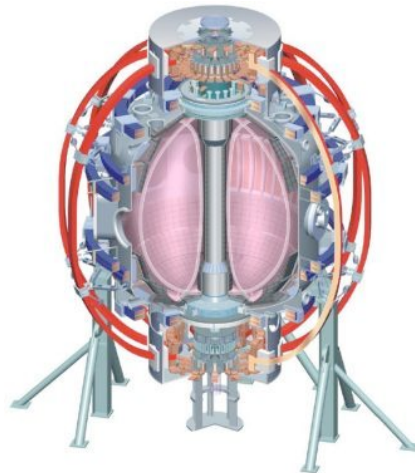


# Effective Sheath Heat Transmission Coefficient in NSTX Discharges with Applied Lithium Coatings

**Josh Kallman**

**Princeton University FPOE  
7/18/2011**

College W&M  
Colorado Sch Mines  
Columbia U  
CompX  
General Atomics  
INEL  
Johns Hopkins U  
LANL  
LLNL  
Lodestar  
MIT  
Nova Photonics  
New York U  
Old Dominion U  
ORNL  
PPPL  
PSI  
Princeton U  
Purdue U  
SNL  
Think Tank, Inc.  
UC Davis  
UC Irvine  
UCLA  
UCSD  
U Colorado  
U Illinois  
U Maryland  
U Rochester  
U Washington  
U Wisconsin

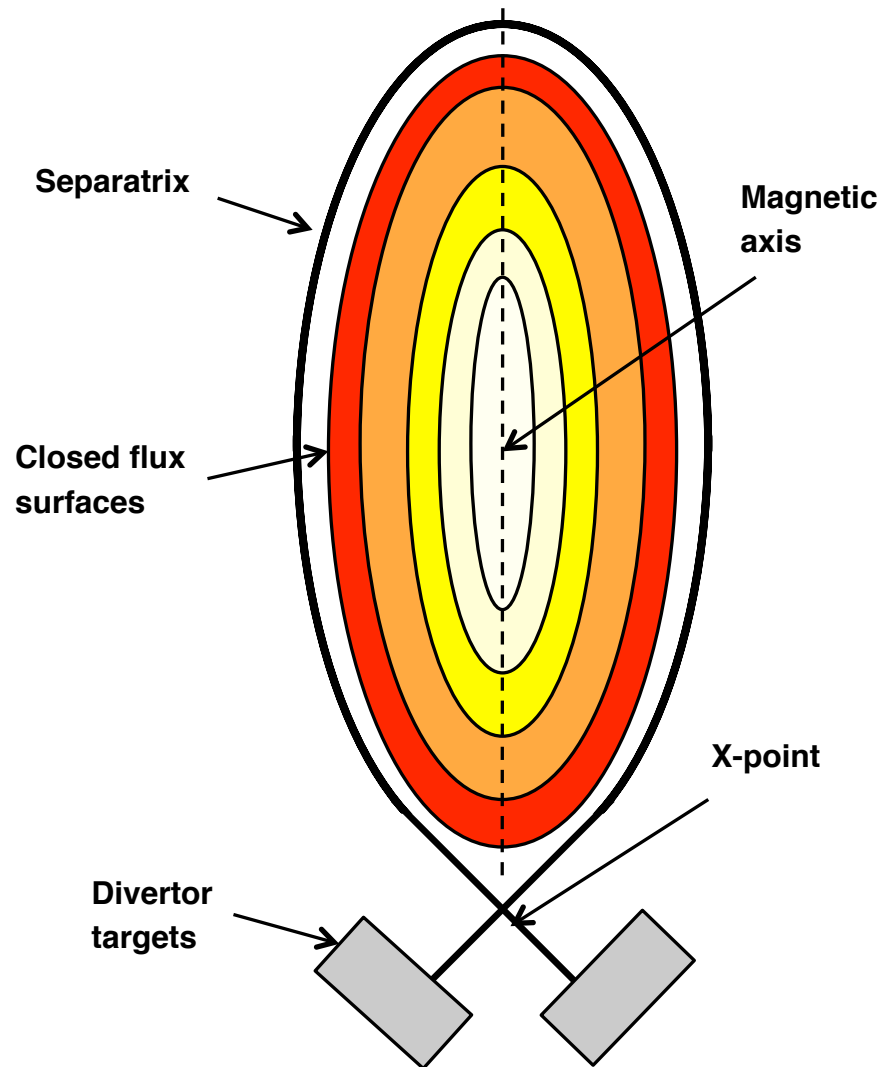


Culham Sci Ctr  
U St. Andrews  
York U  
Chubu U  
Fukui U  
Hiroshima U  
Hyogo U  
Kyoto U  
Kyushu U  
Kyushu Tokai U  
NIFS  
Niigata U  
U Tokyo  
JAEA  
Hebrew U  
Ioffe Inst  
RRC Kurchatov Inst  
TRINITI  
KBSI  
KAIST  
POSTECH  
ASIPP  
ENEA, Frascati  
CEA, Cadarache  
IPP, Jülich  
IPP, Garching  
ASCR, Czech Rep  
U Quebec

# Outline

- The magnetic fusion plasma edge
  - Diverted tokamaks, heat and particle flux
  - Sheath heat transmission
- NSTX and edge measurement capabilities
  - NSTX overview
  - The Liquid Lithium Divertor and associated diagnostics
- Diagnostics for heat flux analysis
  - Dense Langmuir probe array
  - Dual-band IR camera
- Heat flux data and analysis
  - Data acquisition
  - Analysis methods
  - Measurements of sheath heat transmission
- Conclusions and Discussion
- Future work

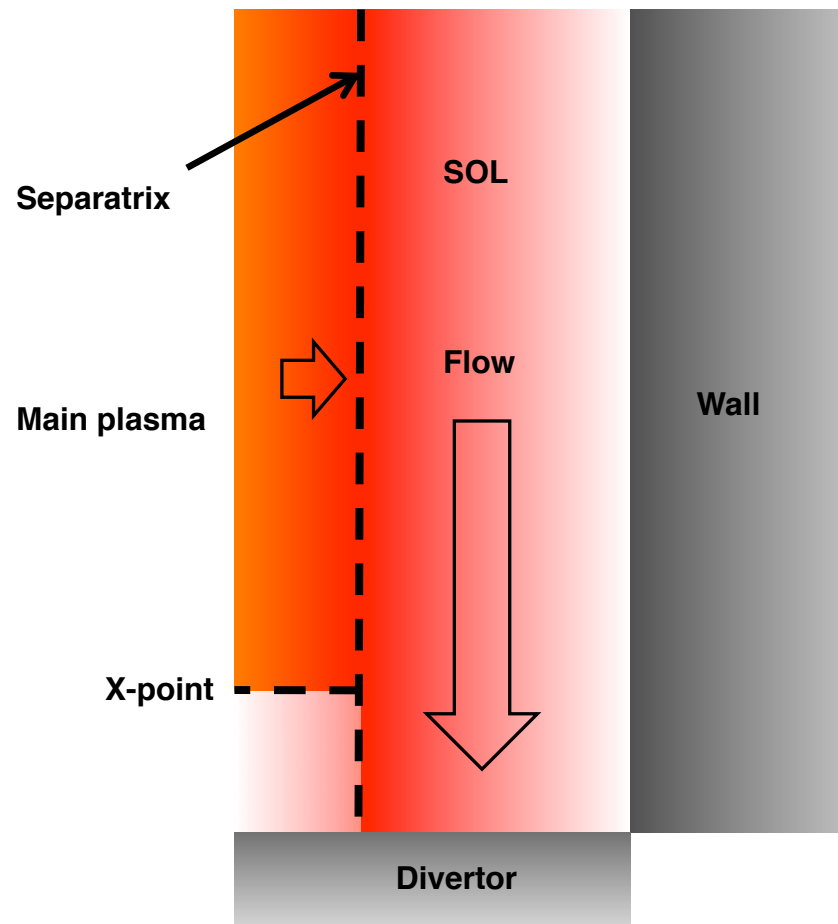
# The diverted tokamak



- The divertor is a magnetic field geometry that directs particle flux to target plates rather than using device walls as limiting structures
- Allows for control of location of particle interaction: isolates from main plasma and can be located near pump ducts for ease of exhaust
- Drawbacks: high localized heat flux magnitudes; in ITER, where average heat flux magnitudes can be expected to be  $>10 \text{ MW/m}^2$ , with peak values from 20-50

## Heat and particle flux at the divertor

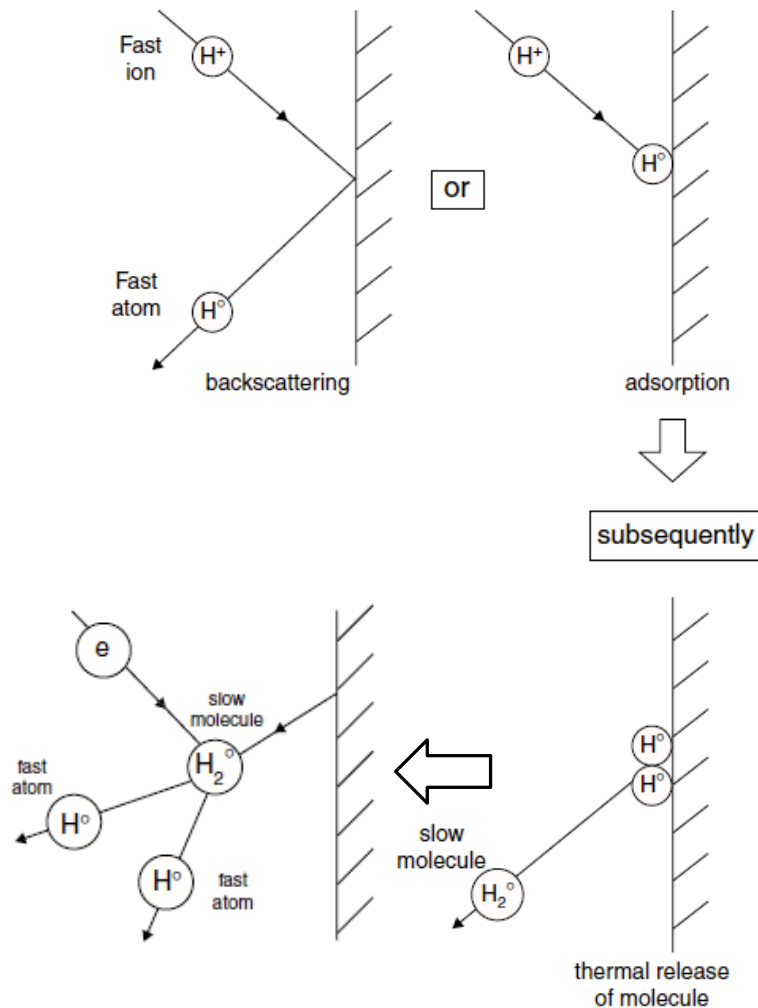
- All power input into plasma (ohmic, NB, RF) must be exhausted by particles or radiation
- Particles escaping core confinement enter the scrape-off layer (SOL), and are transported to divertor surfaces
- Wall-interactions are localized, but can have far-reaching consequences for the plasma





## Recycling – not the good kind

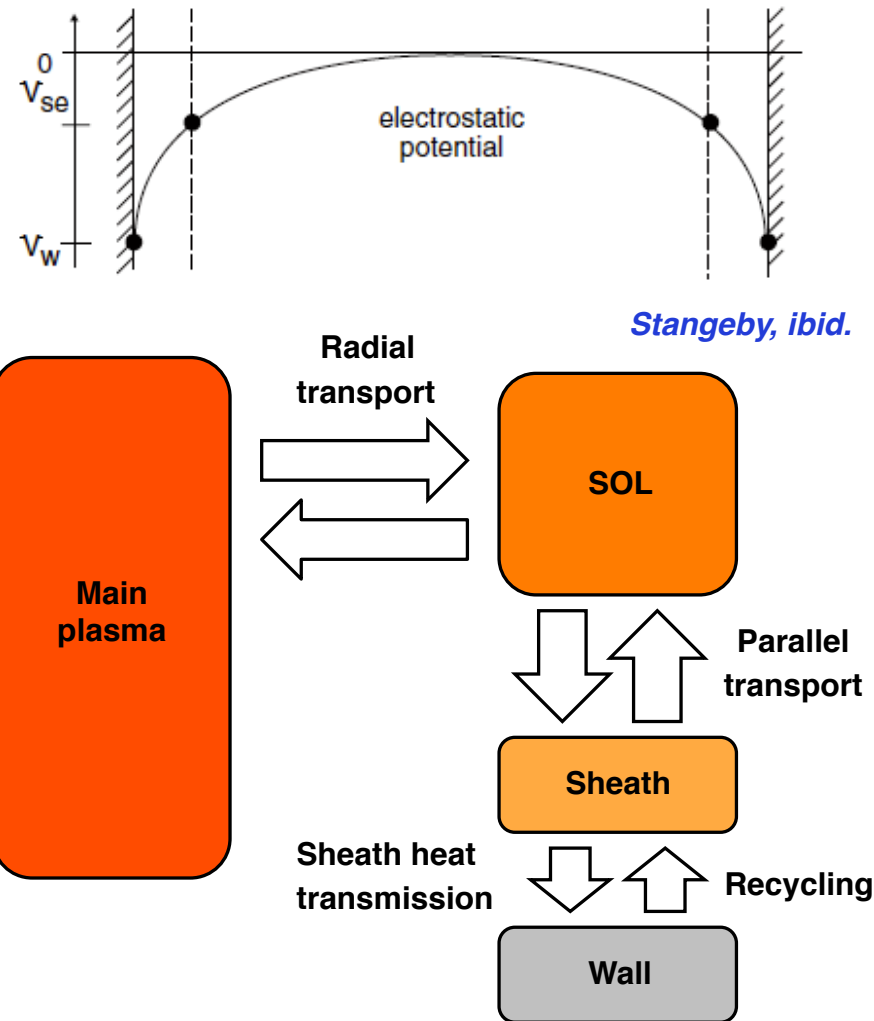
- Graphite, a traditional tokamak wall material, is an excellent heat handler, but a poor particle sink
- Incoming particles will saturate the wall tiles, and subsequent incoming ions will lead to ejection of cold neutrals
- Recycling coefficients for graphite can exceed 98% when saturated
- Recycled neutrals re-enter the SOL, and consume energy as they are re-ionized, increasing edge density and lowering edge temperature



*Strangeby, The Plasma Boundary of Magnetic Fusion Devices*

## The plasma sheath links the wall to the main plasma

- An electrostatic sheath forms around all plasma-facing surfaces
- All particles impacting the divertor surface must pass through the sheath
- The surface is linked to the SOL through the sheath, and the SOL parameters set the boundary conditions for the bulk plasma
- Modeling efforts can quantitatively establish this relationship, but require knowledge of how power is transmitted through the sheath



## Classical picture of sheath heat transmission relies on several key assumptions

- The sheath heat transmission coefficient,  $\gamma$ , links the material walls with the plasma
- SOL electrons are modeled as a Maxwellian distribution, while the ions are treated as a drifting Maxwellian moving at the sound speed
- The net heat flux to the sheath is the contribution from these populations and is given by:  $q = \gamma k_B T_e \Gamma$

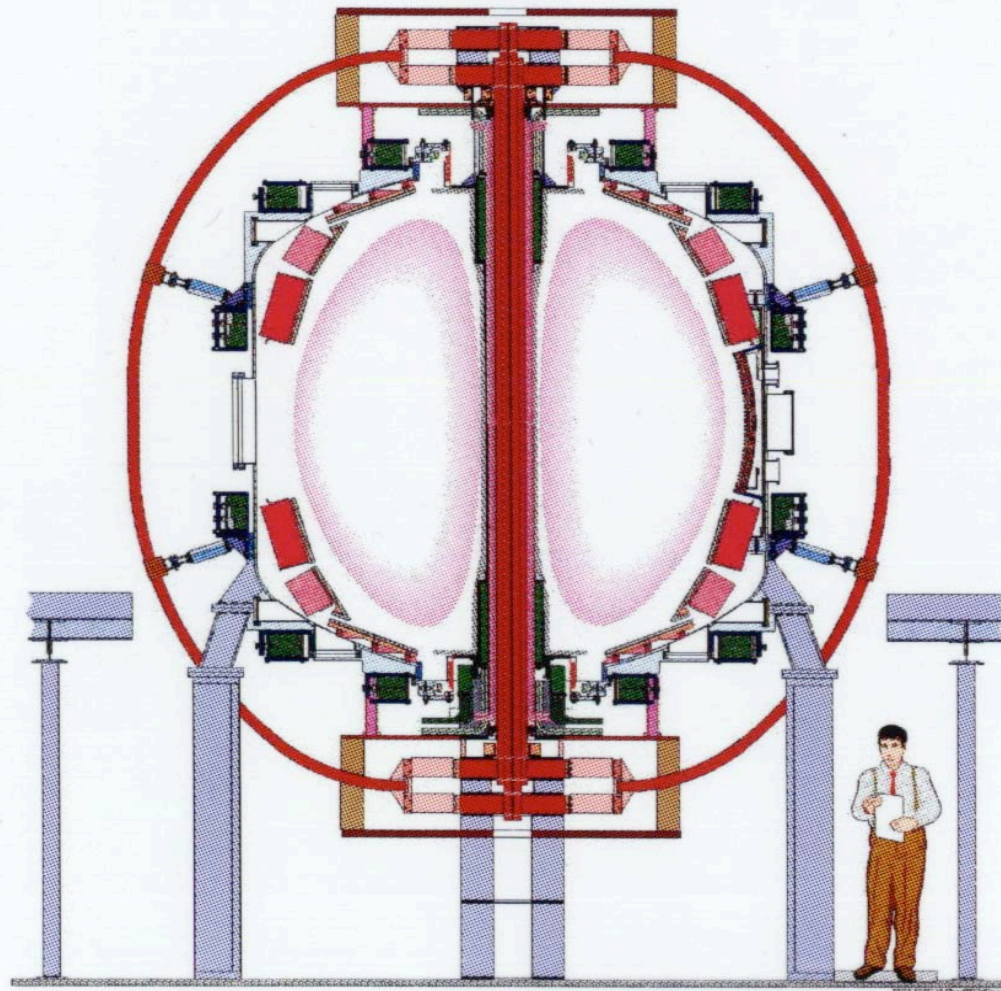
$$\gamma = 2.5 \frac{T_i}{T_e} + \frac{2}{1 - \delta_e} - 0.5 \ln \left[ \left( 2\pi \frac{m_e}{m_i} \right) \left( 1 + \frac{T_i}{T_e} \right) \frac{2}{(1 - \delta_e)^2} \right]$$

- The classical value of  $\gamma$  is  $\sim 6.9$  if  $T_e = T_i$ , and  $\delta_e = 0$
- Does the experimental sheath heat transmission match the classical value, or does an 'effective' value need to be used?
- Requires multiple diagnostics for measurement, good cross-calibration abilities

## Measurements necessary for determining sheath heat transmission

- Heat flux is given by:  $q = \gamma k_B T_e \Gamma$
- The necessary quantities are therefore:
  - a direct measurement of the heat flux to the surface,  $q$
  - the electron temperature,  $T_e$
  - the particle flux to the surface,  $\Gamma$
- The heat flux measured via IR camera readings
- Electron temperature and particle flux determined from Langmuir probe measurements
- Empirical value of  $\gamma$  given by ratio of IR and Langmuir probe measurements

# NATIONAL SPHERICAL TORUS EXPERIMENT U.S.A.



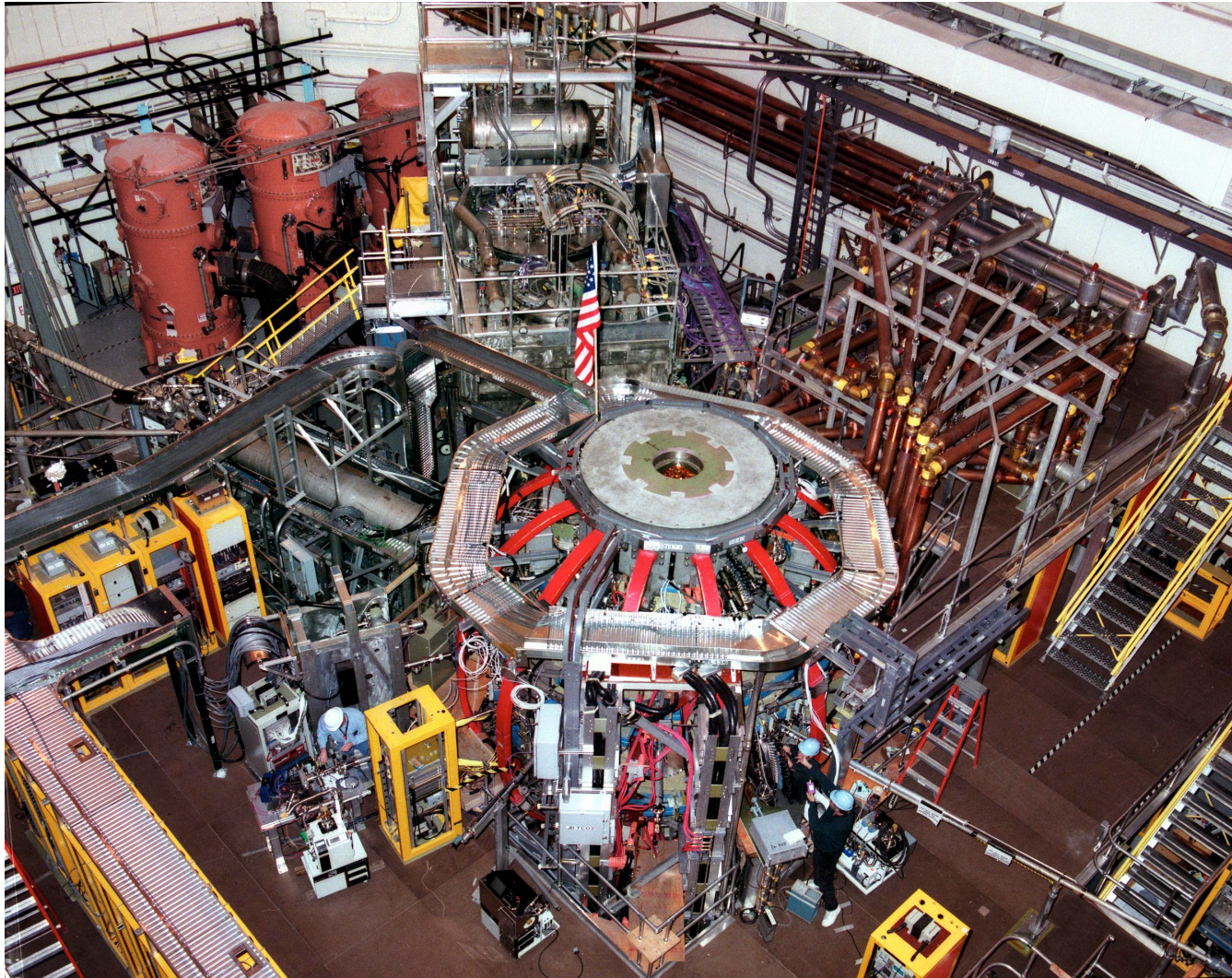
## Baseline Parameters

- Major radius  
 $\leq 85 \text{ cm}$
- Minor radius  
 $\leq 68 \text{ cm}$
- Plasma current  
**1 MA**
- Toroidal field  
**0.3–0.6 T**
- Heating and  
current drive  
**6–11 MW**
- Flat-top time  
**.5–1.6 s**





# The NSTX Facility

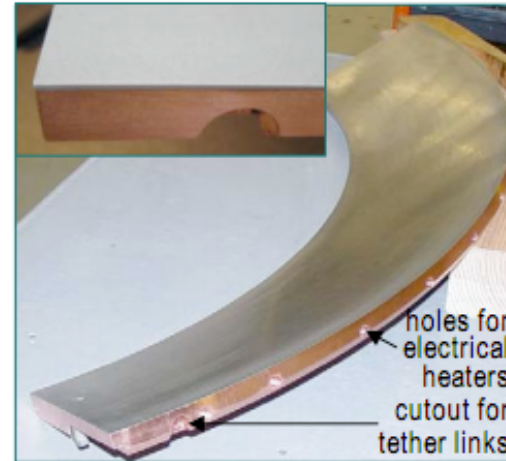




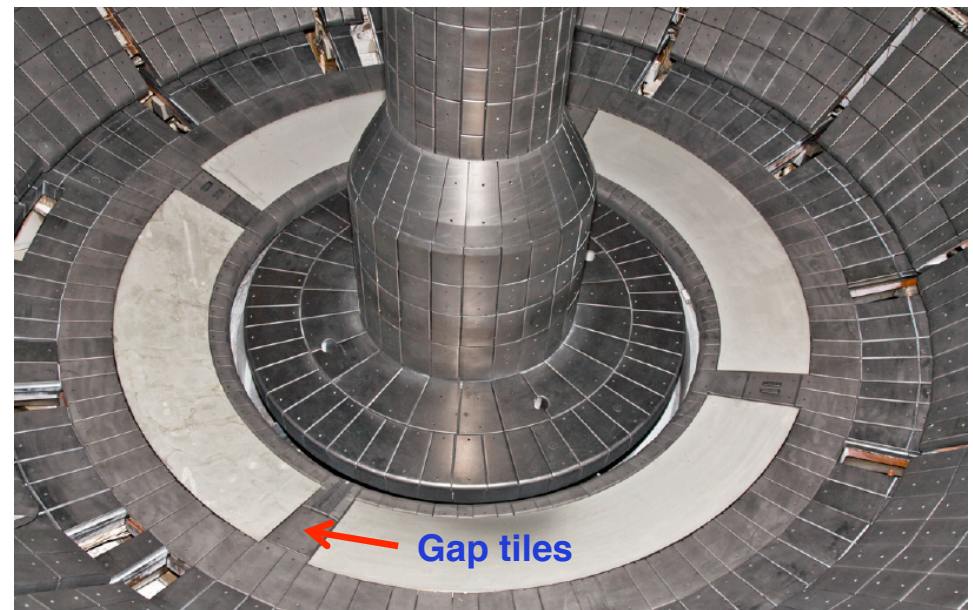
# The Liquid Lithium Divertor (LLD)

- Lithium research in NSTX has produced many favorable results linked to reductions in recycling<sup>1</sup>
- This motivated the installation of a substrate specifically designed for liquid lithium surfaces
- A suite of diagnostics was installed to support this research, making possible the measurements necessary to determine sheath heat transmission

<sup>1</sup>M. Ono. Fus. Eng. and Design. 85:6 (2010)

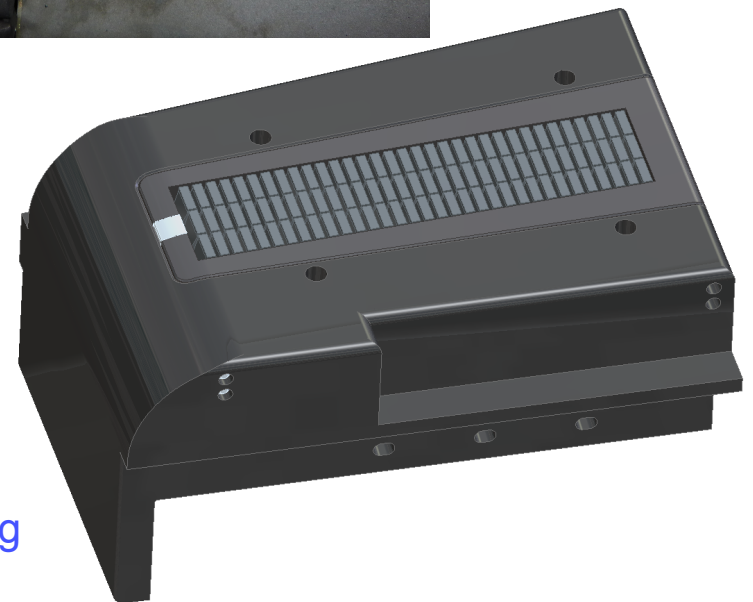
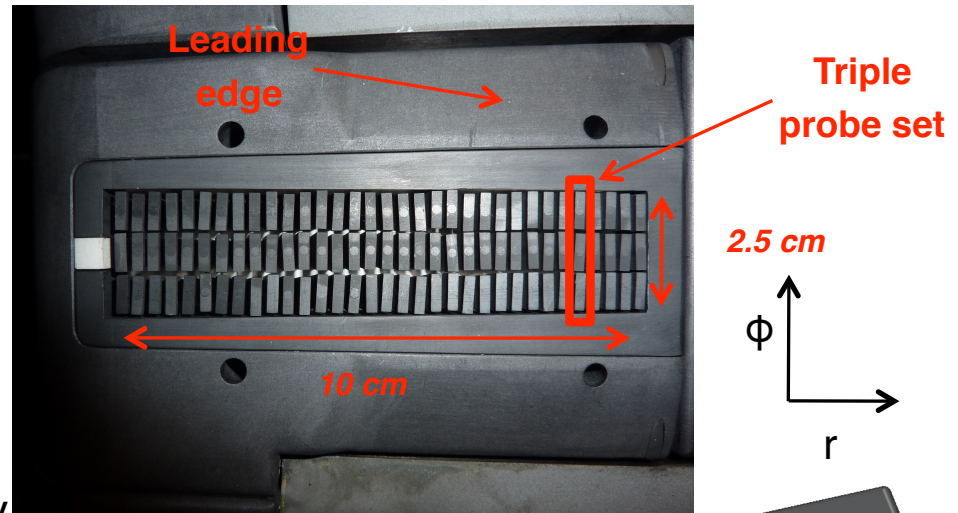


R. Nygren. Journal Nucl. Mat. (2011)



# Triple Langmuir probe array addresses edge diagnostic needs

- 99 individual electrodes arrayed as 33 rows of triple probes, providing density and temperature on a continuous basis
  - can also be operated as swept or SOL current probes
  - triple probes acquire at 250 kHz, swept at 500 Hz
- Probes based on MAST design utilizing a Macor cassette of closely spaced probes embedded in a carbon tile
  - tile mount with radial coverage of divertor
  - electronics provided by UIUC
  - described in RSI papers<sup>1</sup>
- Close spacing of probes provides better resolution in high-gradient (strike point) regions
  - each probe covers 3 mm radially, including spacing
  - probe heads are 2mm radial x 7mm toroidal rectangles

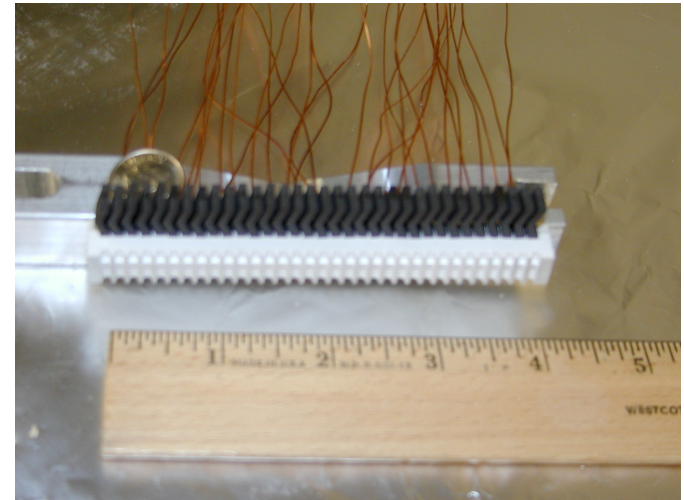
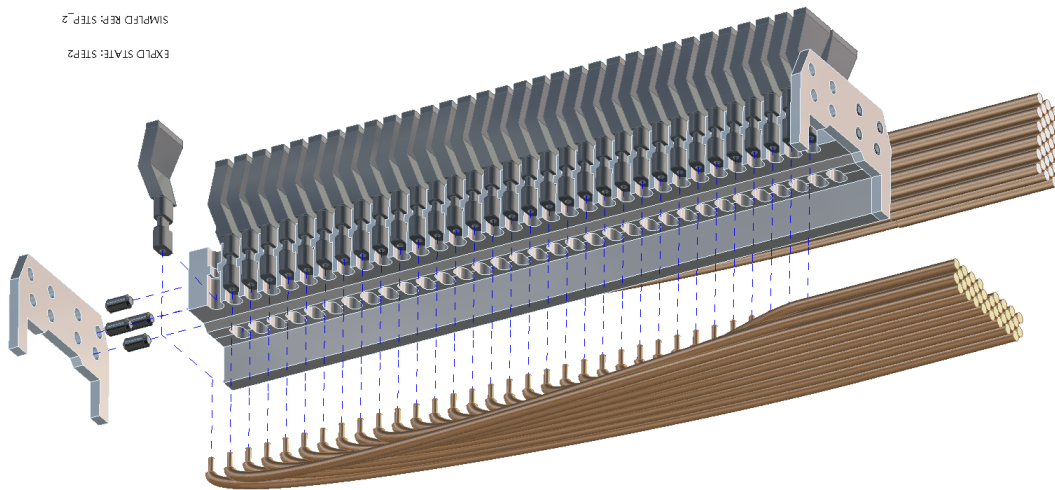


<sup>1</sup>Kallman; Jaworski, RSI (2010); **81**:10

## Heavy lithium depositions on divertor surfaces present materials challenge

- Graphite chosen as probe material due to ease of machineability and previous experience
  - previously installed (graphite) Langmuir probes showed no appreciable loss of signal with heavy lithium loading – but LLD operations necessitated large lithium depositions
- Elemental lithium is conductive, providing a possible path for shorting electrodes to each other or ground
- Lithium also reacts with carbon in the presence of oxygen (residually present in NSTX) to form lithium carbonate, or reacts with water to form LiOH, both insulators
  - beneficial in avoiding grounding and direct conduction, but can provide barrier for incident electrons and ions
- Strike point ablation can remove evaporated lithium, but large integral effect of continuous loading depositions was unknown
  - comparison of signal strength throughout run year showed that signal magnitude did not decrease appreciably

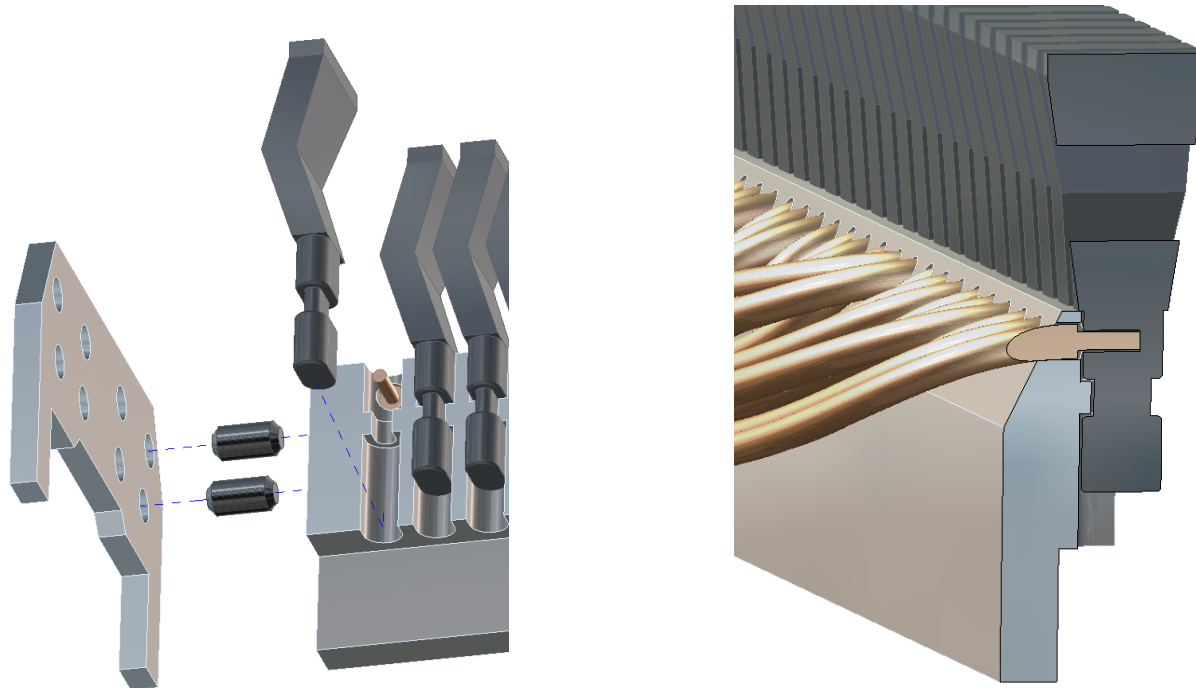
## Cassette design allows for ease of probe mounting and includes channels for wire transport



- Boron nitride cassette features interlocking segments that allow for individual probe seating and securement
  - screwless design reduces mechanical stresses on the probes
- Wiring channels allow for the wires from each group of probes to exit independently
  - wires exit on sides of edge probes and through base of central probes
  - graphite cement used to attach wires to probes; near-identical thermal properties reduce risk of loss of contact due to material expansion
  - Fortafix adhesive used to provide strain relief for wires exiting cassette



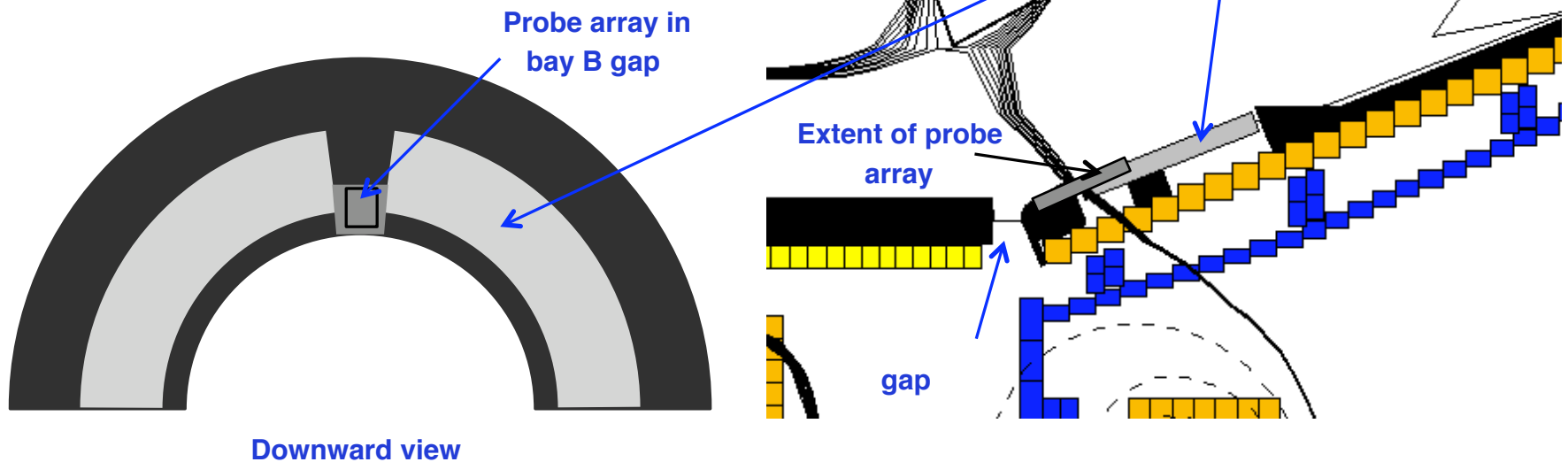
## Probe design includes features to protect underlying surfaces and uses novel materials to facilitate assembly



- Boron nitride offers greater lithium compatibility than original macor design
- Probes are shaped so as to minimize direct exposure of BN to plasma or lithium
  - probe bend prevents direct line of sight for lithium or plasma down to cassette
  - probe widening at top allows for smaller gaps and greater shielding of surfaces below
  - electrode material is vacuum compatible HK-6 Tokai graphite

## Probe array is located near LLD to provide local measurements

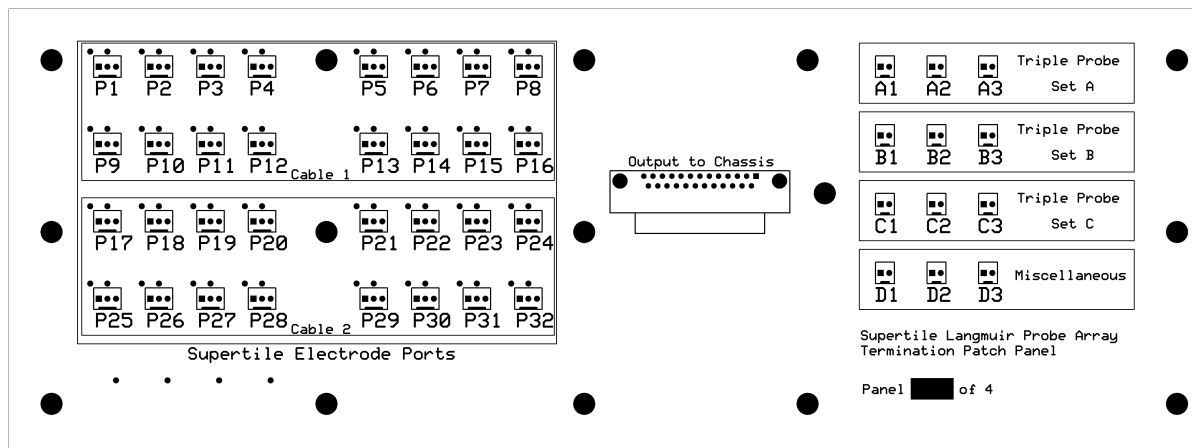
- Probe array begins just outboard of lower divertor gap and extends over roughly  $\frac{1}{3}$  of LLD radially
- Provides local measurements for plasma incident on both carbon and lithium PFCs
- Probe array engineering requirements lead to surface geometry different from surrounding structures





# Langmuir probe data acquisition

- Each electrode is attached to a single insulated copper conductor by means of graphite cement
- Wires run to a vertical 'organ pipe' where they exit the vessel through a vacuum-feedthru
- Wire bundles run to a diagnostic rack where they enter a patch panel, allowing for fast reassignment of electrodes to sensor sets
- Initial package from UIUC provides for 8 triple-probe sets and 4 swept probes
- Triple probes biased through isolated 48V DC power supply
  - acquire floating potential, as well as current and voltage on one of the electrodes
- Single probes swept to -40/+28V using a 200 W Kepco BOP with a sinusoidal waveform
  - acquire current and voltage on electrode only

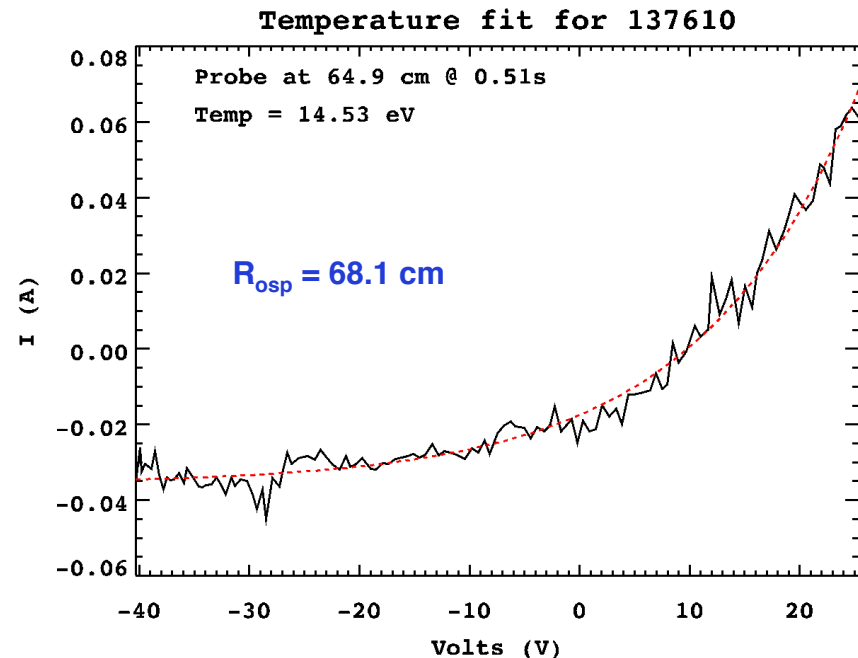


# Swept probe signal interpretation

- Electron current flow to probe tip is given as a Boltzmann distribution dependent on bias voltage, while ions are free-streaming at the sound speed
- Four traces (two periods) are averaged to account for fluctuations (250 Hz effective)
- A 3-parameter least-squares fit is performed to determine  $I_{sat}$ ,  $T_e$ , and  $V_f$
- Several fits are performed on each trace, varying the cutoff voltage to stay in the exponential region
  - resulting temperature is normalized by the chi-square of each fit range to produce error bars
- Once a temperature is fit, density can be obtained from the saturation current equation

$$I_{probe} = I_{sat} \left[ 1 - \exp \left[ \frac{e(V_{bias} - V_f)}{kT_e} \right] \right]$$

$$I_{sat} = \frac{1}{2} en_0 c_s A_{probe} \quad c_s = \sqrt{\frac{k(T_e + T_i)}{m_i}}$$

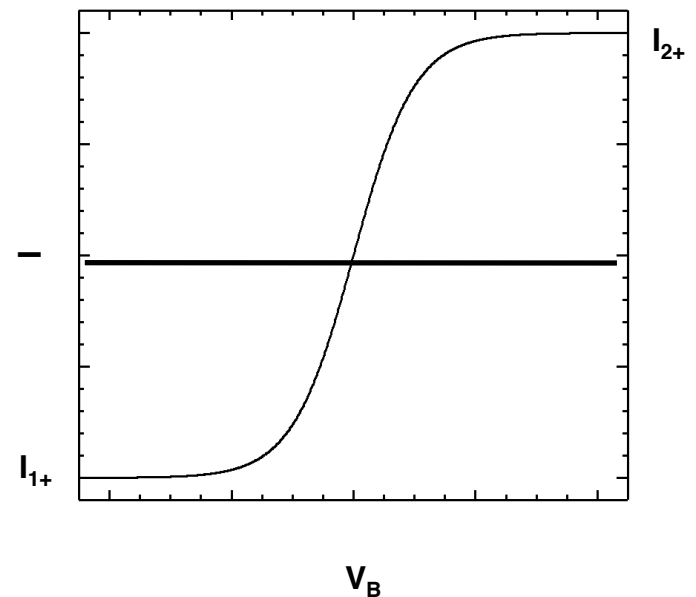
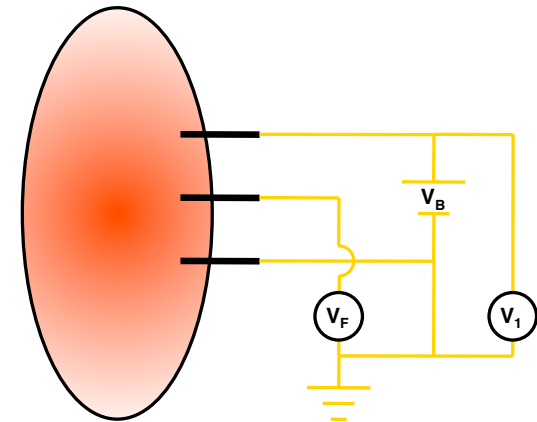


# Triple probe interpretation

- Three individual electrodes can be utilized to give instantaneous temperature readings
  - two probe tips offset by a DC voltage (48 V) referenced to a third tip operating at floating potential
  - the two biased tips are offset around the floating potential so that there is no net current flowing (i.e. electron current = ion current)
- Using the same assumptions about the electron distribution as the swept probes, setting  $I_1 = -I_2$  yields an equation for the electron temperature

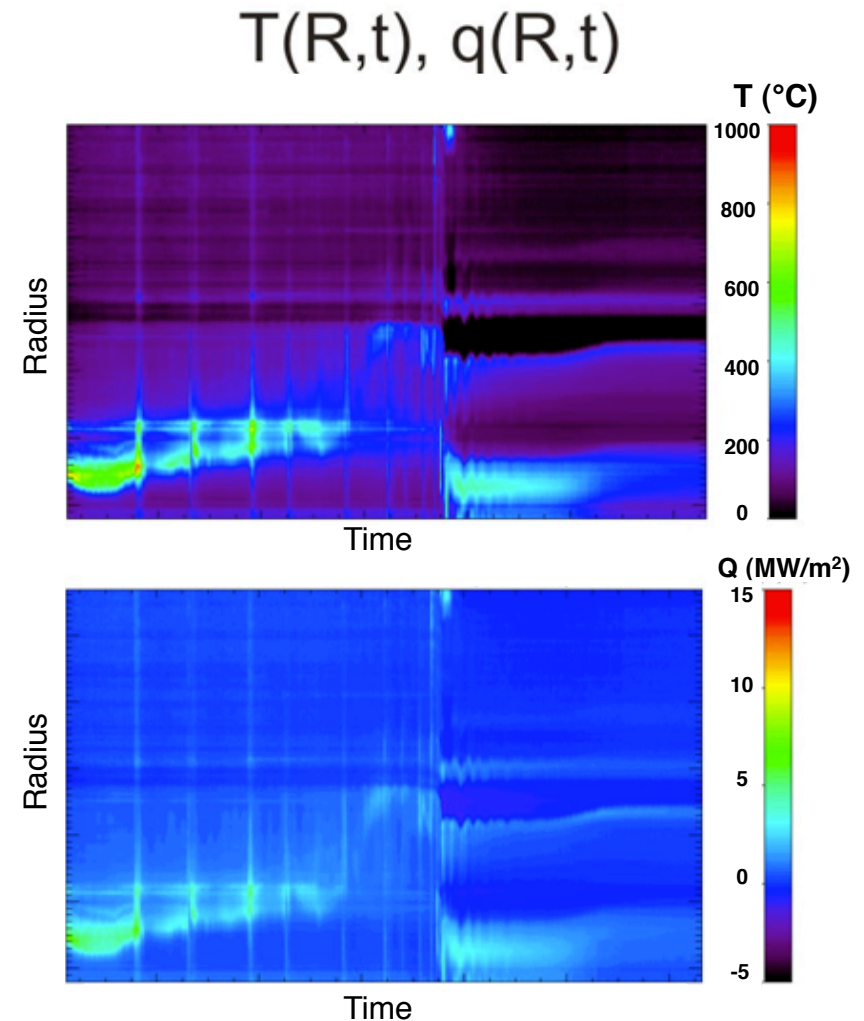
$$T_e \cong \left( \frac{e}{k} \right) \frac{(V_1 - V_f)}{\ln 2}$$

- If bias voltage is sufficiently above electron temperature, measured current is generally ion saturation
- Density can be obtained in the same manner as swept probes
- Provides temperature and density measurements at 250 kHz, but median filtering smoothes noise and reduces effective frequency to ~400 Hz



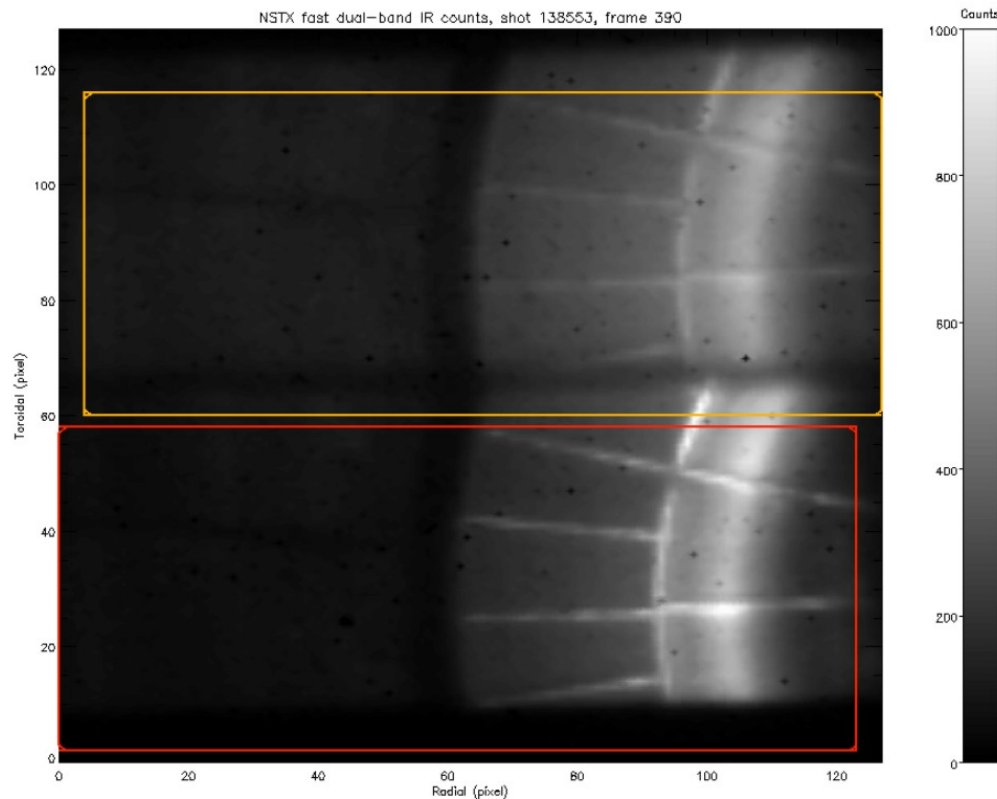
# Dual-band IR camera compensates for emissivity changes in the lithium surface

- ORNL collaboration to take fast camera IR measurements at 2 wavelengths at 1.6 kHz
  - 4-6  $\mu\text{m}$  and 7-10  $\mu\text{m}$
  - ratio of signals cancels out emissivity-dependence in radiated power
- In-situ calibration using heated LLD allows for precise temperature measurements
- 2D heat-conduction code THEODOR determines incident heat flux necessary for observed temperature change



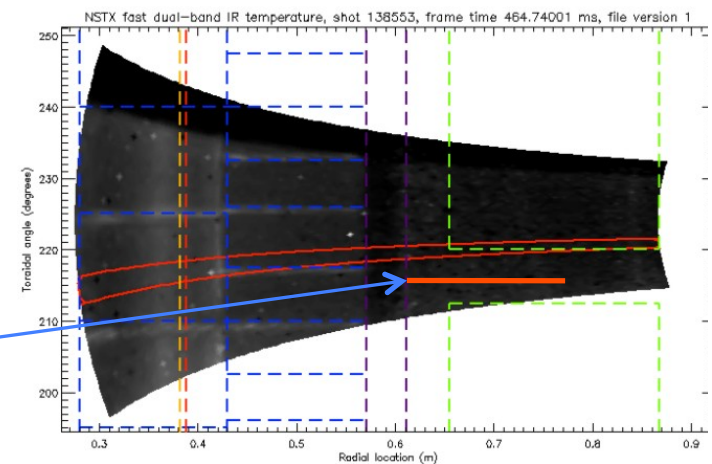
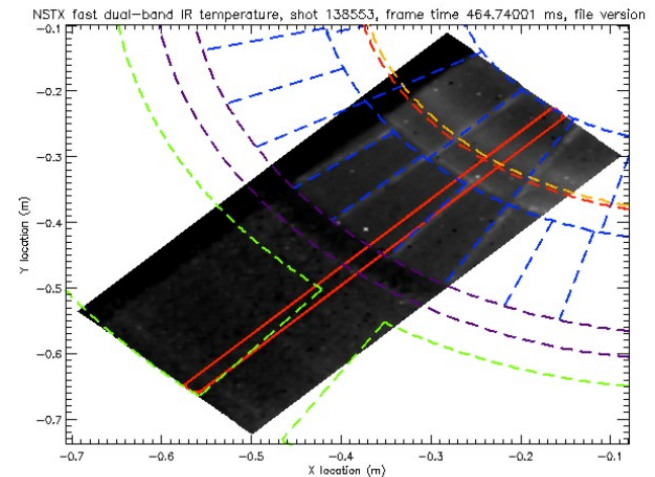
# Raw pixel data is mapped to NSTX geometry

## Raw dual-band data



*Chord used for present study*

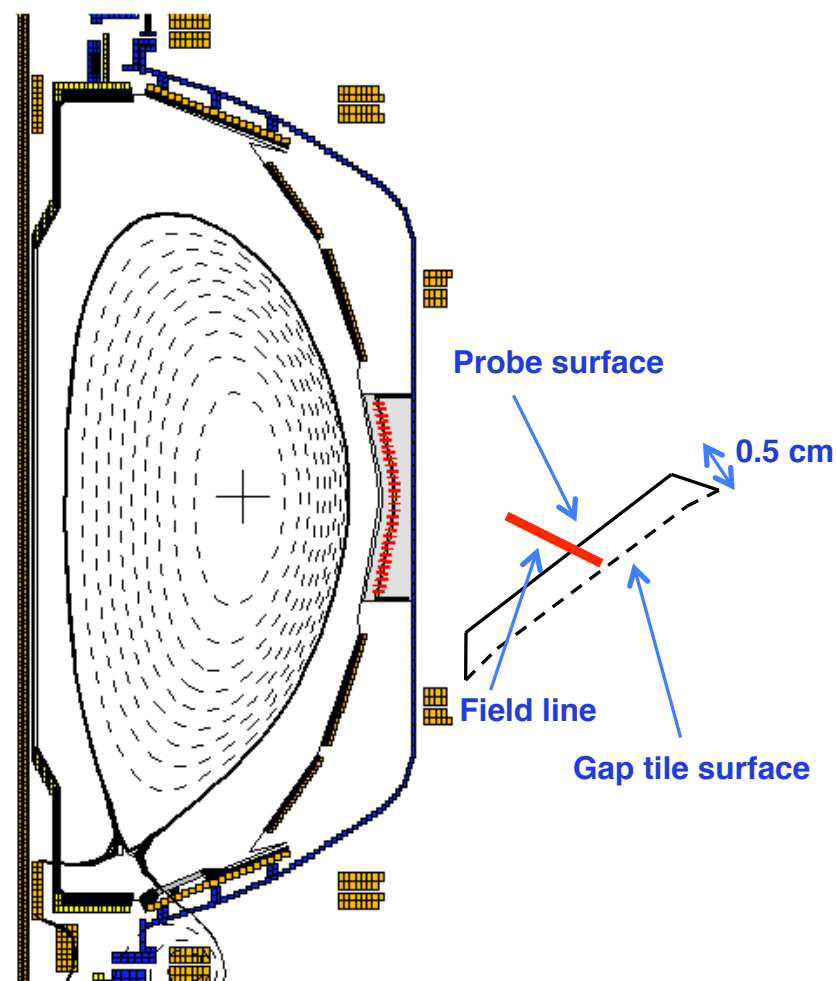
## Aligned, calibrated



## Different toroidal locations, surface geometries necessitate mapping of flux coordinates

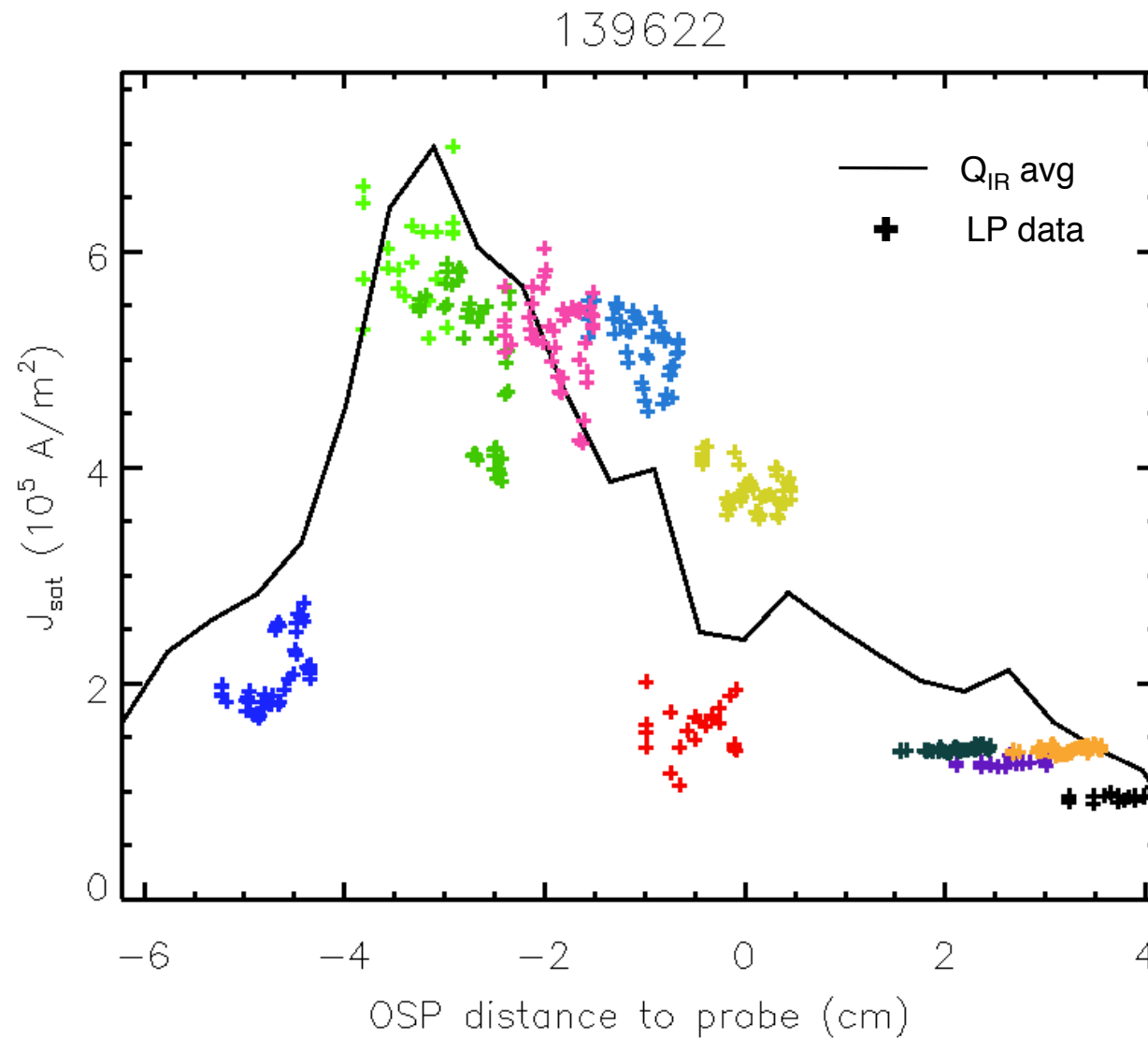
- The probes are approximately  $180^\circ$  away from the IR viewing region and have a unique surface geometry
- FARO arm measurements are utilized to provide an accurate topographical map of the surface at both toroidal positions
- The use of magnetic equilibrium reconstructions in the EFIT code (Grad-Shafranov solver) allows for the determination of poloidal flux at any point in the device
- Using the assumption of toroidal axisymmetry, two positions on the same flux surface experience identical plasma conditions
- Using a fixed probe radius, the flux at each camera point is compared to that at the probe at each time point and the closest match is obtained
  - generally changes effective radius by 2-3 cm, which is significant in high-gradient regions

139625 @ 700 ms

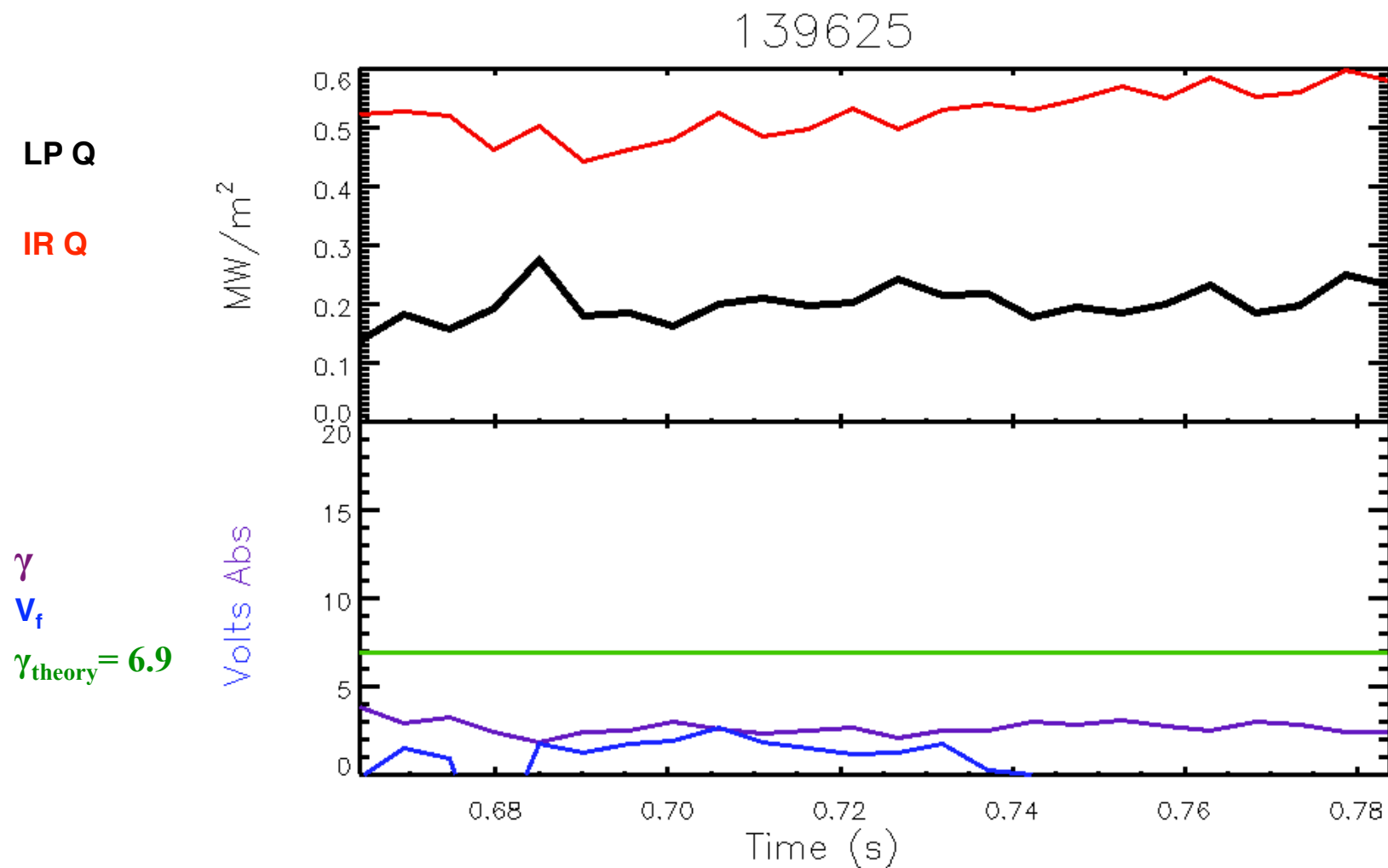




# Heat flux and Langmuir probe radial profiles show good qualitative agreement



## Sample dataset from probe shows heat flux evolution agreement with IR

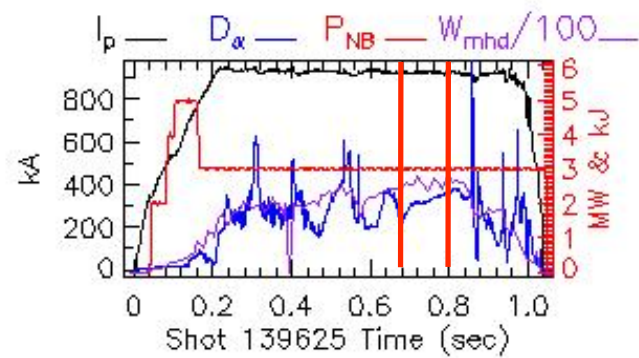
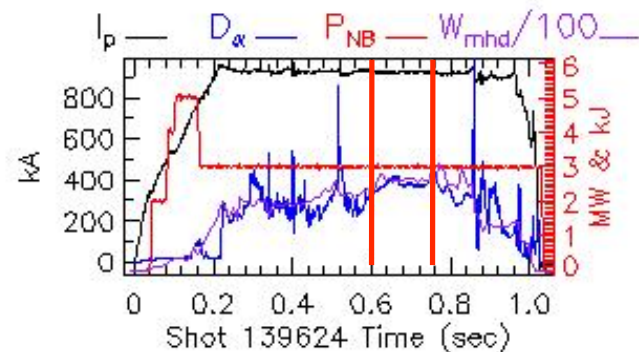
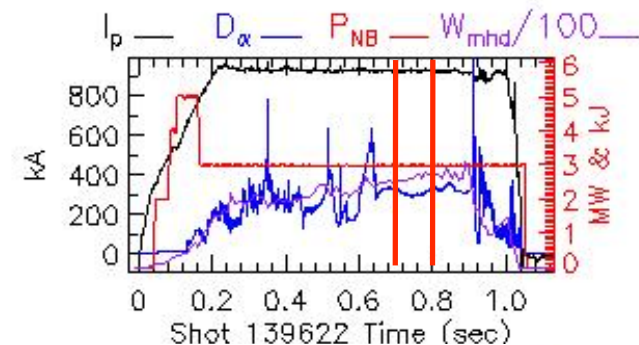


## Data selection criteria

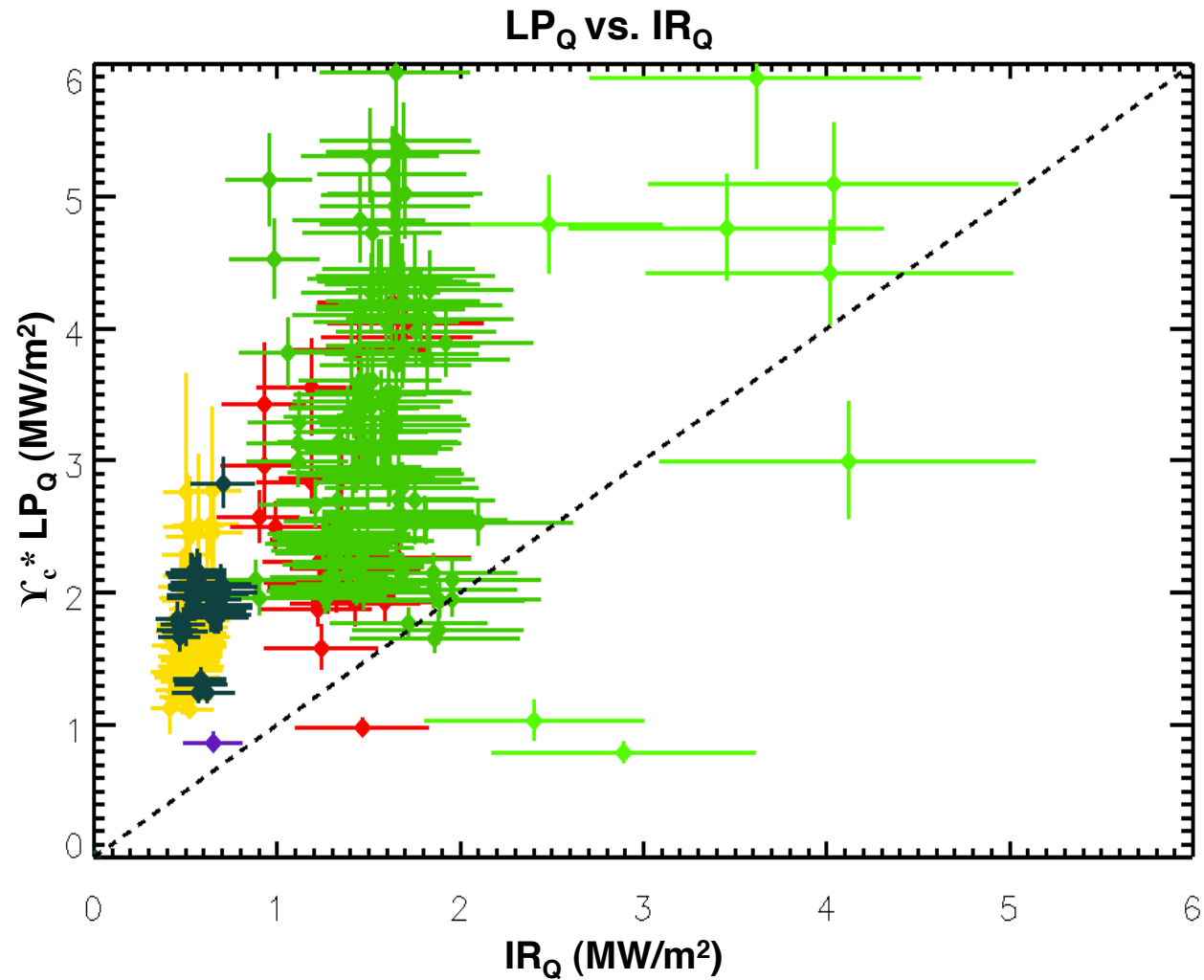
- Discharges must have the strike point position on the probe array to ensure sufficient signal
- Quiescent periods chosen to minimize high transient fluxes due to ELMs
  - generally restricted time windows to 100-200 ms
- IR camera data not available for all shots; sufficient signal desired for optimal comparison
- 2D heat flux model for IR data uses thin lithium film on bulk thermal conductor
  - works better for carbon tiles, motivates chord selection discussed above
- Only looked at signals where floating potential was close to ground
  - floating potential crosses zero at strike point, so this ensures high signal in the relevant region
  - mitigates effects of non-zero floating potential on  $\gamma$

## Case study: combine 3 similar discharges for good statistics

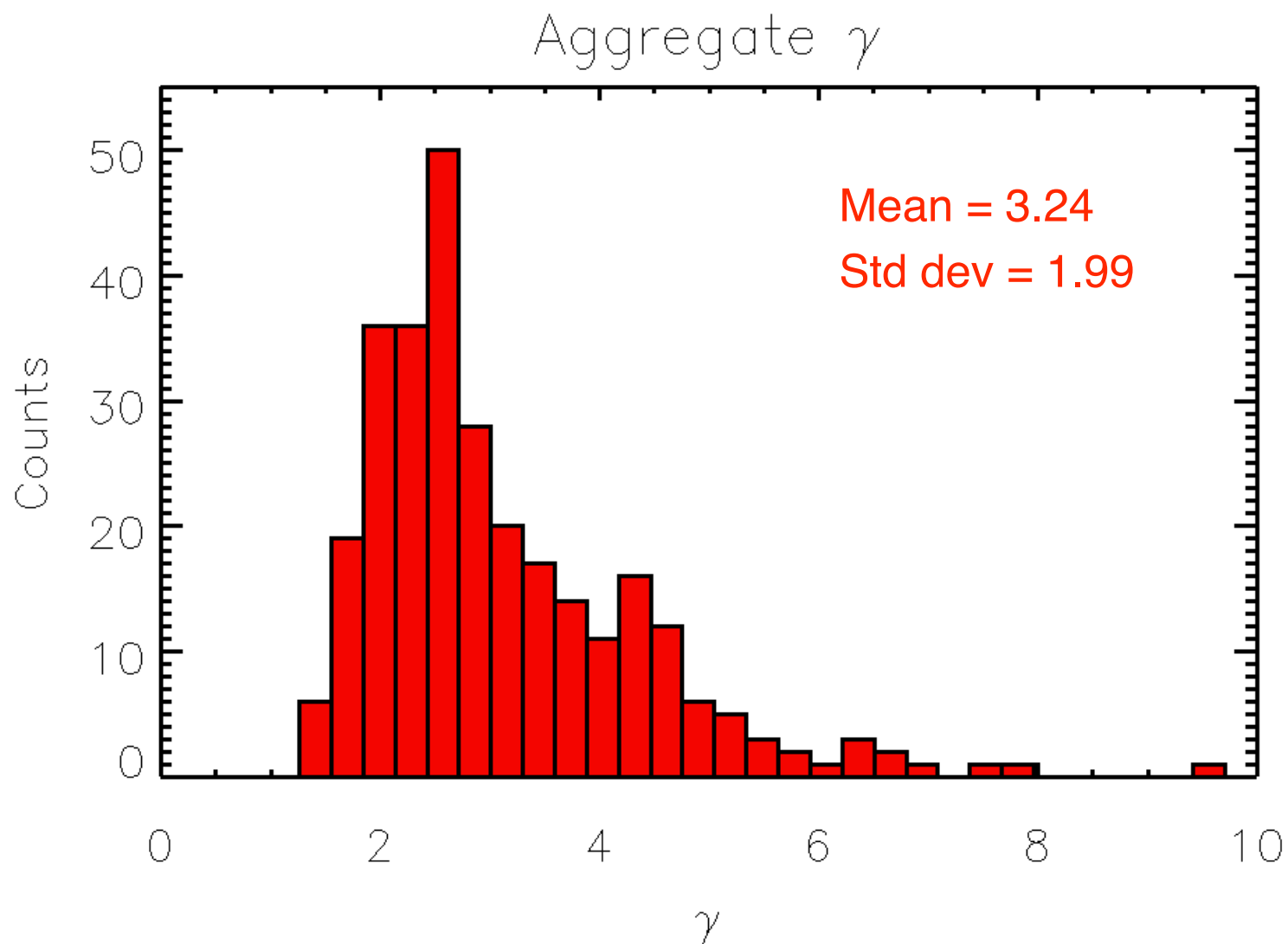
- Selection criteria above limit the number of useable data points for each discharge
- 100-150ms time window limits to approximately 25 points per single probe and 50 points per triple probe
- Floating potential criteria further eliminates data points, sometimes excluding probes altogether
- Signals excluded if low strength or if probes exhibited intermittency/shorting issues
- All 4 single probes kept, but only 4 of 7 triple probe signals included
- Results from three discharges combined to give best possible statistics with remaining data with good confidence interval



## Aggregate data from several shots show deviation from classical predictions

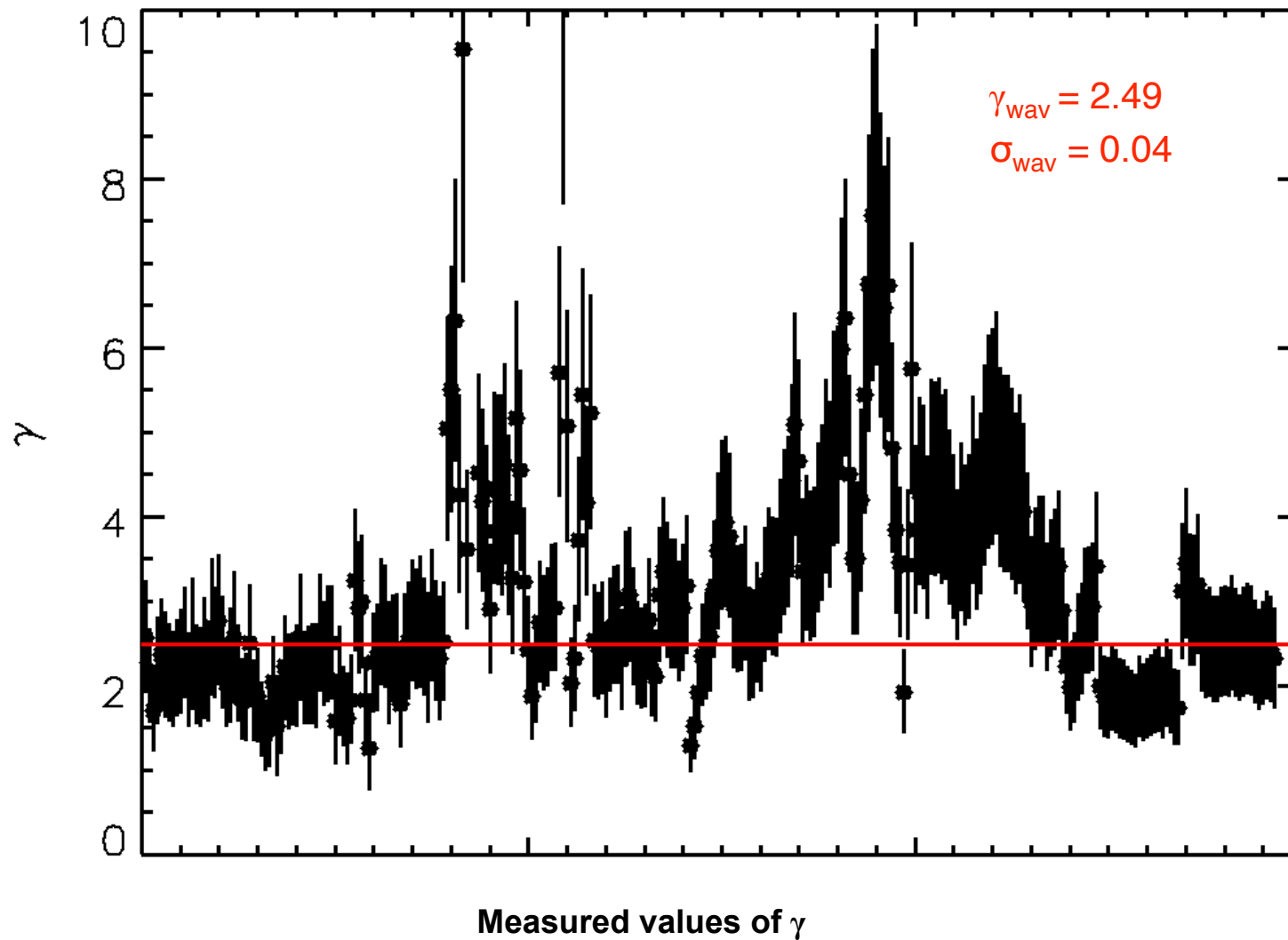


## Distribution of measured values of $\gamma$ inadequately described by Gaussian statistics





## Weighted average method shows that $\gamma \sim$ one third of classical value

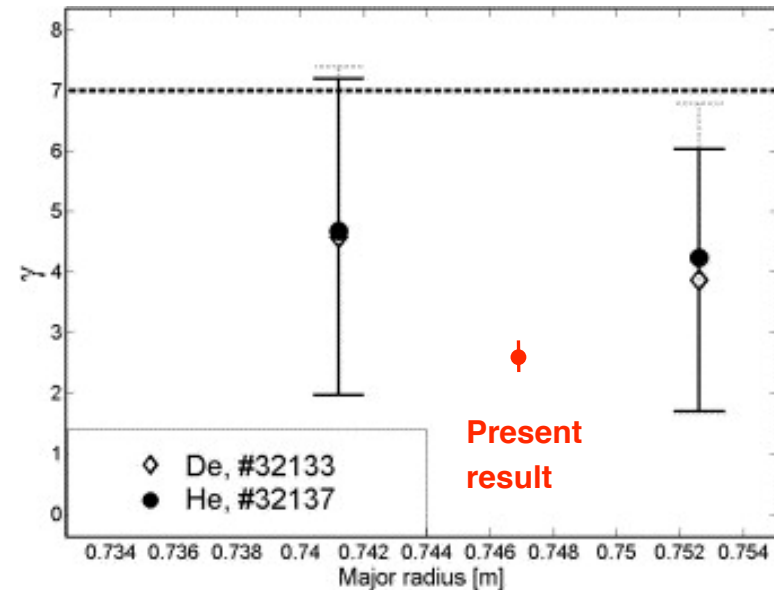


## Conclusions and discussion (I)

- New diagnostic capabilities allow for precision edge measurements – an area traditionally neglected
- High density Langmuir probe array used in conjunction with IR camera data to successfully calculate sheath heat transmission for the first time in NSTX
- Good statistics obtained by overlaying multiple shots with similar profiles
- Sheath heat transmission coefficient is measured to be  $2.49 \pm 0.04$
- This value is about one third of the classical value, so what accounts for the discrepancy?

## Discussion (II)

- Previous measurements of  $\gamma$  have trended lower as well
- Lithium can alter basic picture
  - changes to temperature ratio
  - impurity accumulation/suppression
  - collisionality
- Conception of  $\gamma$  comes from fluid picture of plasma edge in thermodynamic equilibrium
  - does lithium alter the fundamental physics by changing collisionality?



J. Marki et al. Journal Nuc. Mat. 363-365 (2007)

$$\gamma = 2.5 \frac{T_i}{T_e} + \frac{2}{1 - \delta_e} - 0.5 \ln \left[ \left( 2\pi \frac{m_e}{m_i} \right) \left( 1 + \frac{T_i}{T_e} \right) \frac{2}{(1 - \delta_e)^2} \right]$$

## Future research ideas

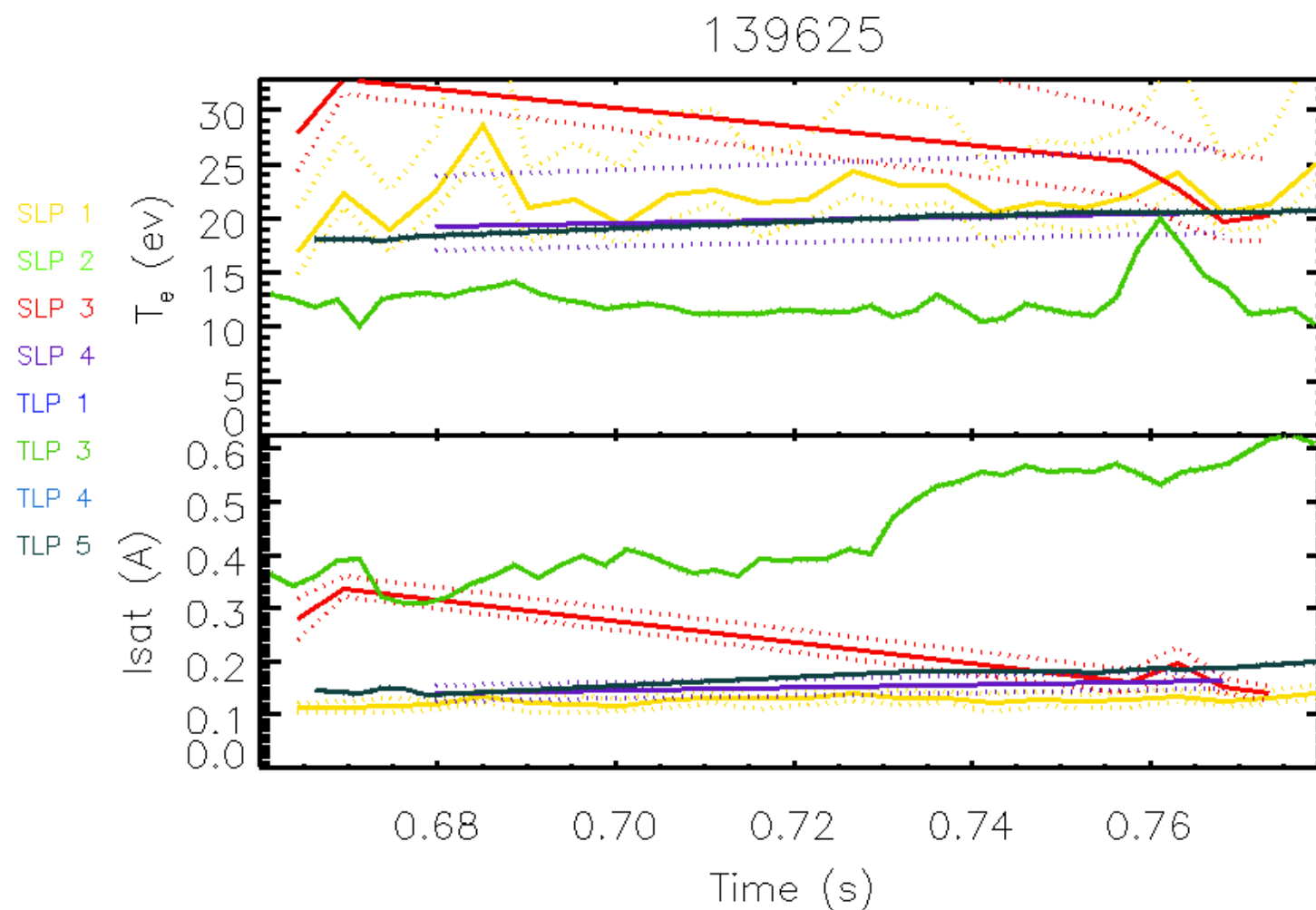
- Link edge heat transport regimes to lithium loading; explicit study with different amounts/quality of lithium surfaces
- Build/install diagnostics to measure edge ion temperature and impurity levels directly
- Test fluid picture with edge modeling; match conditions at probes to mid-plane diagnostics
- Develop and employ kinetic modeling tools with empirical input if fluid picture proves inaccurate

## Acknowledgements

- Bob Kaita, my advisor
- Henry Kugel, LLD project coordinator
- Mike Jaworski for all his work with me on the probe array (my junior advisor)
- Adam McLean for providing the IR camera data
- Tyler Abrams for his collaboration on various projects and measurements of probe resistances
- John Timberlake for his lithium expertise
- Vijay Surla for helping to maintain the probe array
- Larry Guttadora for assisting in the design of the array
- Doug Westover, Scott Gifford, Buddy Kearns, and the other technicians that helped install the array
- Jess Baumgaertel, Craig Jacobson, and Luc Peterson for discussions regarding the data and for moral support
- Viewers like you



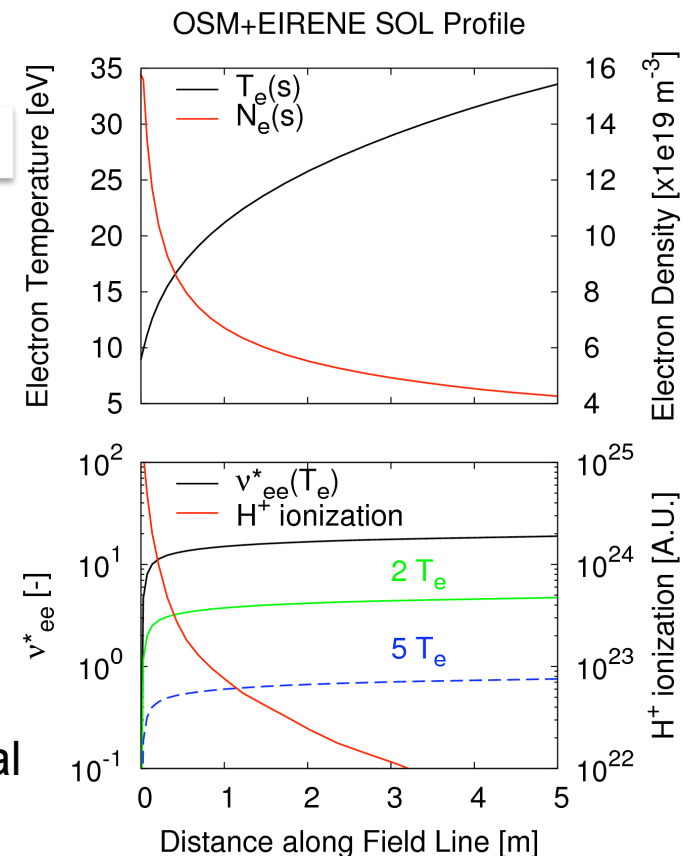
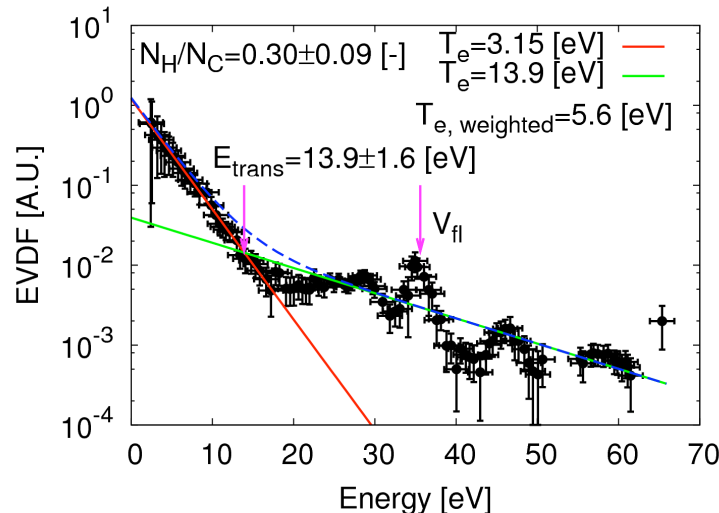
## Raw temperature and saturation currents used



# Mike Jaworski's measurements of kinetic effects in NSTX SOL

.Langmuir probes with non-local interpretation infer non-Maxwellian features in EVDF

•*Courtesy of M. Jaworski*



.Preliminary fluid code run indicates SOL is at marginal collisionality even for **High-recycling PFC**

- OEDGE = Onion-Skin-Method(OSM) + EIRENE + DIVIMP Edge code
- OSM assumes single fluid equations

• Collisionality calculated based on distance,  $s$ , from PFC along field line:  $\frac{s}{\lambda_{ee}} \approx 10^{-16} \frac{n_e \cdot s}{T_e^2}$