fields + radiation
  4. Compare performance of solid vs. liquid metal plasma facing components
- Advance ST as possible FNSF / Pilot Plant
  5. Form and sustain plasma current without transformer for steady-state ST



# ***Theory addresses NSTX-U research priorities, using analytical and simulation tools that produce novel insights***

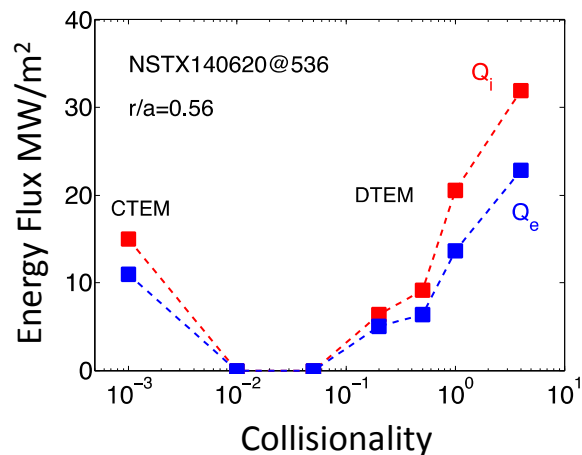
Several aspects of NSTX-U (and more generally, ST) research require rethinking ideas and mechanisms in conventional tokamaks, leading often to new and exciting insights. Examples are:

- Energy confinement scaling at low collisionality, produced by interplay of micro-tearing, trapped electron modes, and strong sheared flows.
- Copious generation of super-Alfvenic ions due to low magnetic fields, exciting Alfvén eigenmodes (AEs) which can provide new mechanisms for beam and thermal transport.
- Novel mechanisms for thermal quench during disruptions. Simulations of new planned experiments for complete maps of halo currents and current ramp-down scenarios, validated by experiments.
- Understanding new physics underlying pedestal structure, and SOL heat flux profiles.

• Directly ITER relevant

# Global, non-linear GTS simulations giving insight into causes of thermal electron and ion transport

- Previously reported role of ITG/K-H in driving ion thermal transport in NSTX L-mode
- Recent simulations have shown possible role of DTEM in contributing to observed favorable collisionality scaling ( $B\tau_{th} \sim v_{*e}^{-0.8}$ )
  - In addition to microtearing
  - Synergy with DIII-D work
- E-M capability presently being implemented in GTS (Startsev, Wang)
  - Supported by Partnership
  - Theory “Notable Outcome” 2015 (available as a pedagogical report)

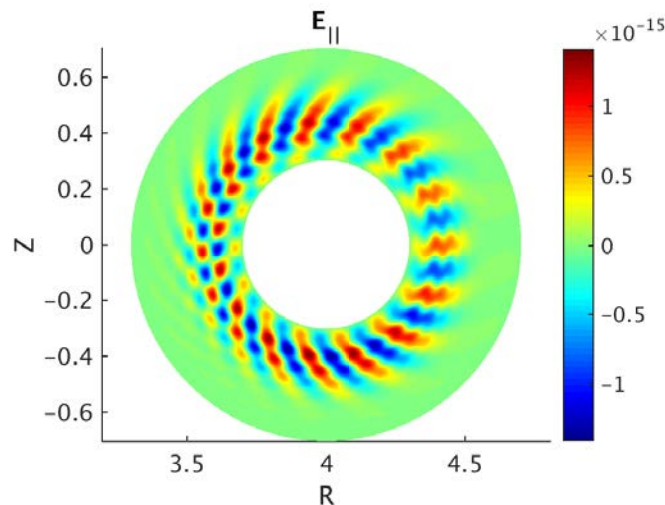


W. Wang et al., NF Letters (2015)

R16-1, 17-3, 18-1, ITER

# Latest Development of EM capabilities in GTS

- Recently, Startsev-Lee EM scheme has been extended to including key toroidal effect and ion dynamics and has been implemented into GTS.
- Currently the scheme is being tested in GTS against recently available results by A. Swamy (IPR India) for collisionless MTM and KBM eigenmodes in tokamaks with circular flux surfaces.



- Result of the GTS simulation with  $n=10$  most unstable mode with tearing parity.

$$R / L_{Te} = R / L_{Ti} = 20, R / L_n = 5, v_e = 0$$

$$\beta_e = 0.5\%$$

$$R / a = 4$$

$$a / \rho_s = 50$$

$$m_i / m_e = 1836$$

- The general geometry version of the Startsev-Lee em scheme including all magnetic drifts has been formulated and is currently being implemented into GTS.

# *Theory addresses NSTX-U research priorities, using analytical and simulation tools that produce novel insights*

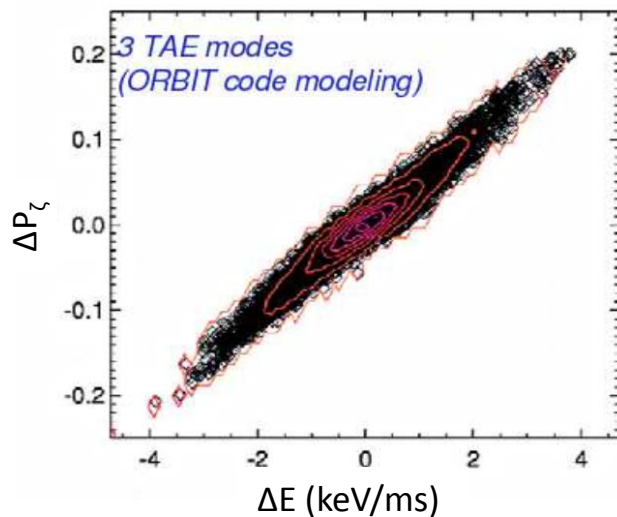
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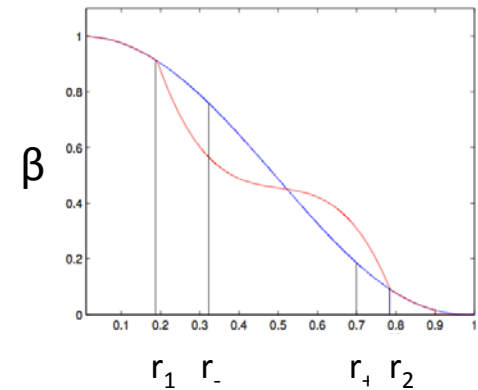
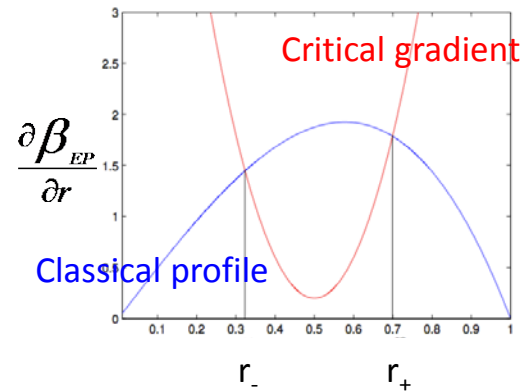
• Directly ITER relevant

# Beam ion confinement: Models for fast ion profile relaxation in presence of AEs under development

- “Kick” model (Podesta)
  - PDF computed by ORBIT (White) in presence of TAEs
  - Mode structures computed by NOVA (Gorelenkov)
  - Kicks  $\sim$  mode amplitude
  - Being used in DIII-D analysis
- Critical Gradient Model (Gorelenkov)
  - Compute critical  $\frac{\partial \beta_{EP}}{\partial r}$  due to AE
  - Mode growth/damping computed by NOVA-K (Fu, Gorelenkov)
  - Originally benchmarked on DIII-D



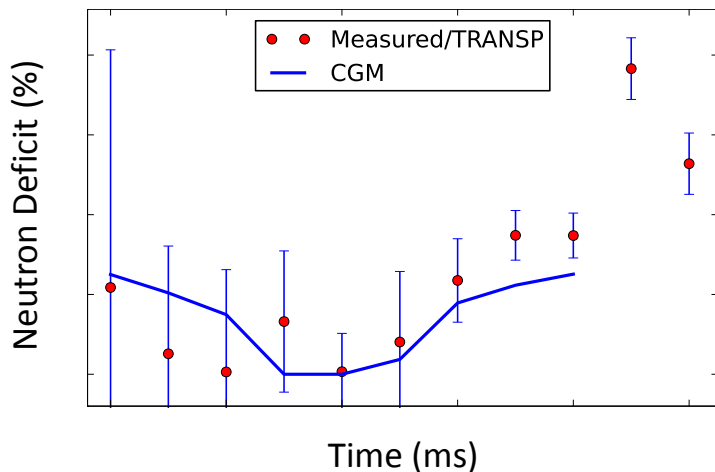
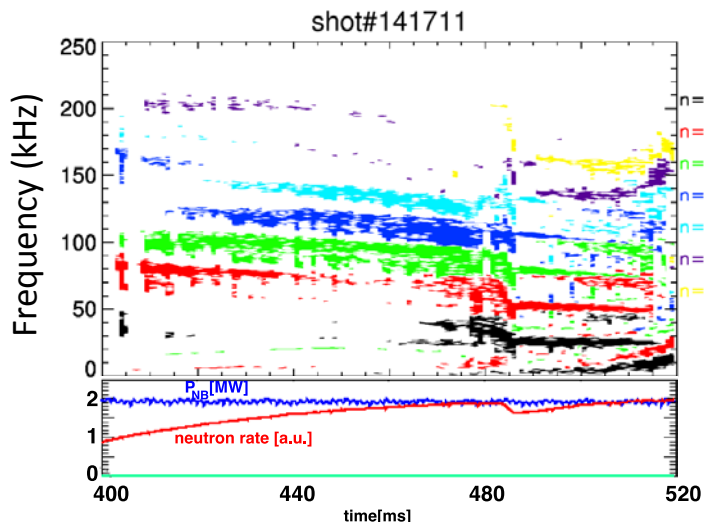
R16-2, R18-2, IR18-2, ITER



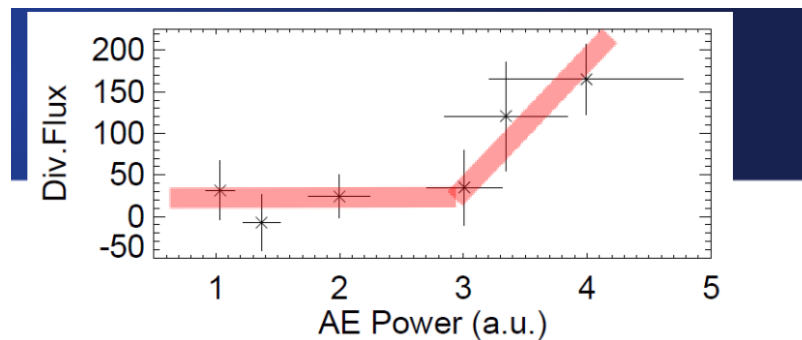
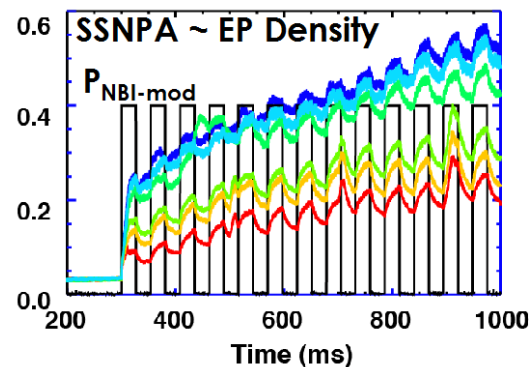
CG used to relax EP profile. It is broadened from the initially unstable one between  $r_\pm \rightarrow r_{1,2}$

# CGM recently validated for NSTX

- Multiple TAE unstable discharge chosen for validation



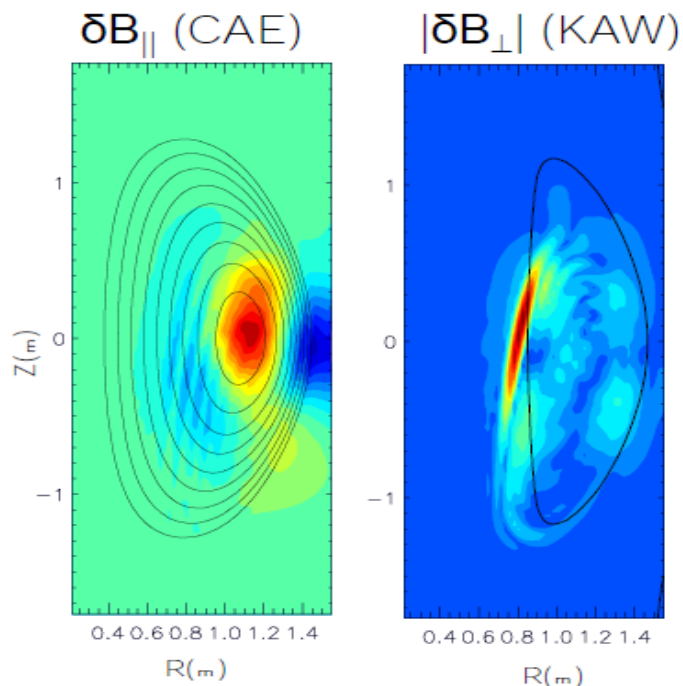
- XP1524 (Heidbrink) directly tests CGM
  - Fashioned after successful DIII-D expt.
  - Different AE activity on NSTX(-U) crucial test for validating theory



Other related XPs include XP1522, 1523 (characterize 2<sup>nd</sup> beam, beam confinement)

# Non-linear HYM simulations of CAE/KAW coupling address power channeling and effect on $T_e$ profile

- Full non-linear simulations including multiple ( $n \geq 4$ ) unstable modes have been performed (Belova)
- $n=4$  (largest mode) saturation amplitude ( $\delta B/B \sim 6.6e-3$ ) comparable to that inferred from reflectometer displacement measurements ( $\delta B/B \leq 3.4e-3$ )



- Calculate channeling of 0.3 to 0.5 MW for  $n=4$
- Can result in several hundred eV modification of  $T_e$  profile ( $T_e$  broadening observed during periods of high CAE activity, with no  $TM_{e0}$ )
- XP1525 (Crocker): Rotation effects on CAE/GAEs
- XP1520 (Kaye), 1523 (Podesta): Effects of collisionality, power on transport



# *Theory addresses NSTX-U research priorities, using analytical and simulation tools that produce novel insights*

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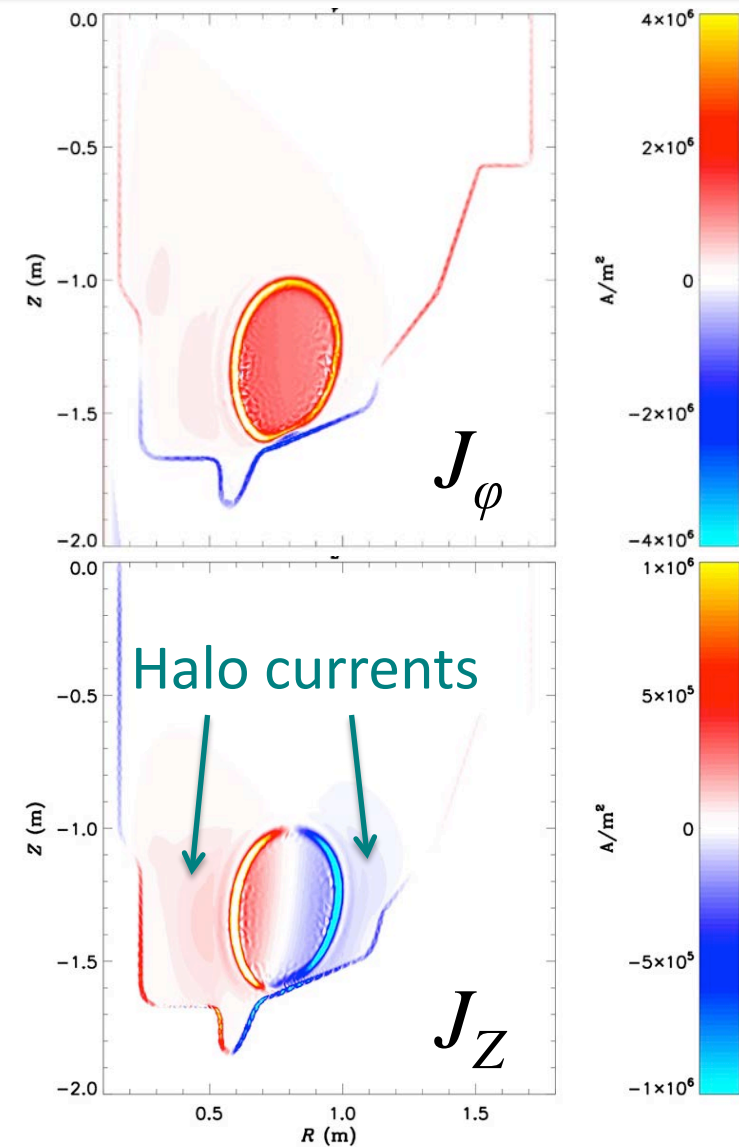
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- **Novel mechanisms for thermal quench during disruptions. Simulations of new planned experiments for complete maps of halo currents and current ramp-down scenarios, validated by experiments.**
- Understanding new physics underlying pedestal structure, and SOL heat flux profiles.

• Directly ITER relevant



# Simulating Disruptions and Halo Currents Using New Modeling Capabilities

- New resistive wall model in M3D-C1 allows unique capability to simulate mode locking and current-quench
- Includes self-consistent treatment of rotation and Halo currents
  - We will seek to validate these models against NSTX-U data

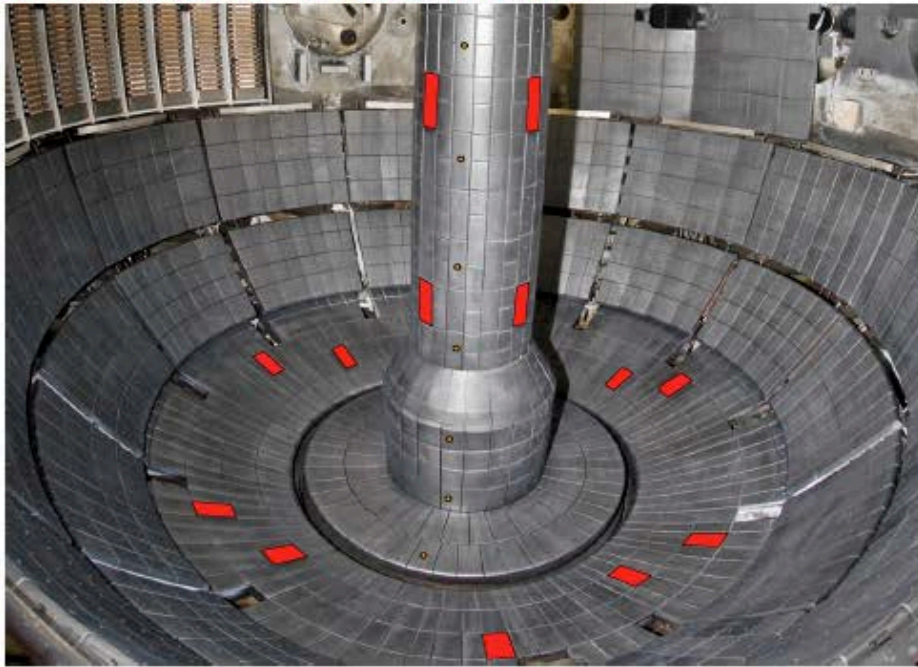


# *M3D-C<sup>1</sup> with RW capability will be used to map out optimal placement of new B-sensors to study halo currents*

- Dynamics of halo currents and forces critical for ITER: particular concern are halo current asymmetries and rotation
- New sensors will measure halo currents, B-fields and JxB forces in NSTX-U
- Critical theoretical issues: (i) role of boundary conditions (ii) halo current distributions in 3D conducting structures (D. Pfefferle)

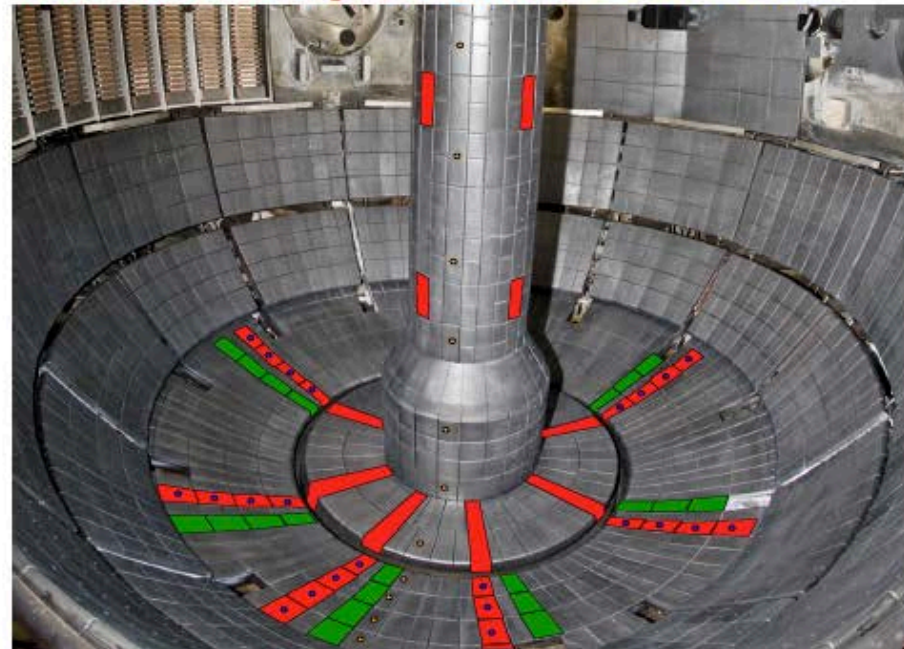
**Planned NSTX-U Base Configuration**

Normal Current Tiles    Single Axis B Sensors



joint NSTX-U/Theory task

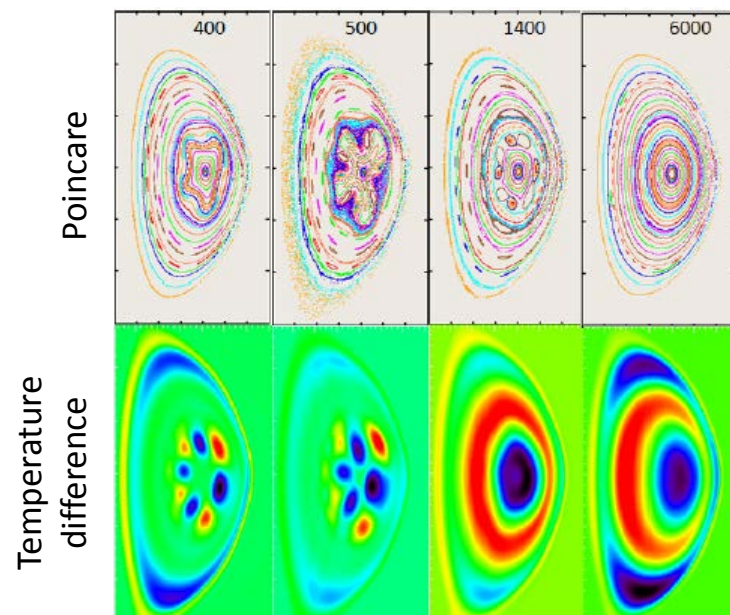
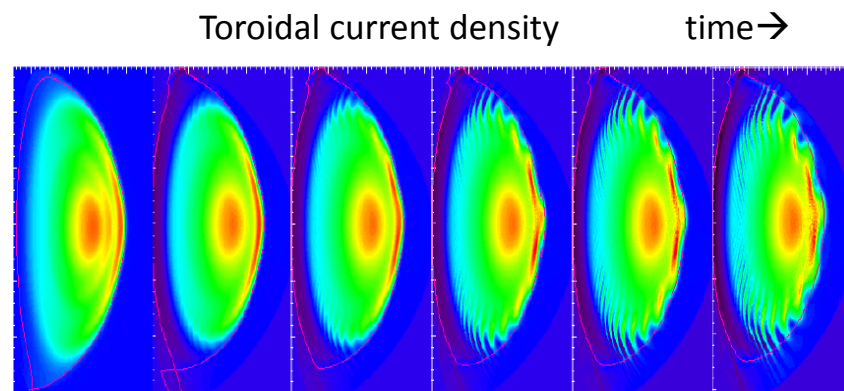
**Potential NSTX-U Expanded Configuration**    Tangent Current Tiles  
Normal Current Tiles    Single Axis B Sensors    Multi-Axis B sensors



R16-3, 17-2, 18-2, 2016 JRT, ITER

# Which linear instabilities lead to thermal quench and disruption?

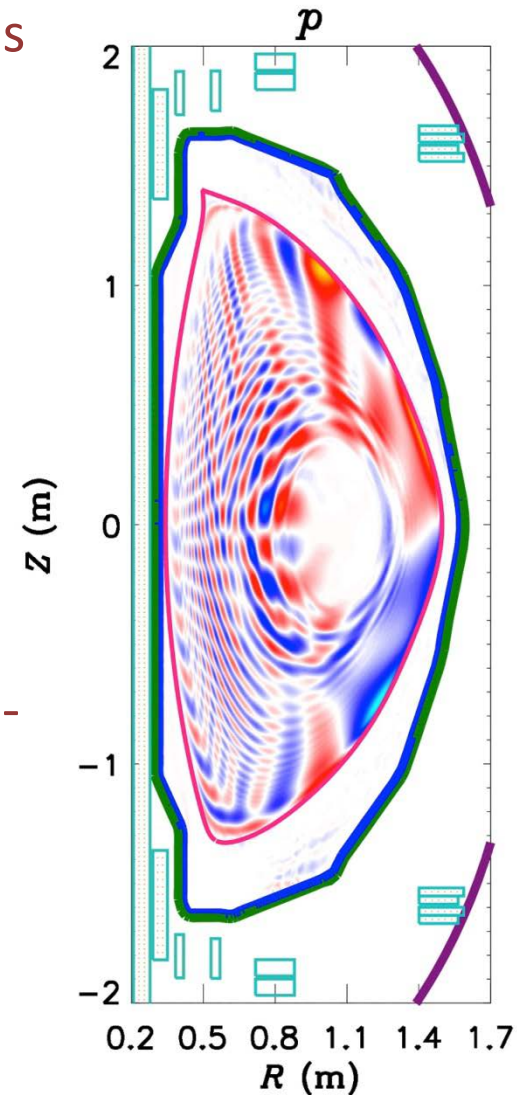
- Rapid end-of-shot current ramp-down in NSTX led to disruption. Nonlinear modeling of this is improving understanding.
- Heating NSTX past the ideal MHD  $\beta$ -limit in some cases just leads to enhanced transport
  - Goal is to predict when this leads to a thermal quench





# Understanding RMP ELM Mitigation in New Regimes

- ELMs in STs and tokamaks respond to applied 3D fields in opposite ways
  - Understanding this discrepancy would substantially improve our confidence in scaling RMP ELM mitigation to ITER
- NSTX-U offers a unique capability to validate ELM suppression models across the ST and tokamak regimes
- Combining models of 3D perturbed equilibrium (M3D-C1) and transport (XGC) will yield a quantitative, predictive model for transport in 3D fields
- M3D-C1 is being used to predict and optimize plasma response to NCC coils in NSTX-U

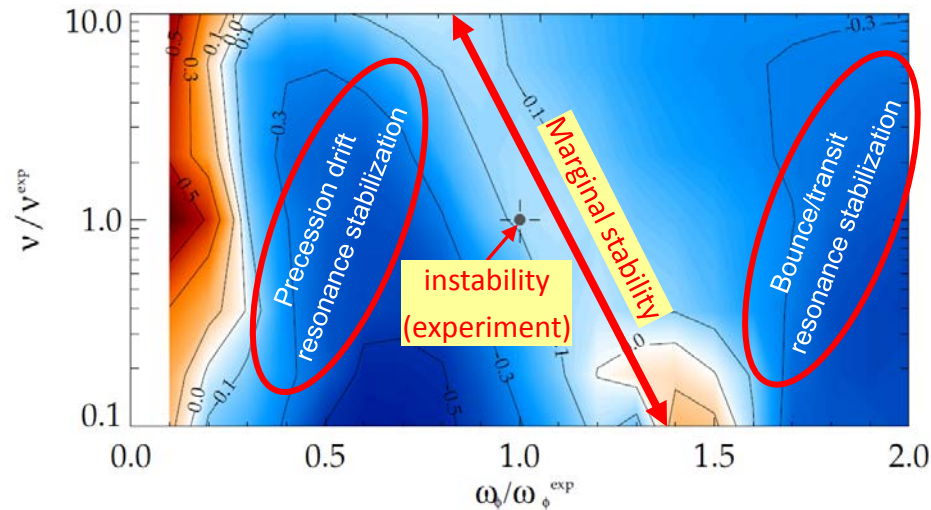


# NSTX-Theory collaborations have led the advancement in kinetic global mode stability physics

## RWM kinetic stability physics theory

$$\gamma\tau_w \approx -\frac{\delta W_\infty + \delta W_K}{\delta W_b + \delta W_K} \quad (\text{Betti, Berkery, Sabbagh})$$

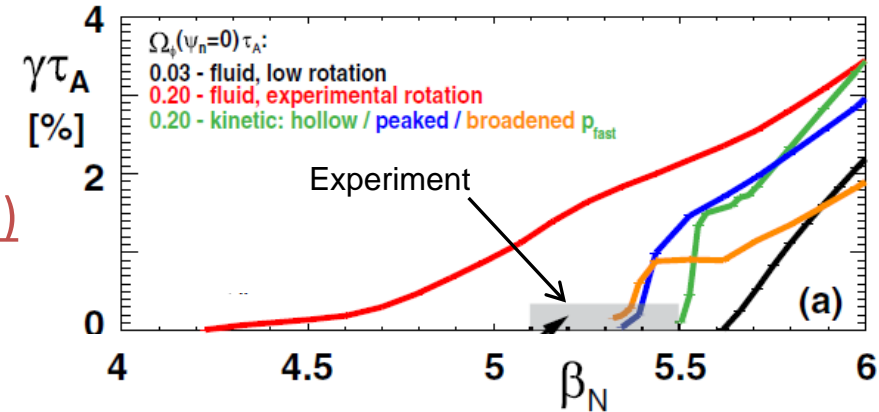
## MISK: RWM kinetic stability (Berkery, Sabbagh)



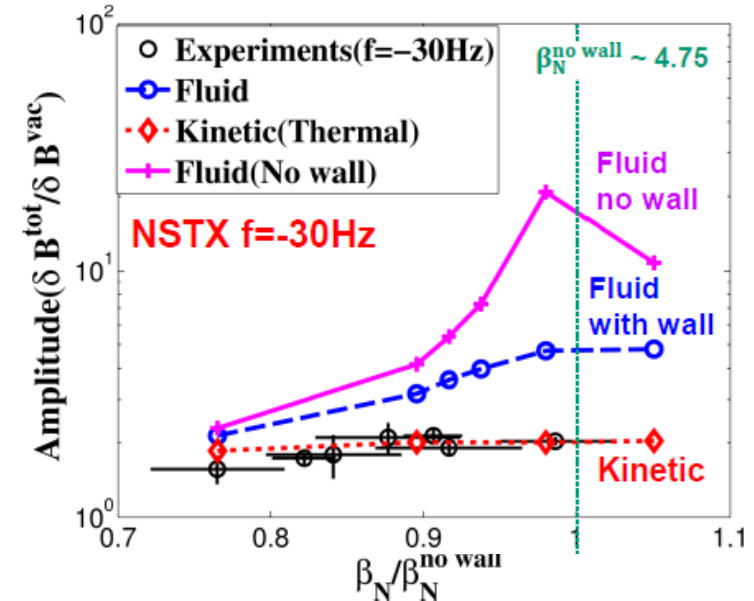
↳ RWM physics with M3D-C<sup>1</sup> (Ferraro)

- Resistive wall implemented

## MARS-K: Ideal wall stability (Menard)



## MARS-K: Resonant field amp. (Wang)



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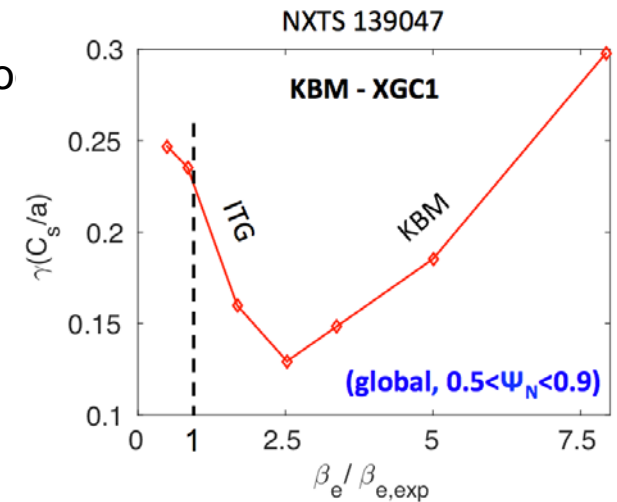
• Directly ITER relevant

# XGC results for NSTX edge plasmas

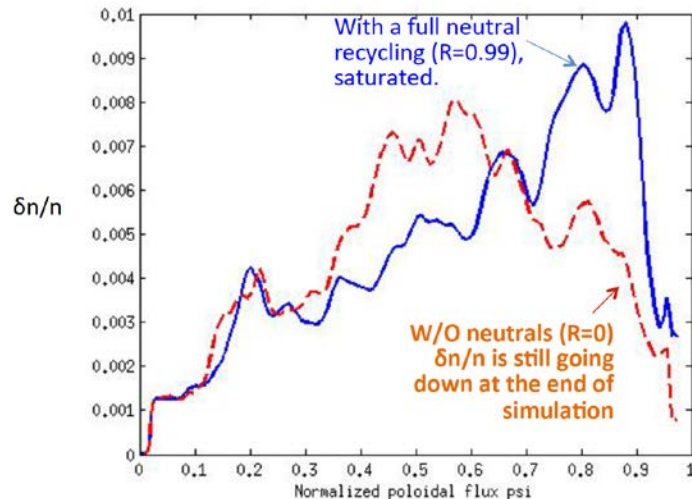
- **Understanding turbulence in NSTX edge plasma**

- Most likely, the turbulence seen at pedestal top are ITG modes (*Diallo, Nucl. Fusion 2013*), not KBMs (*recent study*)
  - Cross verification between E&M XGC1 (*global, S. Ku*) and GYRO (*flux-tube, W. Guttenfelder*) confirmed that  $\beta$  is well below  $\beta_{\text{KBM}}$
- Peeling type kinetic e.m. modes can exist in steep NSTX pedestal, but KBMs are not found (*PPPL-Theory Notable Outcome report to DOE, 2015*)
- Verified existence of blobby turbulence in extended SOL

- **Divertor heat-flux width:** The  $1/l_p$  type scaling in NSTX is from neoclassical orbit spread. Only ~30% effect from blobby spread (*DOE-FES National Theory Performance Target, led by Chang; Chang, IAEA16*)
- **Bootstrap current in NSTX pedestal** is significantly different from Sauter (*Hager, submitted to Phys. Plasmas*), and can alter the NSTX edge stability diagram (*Sabbagh, Nucl. Fus. 2013*)
- **Verified importance of the neoclassical ExB shearing in L-H transition** in NSTX (*Battaglia, Nucl. Fusion 2013*)



# XGC plans for NSTX-U



**Large difference in edge turbulence seen by XGC1 between zero and full neutral recycling.**

## Additional plans for NSTX-U by XGC are

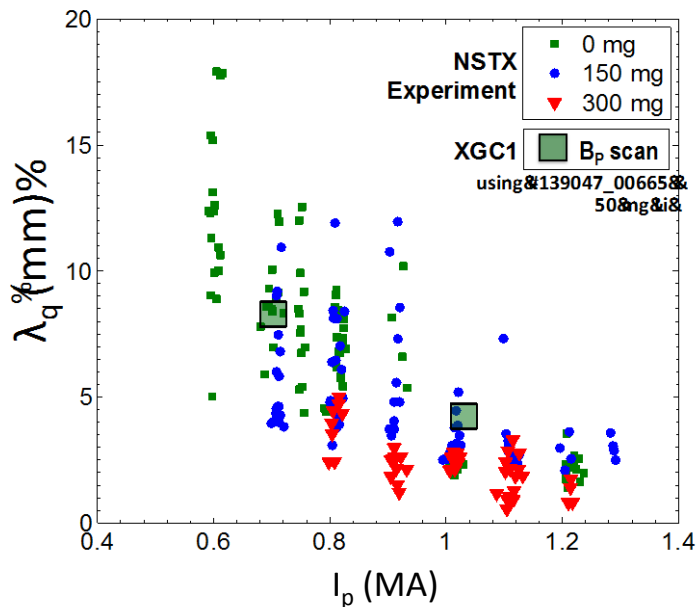
- Impurity studies (turbulence, including neoclassical effects )
  - How will the high and low Z impurity particles from SOL penetrate through the pedestal?
  - What will be their saturated radial profiles in the core?
  - What will be the impact of high and low Z impurities on pedestal confinement? On ExB shearing rate?
- Kinetic study of 3D field penetration into plasma, and effect on pedestal transport and shape.
- Enhancement of the XGC e. m. to include micro-tearing modes
- L-H transition physics
- Pedestal shape up against turbulence criticality and ELMs
- Incorporate DEGAS in XGC, including high-Z atomic physics



# XGC1 simulations aid in understanding basis for SOL heat flux trends in NSTX

- Experiment shows contraction of SOL heat flux width at midplane with  $I_p$  as well as influence of Li conditioning

XGC1 calculations with collisions show similar trend with  $I_p$



Heat flux width determined primarily by neoclassical processes

## XP 1514:

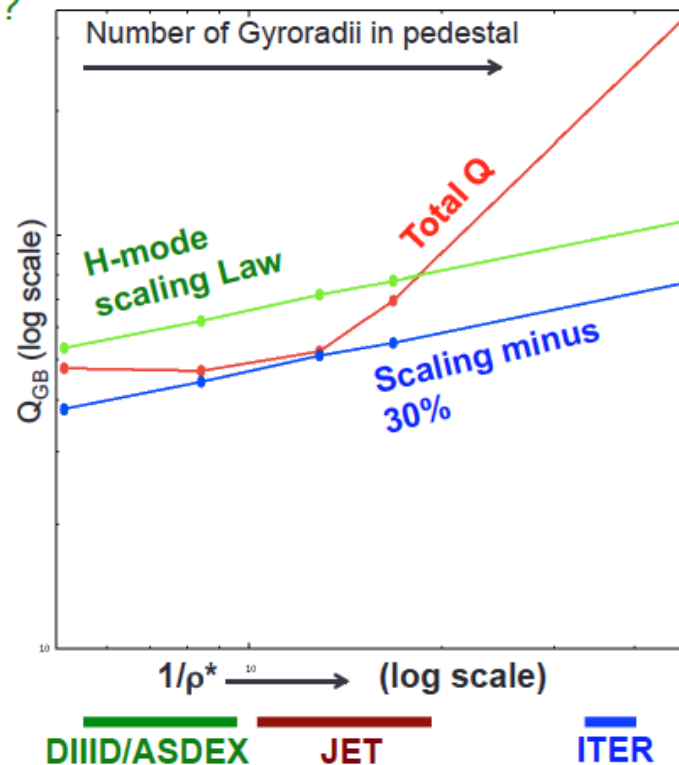
- (Gray) – Heat flux width scaling in NSTX-U; extend range of  $I_p$ , shape, Li dependence
- (Gray) – Is interchange drive responsible for SOL contraction with Li (in collaboration with LODESTAR)?

R16-1, 17-1, ITER

# IFS Collaboration with NSTX-U-Theory Group

How does the magnitude of the gyrokinetic heat flux compare to typical experimental values ?

- *Employ a rough estimate: use the H-mode scaling law to estimate the power required to produce these plasmas*
- *The transport found in GENE for these model profiles is somewhat lower than the scaling law, but within ~ 30%*
- *But beyond some  $\rho^*$  of JET, the transport trend is clearly much worse than the scaling law*



Kotschenreuther (2015)



Preliminary results indicate a surprisingly optimistic prediction for ST plasmas due in part to the localization of unstable eigenfunctions in the region of strong velocity shear: to be studied by XGC in a complementary study, including X-point geometry.

# Flux closure and reconnection studies during CHI in NSTX/NSTX-U

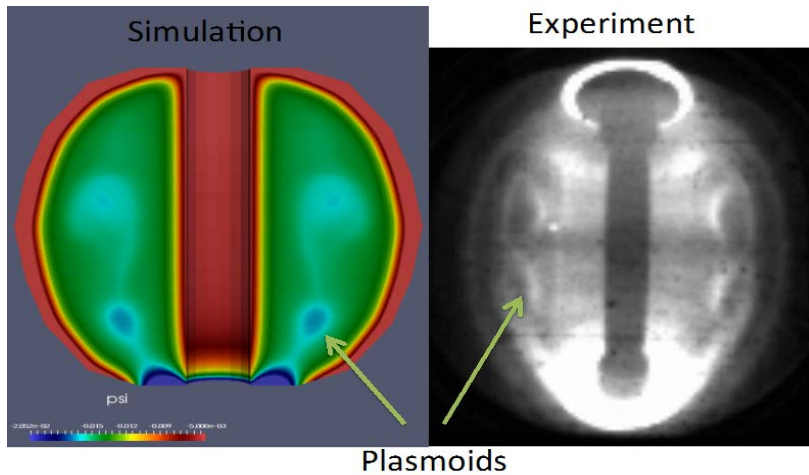


Fig2: Left: Plasmoid formation in simulation of NSTX plasma during CHI / Right: Fast-camera image of NSTX plasma shows two discrete plasmoid-like bubble structures.

Comprehensive resistive MHD simulations have been conducted for the NSTX and NSTX - U:

The goals:

- To achieve maximum flux closure and CHI-generated current
- How transient CHI scales as it is extrapolated to future (larger) devices such as the ST-FNSF
- Understanding the physics of magnetic reconnection during CHI is of great importance for the viability of this concept.

Ebrahimi et al PoP 2013, 2014. PRL 2015, submitted to NF.

- CHI experiments provide a very rich platform for studying magnetic reconnection
- Fundamental reconnection physics in NSTX-U, in particular reconnecting plasmoids physics will be explored in both the simulations and the experiment.

# *NSTX-U/Theory Partnership is focusing on issues central to achieving NSTX-U goals*

- Both over-arching goals and specific yearly Research Milestones (RMs)
  - Matched personnel to topical choices
  - Attempted to provide mapping to Goals, Milestones
- Partnership funding now integrated into the NSTX-U budget
  - No longer incremental
- Theory work not limited to PPPL Theory/NSTX-U partnership
  - Augmented by project theorists, collaborations, SCIDAC, direct Theory coverage of personnel
- Significant synergy with work on DIII-D by same personnel – aspect ratio,  $\beta$ ,  $v_*$ ,  $\rho_*$  leverage on theory validation
  - Confinement/effects of AE modes on fast-ion distribution, RMP/NTV/RWM physics, RF heating and current drive, pedestal/SOL characteristics, core transport

## ***Additional Slides***

# Other related Theory-Experimental work

## Theory

1. Develop and implement self-consistent fast ion distribution for calculating HHFW heating and CD using TORIC/NUBEAM/TRANSP (Valeo)
  - R16-3, 17-4, IR17-1, R18-2, ITER
2. RMP/NTV physics for rotation control (Ferraro, Z. Wang, + CU, PU)
  - R16-3, 17-4, 18-2, 2016 JRT, ITER
3. RWM physics with M3D-C<sup>1</sup> (Ferraro)
  - Resistive wall implemented
  - R16-3, 18-2, ITER
4. Edge neutral density calculations using ENDD diagnostic + DEGAS2 (Stotler)
  - R16-1, 17-3, 18-1, ITER

## Experiment

1. XP1507 (Gerhardt): High non-inductive fraction, high performance discharges
1. Multiple XPs study effect of rotation, NTV (T&T, CAE/GAE, TAE gap structure, RWM control and disruption mitigation)
1. Multiple RWM XPs
1. Critical info for TRANSP (beam deposition, particle transport), XGC1 (collisional effects in edge)