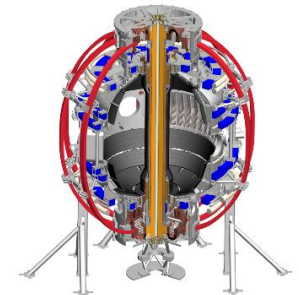


# Liquid Metals: Near and Long-term Plans

M.A. Jaworski for the NSTX-U Team

NSTX-U Program Advisory Committee Meeting  
Princeton Plasma Physics Laboratory, Princeton, NJ  
January 9<sup>th</sup>, 2018



# Outline

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1. Why evaluate liquid metals as a plasma-facing component (PFC)?
  - Liquid PFCs resolve several problems plaguing solid PFCs
  - Liquid PFCs demonstrated to enhance confinement and can absorb more heat flux than do solid PFCs
2. Why study liquid metals now and why in NSTX-U?
  - PMI Community workshop highlighted liquid PFCs in numerous priority research directions and high-impact cross-cutting activities
  - Integrated scenario assessment of ST requires movement away from C

# Liquid metals present *intrinsic* advantages over solid PFCs in several areas

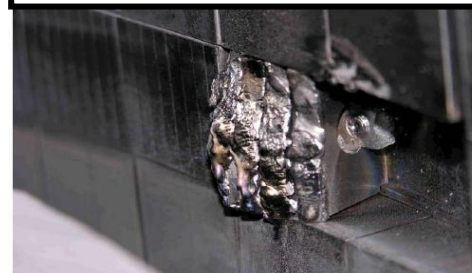
- Liquid metals provide a self-healing/renewable plasma-facing material
  - Immune to thermo-mechanical stresses
  - Returns to equilibrium after perturbations
  - Replenishment eliminates net-reshaping by plasma bombardment
- Separates neutron damage effects from plasma-material interactions
- Eliminates long-time constants associated with solid-wall material transport and evolution<sup>a</sup>
- **Demonstrated confinement effects and power handling capabilities**

**Cracking after thermal shock loading**



Wirtz, et al., JNM 2013

**CMOD W-lamellae**



Coenen, et al., JNM 2013

**Mo first-wall**

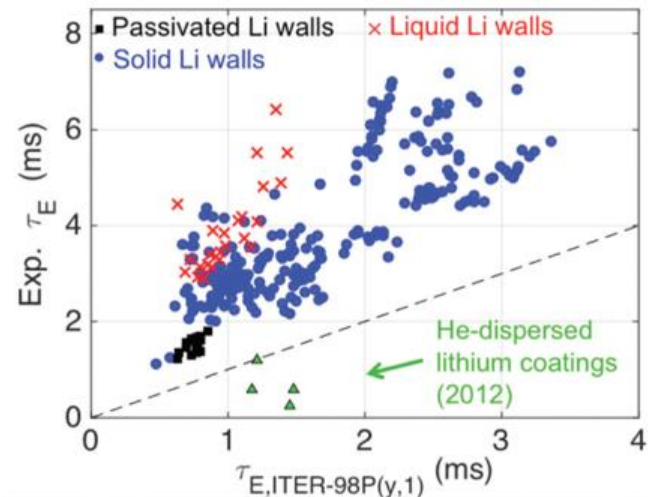
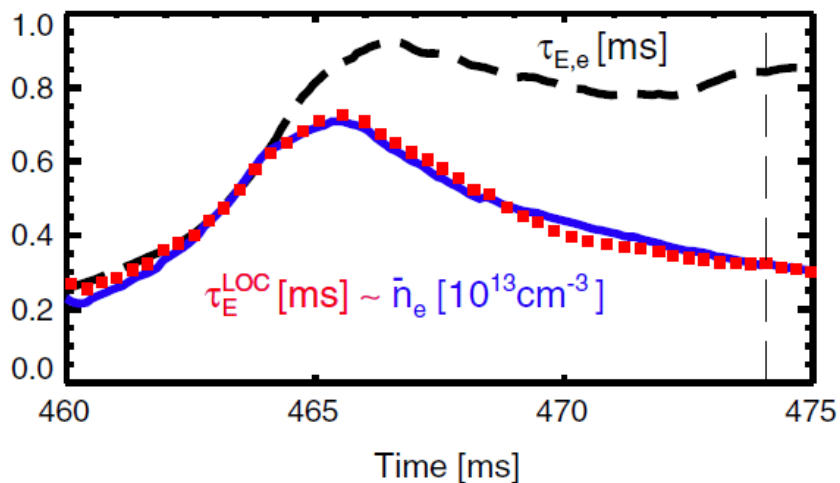


M. Reinke, private comm.

<sup>a</sup>Stangeby, JNM 2011

# LTX demonstrated (again) confinement gains even with partial oxidation of Li coatings and Li pool

- Measured electron energy confinement exceeds neo-Alcator Linear Ohmic Confinement scaling  $\sim 3\times$  (Boyle PRL 2017)
- Evaporated coatings monitored with MAPP diagnostic correlated with good performance to  $\sim 100$  hrs (Lucia PhD 2015, Schmitt PoP 2015)
- **Key question: will flowing liquids perform like coatings (with impurities)?**



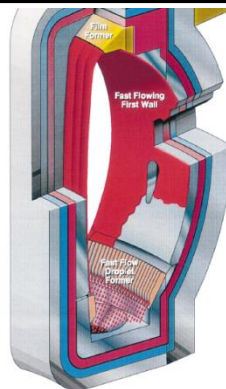
D. Boyle, et al., *Phys. Rev. Lett.* **119** (2017) 015001.

J. Schmitt, et al., *Phys. Plasmas* **22** (2015) 056112.

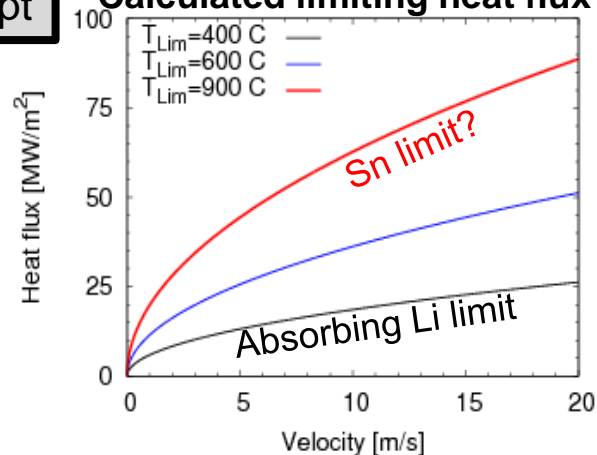
# Liquid metal PFCs can absorb more heat flux than do leading tungsten technologies

- Actively cooled tungsten expected to survive **5-15 MW m<sup>-2</sup>** steady-state
- Fast-flow systems advect power away from heating zones
  - Limiting temperature, heating size, and velocity determine limiting heat flux
  - Li, Ga, Sn all possible metals for use
- Slowly flowing liquid targets recently demonstrated in multiple configurations
  - Non-vaporizing, water-cooled tin demonstrated at **20 MW m<sup>-2</sup>** in Magnum-PSI<sup>a</sup>
  - Vapor-shielded tin achieved self-regulated temperature up to **22 MW m<sup>-2</sup>** in Magnum-PSI<sup>b</sup>

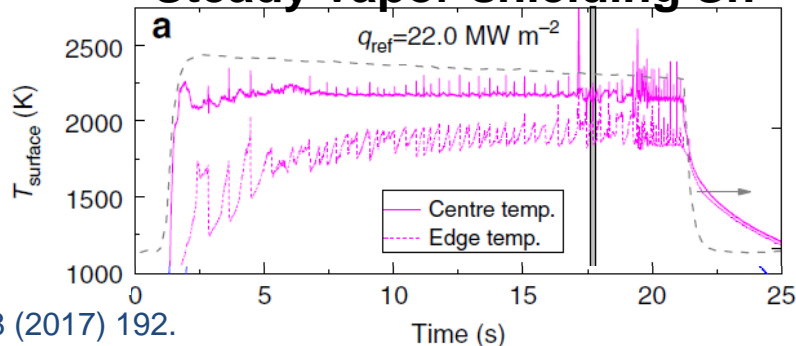
## Fast Flow Concept



## Calculated limiting heat flux

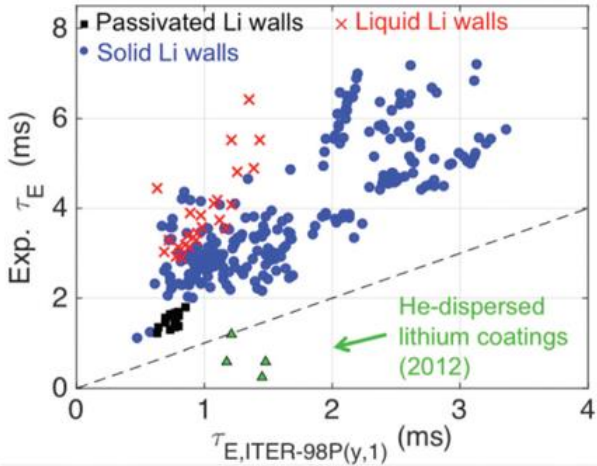


## Steady vapor-shielding Sn



<sup>a</sup>Morgan, 2017 Nucl. Mater. Energy; <sup>b</sup>S. Van Eden, et al., Nature Comm. 8 (2017) 192.

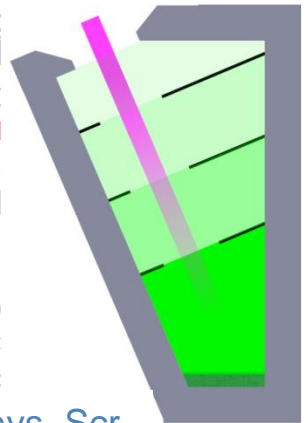
can absorb more heat flux than tungsten technologies



- Active 5-15
- Fast zone
- Li de
- Li

Enhanced confinement with vapor-box divertor<sup>a</sup>?

- Non-vaporizing **20 MW m<sup>-2</sup>** in
- Vapor-shielded up to **22 MW m<sup>-2</sup>**
- Vapor-shielding inspired pre-fill



**Only in a confinement device can one assess the core-edge integrated scenarios**

demonstrated regulated temperature with NSTU and PSI<sup>c,d</sup>

Eden, et al., 2017;

<sup>a</sup>Morgan, 2017 Nucl. Mater. Comm. 8 (2017) 192. <sup>c</sup>DP Rindt et al. Nucl. Fusion

<sup>a</sup>Goldston, 2016 Phys. Scr.

Time (s)

# Outline

1. Why evaluate liquid metals as a plasma-facing component (PFC)?
  - Liquid PFCs resolve several problems plaguing solid PFCs
  - Liquid PFCs demonstrated to enhance confinement and can absorb more heat flux than do solid PFCs
2. Why study liquid metals now and why in NSTX-U?
  - PMI Community workshop highlighted liquid PFCs in numerous priority research directions and high-impact cross-cutting activities
  - Integrated scenario assessment of ST requires movement away from C

# US PMI community workshop highlights the importance of developing tools that can extrapolate to a reactor

- Multiple priority research directions (PRDs) highlight need to advance concepts (including **liquids**):
  - PRD-A: limits on power and particle handling, including **liquids**
  - PRD-B: demonstrate dissipative divertor solutions, including **liquids**
  - PRD-C: develop solutions for main chamber components and tools for sustained operation
  - PRD-D: science of evolving materials, including **liquids**
  - PRD-E: understand impact of materials on core performance
- Multiple high-impact, cross-cutting activities also identified:
  - CC-1: enhance exploitation of existing machines for PMI studies
  - **CC-3: understand the science of liquid surfaces and examine feasibility of liquid PFC solutions**

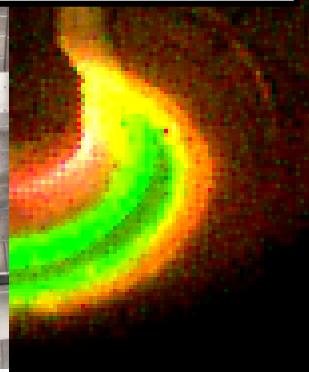
See also Reinke's talk



# NSTX & PPPL expertise, and need for integrated solutions motivates for liquid metal mission in NSTX-U

- Years of developing Li science...
  - Diagnostics: dual-band IR (ORNL), divertor imaging/spect. (LLNL), MAPP (U-Illinois)
  - Theory: liquid stability, quantum molecular dynamics (SBU) and DFT (PU), whole-device material evolution modeling
  - Supporting surface science laboratories
- ...and technologies
  - Lithium experimental methods: LITER, U-LITER, granule & aerosol injection, Liquid Lithium Divertor
  - Large area tray limiters, LTX shell system, etc.
- **NSTX-U can easily create  $q_{\perp} > 20 \text{ MW m}^{-2}$** 
  - Full heating power:  $P/R \sim 20$

**NSTX Liquid Li Divertor**



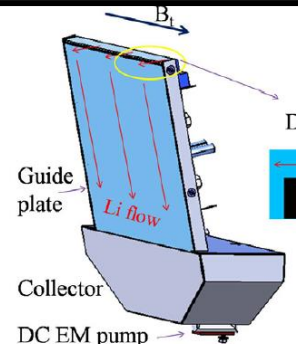
**CDX-U large-area tray limiter**



Kaita 2005 JNM



**EAST FLiLi**



Hu 2016 NF

# Positive NSTX results motivate near-term plan with Upward LITER in NSTX-U

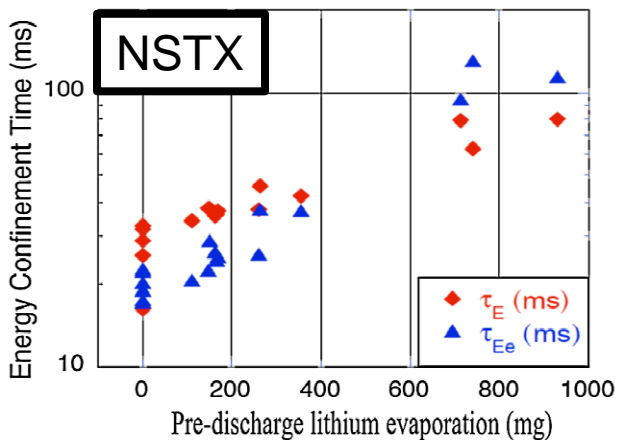
- Confinement improvement attributed to reduced recycling

- Machines at PPPL showing benefits of lithium conditioning over increasing areas

- TFTR Li “super-shots”
- CDX-U, LTX
- **NSTX<sup>a</sup>**

- Others examining Li:

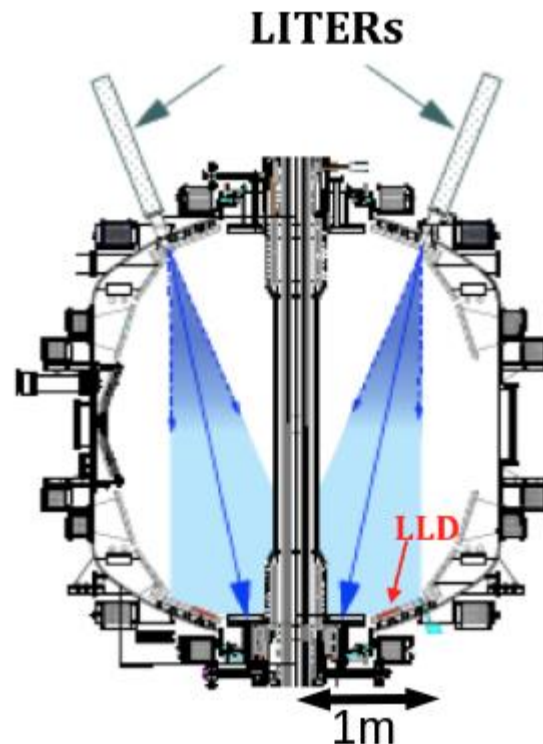
- FTU, T11-M, HT-7, EAST, DIII-D
- TJ-II, RFX



## Upward LITER Development



## NSTX Configuration



<sup>a</sup>R. Maingi, et al., *Phys. Rev. Lett.* **107** (2011) 145004.

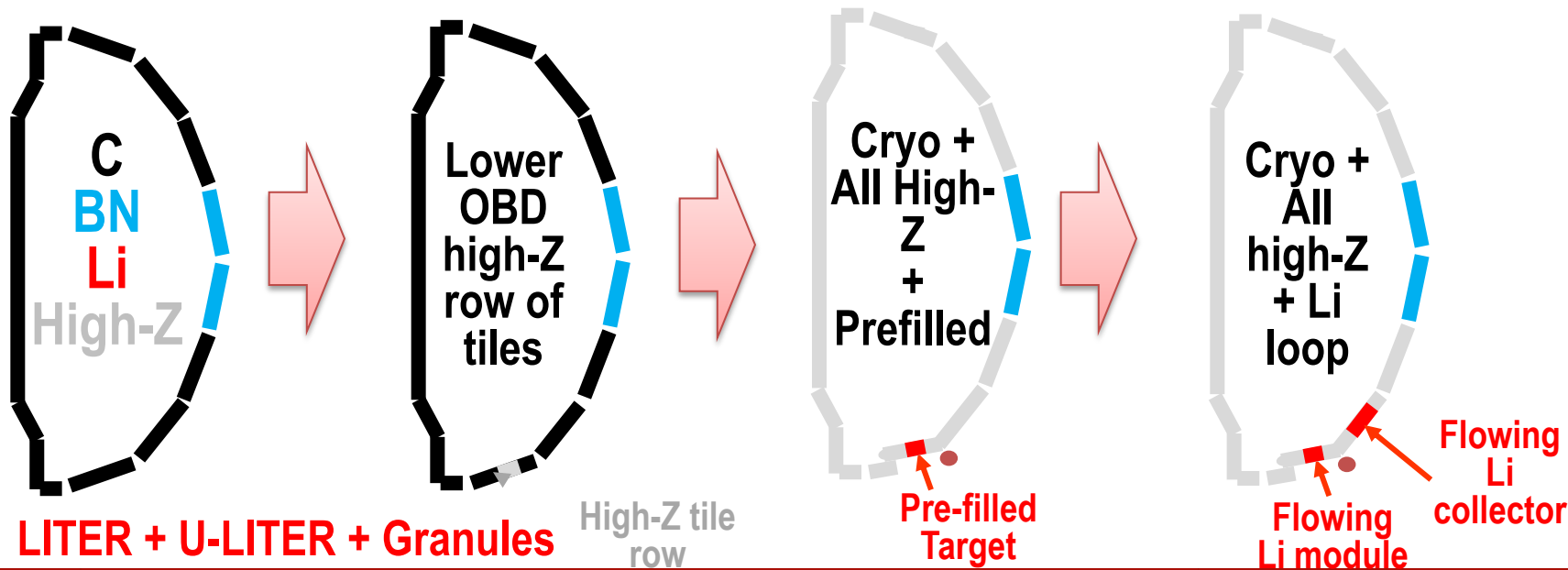
Cai, 2017 IEEE TPS

# High-Z + Li NSTX-U would advance multiple Priority Research Directions simultaneously in unique ST facility

- NSTX and NSTX-U has emphasized surface science understanding of plasma-material interface (PRD-C,D)
  - MAPP deployed in 2016: year 1 capability
  - Studying impact of materials on plasma and vice-versa essential to whole-device modeling of material evolution
- A liquid metal-capable NSTX-U would enable high-Z vs. lithium comparison studies in a high-power device (PRD-A,B,C,E)
  - Extend LTX-beta studies
  - Builds on and complements continuing studies in higher A devices like EAST
- Heating power available to NSTX-U enables testing of liquid concepts to extreme power levels (PRD-A, B, C, E)
  - Potentially critical to ST line
  - Potentially critical to economical fusion energy concepts

# One possible progression: staged conversion to mitigate risk while enabling integrated performance assessment of the ST

- Full high-Z conversion enables comparison with conventional high-Z
- Provides high-power analog to evaluate LTX confinement results
- Open divertor and flexible magnetic configuration mitigates risk

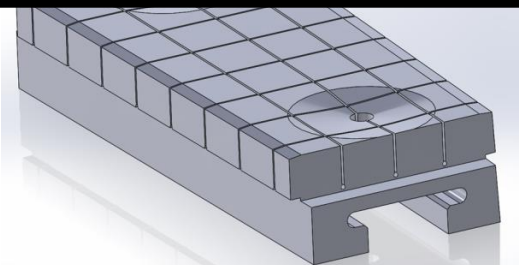


# Most mature technologies proposed for current development path (emphasis on qualification testing)

2015-16 NSTX-U High-Z Design

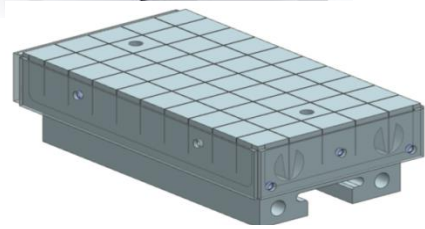
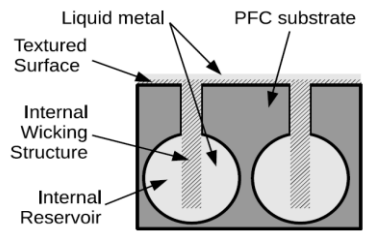
- Bulk high-Z for high heat flux regions, Mo/W coatings may be possible in low-heat flux regions

~2 yr deploy



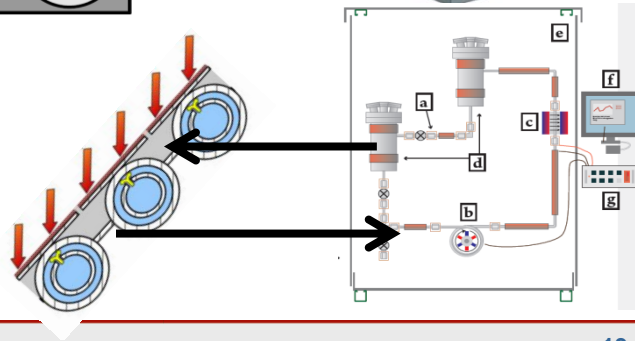
- Pre-filled targets build on high-Z, high-heat flux designs

~2.5 yr deploy after high-Z



- External Li feed into reservoir region with inertial cooling provides near-term similarity to DEMO-relevant concepts

~10 yr deploy



# The integrated assessment of the ST can be improved with near- and long-term PMI studies (Li + high-Z)

- Liquid metals have potential to solve several long-standing issues associated with solid PFCs
- Recent results continue to show confinement gains and potential for steady power exhaust
- US PMI Community identified the need to advance liquid metal science and technology, and evaluate feasibility
- A staged upgrade for NSTX-U can enable an integrated (plasma-material) assessment of the ST
  - New PFCs require sustained R&D starting now for second 5-year plan

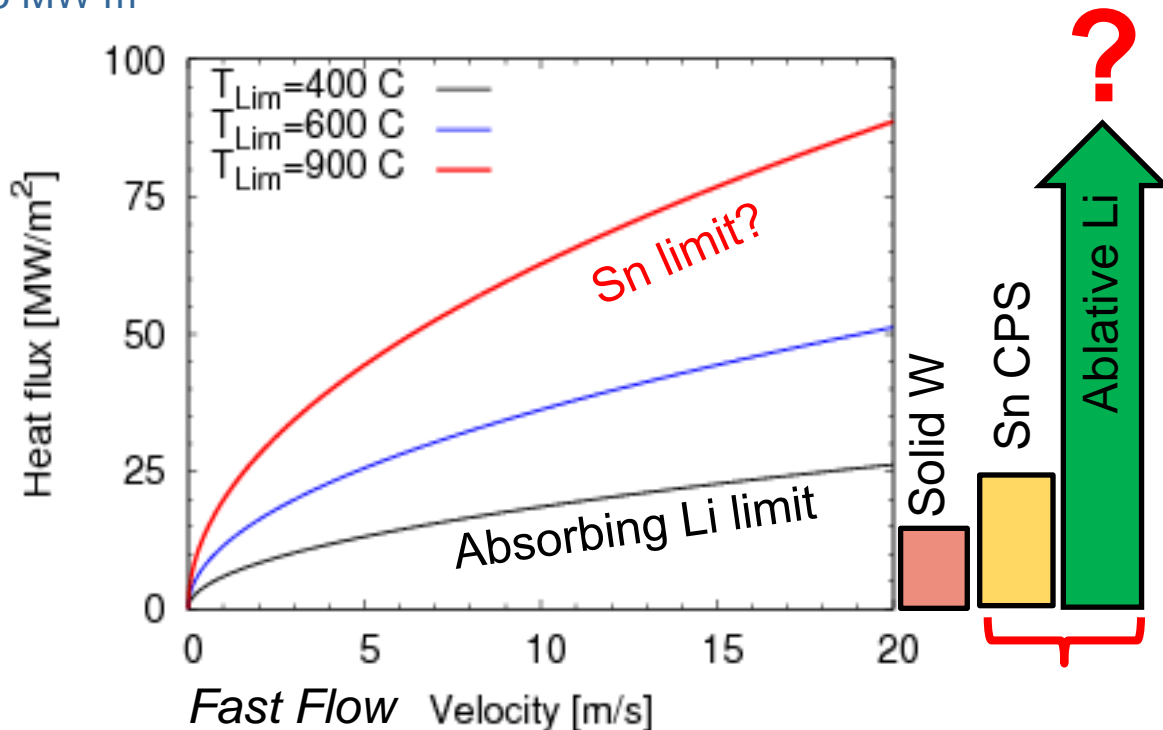
# Thank you!

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# Liquid metal PFCs can absorb more heat flux than leading tungsten technologies

- Actively cooled tungsten expected to survive 5-15 MW m<sup>-2</sup> steady-state
  - Transients and neutrons lead to ~5 MW m<sup>-2</sup>
  - Technological limitations lead to 9m major radius in EU DEMO1 (P/R=17)<sup>a</sup>
- Tin CPS water-cooled target demonstrated 20 MW m<sup>-2</sup> in Magnum-PSI<sup>b</sup>
- Unknown vapor-shielding limit*
  - Dependent on replenishment rate to surface (technology)
  - Dependent on core plasma compatibility with divertor**

## Perpendicular Heat Flux Limit



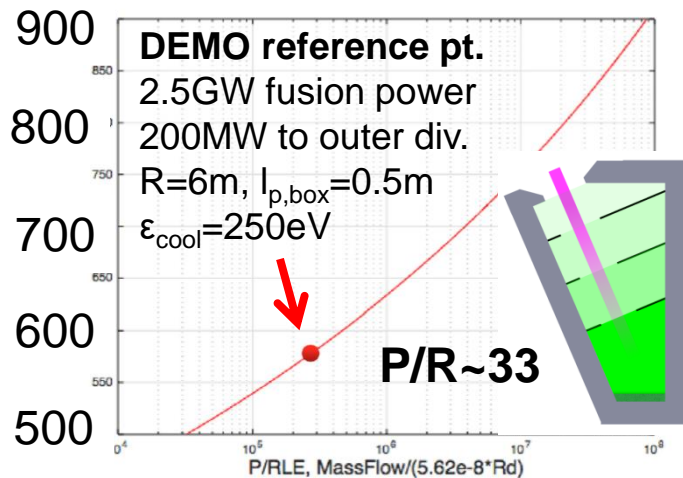
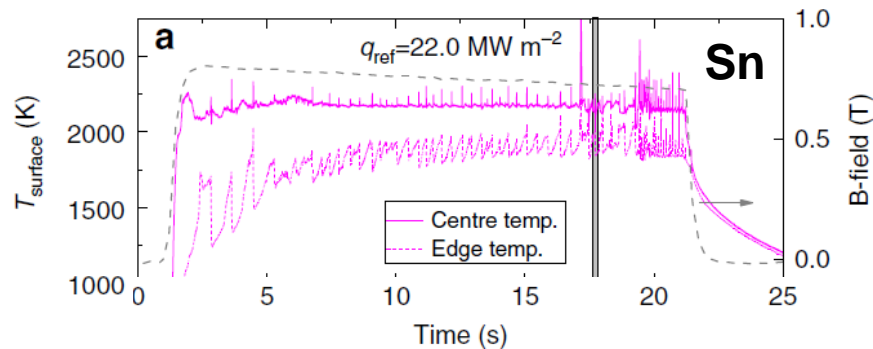
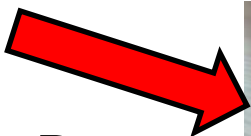
<sup>a</sup>Wenninger 2017 Nucl. Fusion;

<sup>b</sup>Morgan, 2017 Nucl. Mater. Energy



# Vapor shielding experiments in linear plasma device; shows path forward for vapor-box concept

- Liquid Sn and Li targets tested
  - Magnum-PSI linear plasma
  - Both exhibit temperature clamping
- Li-target tested based on pre-filled concept
- Vapor box 0D calculations indicate very high heat flux potential

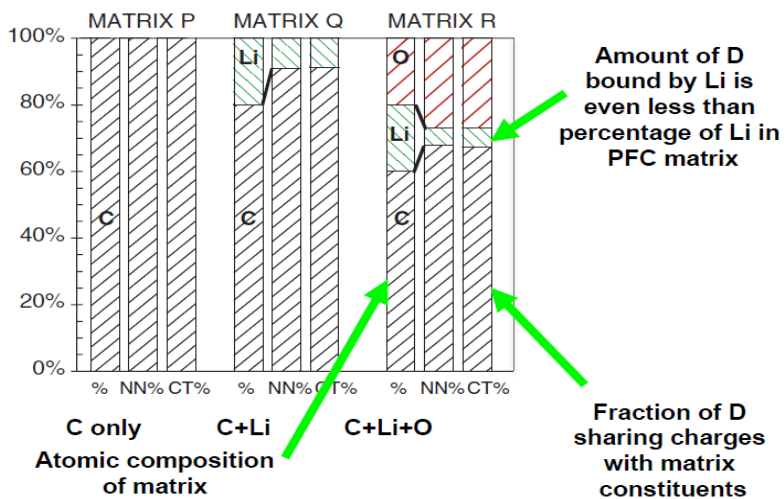


S. Van Eden, et al., *Nature Comm.* **8** (2017) 192.

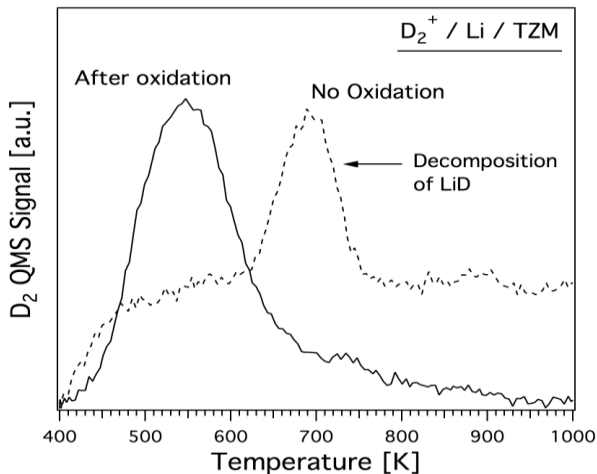
R. Goldston, et al., *Nucl. Mater. Energy* **12** (2017) 1118.

# Multiple tools developed to study complex, substrate-dependent lithium chemistry

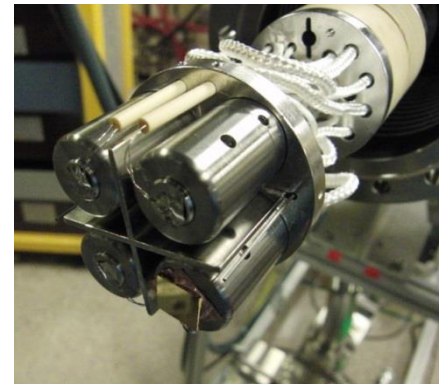
- Quantum modeling by Krstic indicates preferential bonding of deuterium to oxygen in carbon matrix (**theory**)
- Laboratory studies by Capece show increased absorption by oxidized Li, but lower thermal decomposition temperature (**laboratory studies**)
- MAPP **diagnostic** successfully deployed in NSTX-U 2016 run campaign



Krstic, 2013 PRL



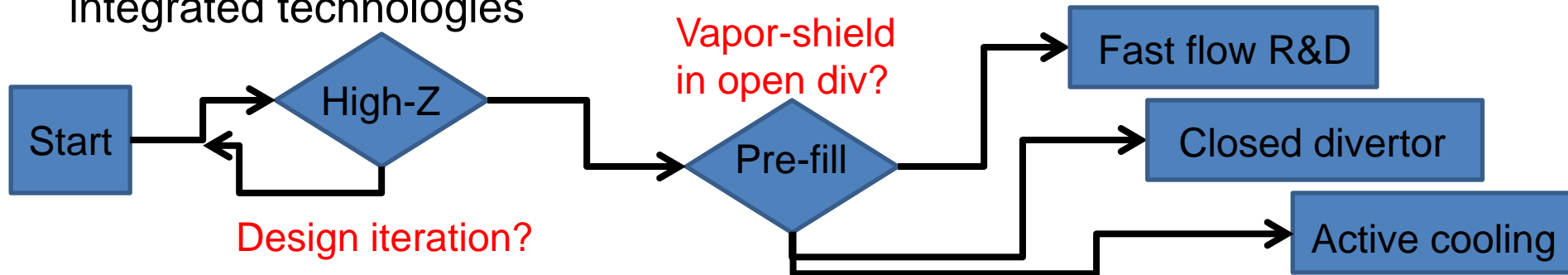
Capece, 2015 JNM



MAPP Diagnostic (U-Illinois)

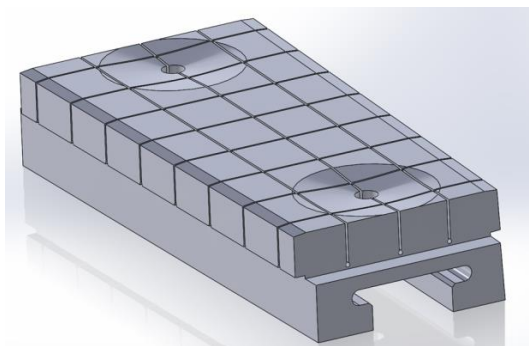
# Multiple decision points in the research program leading to new science and technology opportunities

- NSTX-U PFC Recovery project emphasis on prototypes builds confidence
- High-Z introduction enables early evaluation of operational impact of new PFC materials
  - LITER and U-LITER allows near-term examination of increased areal coverage with lithium conditioning
- Pre-filled target performance presents decision points:
  - Is a closed divertor upgrade necessary for high-temperature (i.e. vapor box)?
  - Is active cooling or fast-flow required for long-pulse particle flux handling?
- Closed loop facility enables broadened facility scope for research on integrated technologies

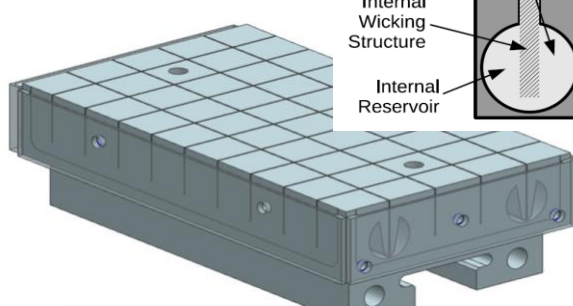


# Most mature technologies emphasized for current development path

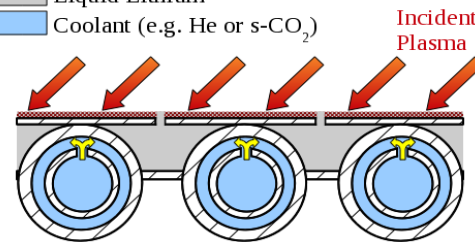
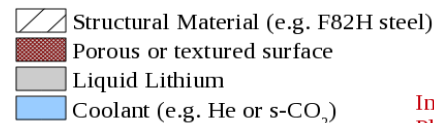
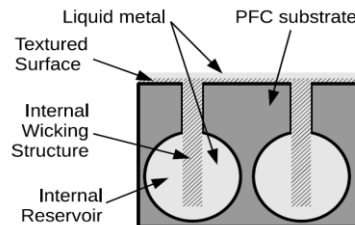
- Bulk high-Z for high heat flux regions, coatings may be possible in low-heat flux regions
- Pre-filled targets build on graphite and high-Z substrate designs
- External Li feed into reservoir region with inertial cooling provides nearest target technology for NSTX-U



High-Z PFC T-bar design



Pre-filled target concept



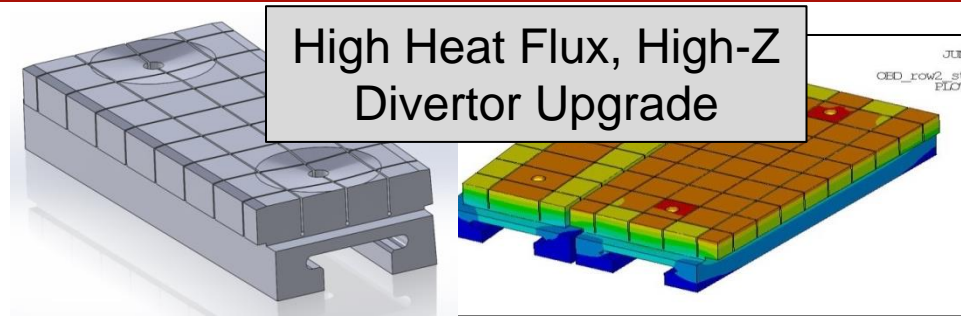
DEMO-relevant PFC concept

# Sustained effort needs to start now if this is to be implemented in 2025-2030 timeframe

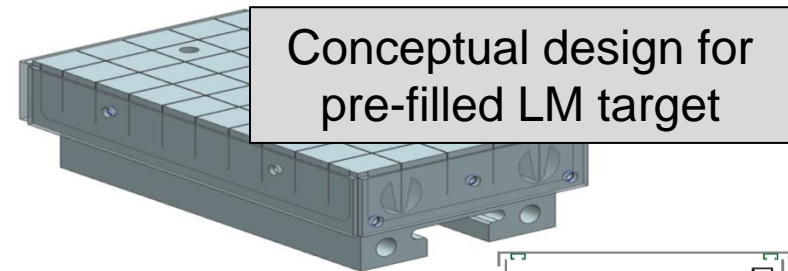
- Reverse schedule developed to identify dependencies in R&D
  - Pre-filled high-heat flux tiles build on High-Z HHF
  - High-Z high-heat flux and low-heat flux tiles build on PFC Recovery activity
- Pre-filled tiles could be used in NSTX-U campaigns ~**6 years** from start (High-Z HHF in ~2 years)
- Li R&D facility recommended to accelerate development
  - Serial R&D processes indicate ~10 years of development to integrated PFCs
  - Fully flowing modules require Li loop and handling R&D & ops experience
  - Fully flowing modules required for Li facility that “**runs as long as we want it to run**”
  - Building/facility recommended in PPPL Li Corrective Action Plan to consolidate Li activities as a best practice

# A three-step progression can accelerate tests of flowing, liquid metal PFCs

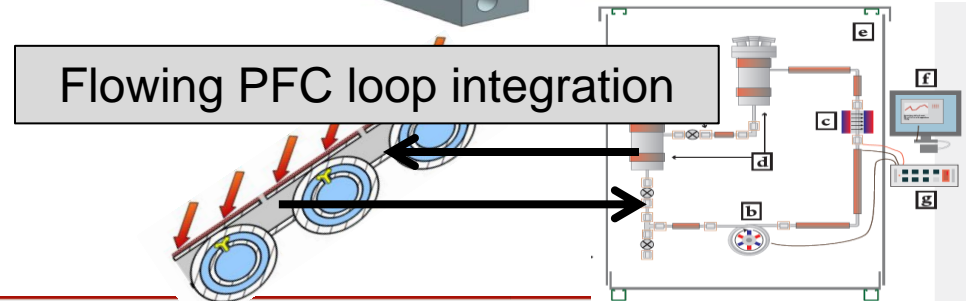
1. High-Z divertor tiles + LITER



2. Pre-filled liquid-metal target



3. Flowing LM PFC

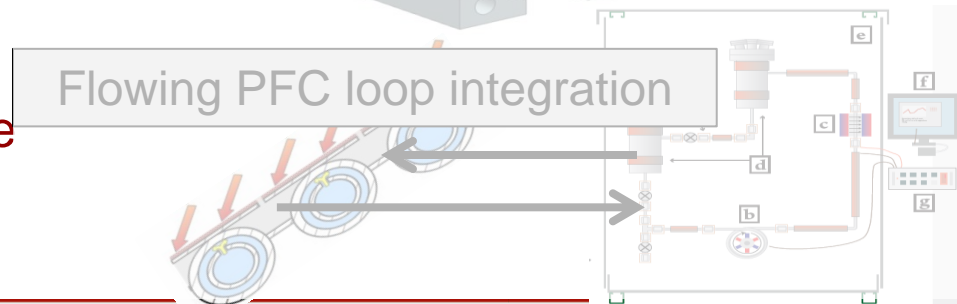
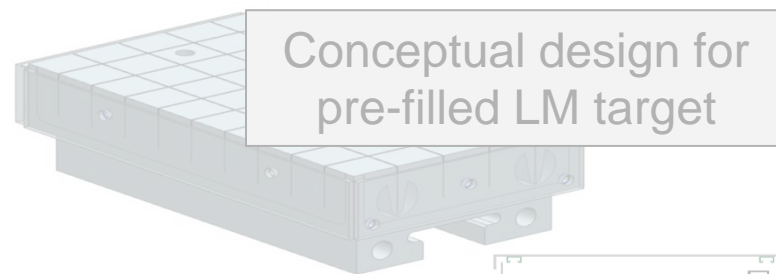
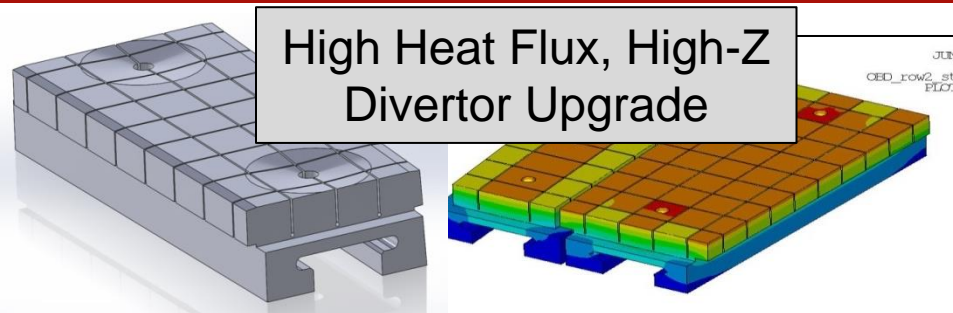


# High-Z divertor tiles + Li evaporated coatings provide divertor analogue of Magnum-PSI experiments

## 1. High-Z divertor tiles + LITER + U-LITER

### – Scientific goals:

- Short-period particle-flux handling through temperature ramp
- Quantify maintenance of Li on high-temperature substrate and protection of substrate
- Re-examine suppression of erosion in high-flux divertor
- Understand impact and core-edge compatibility of high-temp. target with limited inventory of Li

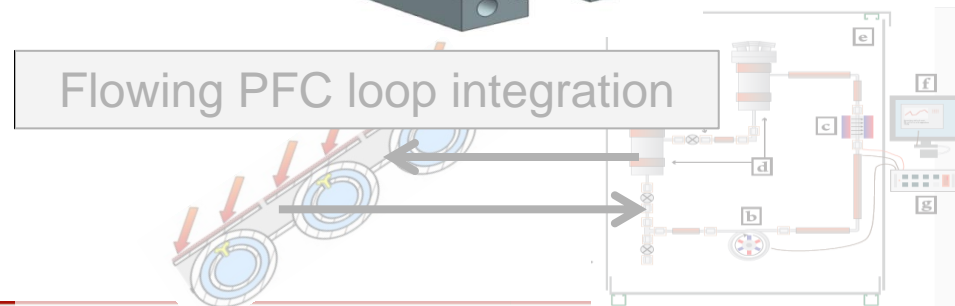
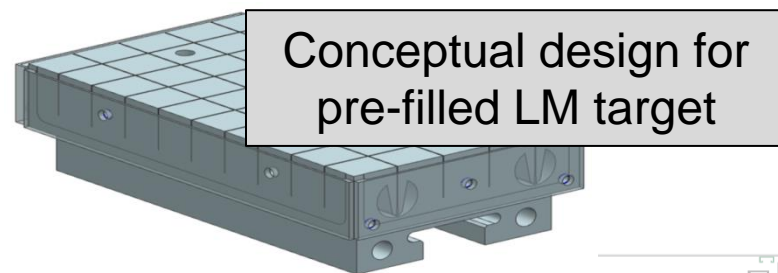
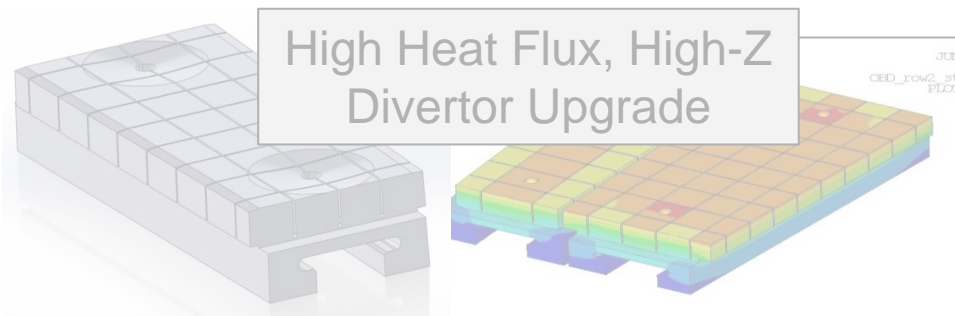


# Pre-filled targets test LM coverage, resupply and impact of significant Li source

## 2. Pre-filled liquid-metal target

### – Scientific goals:

- Short-pulse particle-flux handling with *localized* Li source
- Test models of maintenance of LM wetting and coverage
- Understand limits of LM passive resupply
- Understand impact and core-edge compatibility of high-temp. target with **larger** inventory of Li



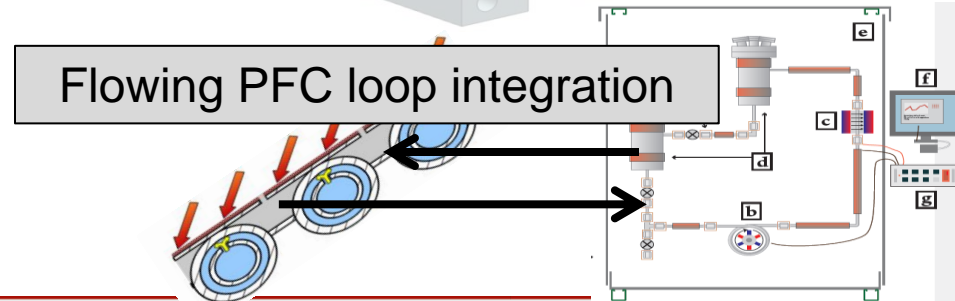
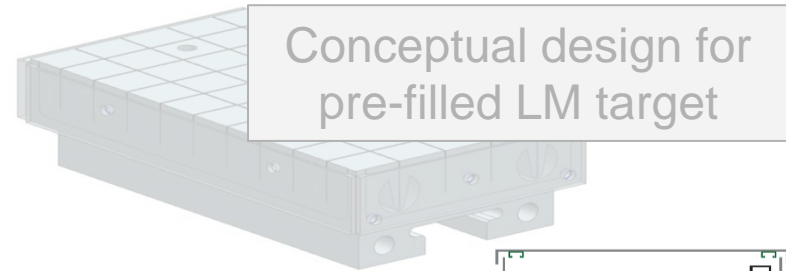
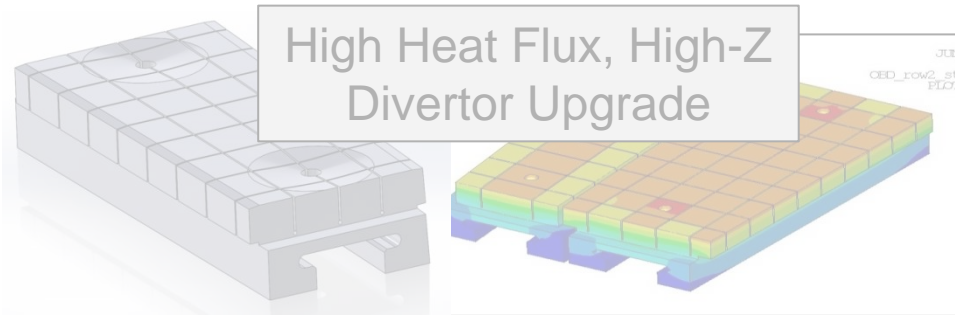


# Final integration demonstrates LM introduction/extraction and inventory control

## 3. Flowing LM PFC

### – Scientific goals:

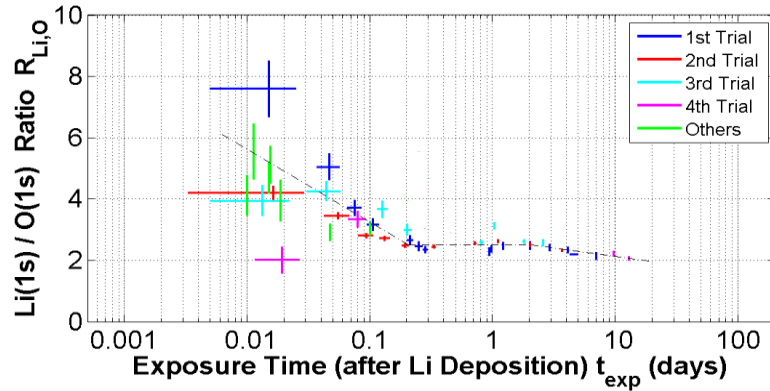
- Short pulse particle flux handling with localized, constant material
- Assess material inventory control to/from LM target
- Understand performance of passive + active replenishment techniques
- Understand impact and core-edge compatibility of high-temp. target



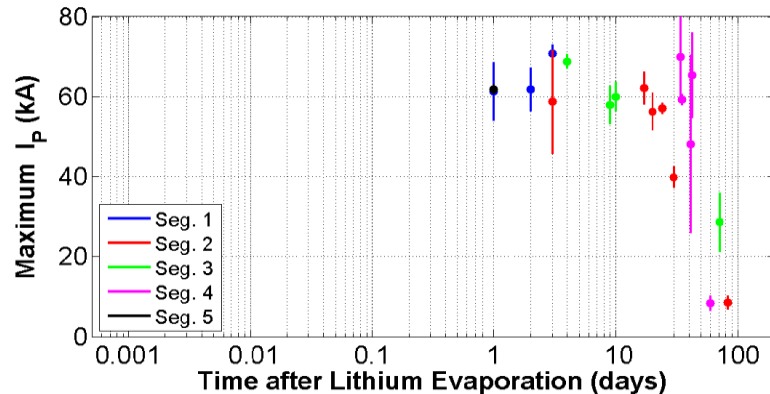
# Measurements of surface composition after lithium application indicate formation of stable oxide layer



Lithium/Oxygen Ratio - Vacuum,  $T_s = 30 \pm 10$  °C



Temporal Evolution of Maximum  $I_p$



- Results shown from several lithium evaporation experiments
- Surface exclusively  $Li_2O$  for  $\sim 100$  hours at  $H_2O$  pressure of  $2 \times 10^{-9}$  T (upper figure)
- Persistence of maximum achievable plasma current – good plasma performance – suggests ability of  $Li_2O$  to retain hydrogen (lower figure)
- Low resolution of MAPP XPS system means more time needed for scans
  - $Li_2O$  formation from lithium unresolved for  $t < 1$  hour

# Hydrogen retention in lithium and lithium oxide measured under controlled laboratory conditions

LTX

- Used stainless steel ultrahigh vacuum chamber with  $2 \times 10^{-10}$  Torr base pressure
- Ni(110) single crystal high-Z substrate
- Lithium films created by thermal evaporation onto Ni substrate
  - Lithium oxide formed by  $O_2$  exposure
- Lithium and lithium oxide films exposed to 500 eV  $H_2^+$  ion beam for total total fluence of  $4 \times 10^{15} H^+ cm^{-2}$ 
  - Beam defocused over surface to provide total fluence of  $4 \times 10^{15} H^+ cm^{-2}$
- Hydrogen retention determined with temperature programmed desorption
  - Measurements made after irradiation with hydrogen at various lithium and lithium oxide film temperatures



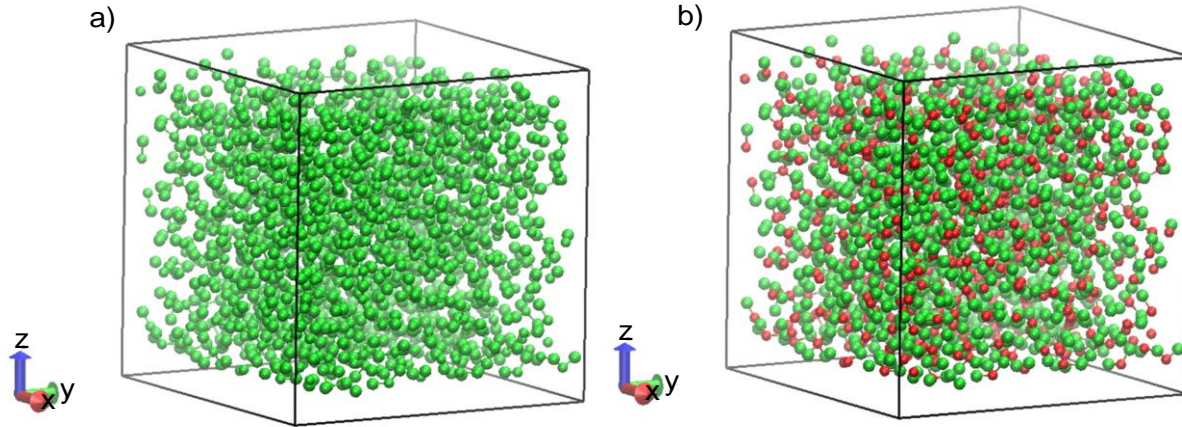
# Experiments simulated with molecular dynamics calculations

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- **Classical molecular dynamics (MD) used**
  - Each atom treated as point mass
  - Newton's equations integrated to compute motion
  - Individual force equation for each atom derived from potential energy functional for system
- **Amorphous target surfaces of pure Li and Li<sub>2</sub>O prepared for 90, 300, 400, 500, and 600K temperatures**
- **Computational cells of about 2000 atoms initially created at 300K for two cases**
  - Random distribution of lithium atoms only
  - Predefined random distribution of 33% O and 67% of Li atoms

# Amorphous distribution of lithium and lithium oxide in unit cells

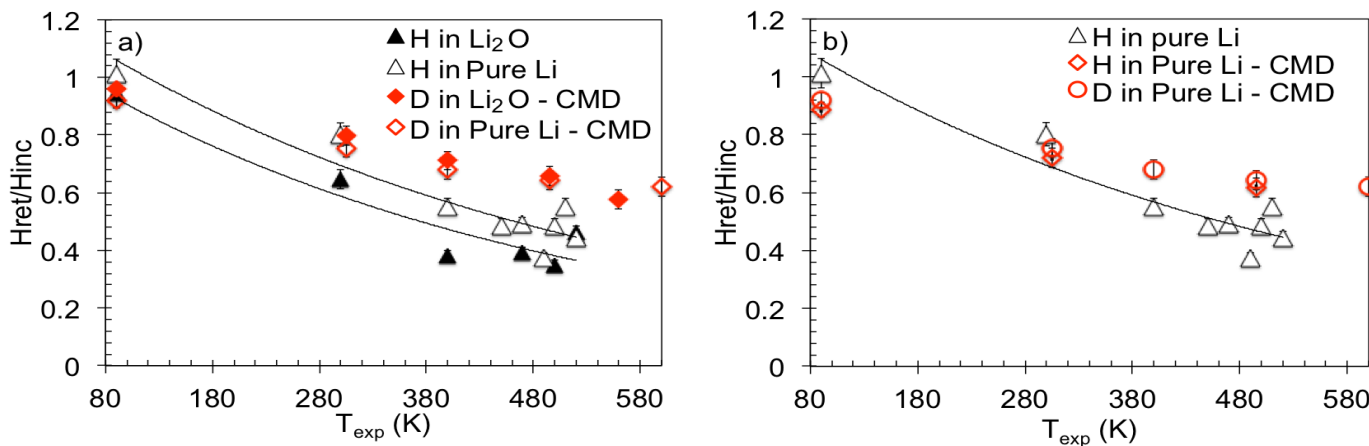


a) Li system (2000 lithium atoms)

b)  $\text{Li}_2\text{O}$  system: 33% of O (660 atoms) and 67% of Li (1340 atoms)

- Green and red symbols represent Li and O, respectively

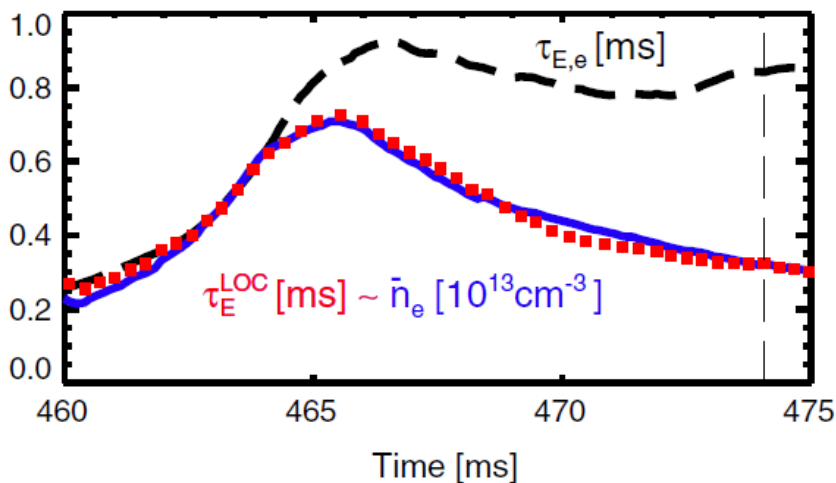
# Hydrogen retention in lithium and lithium oxide seen in laboratory experiments and MD simulations



- a) H retention fraction (H retention/H incident) in Li and  $Li_2O$  as function of exposure temperature and comparison with D retention from MD calculations
- b) MD and experimental data on H retention fraction only in Li and  $Li_2O$  as function of exposure temperature.

# LTX demonstrated (again) confinement gains even with partial oxidation of Li coatings and Li pool

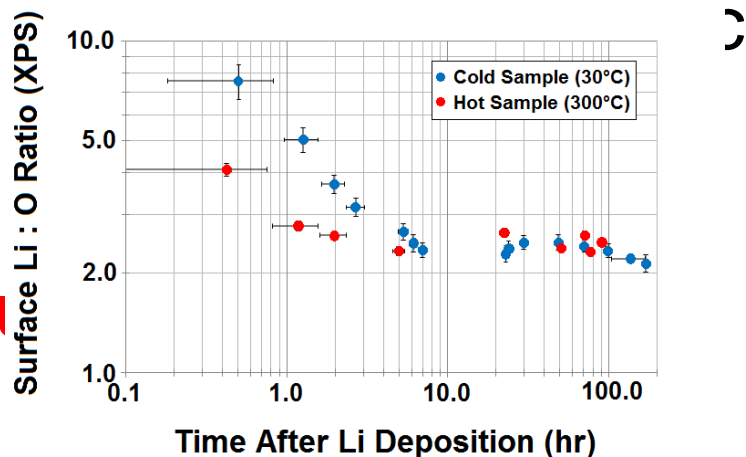
- Measured electron energy confinement exceeds neo-Alcator Linear Ohmic Confinement scaling  $\sim 3x$  (Boyle PRL 2017)



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Surface Evolution in LTX Base Vacuum



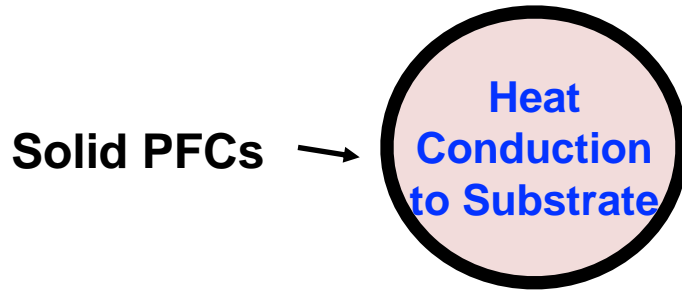
D. Boyle, et al., *Phys. Rev. Lett.* **119** (2017) 015001.

M. Lucia, PhD Thesis, Princeton University. 2015.

# Liquid metal PFCs provide additional pathways for energy transport

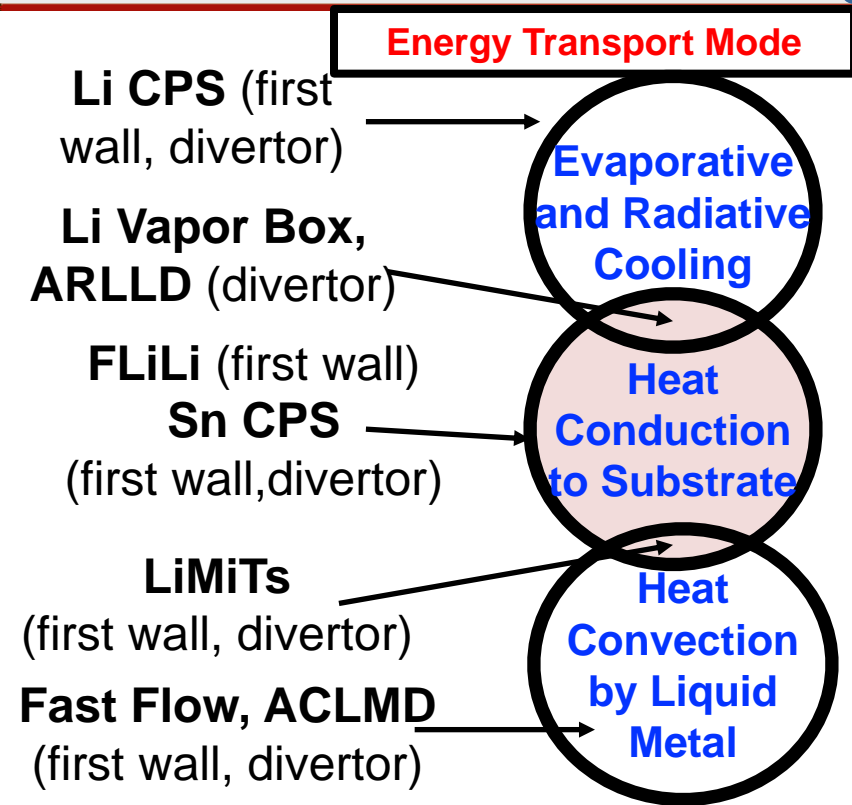
## Energy Transport Mode

- Conventional, solid PFCs utilize extrinsic impurities to enhance radiation





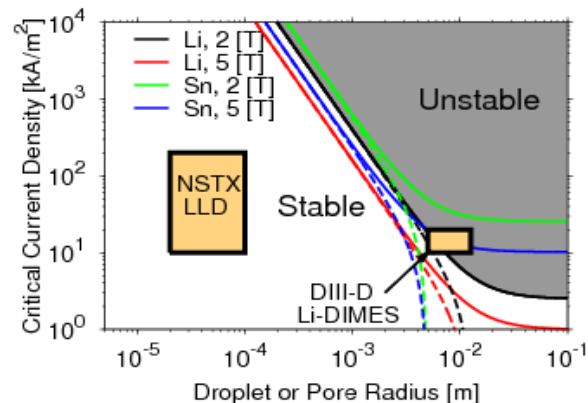
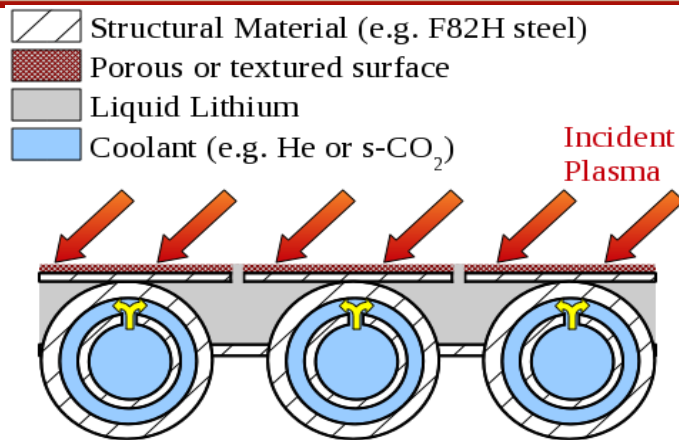
# Liquid metal PFCs provide additional pathways for energy transport



- Conventional, solid PFCs utilize extrinsic impurities to enhance radiation
- Liquid metals access *mobility* as additional tool for energy transport
- Particle retention in Li a strong function of temperature links PMI physics to the technology

# An DEMO-relevant concept provides guidance for relevant technologies in NSTX-U

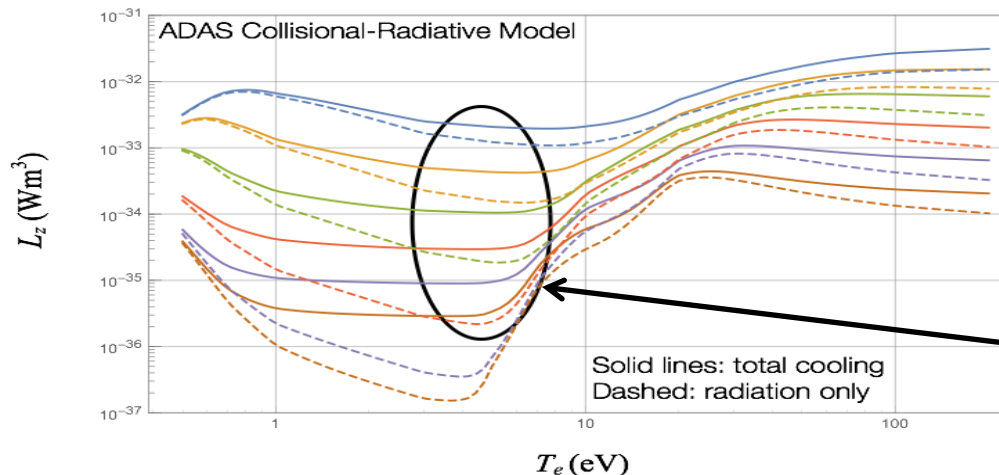
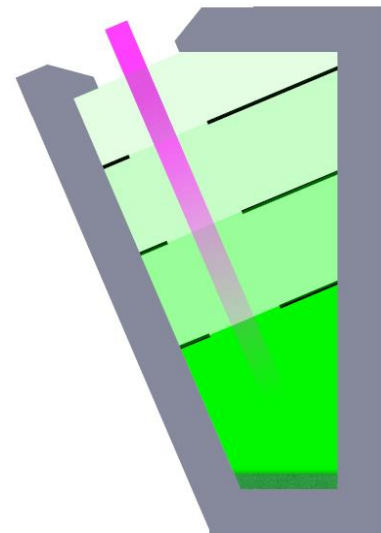
- Stability achieved by porous substrate
- Closely connected primary coolant and liquid lithium reservoir/supply structure
  - Temperature controls evaporation and condensation
  - Temperature controls hydrogen retention
- Continuous flow to/from the surface to flush gettered material and maintain wetted surfaces (substrate protection)



Jaworski, et al., *Plasma Phys. Control. Fusion* **55** (2013) 124040.

# Li Vapor Box (LVB) Divertor

- Li vapor divertor target is constrained in space by differential pumping
  - Bottom chamber reaches equilibrium density of  $\sim 10^{22}/\text{m}^3$  @ 750 C
  - Upper chambers pump by condensation, at much lower temperature
- Plasma energy is dissipated in bottom box

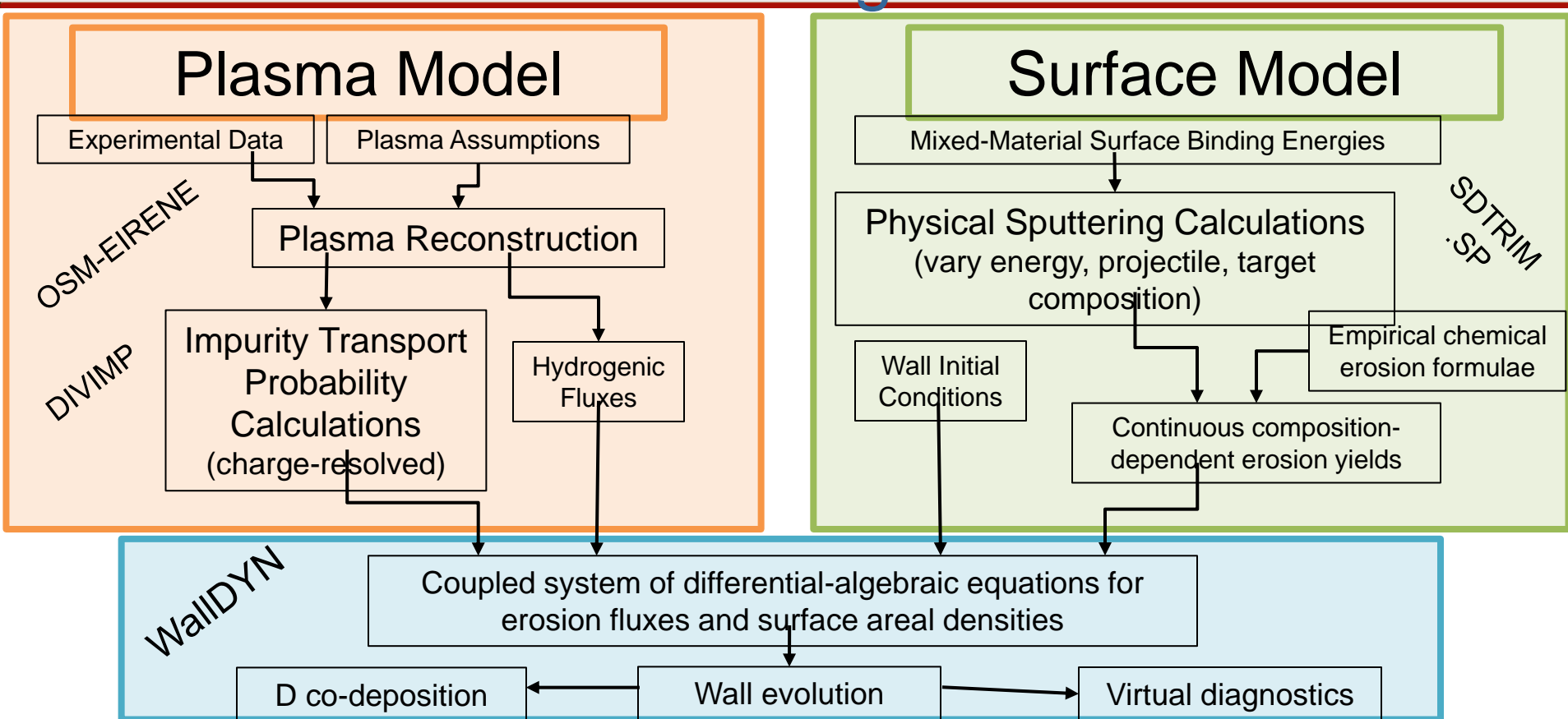


$$\begin{aligned}
 n_e &= 10^{19} \text{m}^{-3} \\
 n_e &= 10^{19.5} \text{m}^{-3} \\
 n_e &= 10^{20} \text{m}^{-3} \\
 n_e &= 10^{20.5} \text{m}^{-3} \\
 n_e &= 10^{21} \text{m}^{-3} \\
 n_e &= 10^{21.5} \text{m}^{-3} \\
 \tau_z &= 100 \mu\text{sec} \\
 n_z &= \tau_z S / V
 \end{aligned}$$

$$\left. \frac{\Delta E}{ptcl.} \right|_{cool} = \frac{\Delta t V p_{cool}}{\Delta t V dn_z / dt|_S} = \frac{n_e n_z L_z}{n_z / \tau_z} = n_e \tau_z L_z \sim 6.2 eV / ptcl.$$

Goldston, ISLA 2015

# WallDYN: An integrated global model for mixed material migration



# WalldYN parameterizes plasma impurity transport with DIVIMP 2D Monte Carlo code

- 20k neutrals launched from each wall bin
  - Energy and angular distribution typical of sputtered particles
  - Deposition location, charge state recorded
- $R \equiv$  Fraction of eroded flux from “source” that ends up in a certain charge state at “destination”
- 1 Algebraic equation for influx of each element & charge state:

$$\Gamma_{wall,elem}^{IN}(t) = \sum_{source} R_{source}^{wall} * \Gamma_{source,elem}^{OUT}(t)$$

- Assumptions:
  - Impurity concentrations are low enough not to disturb plasma (“trace impurity limit”)
  - Plasma transport timescale  $\ll$  wall evolution timescale
  - Plasma does not change in time

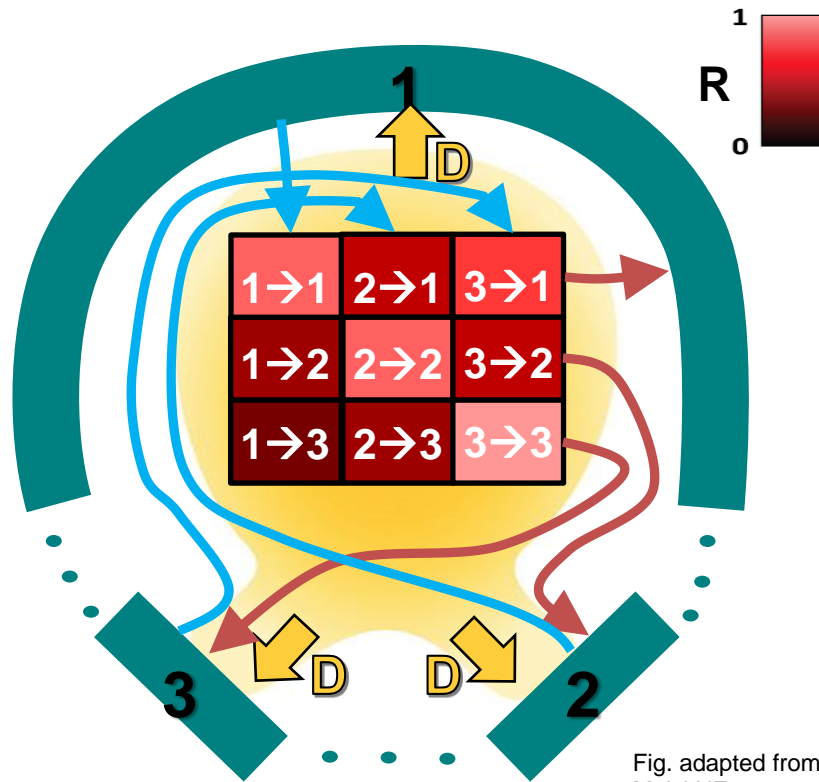


Fig. adapted from Meisl NF 2016

# OSM/EIRENE edge plasma background generated for NSTX-U

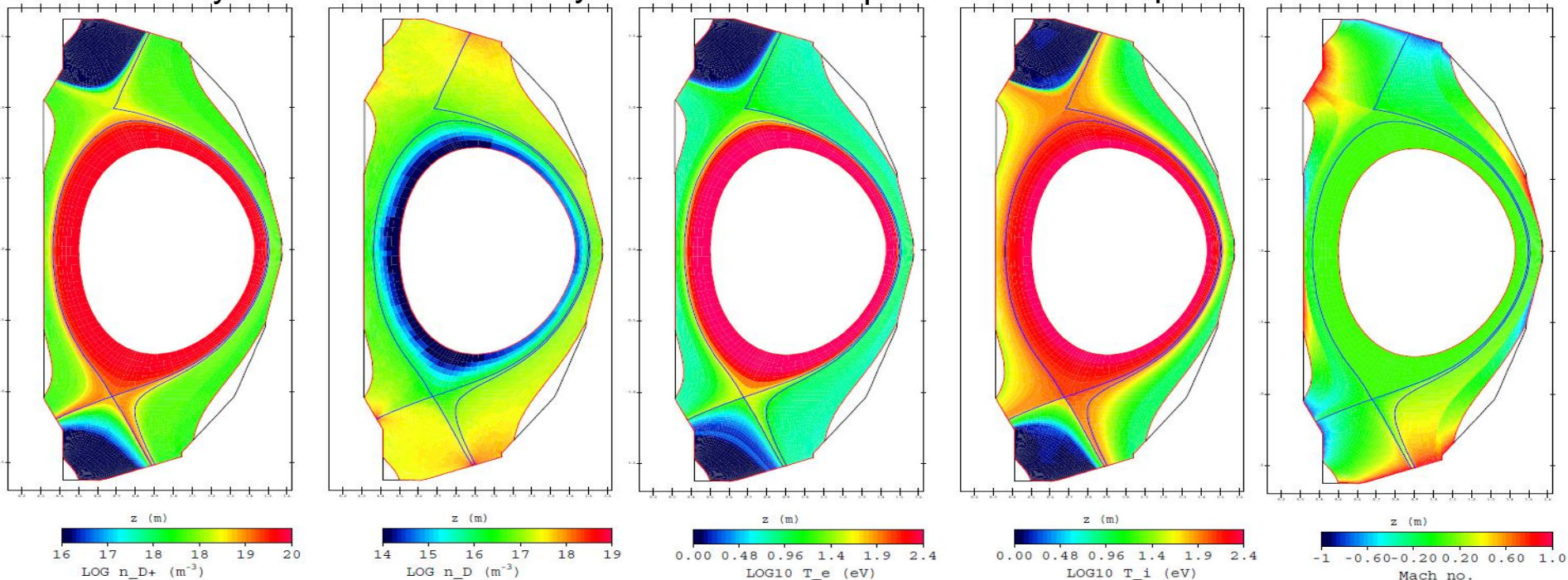
Ion Density

Neutral Density

Electron Temp.

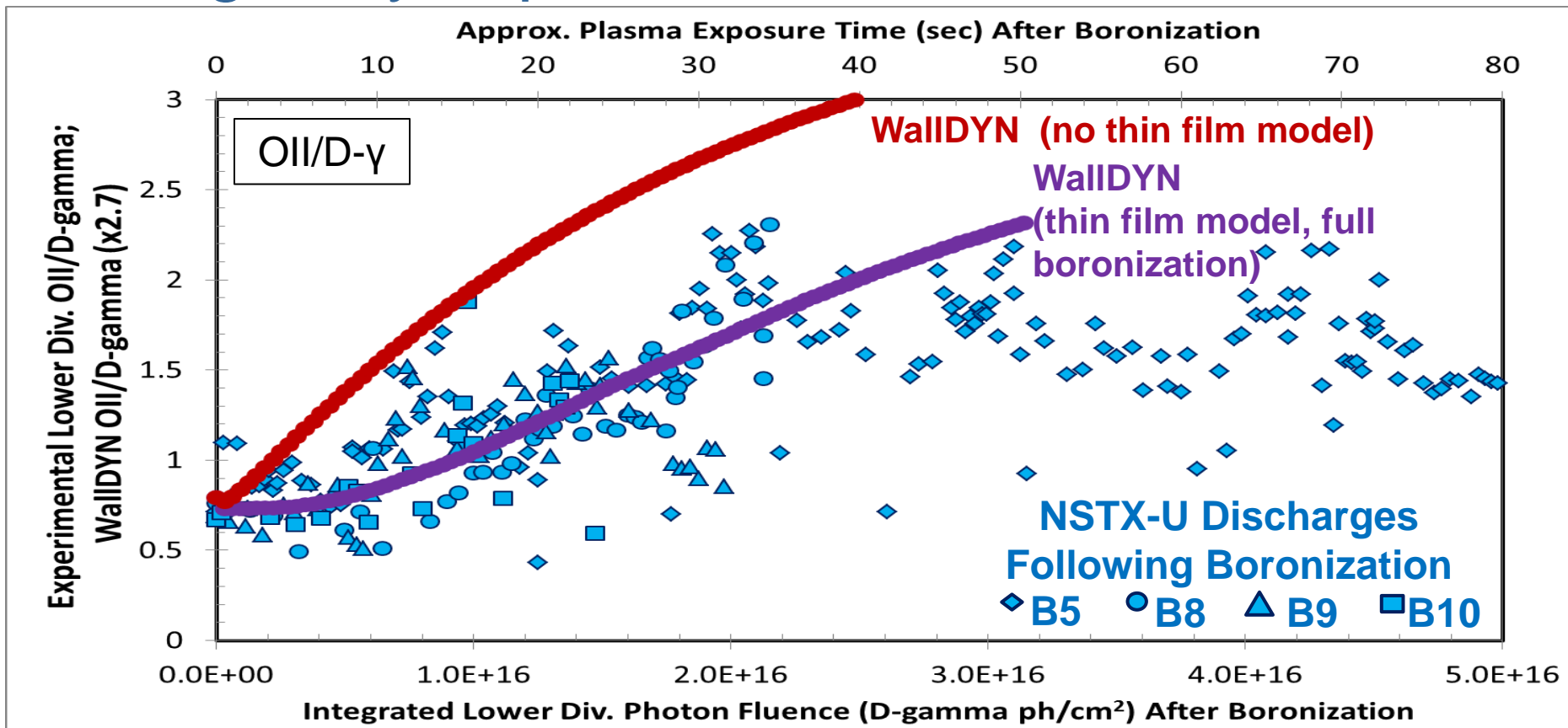
Ion Temp.

Parallel Flow

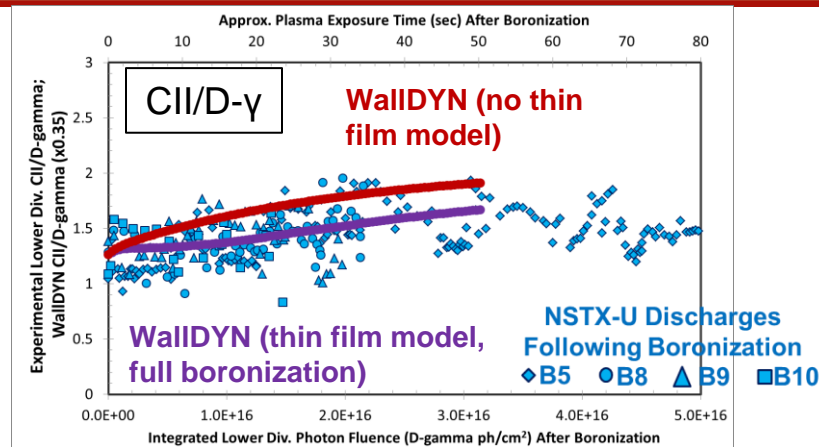
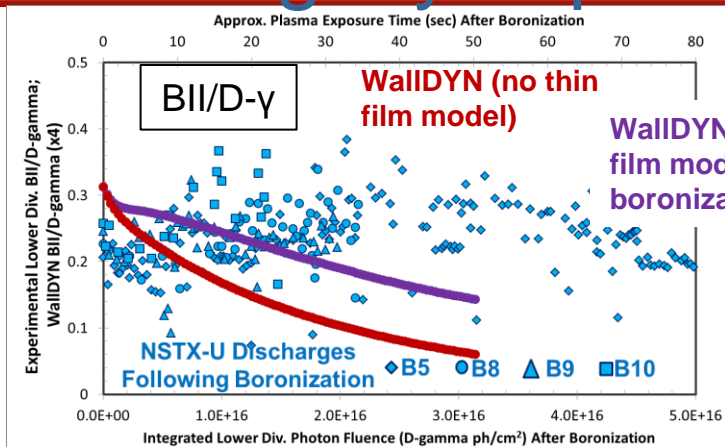


- High strike point density/temperature, large radial gradients (H-Mode like)
- No external flow/pinch applied

# Results: agreement with OII emission trends greatly improved with thin film model



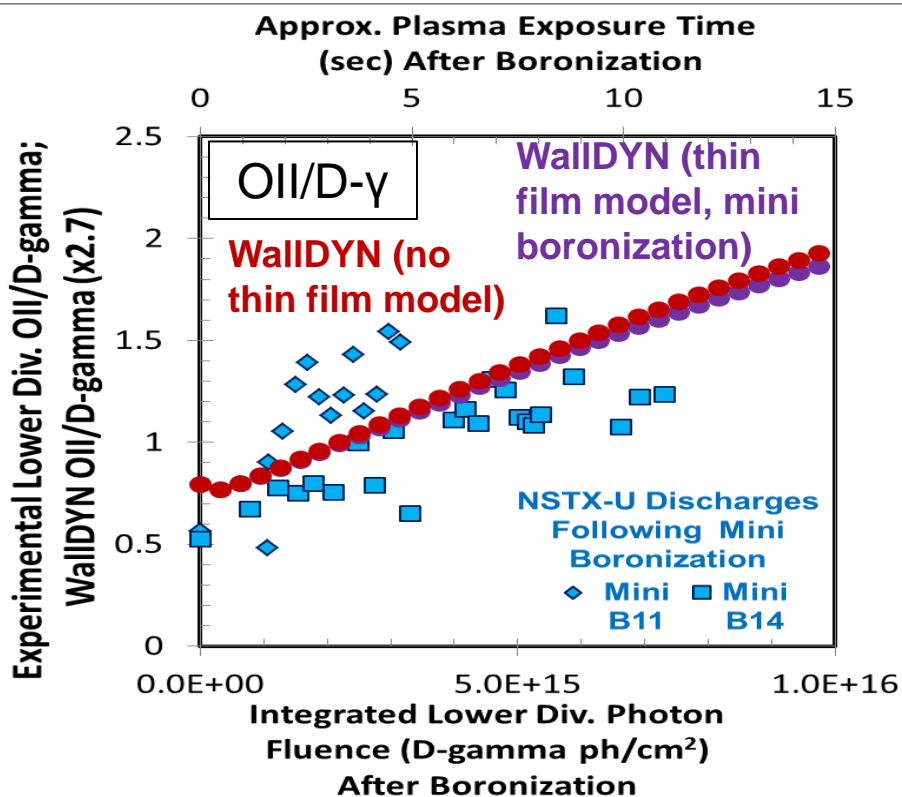
# Results: agreement with BII, CII emission trends slightly improved with thin film model



- Thin film model brings WalIDYN into good agreement with experimental oxygen emission trends following full boronizations
- No experimental trend in carbon emission following boronization
  - Good agreement with both WalIDYN models
- No experimental trend in boron emission following boronization
  - Poor agreement with both WalIDYN models
  - Reason for continued disagreement is under investigation

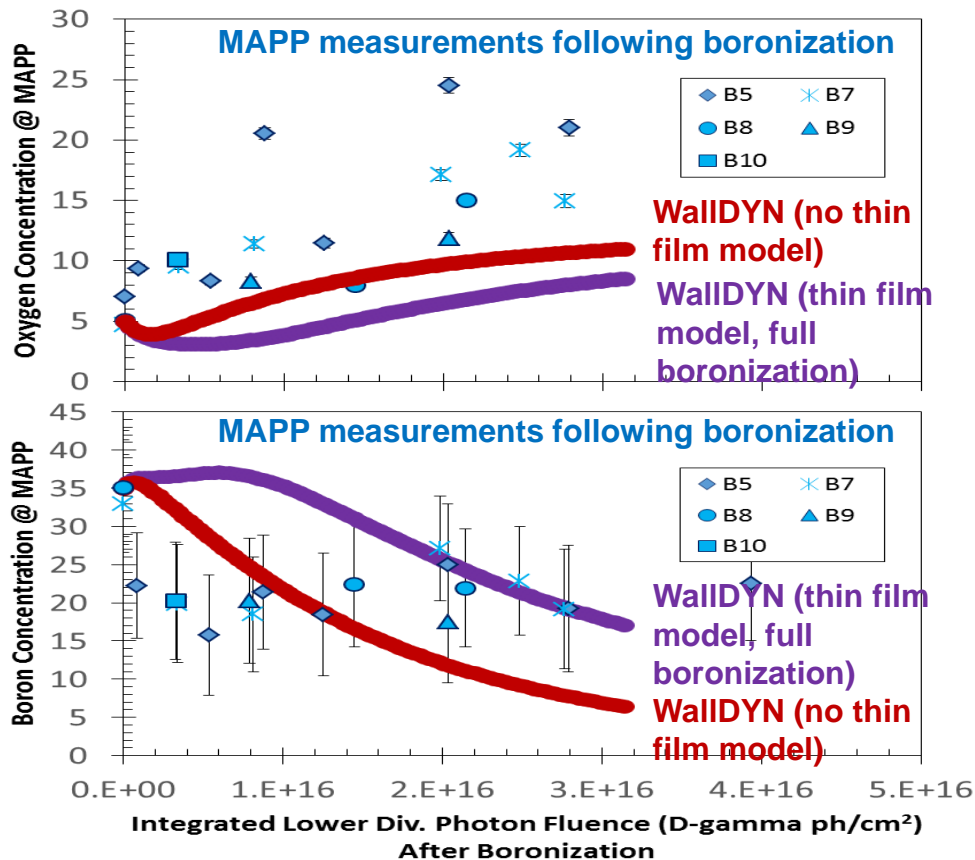


# Results: agreement with mini boronization emission trends similar with either model



- Thin film WalIDYN model capable of differentiating between full and mini boronizations
  - Captures faster rise in impurity influx following mini boronization
- Both model and experiment show degradation of mini boronization coating on time scale of  $\sim 1$  run day (15 sec plasma exposure)
- For mini boronization, thin film WalIDYN model is nearly identical to old WalIDYN model
  - Makes sense, because boron film applied to divertor ( $\sim 30$  A) is thinner than reaction layer (ion penetration depth  $\approx 40$  A)

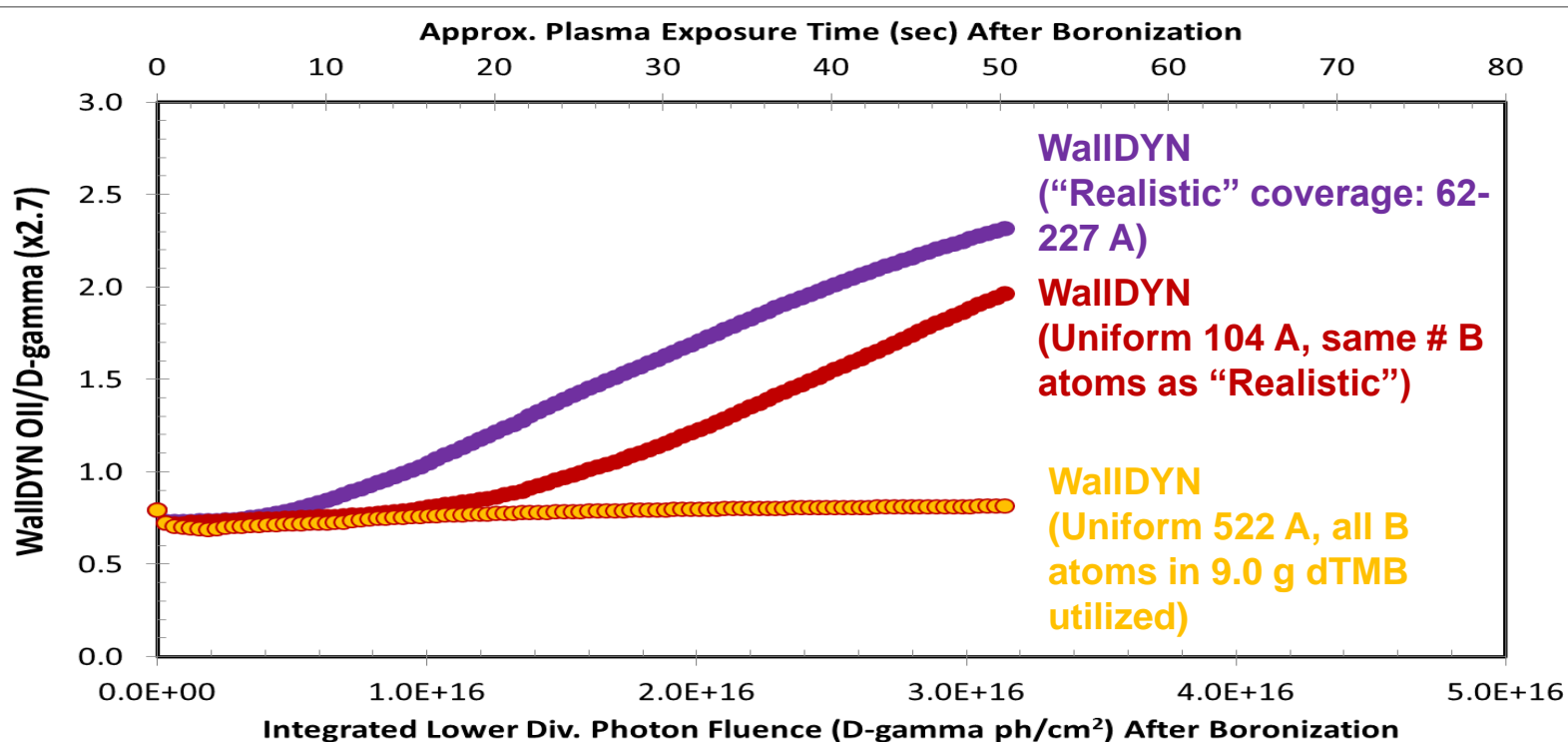
# Results: surface concentration evolution during simulations



- Both WalIDYN models underestimate the rise in oxygen surface composition observed in MAPP
- Surface boron agreement inconclusive
- Models do not take into account chemical state of surface
  - e.g. B-O vs. B-B vs. B-C bonds
  - May be important!

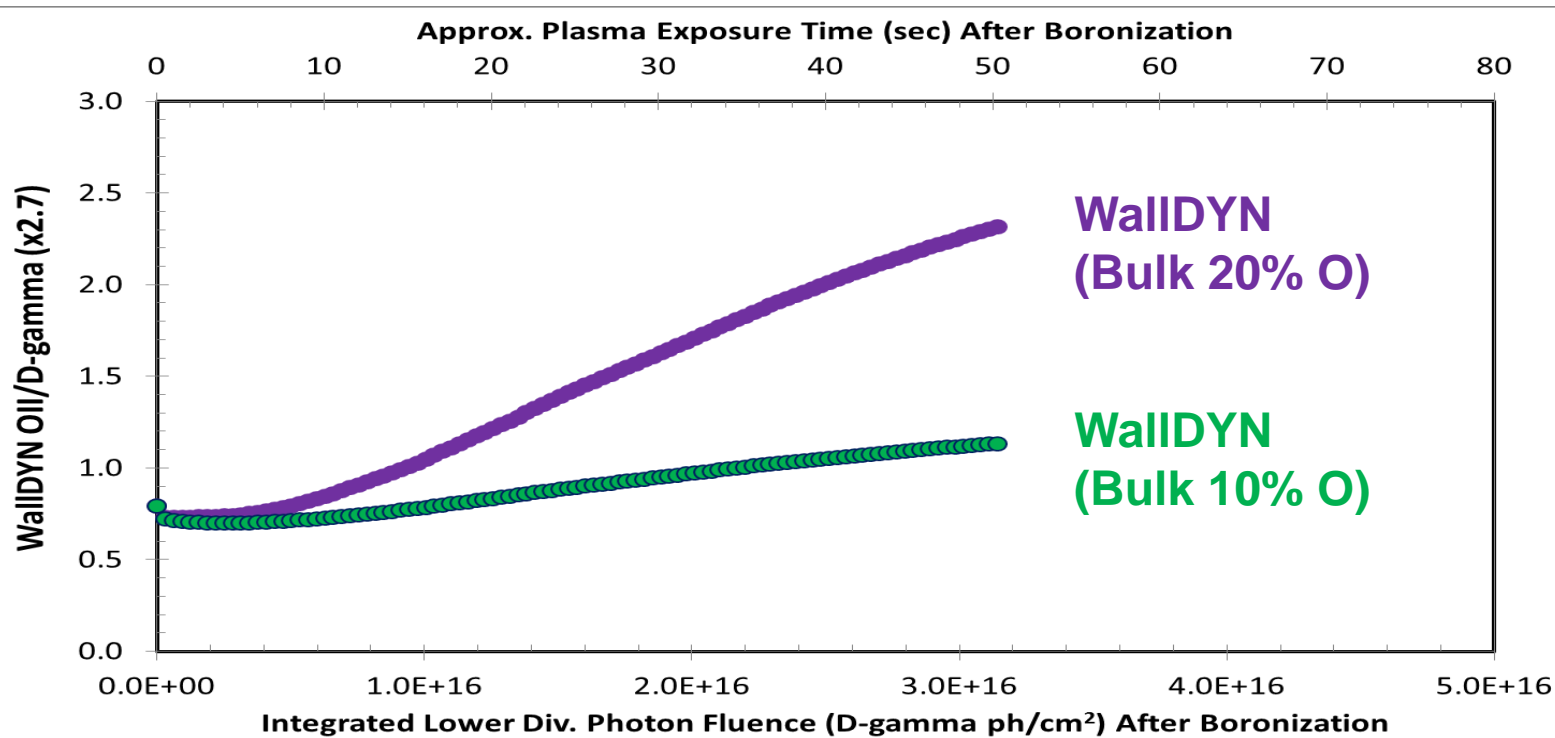
*For more B/C/O surface discussion see:  
F. Bedoya JO4.10 Tue PM  
H. Schamis PP11.57 Wed PM*

# Effect of more uniform boron coverage on oxygen evolution (*aka more GDC electrodes*)



- Uniformity of boron coatings is controlled by properties of He glow discharge and the number of anodes

# Effect of lower bulk oxygen concentration on oxygen evolution (*aka better bakeout*)



- Bulk oxygen content is typically controlled by pre-campaign conditioning (bakeout, etc.)