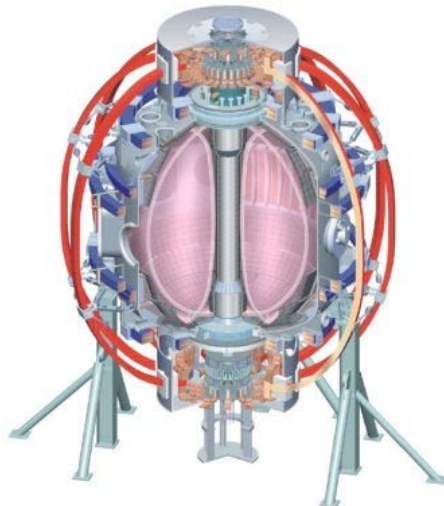


# Research on Lithium-Coated Plasma-Facing Components in NSTX

**Michael Bell**  
for the NSTX Research Team

*PPPL Theory Department Review,  
September 22, 2010*

College W&M  
Colorado Sch Mines  
Columbia U  
CompX  
General Atomics  
INL  
Johns Hopkins U  
LANL  
LLNL  
Lodestar  
MIT  
Nova Photonics  
New York U  
Old Dominion U  
ORNL  
PPPL  
PSI  
Princeton U  
Purdue U  
SNL  
Think Tank, Inc.  
UC Davis  
UC Irvine  
UCLA  
UCSD  
U Colorado  
U Illinois  
U Maryland  
U Rochester  
U Washington  
U Wisconsin



Culham Sci Ctr  
U St. Andrews  
York U  
Chubu U  
Fukui U  
Hiroshima U  
Hyogo U  
Kyoto U  
Kyushu U  
Kyushu Tokai U  
NIFS  
Niigata U  
U Tokyo  
JAEA  
Hebrew U  
Ioffe Inst  
RRC Kurchatov Inst  
TRINITI  
KBSI  
KAIST  
POSTECH  
ASIPP  
ENEA, Frascati  
CEA, Cadarache  
IPP, Jülich  
IPP, Garching  
ASCR, Czech Rep  
U Quebec

# NSTX has a Continuing Research Program into Effects of Lithium-Coated PFCs in *Divertor* Plasmas

**2005:** Injected lithium pellets (~5mg) into He discharges prior to D NBI shot

**2006:** LITHium EvaporatoR (**LITER**) deposited lithium on center column & lower divertor

**2007:** Larger evaporator re-aimed to increase deposition rate on lower divertor

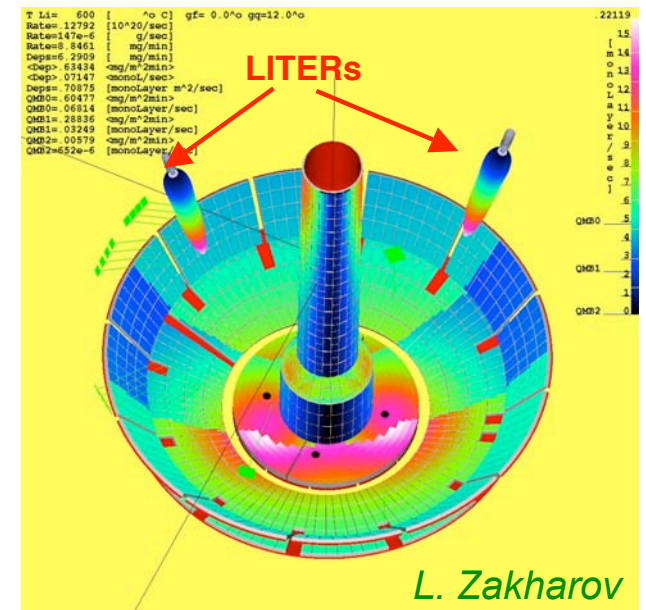
**2008:** Dual LITERs eliminated shadowed regions – Also tried “lithium powder dropper”

**2009:** Routine use of dual LITERs

**2010:** Liquid lithium on part of lower outer divertor

- NSTX program complements and extends lithium research on **TFTR**, **CDX-U**, **T-11**, **FTU** (limiter tokamaks), **TJ-II** (stellarator)
- Lithium research beginning on **LTX**, **HT-7**, **EAST** (divertors) and **RFX** (RFP)

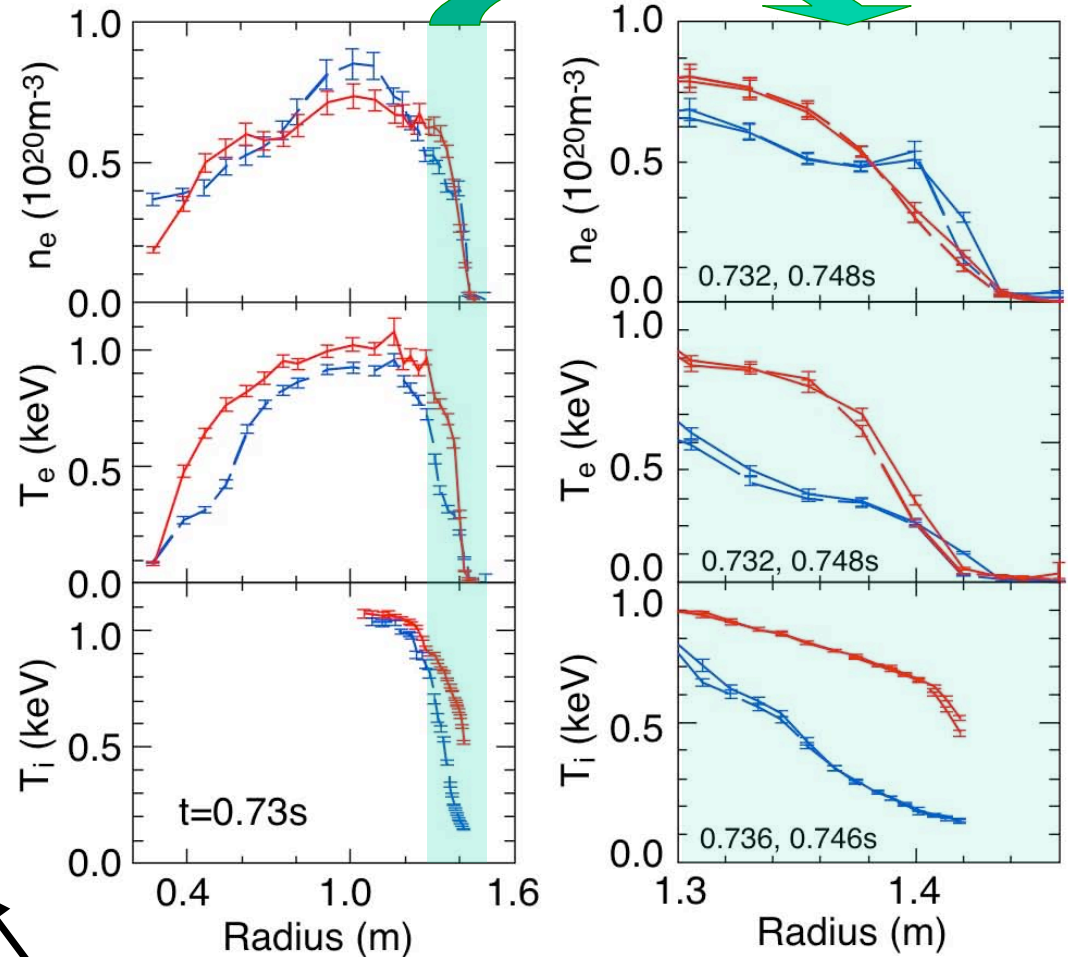
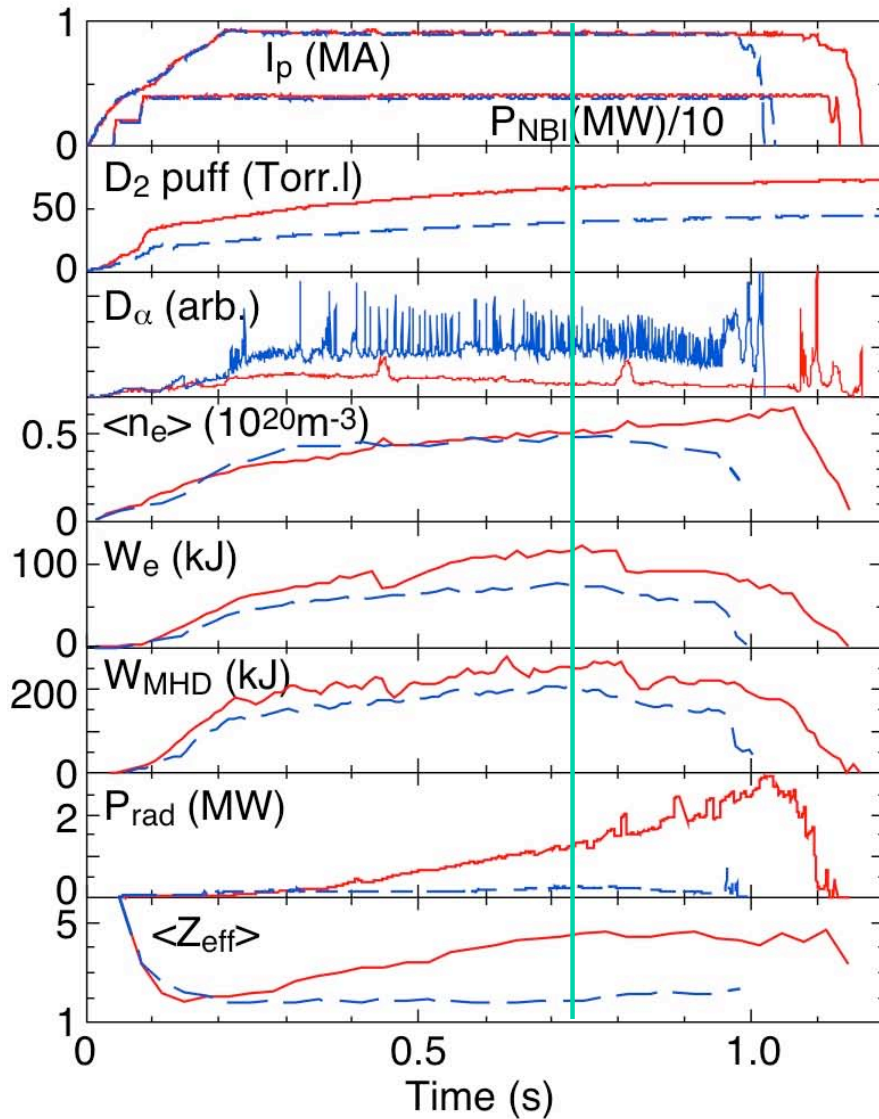
## Modeled 2-LITER deposition



- Typically apply 100 – 600 mg lithium between discharges
  - Lithium thickness on inner divertor 5 – 160 nm

# Lithium Coating Reduces Deuterium Recycling, Suppresses ELMs, Improves Confinement

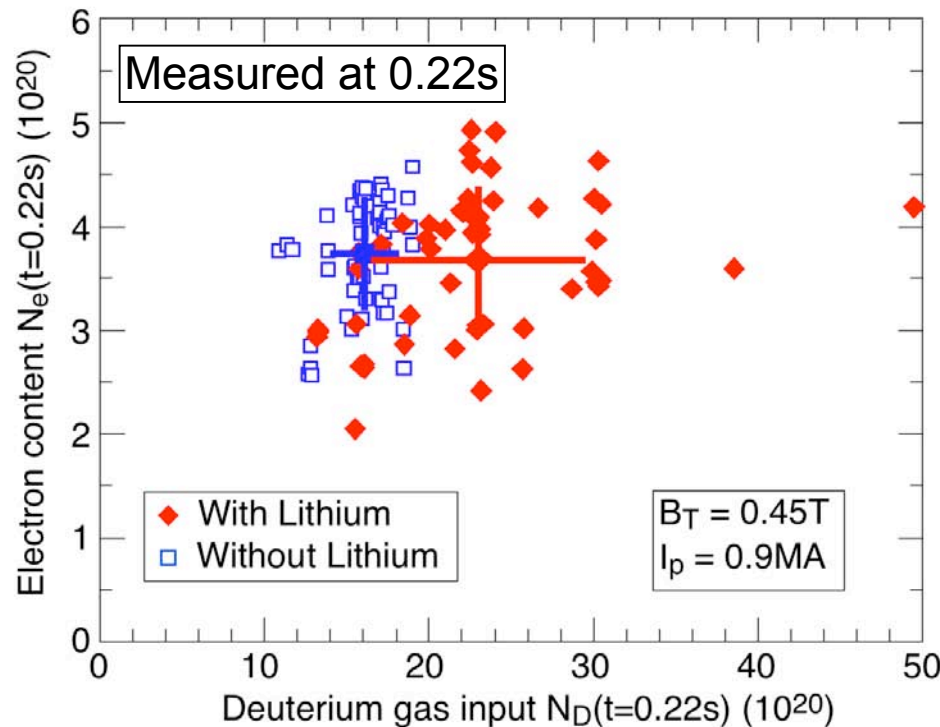
No lithium (129239); **260mg lithium (129245)**



Without ELMs, impurity accumulation increases radiated power and  $Z_{eff}$

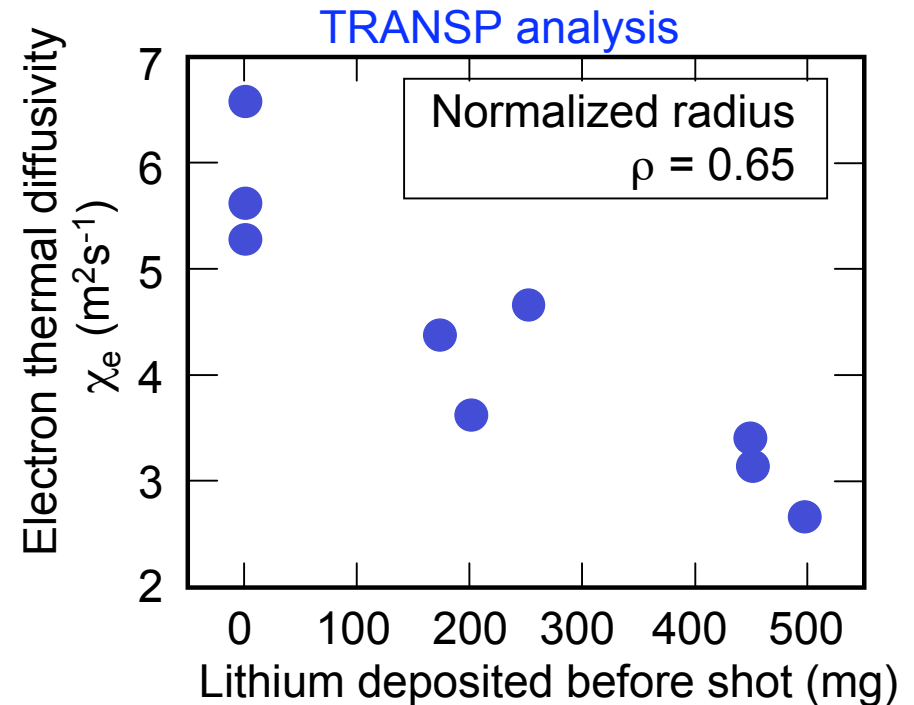
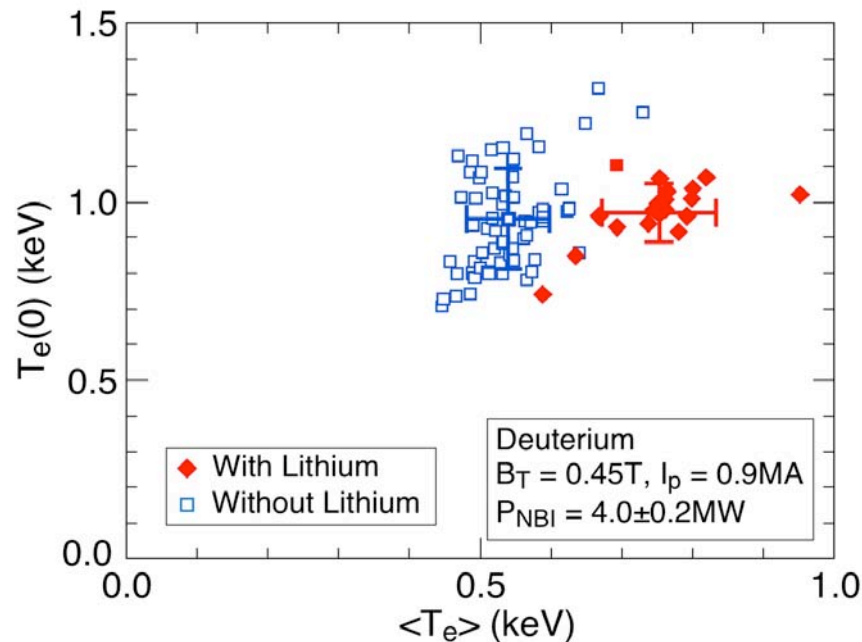
# Although Lithium Reduces Deuterium Recycling, Need to Increase Fueling to Avoid Locked Modes

- Measurements of  $D_\alpha$  emission profile show greatly reduced neutral D density across outboard midplane with lithium
- Lower density achievable early in discharges both with and without lithium but likelihood of deleterious locked modes increases
  - Lower density desirable to achieve NSTX goal of non-inductive sustainment



- *MHD stability of low-density shaped plasmas a critical issue*

# Broadening of Electron Temperature Profile Implies Reduction of Thermal Diffusivity in Outer Region

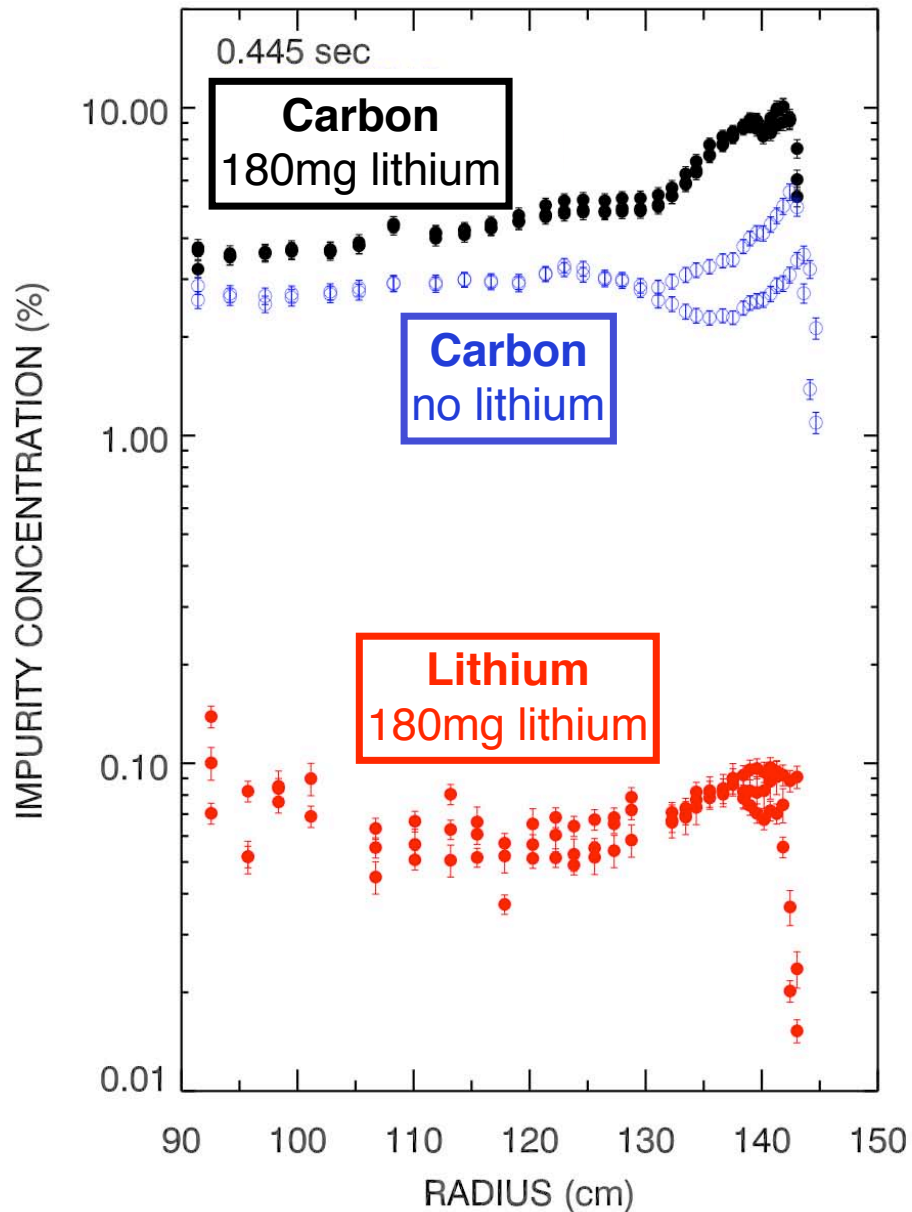


- All plasmas, both with and without lithium, in H-mode
- H-mode threshold reduced by lithium coating by up to factor 4
- Fast-ion contribution to total energy ( $\propto T_e^{3/2}/n_e$ ) also increases
- Thermal ion confinement remains close to neoclassical level both with and without lithium

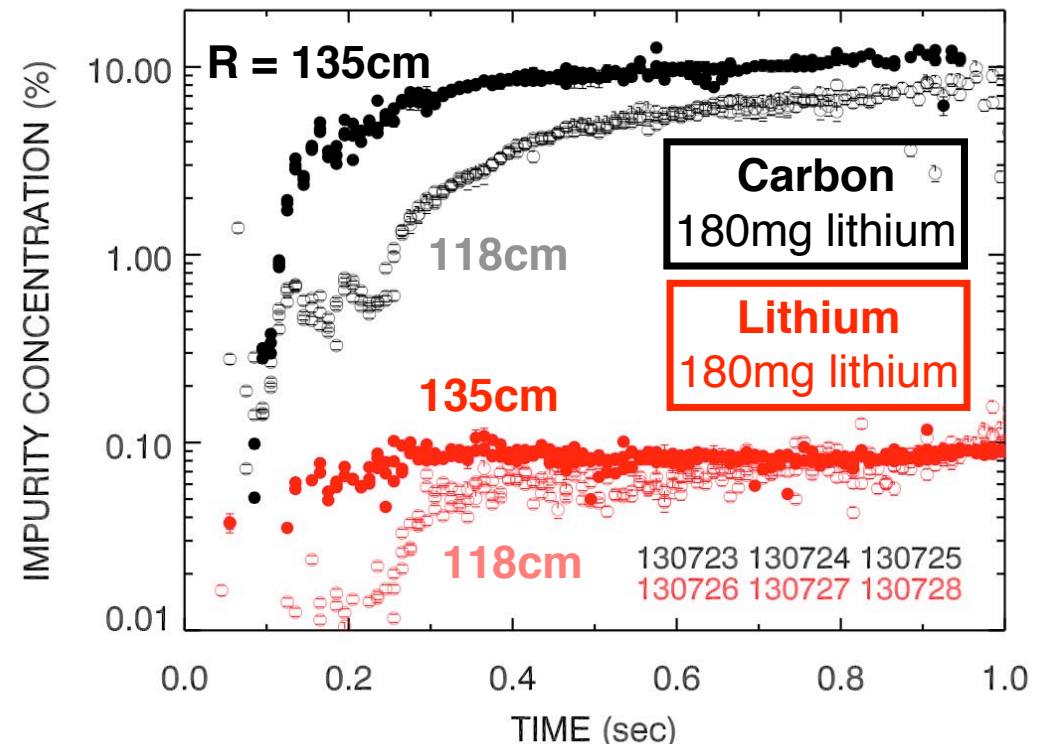
# Improved Confinement Provides Opportunities for Theoretical Explanation and Model Testing

- In TFTR, improvement with lithium was in the “supershot” regime
  - Anomalous transport associated with ITG turbulence:  $T_{i,\max}(0) \propto T_i(a)$
  - Lithium reduced recycling below what was achievable with “conditioned” carbon PFCs allowing higher edge temperature *and*
  - Larger E/B shearing with peaked  $p(r)$  stabilized modes for  $\omega_{\text{EXB}} > \gamma_{\text{ITG}}$
  - Electron temperature profile also broadened with lithium coating
- Also note theoretical predictions of enhanced ion neoclassical transport driven by cold ions from edge [A.A. Ware, PFB **2** (1990) 1435]
- In NSTX, transport appears to be reduced in electron channel
  - Ion transport already close to neoclassical: high E/B shearing at lower B
  - Suppression of ETG modes has been associated with reduced  $\chi_e$  in NSTX reversed-shear plasmas *but*
  - We do not have measurements confirming ETG suppressed by lithium
  - $T_e$  profile is tending towards predictions of flat  $T_e$  with fully absorbing wall

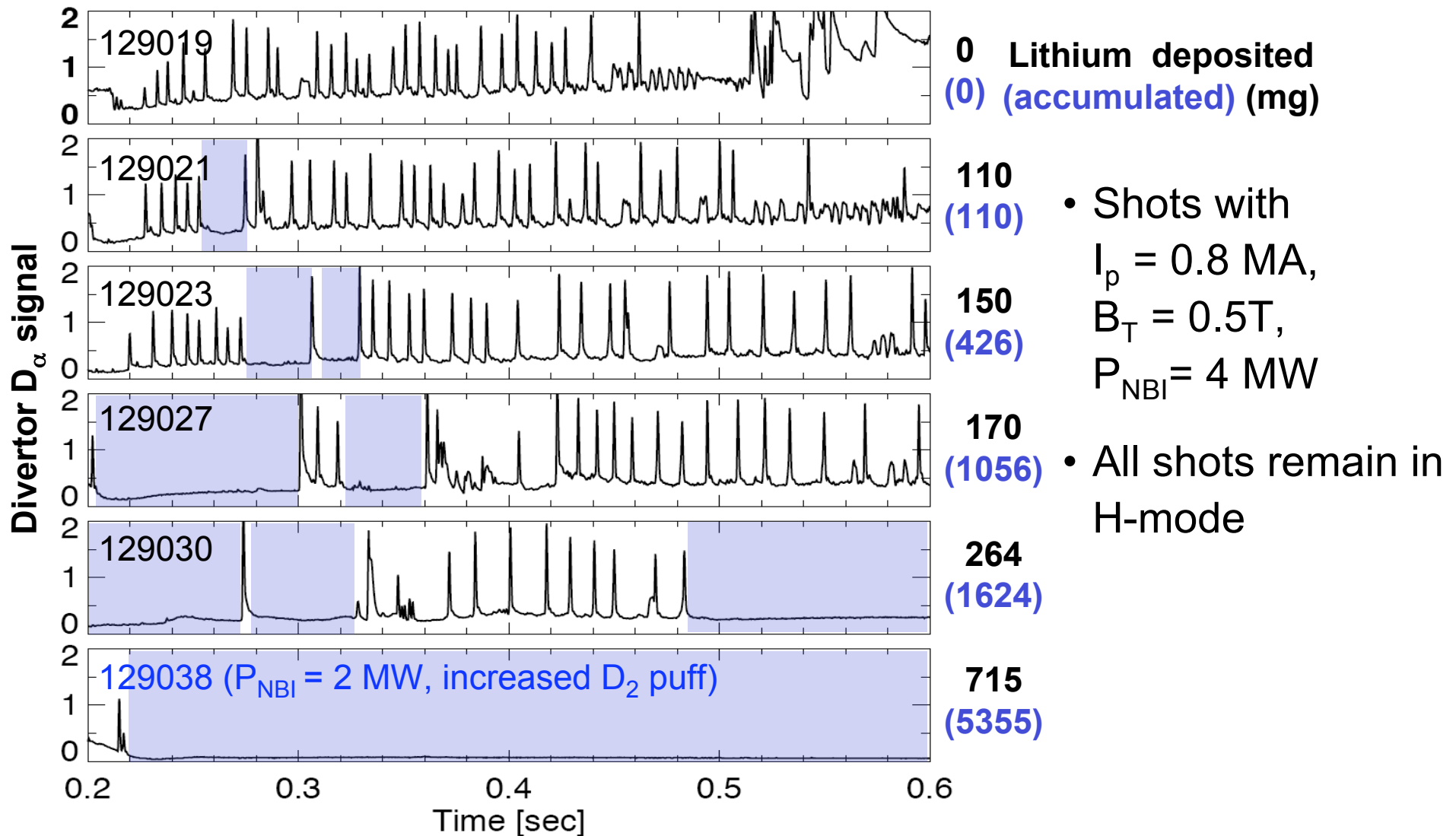
# Lithium Concentration in Plasmas Remains Low but Carbon Concentration Rises with Lithium Coating



- Quantitative measurements of  $C^{6+}$ ,  $Li^{3+}$  with charge-exchange recombination spectroscopy
- $n_C/n_{Li} = 30 - 100$
- Hollow profiles early for both C and Li fill in as time progresses



# Suppression of ELMs Occurs By Lengthening and Coalescence of ELM-free Periods

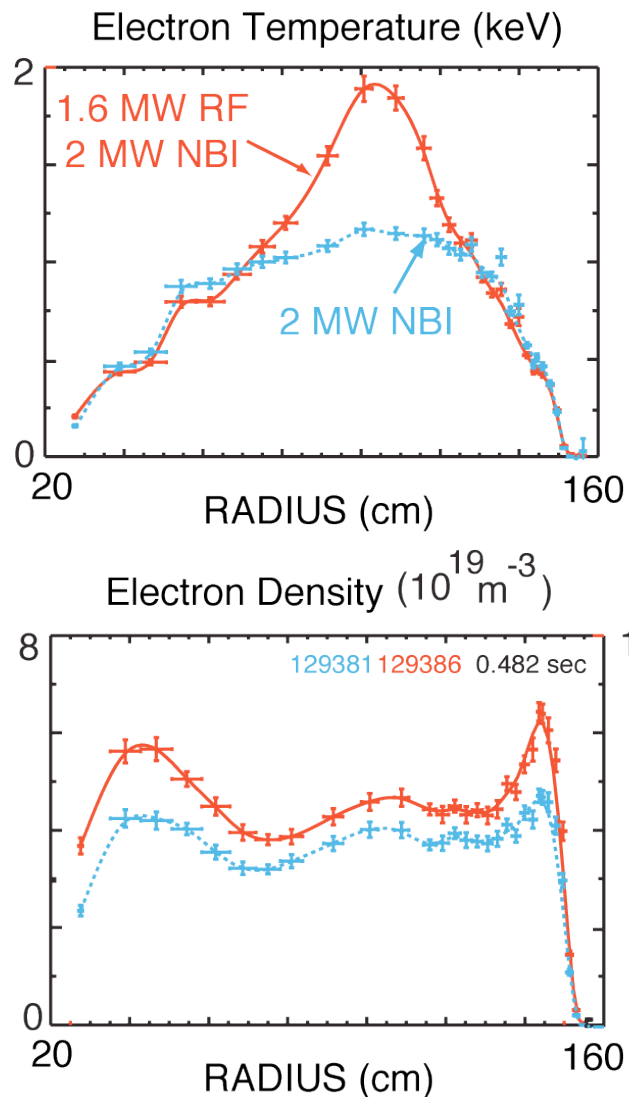


- ELM suppression was predicted through changes in location of current density gradient with respect to mode rational surfaces (Zakharov, 2006)

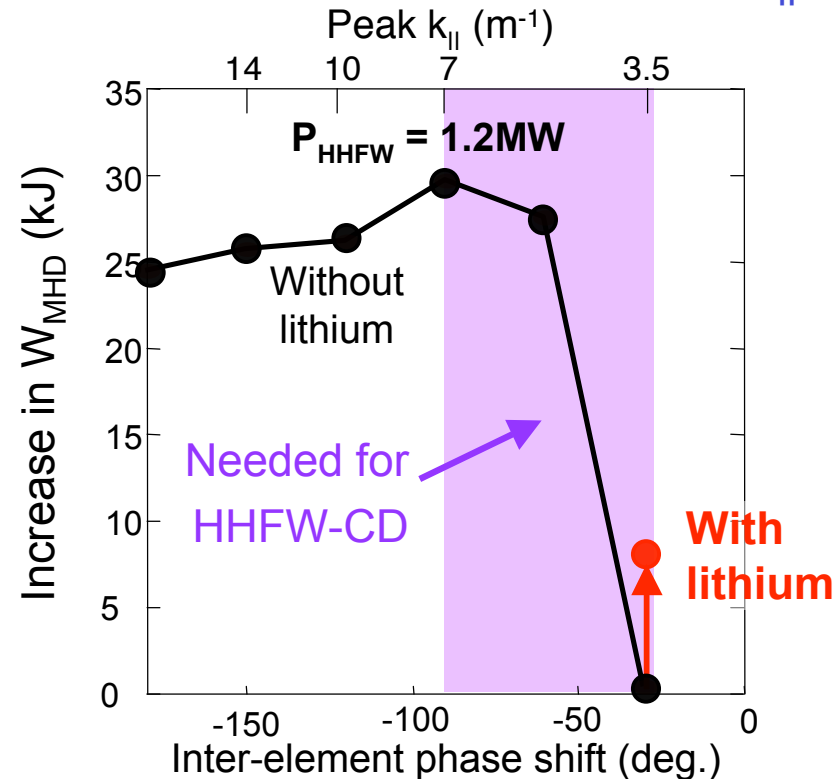


# Lithium Coating Improves HHFW Heating Efficiency in NBI H-Modes and at Low $k_{||}$ for Current Drive

## Core Electron Heating in Deuterium NBI H-Mode



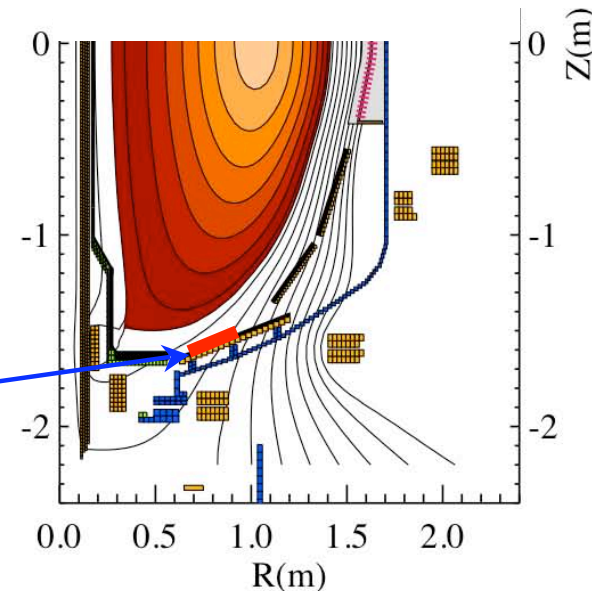
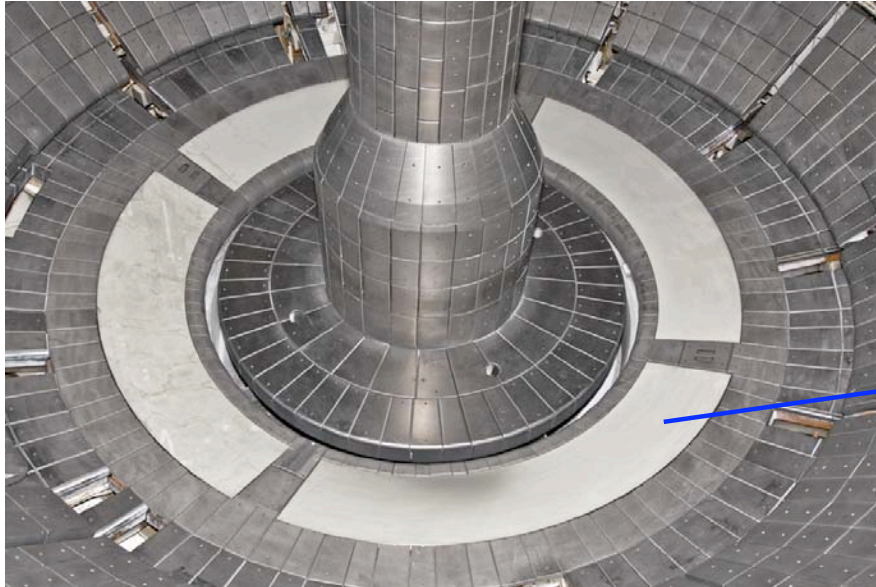
## First indications of heating at low $k_{||}$ in D



- Reflectometer measures reduced SOL density in front of antenna with lithium
  - Results consistent with suppression of parasitic surface waves in SOL
    - $n_{e,\text{SOL}} < n_{e,\text{crit}} \propto B k_{||}^2 / \omega$

# This Year, NSTX Began Investigating *Liquid Lithium* on Plasma Facing Components

- Laboratory measurements in PISCES and experience in CDX-U show that liquid has much higher capacity for deuterium retention than solid



- Replaced carbon tiles in outer lower divertor with 4 segmented plates
  - Plasma-facing surface coated with semi-porous (~50%) molybdenum
  - Surface could be heated to  $>300^{\circ}\text{C}$  (Li melting point  $180^{\circ}\text{C}$ )
  - Supplied lithium with LITERs and lithium powder dropper
- Initial experiments designed to evaluate capability of liquid lithium to sustain deuterium pumping
  - Small differences observed so far at variance with original modeling

# Benefits (& Problems) of Lithium Coated PFCs Provide Fertile Ground for Theory & Modeling

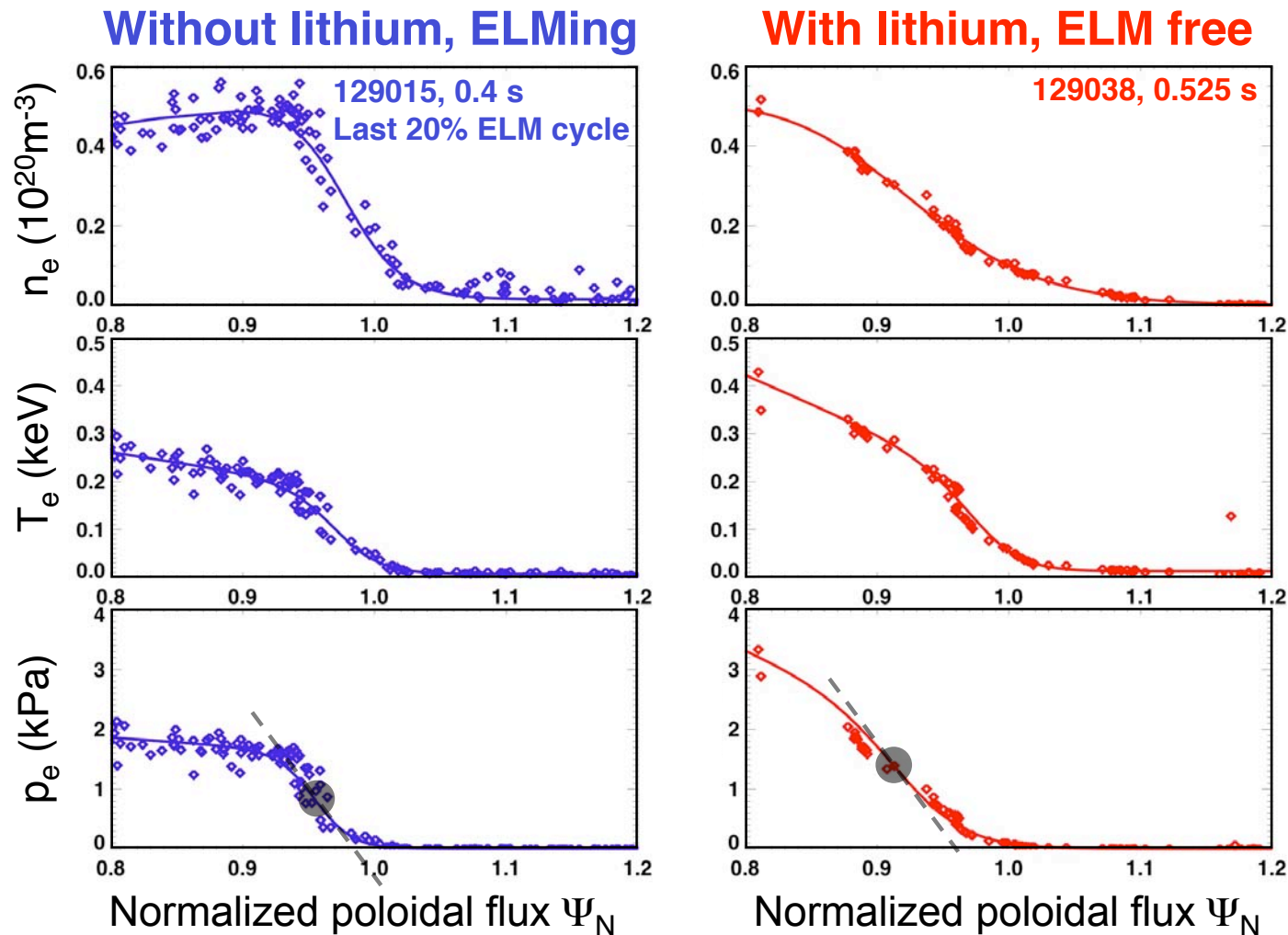
- Reduced hydrogenic recycling
  - Theory and modeling of particle transport including neoclassical and anomalous transport
  - How are edge flows changed by the presence of divertor pumping?
- Reduced H-mode threshold power by up to a factor 4
  - Models for edge barrier formation
  - Z-dependence of particle transport and role in impurity accumulation
- Improved confinement, particularly in electron channel
  - Roles of high- and low- k turbulence & suppression at low aspect ratio
- ELM suppression through changes in edge profiles
  - Is peeling-ballooning model adequate and extrapolable?
  - Roles of edge currents and flows and how these are changed
  - Extrapolable models for ELM triggering by magnetic perturbations
- Optimizing ICRH coupling to core plasma
  - Role of parasitic surface waves and loss processes
- We already have a wealth of data on lithium effects but *guidance on new diagnostics or upgrades to address critical theory needs is encouraged*

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# Supplementary material on ELM Effects of Lithium

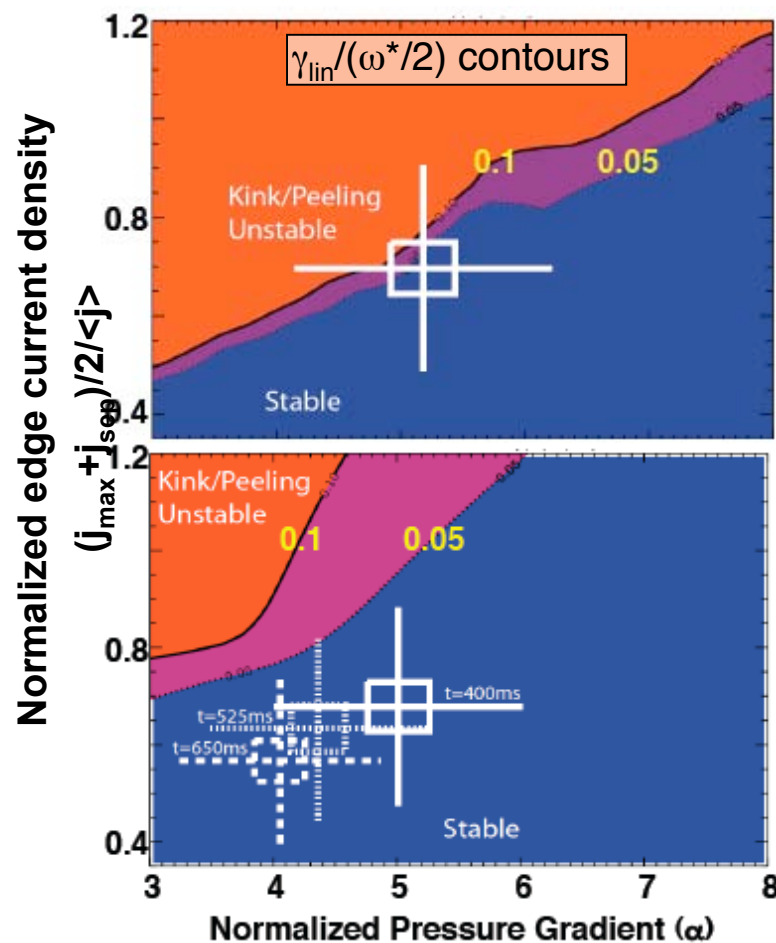
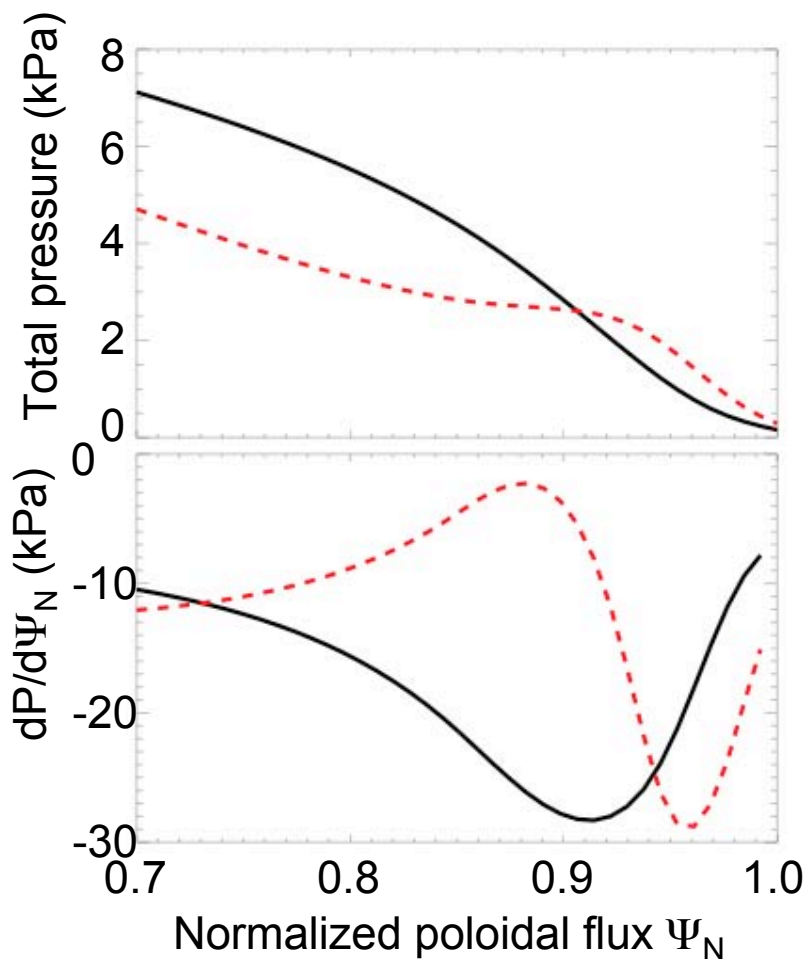
# Lithium Affects ELMs Through Changes in Temperature and Pressure Profile at Edge

- Multiple timeslices mapped into composite profiles using EFIT equilibrium



# Shift of Maximum in $\nabla p_e$ to Region of Lower Shear with Lithium Stabilizes Kink/Ballooning

- Analysis with PEST and ELITE codes
- Change in recycling affects edge current
- Precursor activity with  $n = 1 - 5$  observed before ELM onset



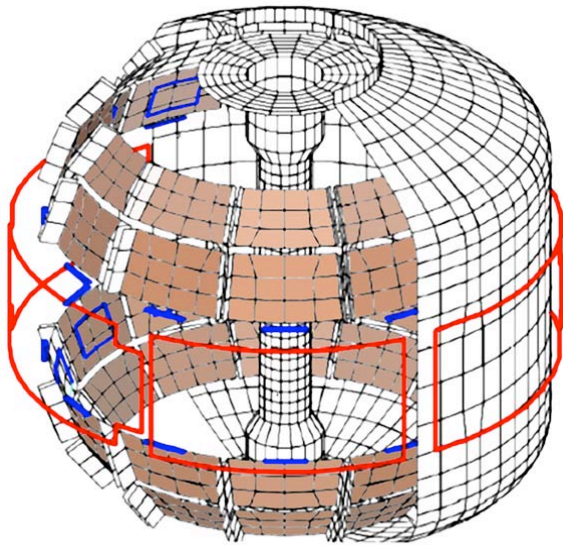
Without lithium  
(end of ELM cycle)

With lithium



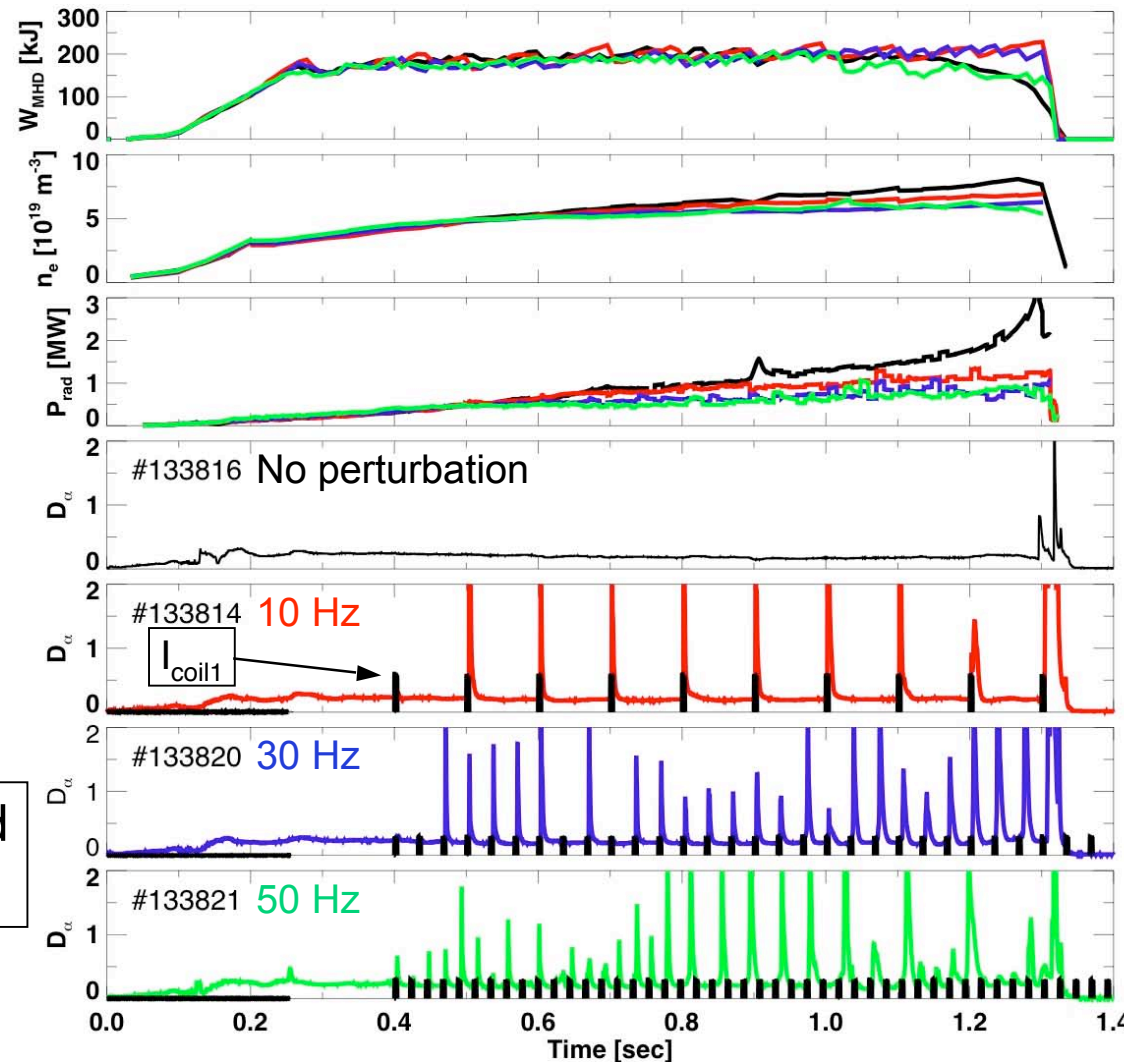
# External Non-Axisymmetric Coils Can *Induce* Repetitive ELMs in Discharges with Lithium Coating

Generate  $n = 3$  resonant radial field perturbations with **3 pairs of midplane coils**



3 Switching Power Amplifiers applied trains of 3kA, 4ms square pulses

Double-null,  $\kappa=2.4$ ,  $\delta=0.8$ , 0.8MA, 0.45T, NBI 4 MW



- Induced ELMs reduce  $n_e$ ,  $P_{rad}$ ,  $Z_{eff}$  with small effect on plasma energy