

Confinement and stability discoveries from high-beta spherical tori

Jonathan Menard (PPPL)

Program Director for NSTX Upgrade

*With contributions from the NSTX, MAST
Pegasus, and other ST research teams*

Norman Rostoker Memorial Symposium

The Fairmont Newport Beach Hotel

August 24-25, 2015



Outline

- **ST overview**
- **Global MHD Stability**
- **Energy Confinement**
- **Energetic Particles**
- **ST Upgrade Status**
- **Summary**

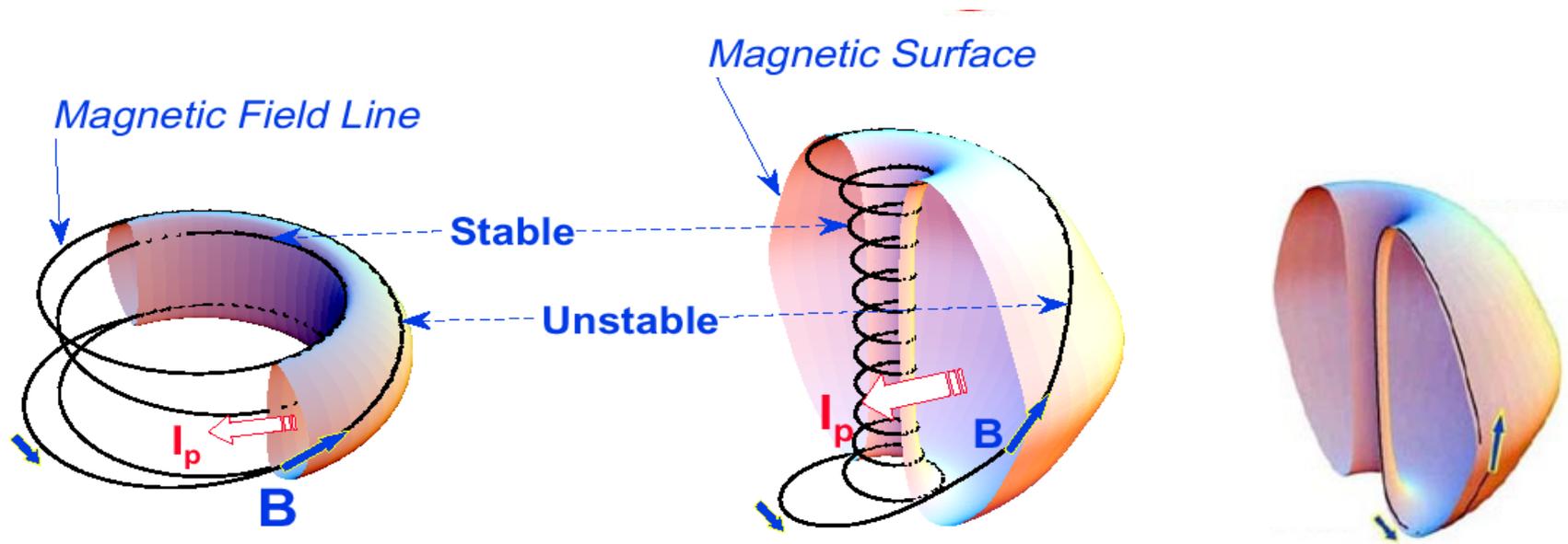
“Spherical” tokamak (ST) has aspect ratio $A < 2$

Aspect Ratio $A = R/a$

Elongation $\kappa = b/a$

Toroidal Beta $\beta_T = \langle p \rangle / (B_{T0}^2 / 2\mu_0)$

- Natural elongation makes its spherical appearance
- Favorable average curvature improves stability at high beta



Tokamak

$A \sim 3-4,$
 $\kappa = 1.5-2,$
 $q_{95} = 3-4,$
 $\beta_T = 3-10\%$

ST

$A \sim 1.2-2,$
 $\kappa = 2-3,$
 $q_{95} = 6-20,$
 $\beta_T = 10-40\%$

FRC

$A \sim 1,$
 $\kappa = 0.5-10,$
 $q \sim 0,$
 $\beta \sim 100\%$

High β_T enables compact Fusion Nuclear Science Facility (FNSF) with high neutron wall loading

$$P_{\text{fusion}} \propto \langle p \rangle^2 \times \text{Vol}$$

$$P_{\text{fusion}} \propto \beta_T^2 \times \text{Vol}$$

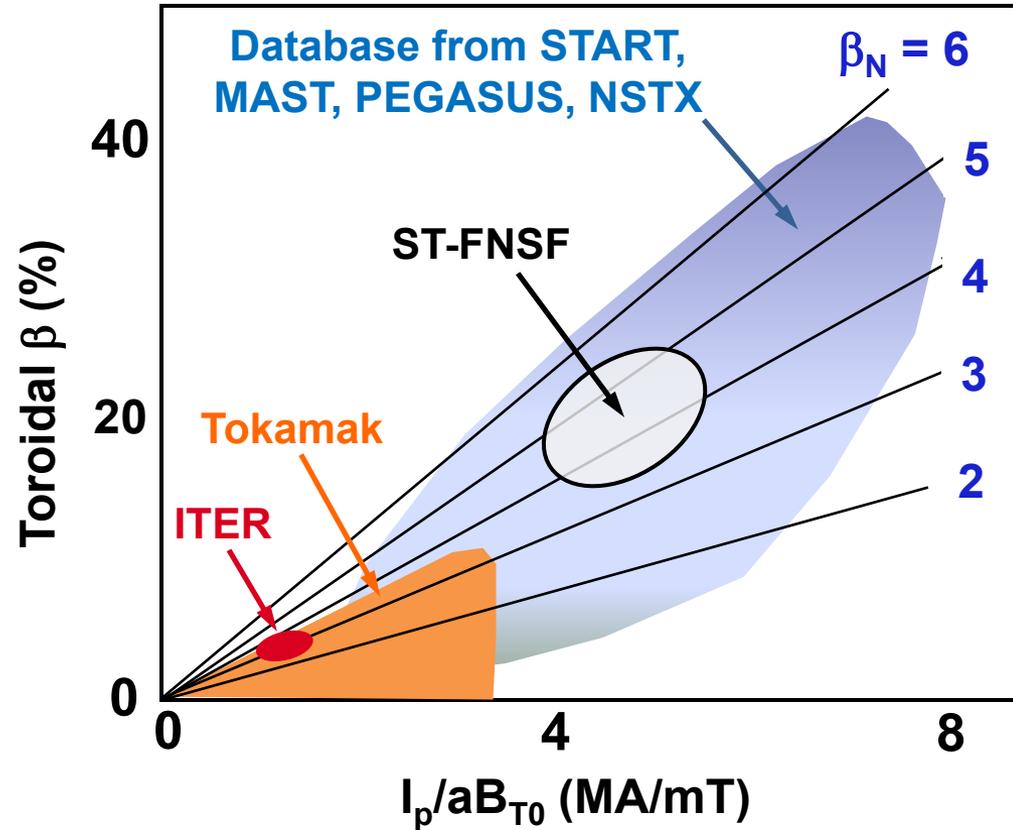
↑
↑

Physics limited
 \$

 Investment & engineering limited

$$W_n \propto P_{\text{fusion}} / \text{Area}$$

$$W_n \propto \beta_T^2 B_{T0}^4 a \quad (\text{not strongly size dependent})$$

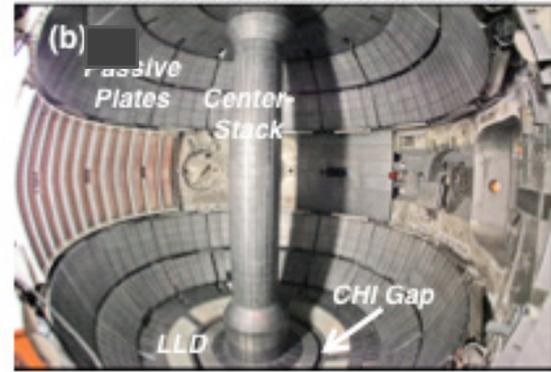
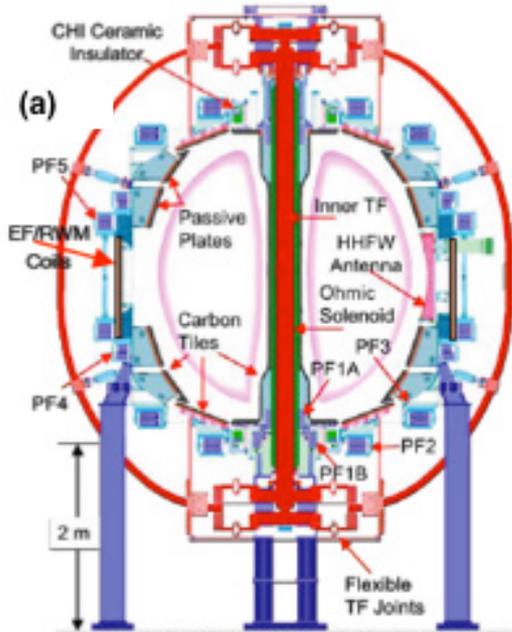


$W_n \sim 1\text{-}2 \text{ MW/m}^2$ with $R \sim 1\text{-}2\text{m}$ FNSF feasible!

MA-Class ST Research Started ~2000

Complementary Physics Capabilities of NSTX and MAST

NSTX



Complementary Capabilities

Passive Plates
Helicity Injection
Fast wave heating
1 x 6 RWM Coils

Large divertor volume
Merging/Compression
Electron Cyclotron
2 x 12 ELM coils

Similar Capabilities

NSTX

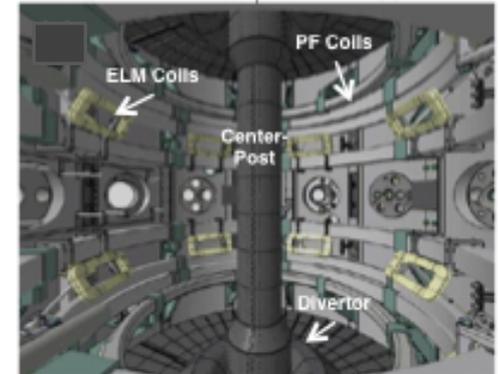
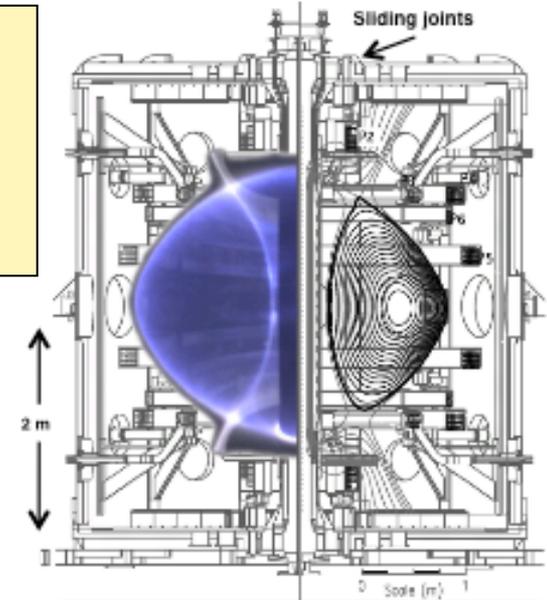
$R = 85 \text{ cm}$
 $A \geq 1.3$
 $\kappa = 1.7 - 3.0$
 $B_T = 5.5 \text{ kG}$
 $I_p \leq 1.5 \text{ MA}$
 $V_p \leq 14 \text{ m}^3$
 $P_{\text{NBI}} = 7.4 \text{ MW}$

MAST

$R = 80 \text{ cm}$
 $A \geq 1.3$
 $\kappa = 1.7 - 2.5$
 $B_T \sim 5.0 \text{ kG}$
 $I_p \leq 1.5 \text{ MA}$
 $V_p \leq 10 \text{ m}^3$
 $P_{\text{NBI}} = 4.0 \text{ MW}$

- Comprehensive diagnostics
- Physics integration
- Scenario development

MAST

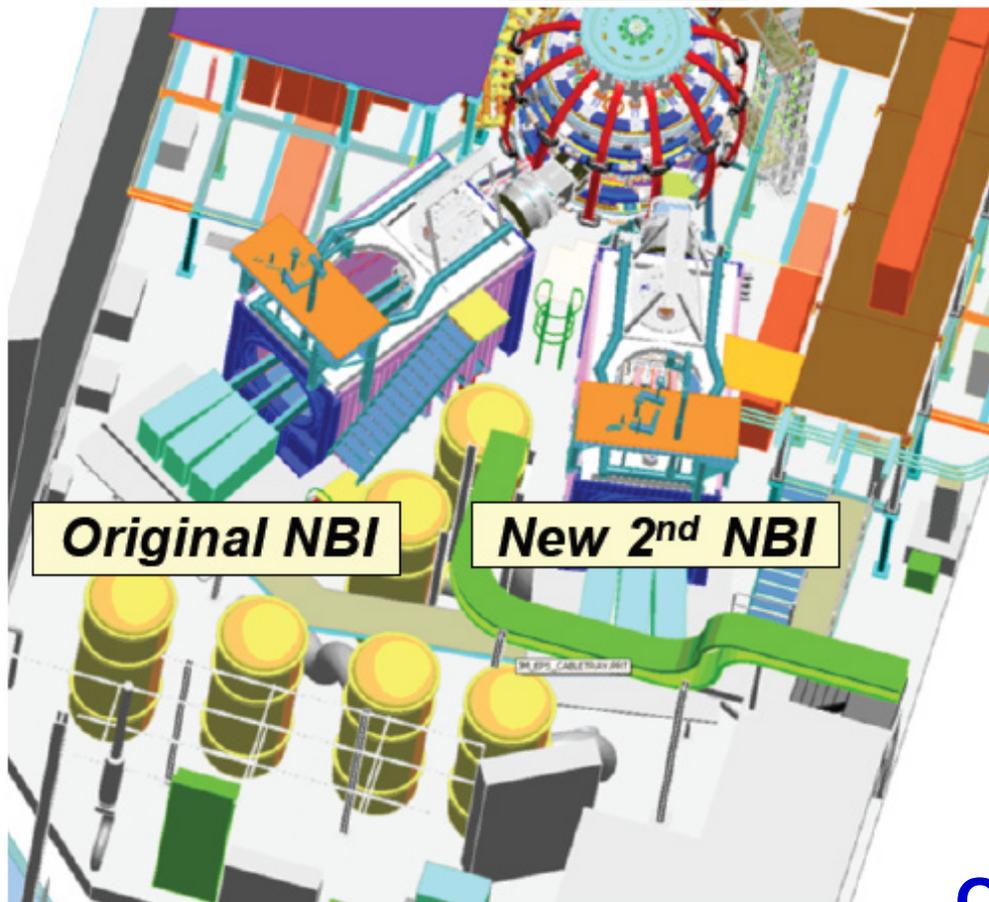


M. Ono, IAEA 2000, NF 2001

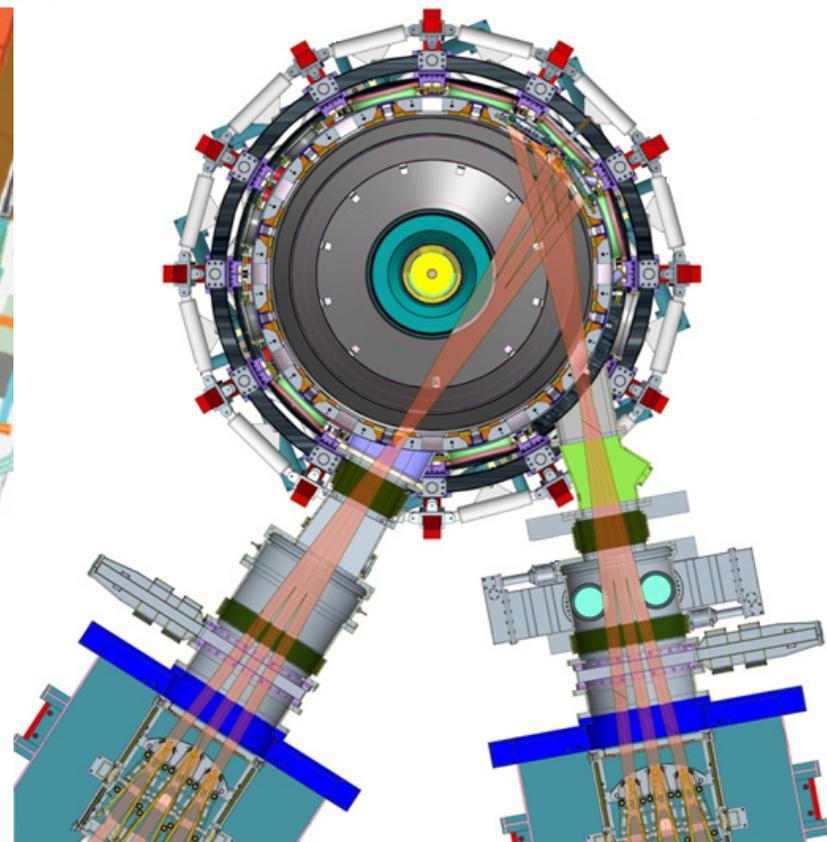
A. Sykes, IAEA 2000, NF 2001

Mega-ampere-class STs rely heavily on co-injected neutral beams for heating, current drive

NSTX



NSTX-U



Original NBI

($R_{\text{TAN}} = 50, 60, 70\text{cm}$)
5MW, 5s, 80keV

New 2nd NBI

($R_{\text{TAN}} = 110, 120, 130\text{cm}$)
5MW, 5s, 80keV

New physics accessed in ST → enhanced understanding of toroidal confinement physics

- Lower A → higher β , strong shaping
- Higher β →
 - Electromagnetic effects in turbulence
 - More potential drive for fast-ion-driven instabilities
 - Simulate fast-ion transport of ITER / burning plasmas
 - Over-dense plasmas: RF heating, current drive
- Low- A / high- β broadly impact transport, stability:
 - Higher fraction of trapped particles (low A)
 - Increased normalized orbit size (high β)
 - Increase flow shear (due to low B , low A)
- Compact geometry (small R) → higher power and particle fluxes relevant to ITER, reactors

Outline

- ST overview
- **Global MHD Stability**
- Energy Confinement
- Energetic Particles
- ST Upgrade Status
- Summary

Simulations find $\langle \beta_N \rangle$ is more aspect ratio invariant than β_N – both with & without wall stabilization

Para / diamagnetic effects and B_p / B ratio important at low-A

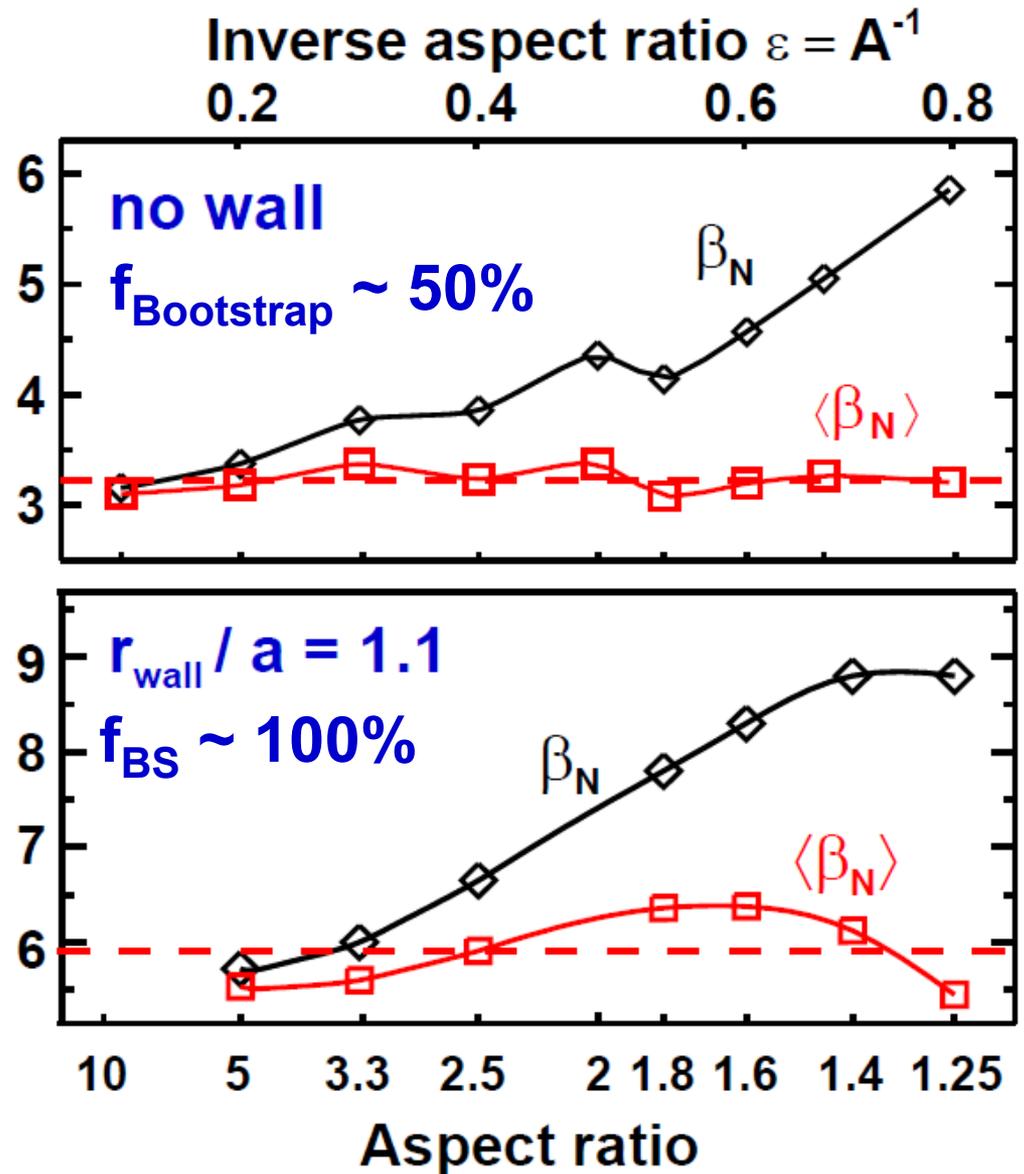
$$\beta_T \equiv 2\mu_0 \langle p \rangle / B_{T0}^2$$

$$\langle \beta \rangle \equiv 2\mu_0 \langle p \rangle / \langle B^2 \rangle$$

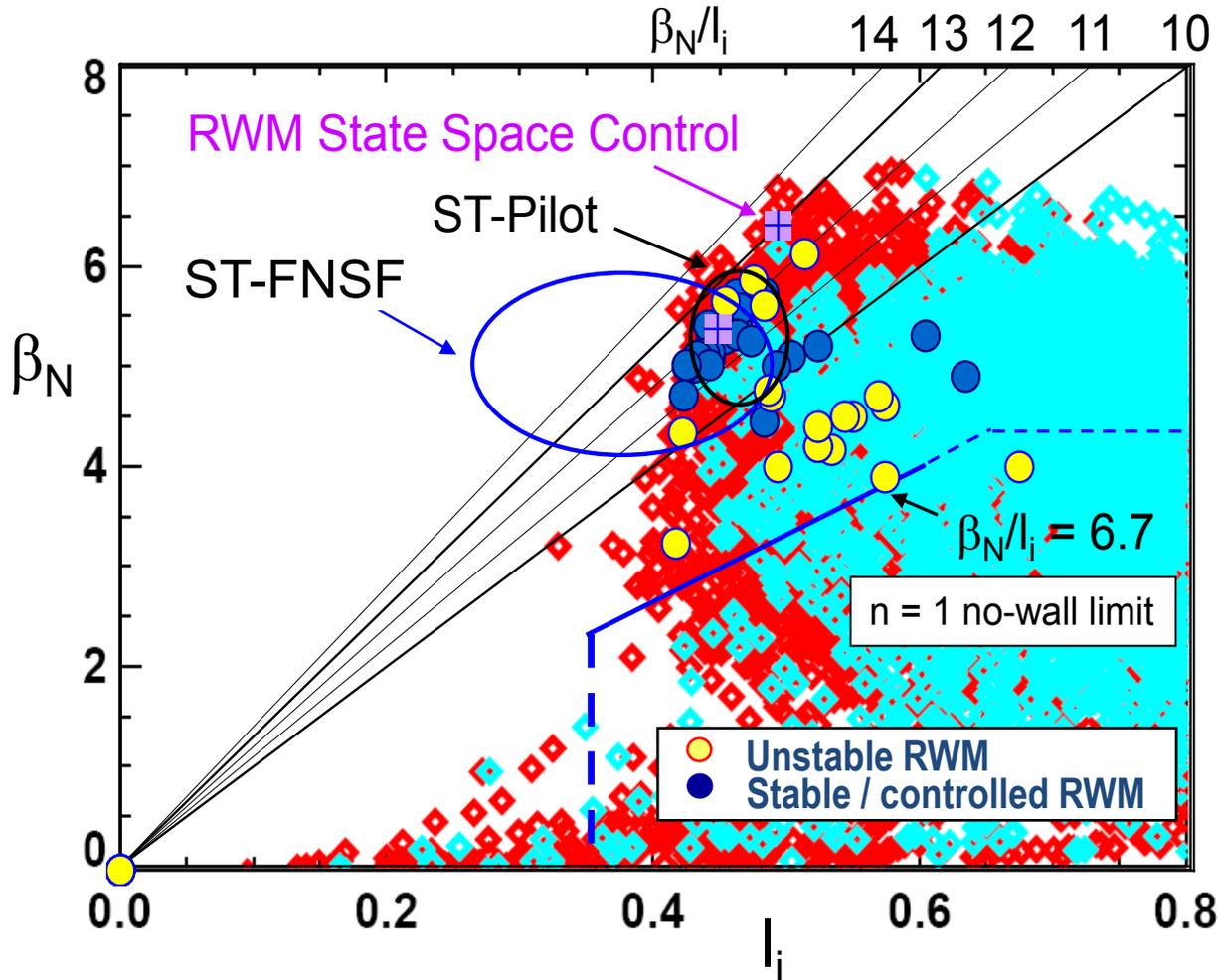
$$I_N \equiv I_p / aB_{T0} \text{ [MA/mT]}$$

$$\beta_N \equiv \beta_T (\%) / I_N$$

$$\langle \beta_N \rangle \equiv \langle \beta \rangle (\%) / I_N$$



Record β_N and β_N / I_i accessed in NSTX using passive + active resistive wall mode stabilization

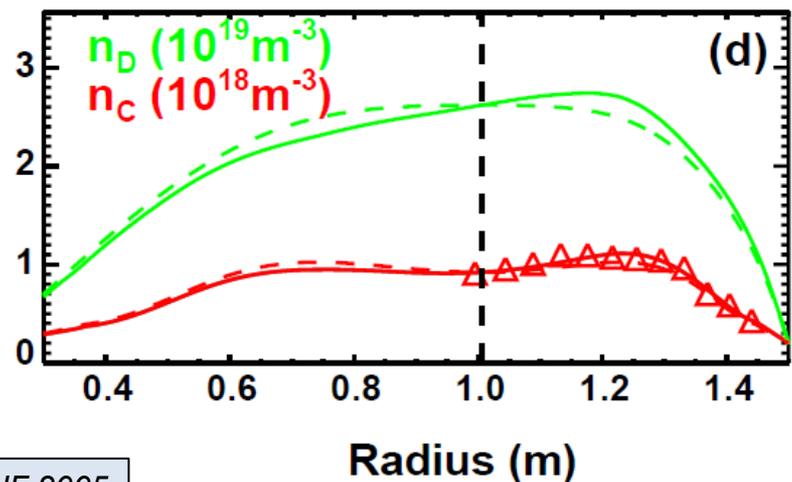
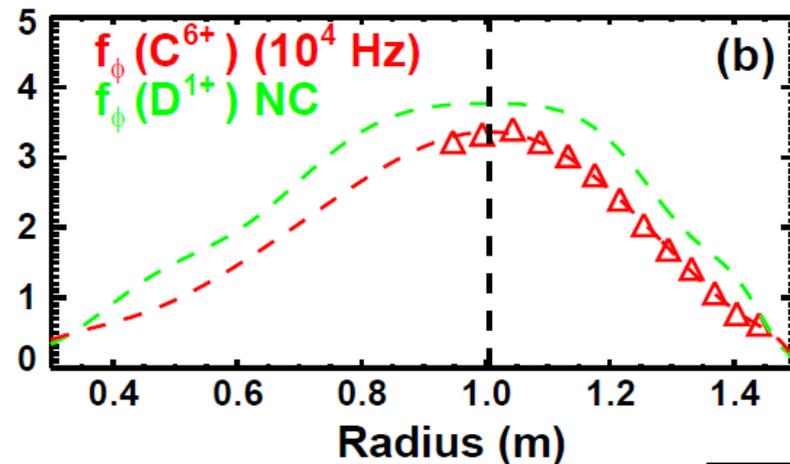
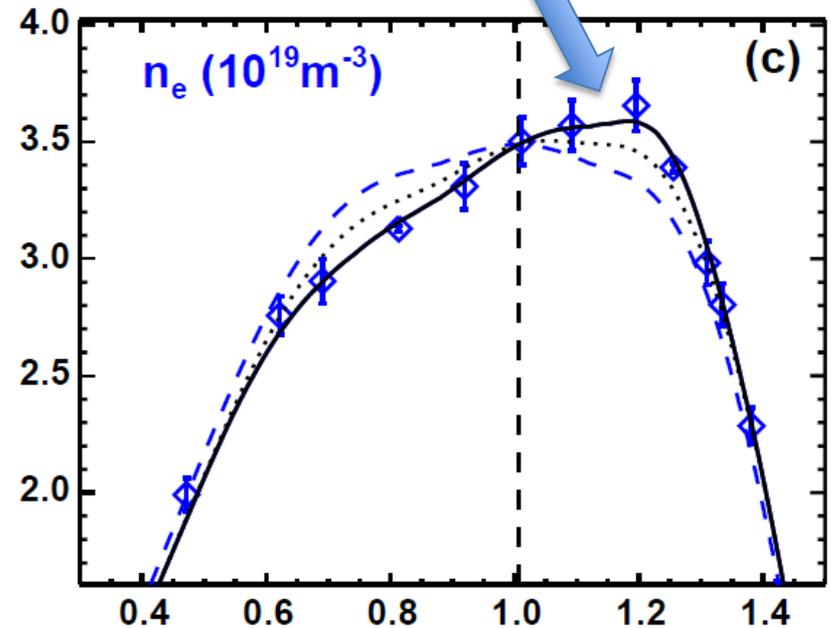
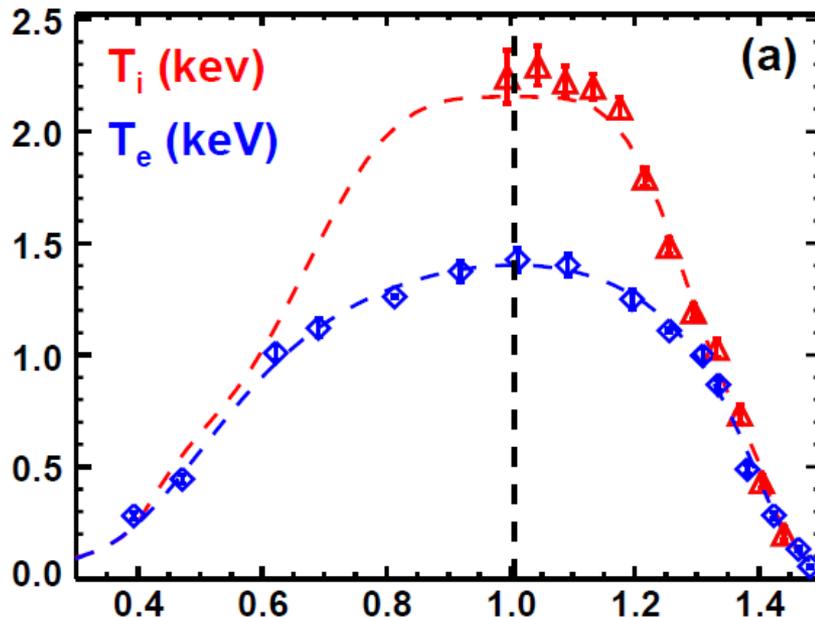


- High β_N regime is important for bootstrap current generation.
- High β_N / I_i regime important since high f_{BS} regime has low I_i .

S. Sabbagh, PRL 2006
J. Berkery, PRL 2011
W. Zhu, PRL 2006

Major NSTX-U mission is to achieve fully non-inductive operation at high β

Rotation / centrifugal effects important and measurable in equilibrium



J. Menard, NF 2005

Kelvin-Helmholtz (KH) instabilities predicted when central sound-speed Mach number $M_s \approx 0.7-0.8$

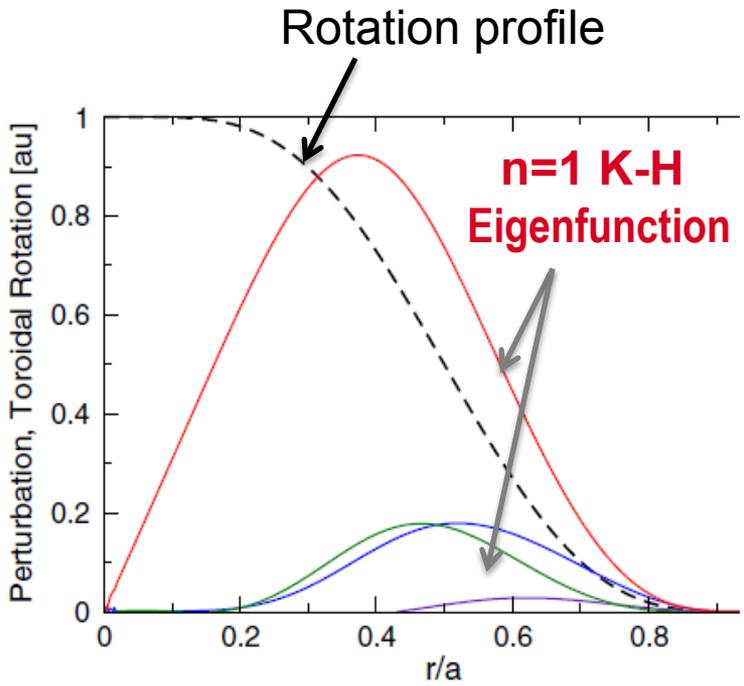


Figure 4. The $n = 1$ KH_{\parallel} eigenfunction when the flow profile (dashed line) is centred at $r/a = 0.5$.

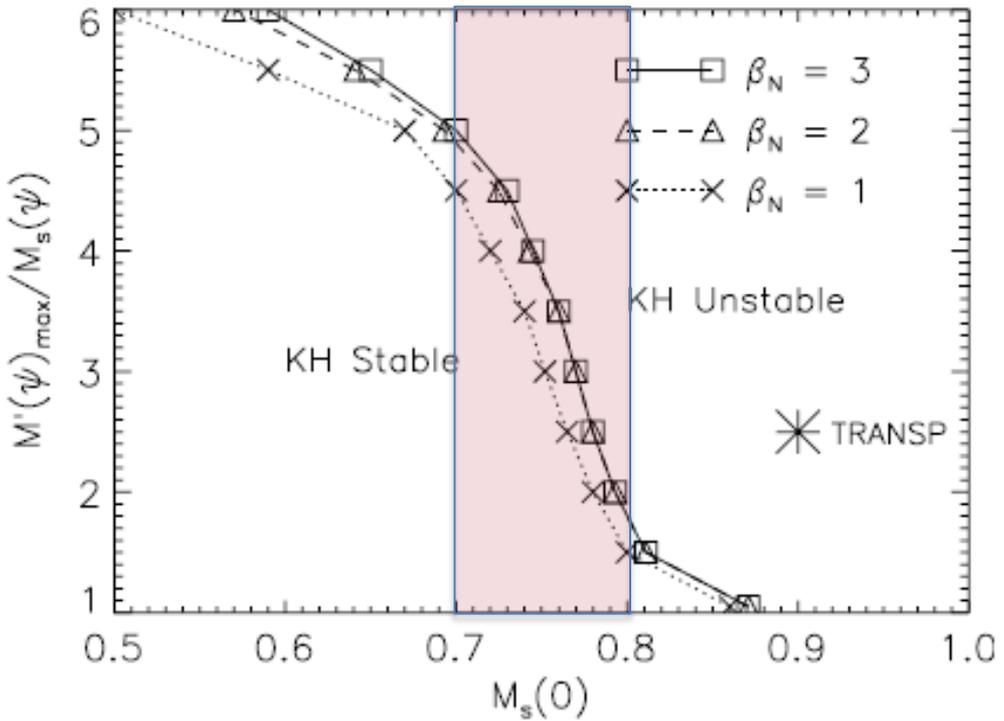
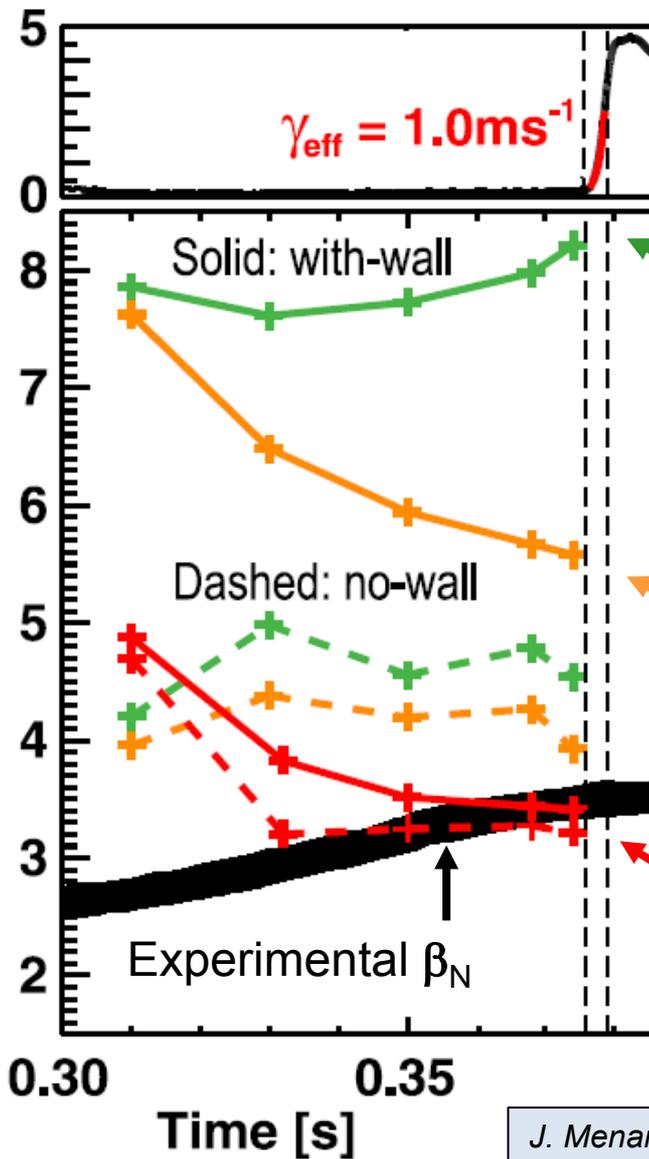


Figure 5. The KH_{\parallel} stability boundary in terms of flow speed and gradient for three different plasma pressures when the safety factor is fixed at $q_{\min} \simeq 1.3$. A typical rotation profile from TRANSP predictions is shown for reference, indicating that CTF with fully uni-directional beams is likely to be KH_{\parallel} unstable.

I. Chapman, NF 2012

Hybrid MHD-drift-kinetic stability calculations find rotation + fast-ions can weaken wall-stabilization of ∇p -driven kink

Mirnov at vessel wall
 $|\mathbf{B}_{n=1}|$
 [Gauss]



- MARS-K code needed / used to explain NSTX instability onset at highest rotation, β_{fast} fractions

Low-rotation fluid with-wall limit is very high \rightarrow marginal $\beta_N \sim 7-8$

Increasing rotation lowers max β_N to ~ 5.5 at mode onset time

Full kinetic treatment including fast-ions \rightarrow marginal $\beta_N \sim 3.5 \rightarrow$ most consistent with experiment

Outline

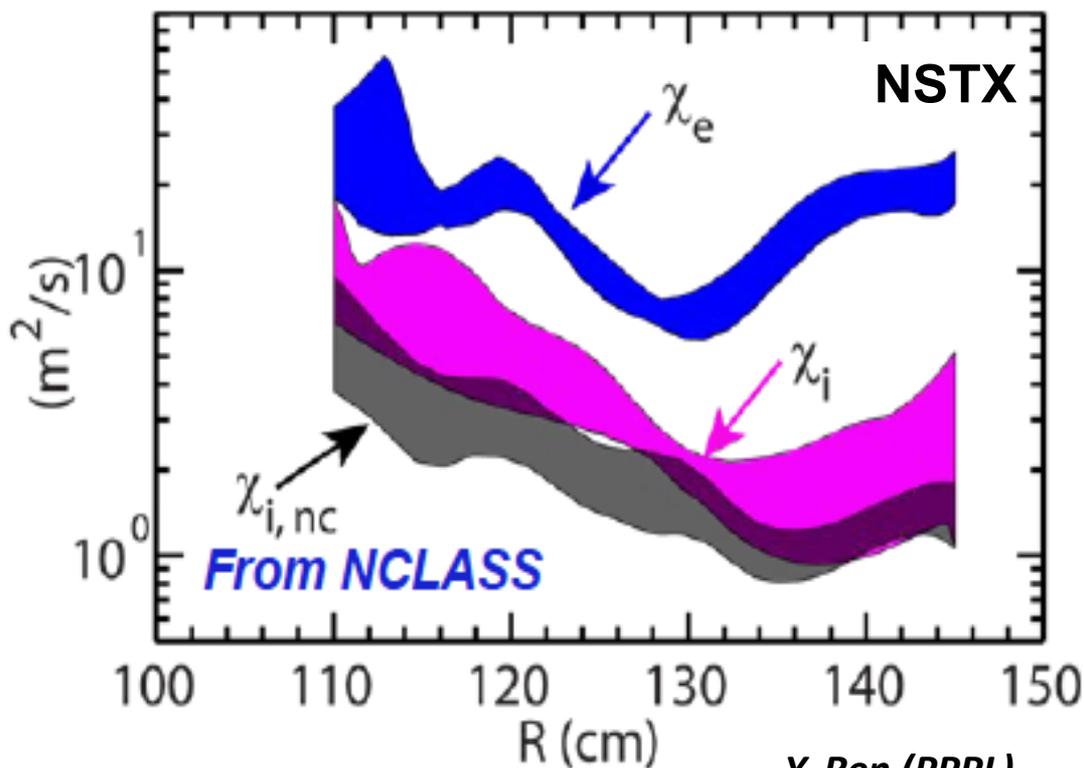
- ST overview
- Global MHD Stability
- **Energy Confinement**
- Energetic Particles
- ST Upgrade Status
- Summary

High confinement multiplier H needed for compact ST

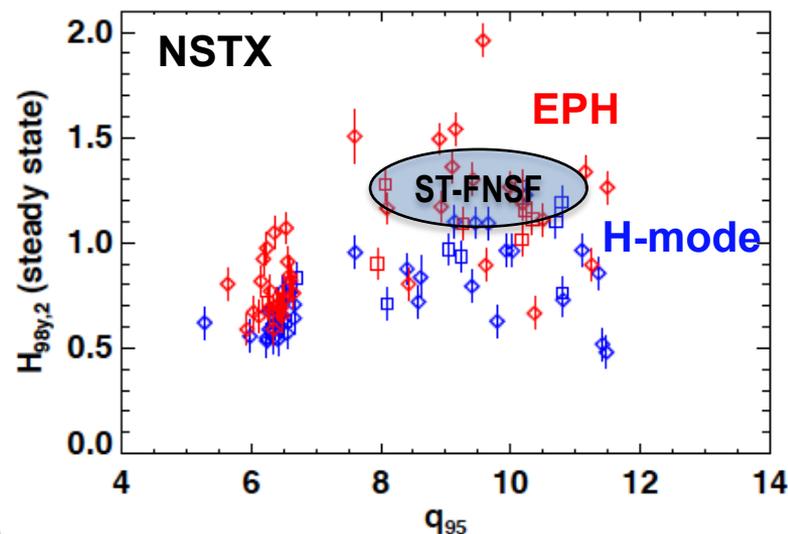
Fusion gain Q depends strongly on “H”, $Q \propto H^{5-7}$

H = 1.2 – 1.3 enables compact FNSF, design flexibility/margin

- Ion energy transport in H-mode ST plasmas near neoclassical level **due to high shear flow and favorable curvature**
- Electron energy transport anomalous (as for all tokamaks)



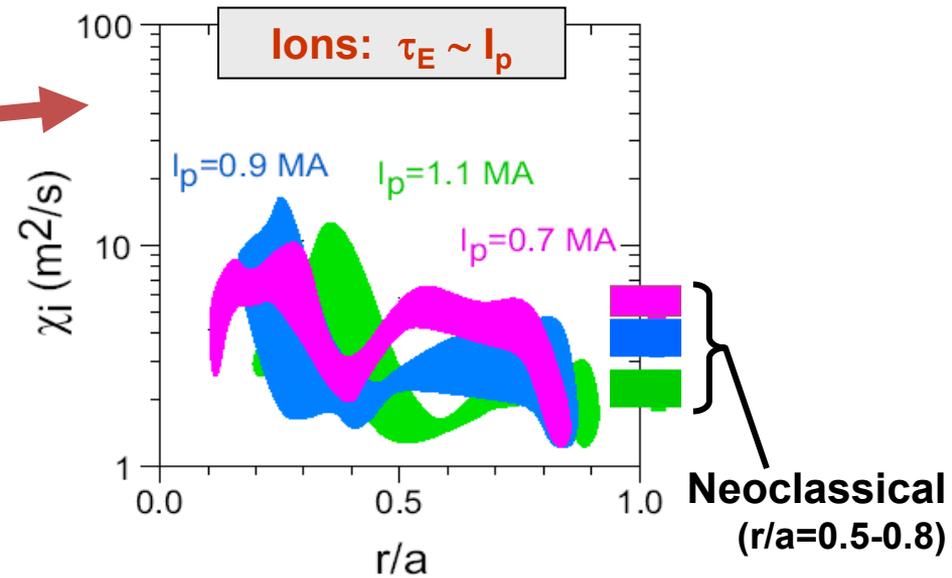
Enhanced pedestal H-mode (EPH) has H up to 1.5-2 \rightarrow attractive for ST-FNSF



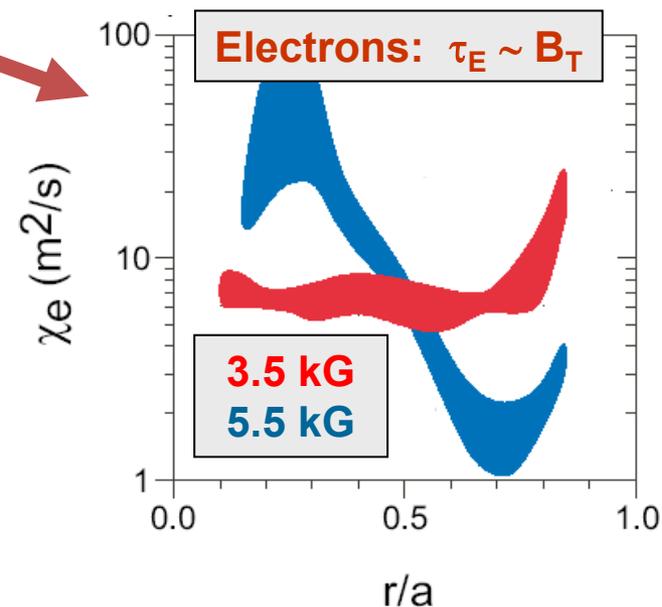
R. Maingi, PRL 2010
S. Gerhardt NF 2014

Electron and ion τ_E scale differently in ST, and different than at higher aspect ratio

- **Ion $\tau_E \sim I_p$, consistent with neoclassical ion transport**
 - Implies ion turb. suppressed by high $E \times B$ shear \rightarrow possibility of isolating causes of e-transport



- **Electron $\tau_E \sim B_T$**
 - Could imply Electron Temperature Gradient (ETG) modes, and/or electromagnetic turbulence



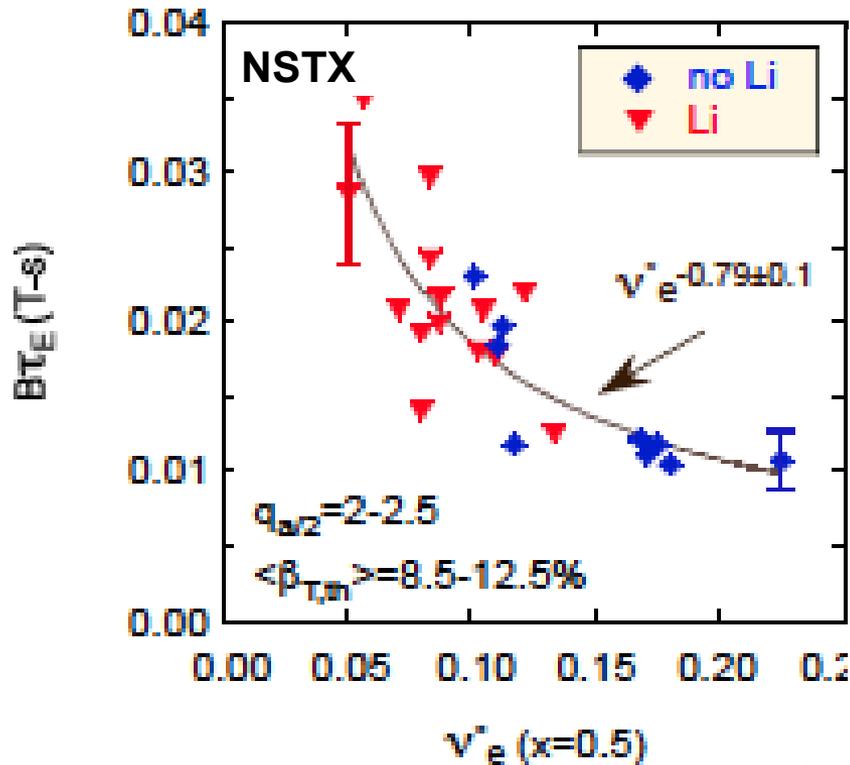
S.M. Kaye, PRL 2007

Favorable confinement trend with collisionality, β found

Important implications for future ST FNSF, Demo with lower ν_*

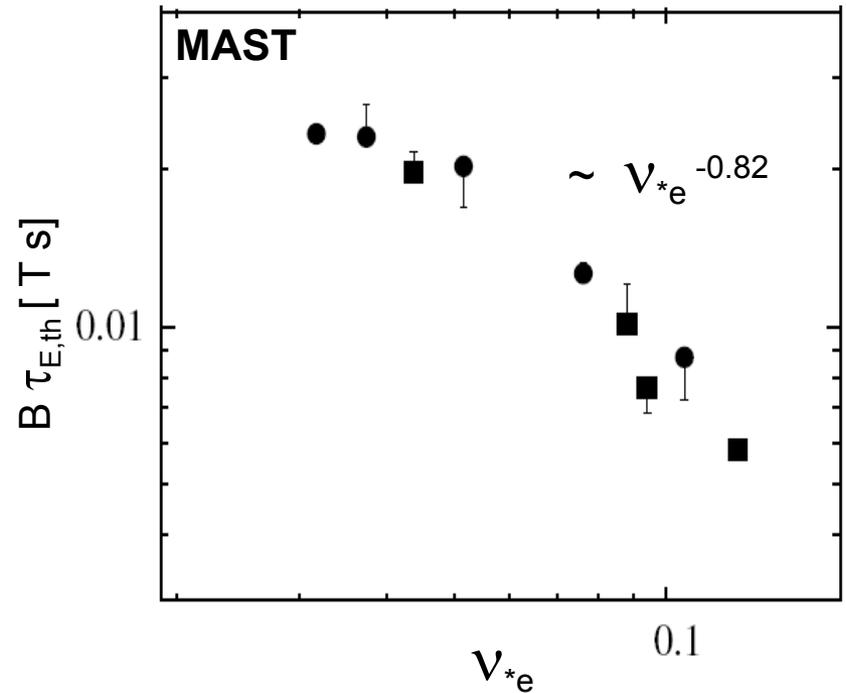
$$\tau_{E,th} \propto \nu_{*e}^{-0.1} \beta^{-0.9} \quad \text{tokamak empirical scaling (ITER 98}_{y,2}\text{)}$$

$$\tau_{E,th} \propto \nu_{*e}^{-0.8} \beta^{-0.0} \quad \text{ST scaling}$$



S.M. Kaye, NF 2007, 2013

$$\nu_* \propto \bar{n}_e / T^2$$

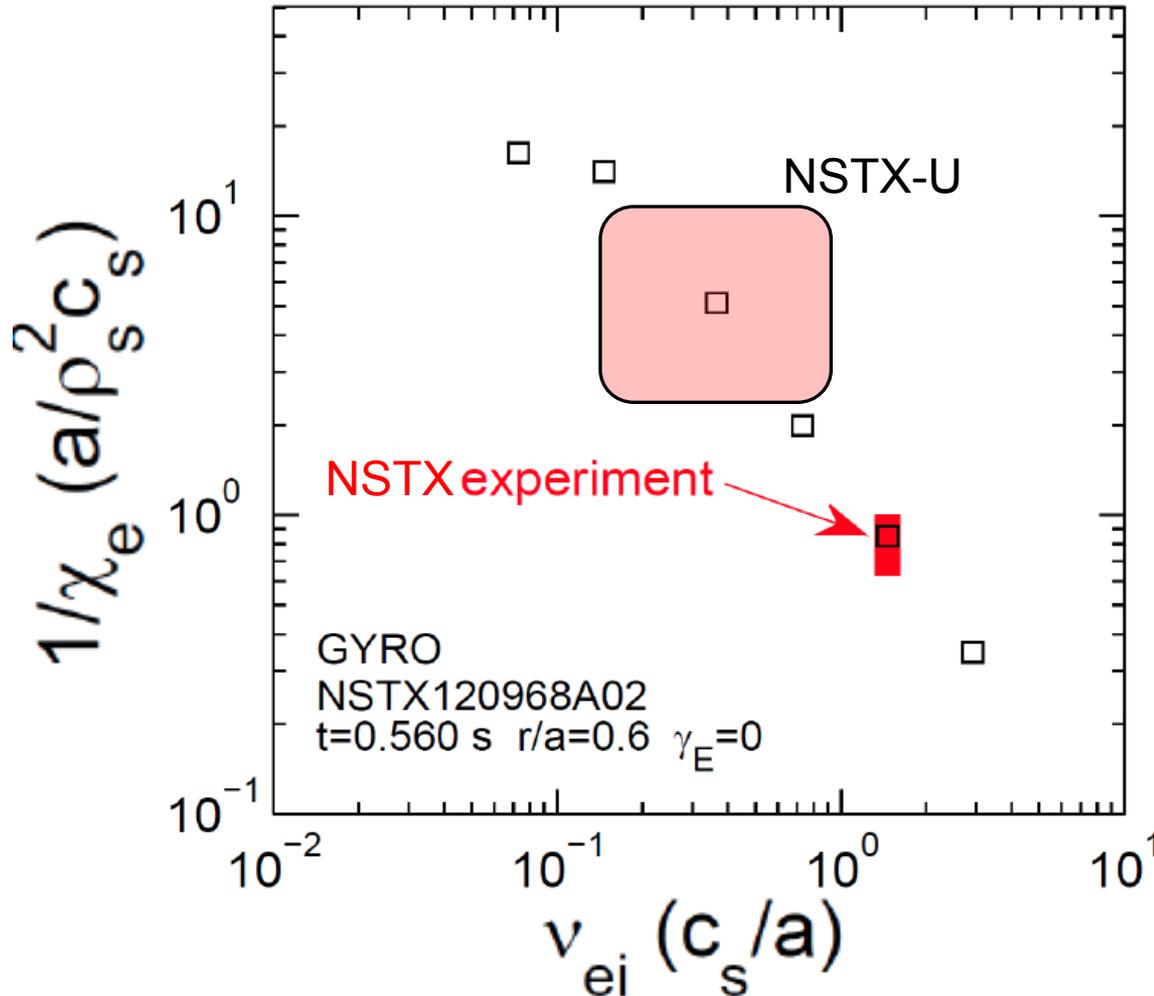


M. Valovic, NF (2011)

Very promising ST scaling to reactor condition, if continues on NSTX-U/MAST-U

Micro-tearing-driven (MT) transport may explain ST τ_E collisionality scaling

MT-driven χ_e vs. v_{ei} using the GYRO code



- MT growth rate decreases with reduced collisionality in qualitative agreement with the NSTX experiment.
- Further electron confinement improvement expected due to reduced collisionality.

W. Guttenfelder, PoP 2013,
PoP 2012, PRL 2011

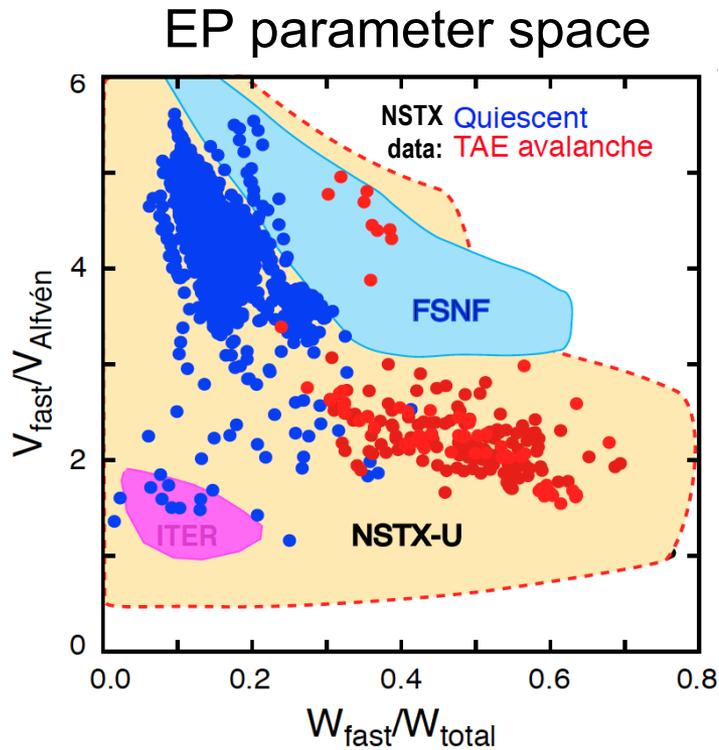
Outline

- ST overview
- Global MHD Stability
- Energy Confinement
- **Energetic Particles**
- ST Upgrade Status
- Summary

NBI-heated STs excellent testbed for α -particle physics

Alfvénic modes readily accessible due to high $V_{\text{fast}} > V_{\text{Alfvén}}$

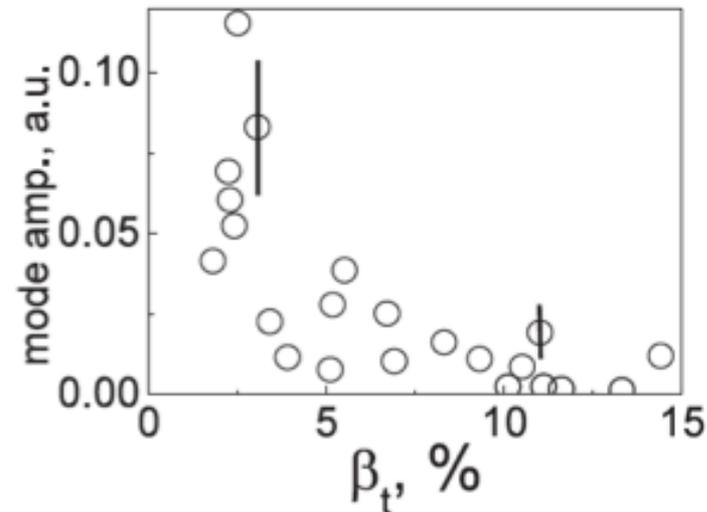
- α -particles couple to Alfvénic modes strongly when $V_{\alpha} > V_A \sim \beta^{-0.5} C_s$
- $V_{\alpha} > V_A$ in ITER and reactors: condition easily satisfied in ST due to high β
- Fast-particle-driven Alfvén Eigenmodes: Toroidal, Global, Compressional
- NSTX-U will also explore $V_{\text{fast}} < V_A$ regime giving more flexibility



E. Fredrickson, NF 2013

TAEs significantly modified at high β as $V_A \rightarrow C_s$

Stabilization of TAEs at high β in MAST

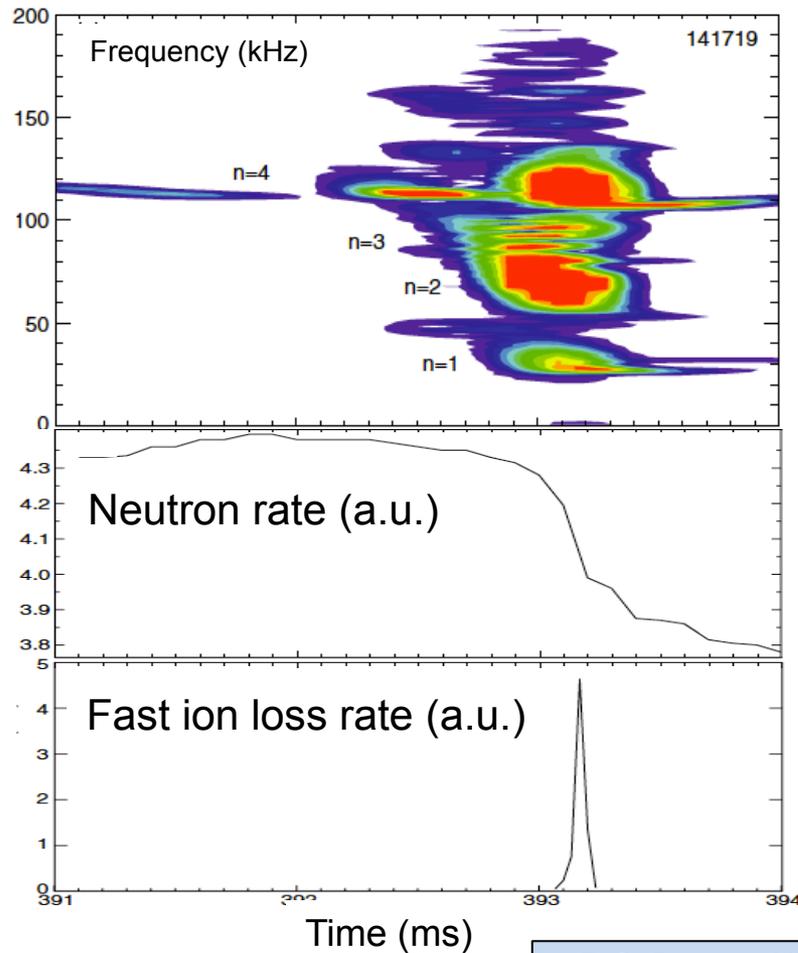


M.P. Gryaznevich, PPCF 2004

“TAE avalanche” shown to cause energetic particle loss

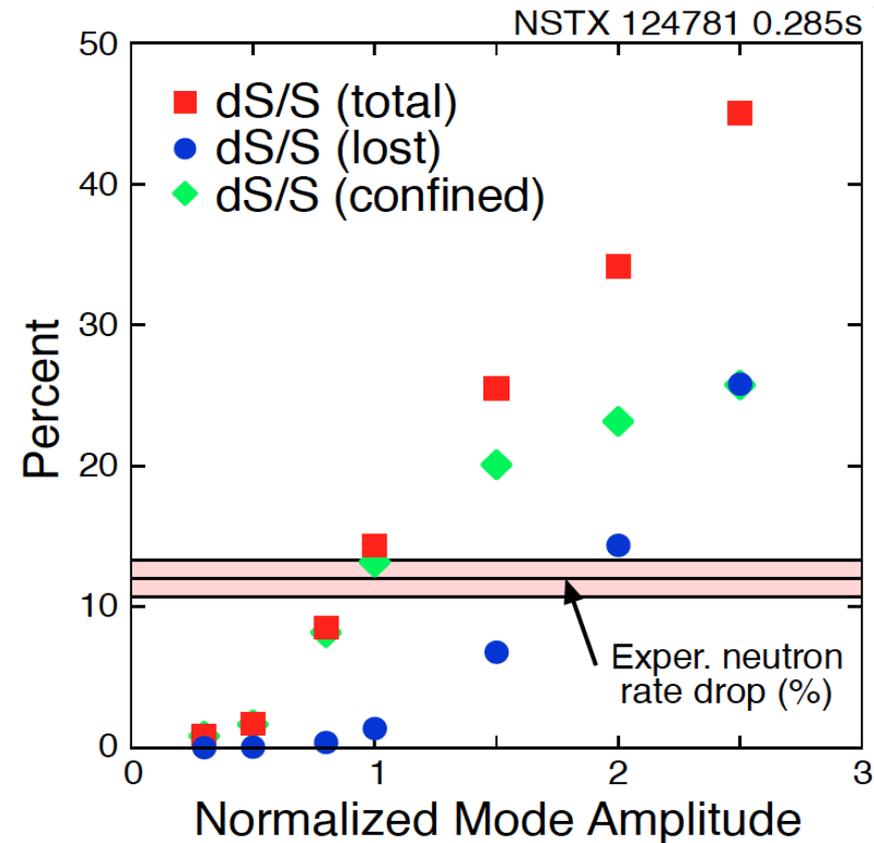
Uncontrolled α -particle loss could cause reactor first wall damage

Multi-mode TAE avalanche can cause significant EP losses as in “sea” of TAEs expected in ITER



D.S. Darrow, NF (2013)

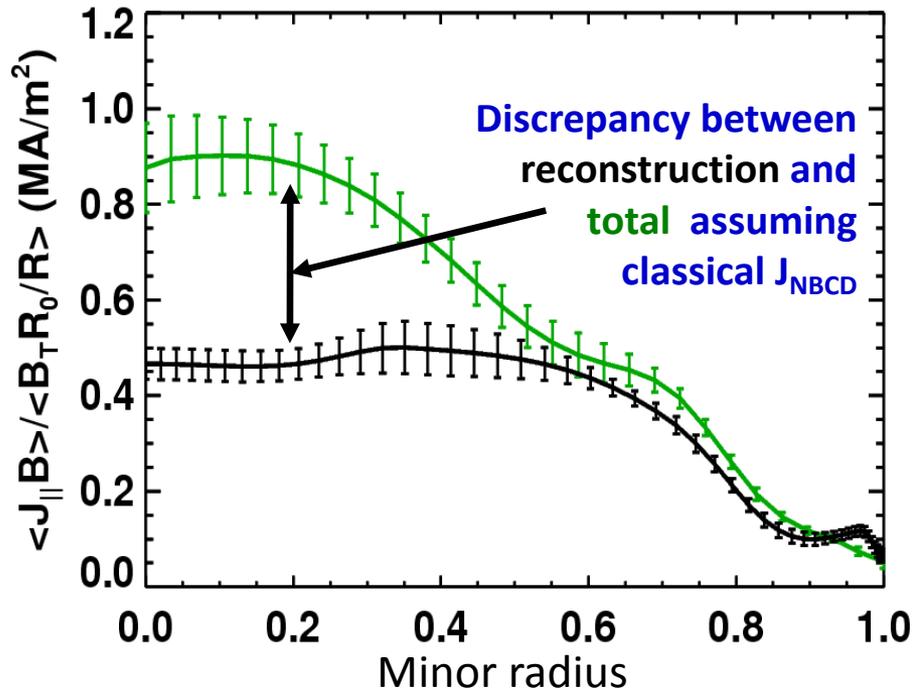
Progress in simulation of neutron rate drop due to TAE avalanche



E. Fredrickson, NF 2013

Rapid TAE avalanches could impact NBI current-drive in advanced scenarios for NSTX-U, FNSF, ITER AT

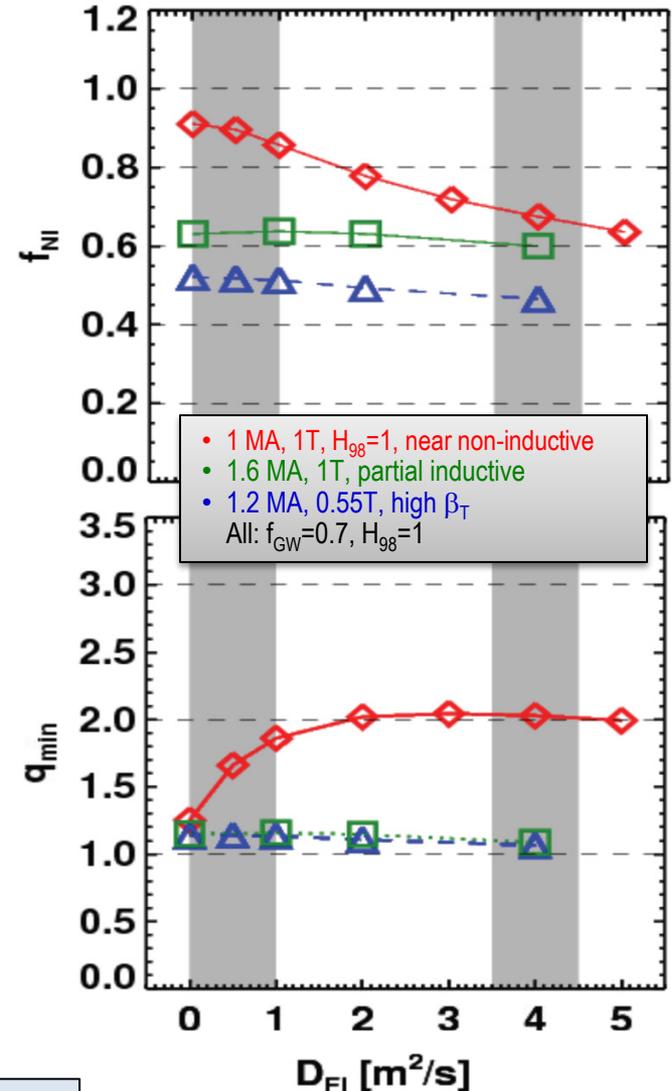
NSTX: rapid avalanches can lead to redistribution/loss of NBI current drive



700kA high- β_p plasma with rapid TAE avalanches has time-average $D_{\text{FI}} = 2\text{-}4\text{m}^2/\text{s}$

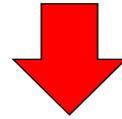
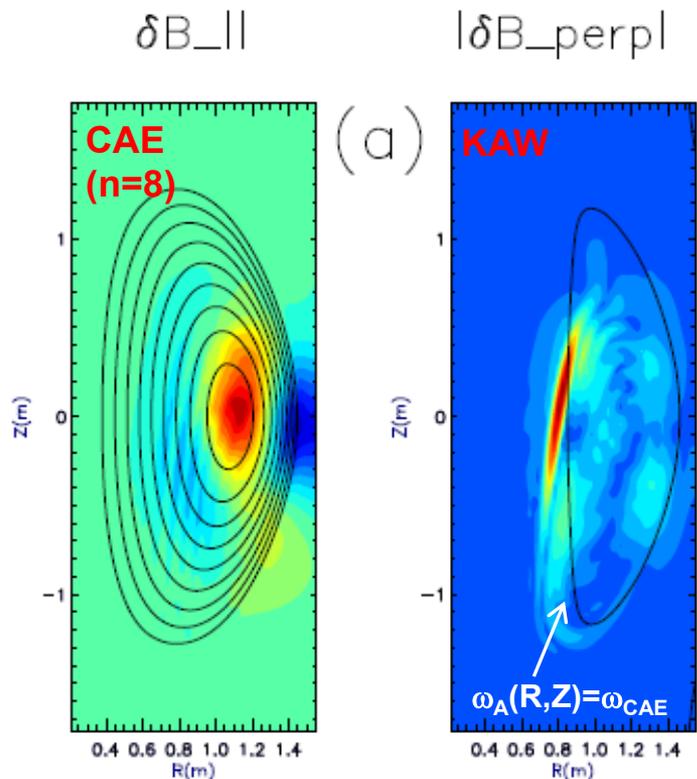
S. Gerhardt NF 2011, NF 2012

NSTX-U TRANSP simulations



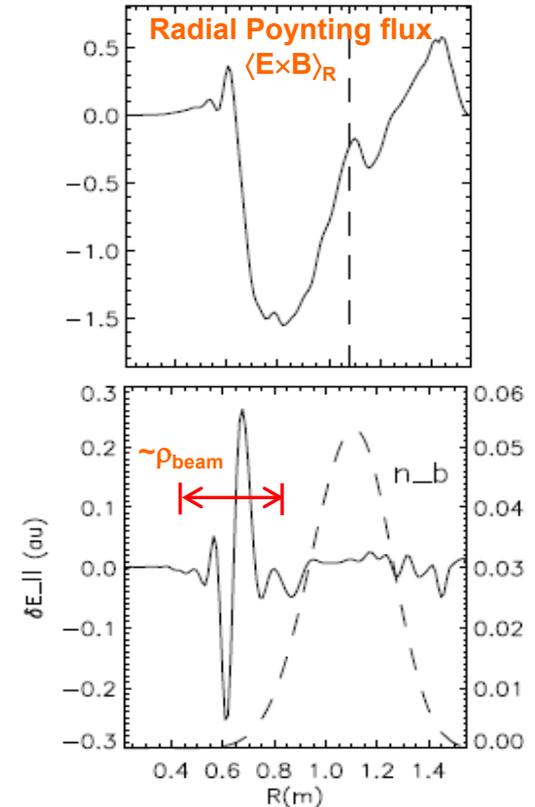
CAE mode-conversion to kinetic Alfvén waves (KAW) predicted to transfer core NBI power to mid- ρ electrons

- 1) GAE/CAEs cause large χ_e through stochastic orbits (N. Gorelenkov, NF 2010)
- 2) CAEs also couple to KAW - Poynting flux redistributes fast ion energy near mid-radius, E_{\parallel} resistively dissipates energy to thermal electrons
 - $P_{\text{CAE} \rightarrow \text{KAW}} \sim \mathbf{0.4 \text{ MW}}$ from QL estimate + experimental mode amplitudes
 - $P_{e, \text{NBI}} \sim \mathbf{1.7 \text{ MW}}$ for $\rho < 0.3$, NBI power deposited on core electrons



Up to 25% of
electron heating
power transferred
to KAW off-axis

HYM code
E. Belova, PRL 2015



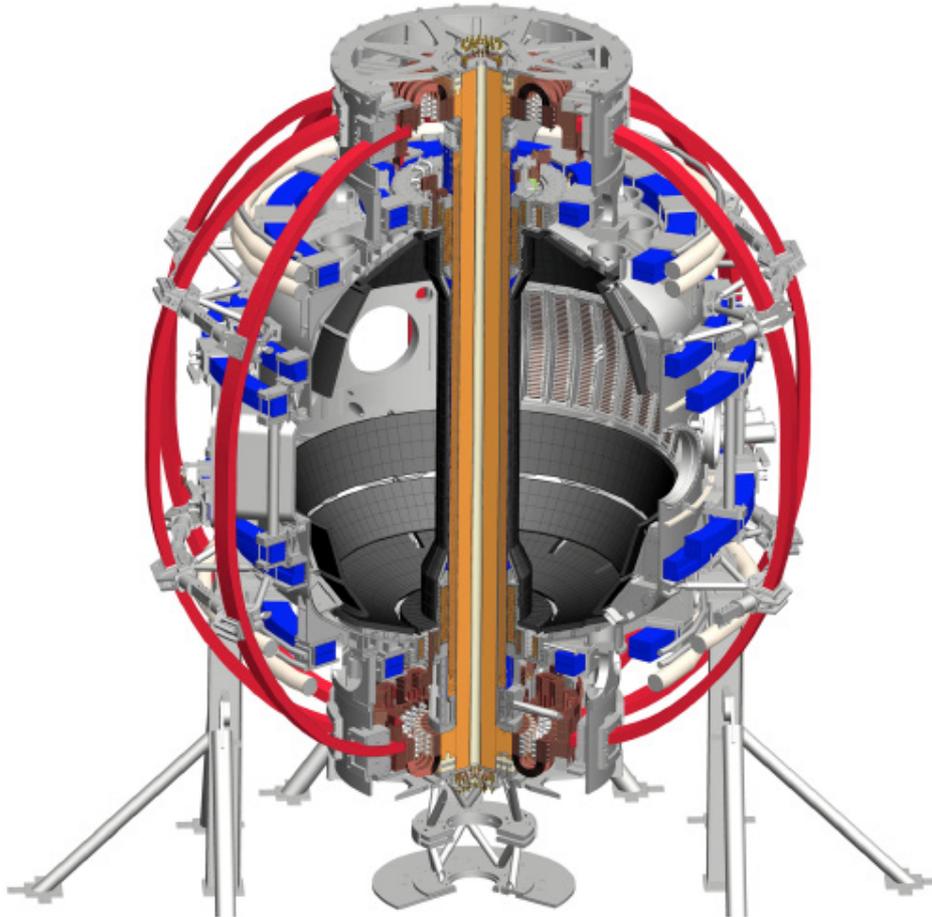
Outline

- ST overview
- Global MHD Stability
- Energy Confinement
- Energetic Particles
- **ST Upgrade Status**
- Summary

NSTX and MAST are undergoing major upgrades

~2x higher B_T , I_p , P_{NBI} and ~5x pulse length vs. NSTX/MAST

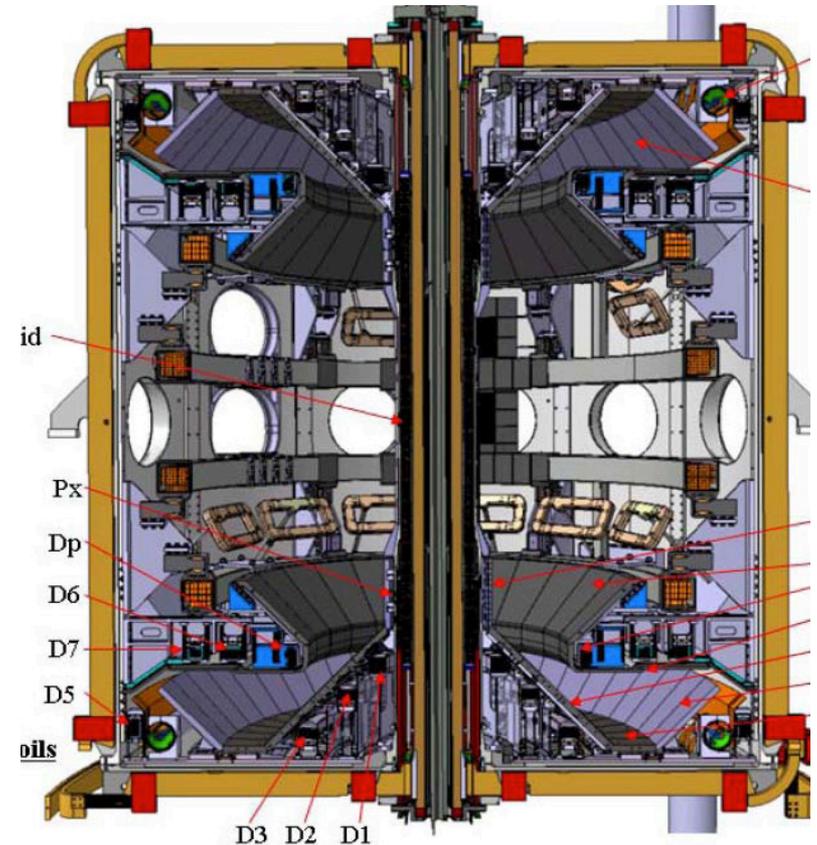
NSTX-U



Highly tangential 2nd NBI for non-inductive sustainment, profile control

First test plasma few weeks ago

MAST-U

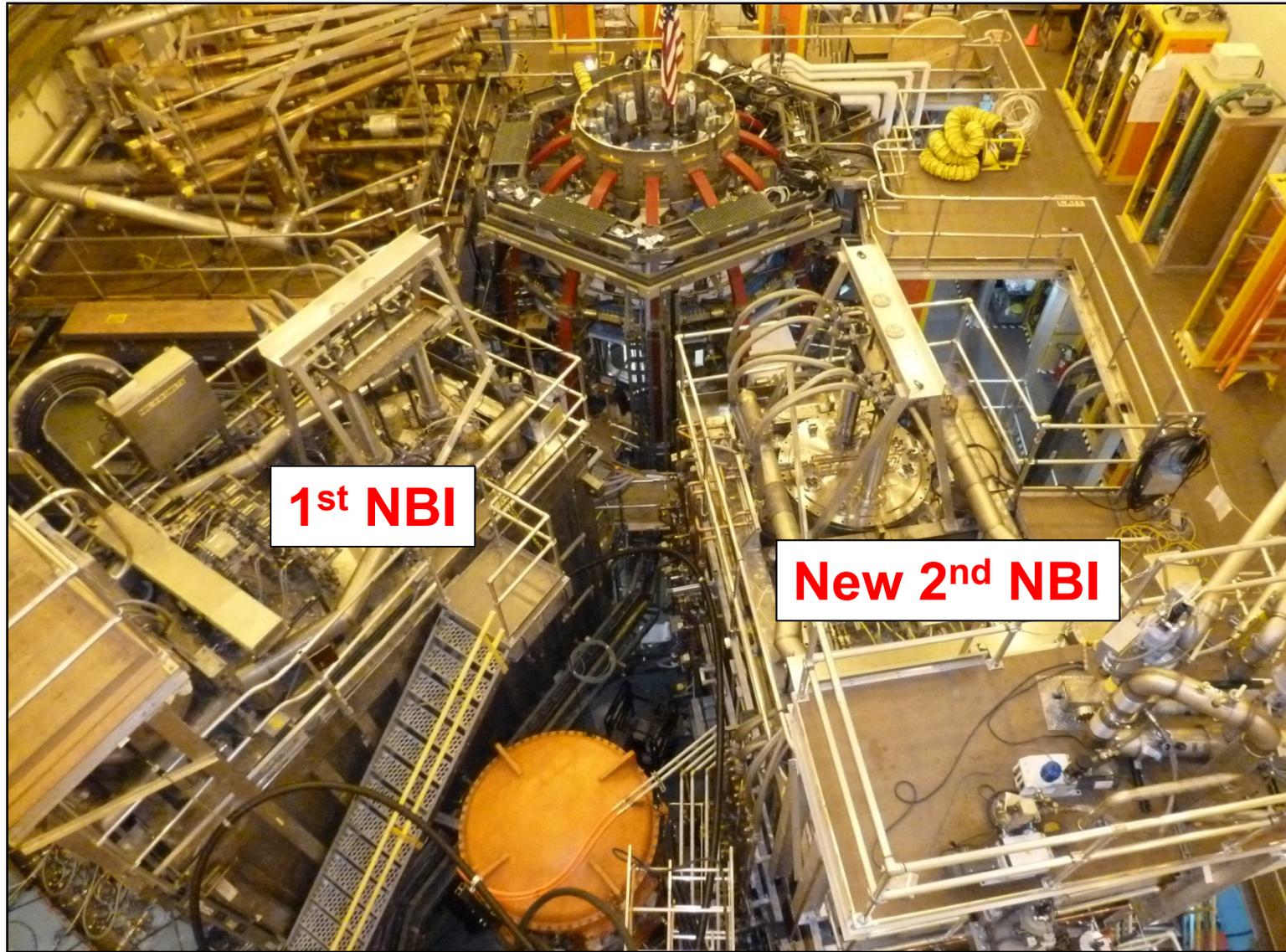


Super-X divertor configuration for FNSF/DEMO divertor solution

First test plasma 2017

NSTX Upgrade project recently completed

On cost and schedule, first test plasma $\sim 100\text{kA}$ (Aug. 10, 2015)



New centerstack (CS) highlights: Jan – Aug 2015

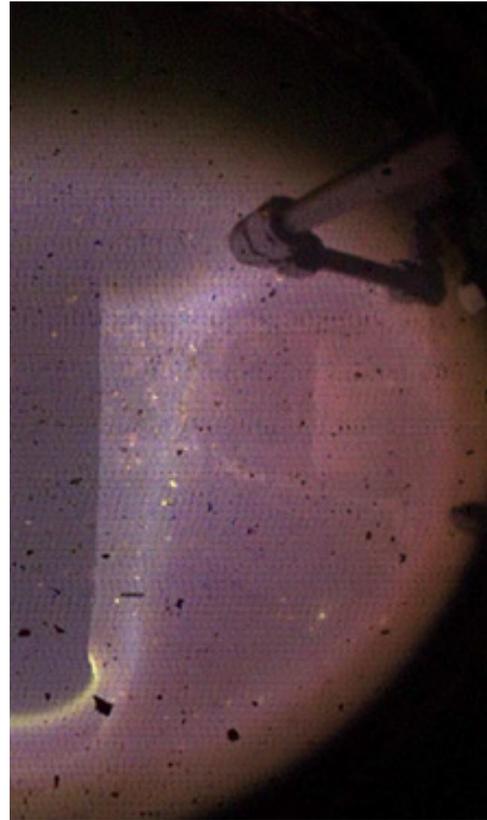
CS crane lift



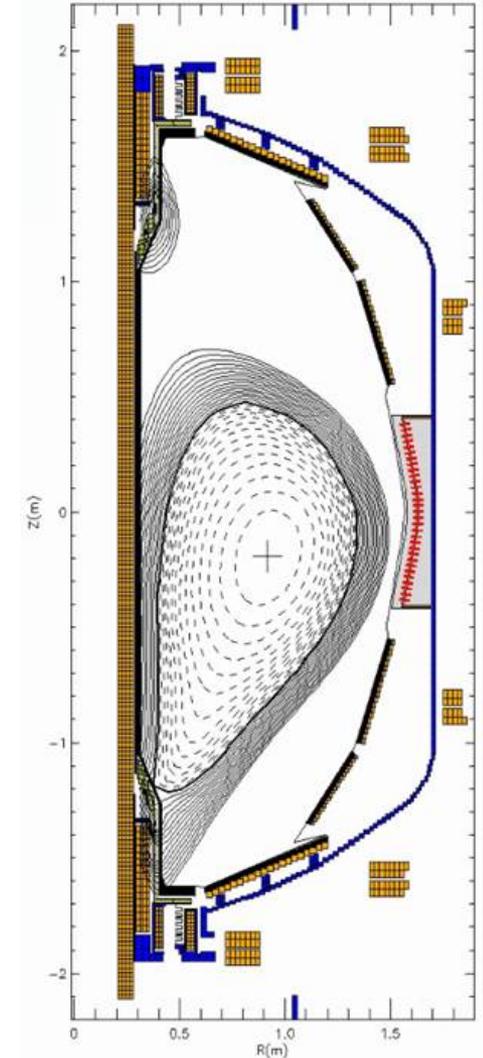
CS installed



**First test plasma
(Ohmic heating only)**



**Magnetics functional →
EFIT reconstructions**



Summary

- Upgraded STs will provide many opportunities to study toroidal confinement physics in new regimes:
 - Low aspect ratio, strong shaping, high β , low collisionality
 - Access to strong fast-ion instability drive, high rotation
 - Advanced divertors, lithium walls, high-Z PFCs
- There are potentially interesting linkages between ST and CT / FRC physics that could be explored further:
 - Role of rapid rotation, strong beams, kinetic effects, ...
- Thank you!