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Phase space effects on fast ion distribution function modeling in tokamaks

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Integrated simulations of tokamak discharges typically rely on classical physics to model energetic particle (EP) dynamics. However, there are numerous cases in which energetic particles can suffer additional transport that is not classical in nature. Examples include transport by applied 3D magnetic perturbations and, more notably, by plasma instabilities. Focusing on the effects of instabilities, ad-hoc models can empirically reproduce increased transport, but the choice of transport coefficients is usually somehow arbitrary. New approaches based on physics-based reduced models are being developed to address those issues in a simplified way, while retaining a more correct treatment of resonant wave-particle interactions. The kick model implemented in the tokamak transport code TRANSP is an example of such reduced models. It includes modifications of the EP distribution by instabilities in real and velocity space, retaining correlations between transport in energy and space typical of resonant EP transport. The relevance of EP phase space modifications by instabilities is first discussed in terms of predicted fast ion distribution. Results are compared with those from a simple, ad-hoc diffusive model. It is then shown that the phasespace resolved model can also provide additional insight into important issues such as internal consistency of the simulations and mode stability through the analysis of the power exchanged between energetic particles and the instabilities. Published by AIP Publishing.

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I. INTRODUCTION

Burning fusion plasmas feature a high content of energetic particles (EP) originating from Neutral Beam (NB) injection, rf heating, or fusion reactions. Because of their crucial role, quantitative understanding and accurate modeling of the EP dynamics are required for interpreting present experiments and for predicting scenarios on future devices. Modeling tools already exist to model EP dynamic when energetic particles behave *classically*. Source and sink terms are well known and can be taken into account in the simulations with good accuracy. However, departure from classical behavior can be expected—and is indeed observed, cf. Refs. 2 and 3, and references therein—in the presence of perturbations of the EP evolution. The latter, for example, include magnetic perturbation induced by external coils, rf fields, or plasma instabilities.

Several tools have been developed to study nonclassical EP behavior, ranging from first-principles numerical codes⁴⁻⁹ to reduced models¹⁰⁻¹³ with various degree of simplifications. In particular, reduced models appear attractive for long time-scale integrated simulations of tokamak discharges, which typically require relatively short execution times to enable routine analysis of entire discharges or extensive parameter scans. This work focuses on the use of reduced models to include the effects of plasma instabilities (such as Alfvénic modes, AEs) in integrated simulations of tokamak discharges with energetic particles from Neutral

II. MODELING TOOLS

be discussed.

The main tool used in this work for integrated tokamak modeling is the TRANSP code. 16,17 Energetic particle evolution is modeled through the NUBEAM module 18,19 of TRANSP, which includes several models for additional EP transport in addition to classical EP physics. For simplicity,

Beam (NB) injection. Results from the NSTX device¹⁴ will

is to assess what level of complexity is required in modeling

the evolution of the EP distribution function, F_{EP} , from

which most of the other EP-related quantities are computed.

For example, the latter include NB driven current and current

drive efficiency, whose assessment is one of the major mile-

stones for the NSTX Upgrade device. 15 Results from two EP

transport models are compared. The main difference between

the two models is whether EP transport is simply assumed to

occur as diffusion in the radial coordinate, or modifications

main modeling tools used in this work (Sec. II), followed by

a description of the experimental scenario on which simula-

tions are based upon (Sec. III). Section IV contains the main

results of this work, starting with examples of EP distribu-

tions obtained from the two models. The implications of the differences in EP distribution from the two models on the

consistency of the simulations are then discussed. As an

example, the power balance between energetic ions and the

instabilities they drive is taken as figure of merit to assess

the consistency of the simulations. Section V summarizes

the main findings of this work and concludes the paper.

The following sections include an introduction to the

of particle's phase space are also modeled.

At a more fundamental level, the main goal of this work

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results presented here are limited to the two models described below.

The first model is based on the simplest possible assumption of purely diffusive EP transport, with a particle flux given by

$$\Gamma_{nb} = -D_b \nabla n_b, \tag{1}$$

where ∇n_b is the radial gradient of the beam ion density. The diffusion coefficient D_b in Eq. (1) is an ad-hoc parameter, here assumed to be uniform in radius and with no energy dependence. D_b values are chosen to match measured quantities such as the neutron rate (see below). A time dependence $D_b = D_b(t)$ can be retained to improve the match with the experimental data as a function of time. Typical values are $0 \le D_b \le 5 \text{ m}^2/\text{s}$.

The second model (referred to as kick model) is based on a transport probability $p(\Delta E, \Delta P_{\zeta}|E, P_{\zeta}, \mu)$, which describes changes in particle's energy and to-dal canonical momentum resulting from the instabilities. The transport matrix is pre-calculated through particle-following codes such as ORBIT, using perturbations modeled by MHD codes such as NOVA 22-24 that reproduce experimentally observed instabilities in terms of frequency and mode number spectrum. For the shot analyzed in this work, 6 probabilities are used to model AEs, including 2 reversed-sear AEs (RSAEs) for $t < 200\,\mathrm{ms}$ and 4 toroidal AEs (TAEs) for $t > 200\,\mathrm{ms}$, and the n = 1 kink-like mode throughout the discharge. For each mode, the mode structure is computed at a single time, then rescaled at different times based on frequency-filtered signals from Mirnov coils. On the structure is computed at a single time, then rescaled at different times based on frequency-filtered signals from Mirnov coils.

In practice, during a TRANSP run particles are classified based on their phase space variables E, P_{ζ} , and μ , which indicate energy, toroidal canonical momentum, and magnetic moment according to the normalizations used in Ref. 25. As time evolves, particles experience kicks ΔE , ΔP_{ℓ} according to the probability matrix, based on their phase space location. This probabilistic approach naturally fits with the MonteCarlo approach on which the NUBEAM module is based. Up to 10 probability matrices can be provided as input to model different modes, or sets of modes with similar properties. Free parameters for simulations with the kick model are the mode amplitudes as a function of time. Amplitudes are inferred from experimental measurements, when available. Further iterations may be required to achieve a better agreement with the measured neutron rate and stored energy. More details on the model can be found in Ref. 20.

III. EXPERIMENTAL SCENARIO

Results presented in this work refer to NSTX discharge #139048. Plasma current reaches its stationary level of 0.9 MA at \sim 200 ms. Toroidal field on axis is 0.45 T. Up to 6 MW of NB power are injected, with increasing steps of 2 MW between 50 ms and 200 ms. The discharge transitions into H-mode confinement during the current ramp-up phase and stays in H-mode until its termination.

As shown in Fig. 1, a rich variety of Alfvénic activity is observed throughout the discharge. Reverse-shear Alfvén Eigenmodes (AEs) with low toroidal mode number, n = 1-2,

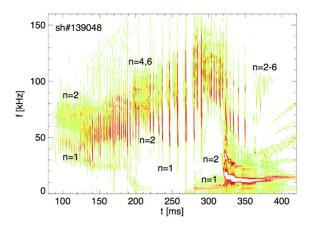


FIG. 1. Frequency spectrum of magnetic fluctuations from Mirnov coils installed at the low-field side vessel wall for NSTX discharge #139048. Toroidal mode numbers of the different instabilities are indicated in the figure.

are present during the current ramp-up. Afterwards, toroidal AEs (TAEs) with n=1-6 become dominant. Alfvénic instabilities co-exist with lower frequency, kink-like modes as the minimum of the safety factor approaches $q_{min} \sim 1$ for $t \geq 320\,\mathrm{ms}$. As typical for strongly NB-driven NSTX discharges, higher frequency Global and Compressional AEs are also measured at frequencies $\gg 200\,\mathrm{kHz}$. Those modes will not be considered in the following analysis.

Instabilities in the TAE frequency range have been analyzed through the ideal MHD code NOVA^{22–24} to infer the radial mode structure, based on the comparison with experimental data from Mirnov coils and a multi-channel reflectometer array.^{27,28} The complete analysis is described in Ref. 29. The inferred radial mode structures for TAEs with toroidal mode number $n_{tor} = 1, 2, 4, 6$ are shown in Fig. 2. Modes cover most of the minor radius, which is typical for TAEs observed on NSTX.^{29,30}

No experimental data are available for the mode structure of the kink-like modes. Therefore, a simple analytical model is used to model their structure.³¹

Data from magnetic pick-up coils installed at the lowfield side plasma wall are used in the following as an initial

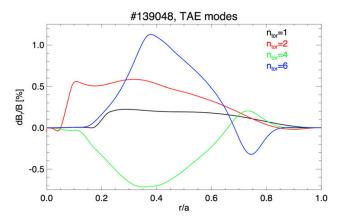


FIG. 2. Radial mode structure computed by the NOVA code for the $n_{tor} = 1-6$ TAE modes observed in Fig. 1 for $t \gtrsim 200\,\mathrm{ms}$. Note the radial extension of the modes, covering the entire minor radius. The amplitude of the radial magnetic field perturbation, $\delta B_r/B$, corresponds to a normalized mode amplitude $A_{mode} = 1$ for the kick model.

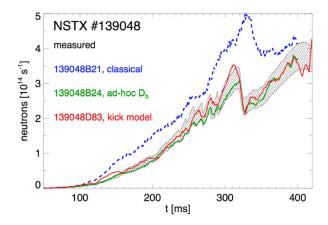


FIG. 3. Neutron rate for NSTX discharge #139048. Dashed region indicates the measured values, assuming a $\pm 5\%$ uncertainty in the measurements. Dashed line is the predicted neutron rate from TRANSP assuming classical fast ion physics. Solid lines are the results with enhanced fast ion transport using the ad-hoc D_b and kick models.

guess to infer the temporal evolution of the mode amplitude for each of the modes shown in Figs. 1 and 2. TRANSP runs are then iterated by modifying the amplitude of each mode to improve the match between measured and simulated neutron rates. For this work, all TAE mode amplitudes are scaled by the same factor in each iteration to preserve the relative mode-to-mode scaling deduced from Mirnov coil signals. However, the amplitude is reduced for specific modes at times for which the power transferred from fast ions to the mode becomes negative (cf. Sec. IV C).

IV. MODELING RESULTS

TRANSP runs based on the same experimental profiles for NSTX discharge #139048 and different assumptions on fast ion transport provide the main results for the following analysis. Runs assuming *classical* transport are used as reference and set an upper limit for quantities such as fast ion density, neutron rate, and stored energy in the absence of enhanced EP transport. Runs with the *ad-hoc* diffusive model and the *kick* model are iterated until satisfactory agreement between simulated and measured neutron rate is achieved, see Fig. 3. Free parameters for the two models are the assumed D_b and mode amplitude, respectively.

As seen from Fig. 3, classical simulations over-estimate the measured neutron rate. Measured data show rapid drops caused by bursts of TAE activity. A larger drop caused by a large-amplitude kink-like mode can also be seen around 330 ms. In this case, thermal plasma profiles are perturbed by the mode and contribute to the reduction of neutron rate.

A. EP distribution function

A first comparison for the fast ion distribution as function of energy and pitch $(p \equiv v_{\parallel}/v)$ is the ratio of parallel to total fast ion velocity averaged over particle's orbits) is shown in Fig. 4. NB injection energy is $E_0 = 90 \, \mathrm{keV}$ in the co-current direction, with about 50% of the injected neutrals populating the $E_0/2$ and $E_0/3$ energy components. After the injected neutrals are ionized, the resulting fast ions slow

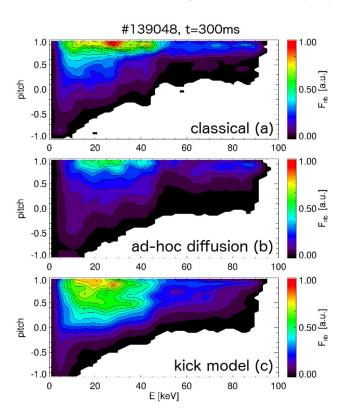


FIG. 4. Fast ion distribution computed by NUBEAM at $t \approx 300$ ms assuming classical fast ion physics and enhanced transport through the *ad-hoc D_b* and kick models. Normalization factor is the same for all distributions.

down in energy and spread in pitch until they are either lost or thermalized.

The addition of enhanced radial diffusivity causes a net depletion in the fast ion population, which appears in the energetic particle distribution function F_{EP} as an overall reduction over the entire energy and pitch range (Fig. 4(b)). If phase space modifications are then introduced through the kick model, more significant differences from the classical case arise (Fig. 4(c)). The distribution broadens significantly at lower energies for instabilities that mainly act on strongly co-passing particles with pitch $p \sim 1$. This populates regions that are otherwise poorly populated, such as trapped and stagnation orbits around $p \sim 0$.

From analysis with the ORBIT code, TAE modes observed in NSTX discharge #139048 have stronger resonances with co-passing fast ions. For example, Fig. 5 illustrates the kick probability $p(\Delta E, \Delta P_\zeta)$ resulting from the $n_{tor}\!=\!4$ perturbation shown in Fig. 2 for ions with energy 75 keV. Resonant wave-particle interactions have increasingly stronger effects as $\mu\!\to\!0$, corresponding to $|p|\to 1$ (i.e., strongly co- or counter-passing particles). Almost no interaction occurs for trapped particles.

The relative change in F_{EP} as predicted by the two models is shown in Fig. 6. The *ad-hoc* diffusivity pushes particles over a broad range of pitch, from regions populated by NB injection at p > 0.6 towards regions of the (E, p) space where no particles are observed in the classical run (cf. Fig. 4(a)). The kick model leads to more localized depletion around the injection pitch, $0.6 \le p \le 0.9$. The region p < 0.4 features a larger increase than in the case of enhanced D_b . These results

FIG. 5. Example of phase space for $E=75\,\mathrm{keV}$ fast ions for NSTX discharge #139048 at $t\approx300\,\mathrm{ms}$. Regions correspond to (1) co-passing, (2) counter-passing, (3) trapped, (4) potato, and (5) stagnation orbits. ²⁵ Colored regions indicate the root-mean-square energy kicks, ΔE_{rms} , computed by ORBIT for a $n_{tor}=4\,\mathrm{TAE}$ mode.

can be understood on the basis of the different effects of TAE resonances vs. pitch angle scattering and slowing down. For example, consider the case shown in Fig. 6(c) with multiple resonances computed for a n = 4 TAE around mid-radius. Resonances are mainly localized at pitch $p \ge 0.4$ and will mostly tend to move particles around the resonant lines that can be identified in the figure. On the other hand, pitch angle scatter and slowing down tend to redistribute particles outside the resonant regions. Eventually, fast ions end up in (E, pitch) regions where resonances are ineffective and from there they continue to scatter and slow down undisturbed. The net effect is the population of phase space zones that may have been previously scarcely populated, such as for $p \le 0.4$ in Figs. 6(b) and 6(c), by fast ions originally located in regions of strong mode-particle interaction. For both transport models, a broad energy range is affected. This is intrinsic in the ad-hoc diffusivity model, which has no energy discrimination in its implementation used in this work. For the kick model, this is a result of the large number of resonances from multiple modes that form a dense net in phase space, cf. Fig. 6(c) and Ref. 32.

Additional details on the changes in distribution function from the three runs are presented in Fig. 7. F_{EP} modifications remain small at energies $E \approx 80\,\mathrm{keV}$, near the injection energy. Here, the EP population is continuously replenished by NB injection, and the source term dominates in the distribution function evolution. At lower energy, $E \approx 20\,\mathrm{keV}$, significant differences in the temporal evolution of F_{EP} are apparent from the departure of average pitch and its broadening with respect to classical simulations. The larger difference with respect to high-energy particles is caused by the cumulative effects of instabilities acting on particles that are slowing down.

The ad-hoc D_b and kick models differ in predicting the average pitch and the width of the distribution (here

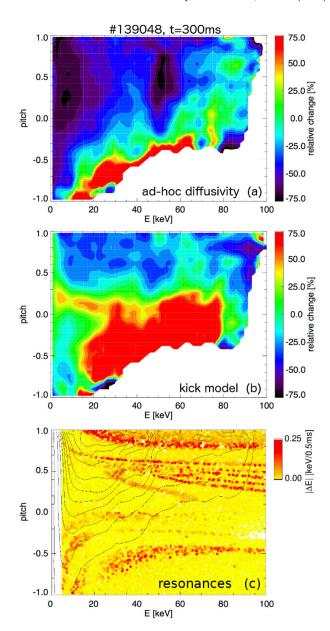


FIG. 6. Relative change of the fast ion distribution function (same shown in Fig. 4) caused by instabilities. Fast ion transport is computed using (a) the ad-hoc D_b model and (b) the kick model. (c) Example of a test particle distribution run with the ORBIT code in the presence of a n = 4 TAE mode around mid-radius. Each dot represents a test particle. Energy change after a 0.5 ms run is color-coded as in the color bar. Solid contour lines show the fast ion distribution (classical case).

quantified as broadening in pitch). Moreover, the kick model shows larger variability in time, depending on the mix and relative amplitude of the modes included in the computation of the kick probability matrix at each specific time.

B. Integrals of F_{EP} : Fast ion density, losses, and NB power to thermal plasma

The effects of modifications of F_{EP} and its temporal evolution propagate to other quantities in whole-discharge simulations, as can be appreciated from Fig. 8. The depletion of the distribution function caused by enhanced transport directly transfers to a reduction in the fast ion density, n_b ,

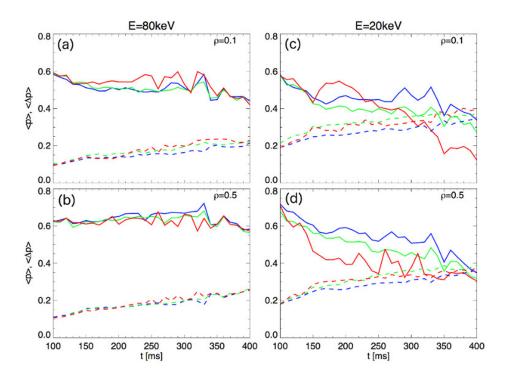


FIG. 7. Time evolution of the average pitch, (solid lines), and average pitch broadening, $\langle \Delta p \rangle$ (dashed lines). Colors refer to classical runs (blue) and enhanced fast ion transport using the $ad\text{-}hoc\ D_b$ (green) or kick (red) models. Panels refer to (a) and (b) 75 < E < 85 keV fast ions and (c) and (d) 15 < E < 25 keV fast ions at two different normalized radii, $\rho = 0.1$ and $\rho = 0.5$.

obtained by integrating F_{EP} over phase space. The radial density gradient also decreases, indicating a flattening of the radial fast ion profile (Fig. 8(a)). A significant flattening with respect to classical simulations is predicted by both transport

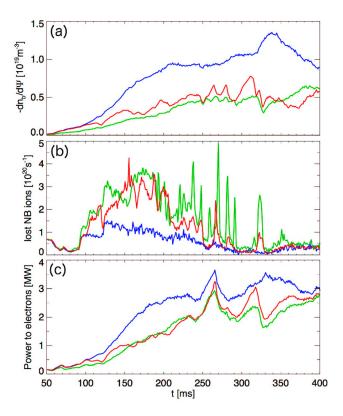


FIG. 8. (a) Radial gradient of the fast ion profile around $\rho = 0.5$ for classical simulations (blue) and assuming enhanced transport through *ad-hoc* diffusivity (green) and the kick model (red). (b) Total fast ion loss rate for the three cases. (c) Total fast ion power damped on the thermal electrons through slowing-down.

models, with shorter time-scale variations resulting from the kick model in response to spikes of the mode amplitude.

Reduced density is accompanied by enhanced EP losses from the core plasma, see Fig. 8(b). The two models predict comparable losses during the initial $\approx 200 \, \mathrm{ms}$ of the discharge. Mode activity is dominated by n=1, 2 AEs in the kick model during this time. After $t \approx 200 \, \mathrm{ms}$, direct losses computed using the kick model gradually decrease toward the classical level. In this case, transport in phase space mainly results in energy redistribution to lower energies and in enhanced charge-exchange losses for particles orbiting through the plasma edge. The ad-hoc diffusive model lacks of energy dependence for D_b , and therefore, the only channel for transport is to increase the losses from the plasma. This is visible from the large spikes in fast ion losses in Fig. 8(b).

Fast ions that are not lost during slowing-down will eventually transfer power to the thermal plasma and thermalize. Focusing on the NB power transferred to the electrons, see Fig. 8(c), results are comparable for the two models. Consistently with the reduced loss rate from Fig. 8(b), the kick model predicts a slightly larger power transferred to the electrons after $t \approx 250 \, \text{ms}$ than the *ad-hoc* diffusive model. Although the difference may appear small, it can still have a significant impact on power balance and thermal transport analyses based on TRANSP results, since local values as a function of radius can vary substantially.³³

To conclude this section, there is evidence that the use of a specific transport model can result in significant differences for the predicted fast ion distribution function. Those differences can propagate to integral quantities such as EP radial density profile and loss rate. More subtle variations can also result from the models such as NB power transfer to the thermal plasma and its radial deposition profile. (More examples of integral quantities computed through the ad-hoc D_b vs kick model can be found in a separate work, cf. Ref.

33). Section IVC shows how the additional information made available by the phase space-resolved transport model can be a powerful tool to assess the consistency of the underlying EP transport assumptions. New physics insight can also be gained, for example, on the instabilities causing the enhanced transport.

C. Consistency of the results from AE power balance analysis

Consider a mode interacting resonantly with some portion of the EP distribution function. For the mode to be unstable, a net positive power must be transferred from the interacting fast ions to the mode, causing the mode amplitude to increase. A simple expression for the time evolution of the mode energy, $E_{\rm w}$, is

$$\frac{dE_w}{dt} = P_{EP} - 2\gamma_{damp} E_w,\tag{2}$$

where P_{EP} is the power from the fast ions to the mode and γ_{damp} is the mode's damping rate. For similarity with the damping term, we take here $P_{EP} \equiv 2\gamma_{gr}E_w$ with γ_{gr} being the growth rate. At small mode amplitude (proportional to $\sqrt{E_w}$), the feedback of the mode on the EP distribution is negligible, so that $\gamma_{gr} = constant$ coincides with the linear growth rate. After this initial *linear* phase, during which the mode grows exponentially in time, the mode starts to affect the region of phase space in which interaction occurs, pushing particles outside that region (cf. Figs. 6(b) and 6(c)). This implies that P_{EP} decreases since γ_{gr} (which is now a function of E_w , in this *non-linear* phase) decreases, eventually leading to saturation with $E_w \approx P_{EP}(E_w)/2\gamma_{damp}$.

The mode evolution for a real case can be more complicated. Each mode can interact with multiple phase space regions, since several resonances can be present even for a single mode. In addition, P_{EP} also depends on EP sources (e.g., from NB injection and EP slowing down) that replenish the fast ion distribution regions depleted by the interaction with the mode. Moreover, the mode damping rate γ_{damp} can vary in time. All this leads to a dynamical balance between mode drive and damping, whether the latter is through damping on the thermal plasma or effective damping by depletion of particles in the resonant phase space regions.

Given all the complications that are present in a real case, it seems plausible to assume that a necessary condition for a mode to be unstable and to reach saturation is given by

$$P_{EP}(E_w) \gtrsim 0 \quad for E_w > 0,$$
 (3)

where $P_{EP}(E_w) \approx 0$ for finite E_w is the minimal condition in the limit $\gamma_{damp} \rightarrow 0$. (A finite γ_{damp} will increase the power P_{EP} required to sustain a finite mode amplitude.)

In its implementation in the NUBEAM module of TRANSP, the kick model does compute the power $P_{EP,j}$ exchanged between fast ions and each of the j=1...N modes provided as input. Pitch angle scattering and slowing down are included as in standard NUBEAM/TRANSP runs. Initial conditions for the EP distribution are set in TRANSP/NUBEAM by the actual NB injection parameters used in the

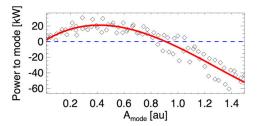


FIG. 9. Example of power transferred from the fast ions to a $n_{tor} = 4$ TAE mode, as computed by the kick model. Mode amplitude is swept with a triangular waveform for $0 \le A_{mode} \le 1.5$. Note the initial increase and successive roll-off of $P_{EP,j}(A_{mode})$, eventually leading to a *negative* power when the mode amplitude exceeds the inferred saturation amplitude, $A_{mode}^{sat} \approx 0.9$ in normalized units.

experiment, providing the time-dependent NB injection voltage and injected power as input. An example is shown in Fig. 9 for the $n_{tor}=4$ TAE mode shown in Fig. 2. To produce those data, the mode amplitude was scanned in the range $0 \le A_{mode}(t) \le 1.5$ with a triangular waveform over time windows of ~ 50 ms. (Amplitudes are normalized to their value at the time mode structures are computed, cf. Fig. 2.) The modulation period of 50 ms is chosen to be sufficiently larger than the typical EP slowing down time (typically 15–30 ms), so that the EP distribution has time to respond to the variations of mode amplitude. (For instance, hysteresis is observed in the EP response to large A_{mode} variations if the

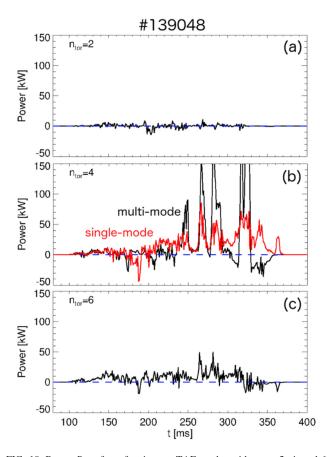


FIG. 10. Power $P_{EP,j}$ from fast ions to TAE modes with $n_{tor} = 2$, 4, and 6 computed by the kick model with all three modes included in the simulation. Panel (b) shows a comparison of $P_{EP,j}$ from multi-mode vs. single-mode simulations.

modulation period is too short, thus leading to ambiguous results for the computed power).

Note the roll-over of P_{EP} in Fig. 9 as the amplitude is increased above $A_{mode} \approx 0.5$. Eventually, P_{EP} becomes negative indicating that the un-physical condition for which the mode *transfers* power back into the EP population is achieved. For this example, the saturation amplitude would then be $A_{mode} \approx 0.9$ assuming $\gamma_{damp} = 0$.

Combined with the condition in Eq. (3), the knowledge of $P_{EP,j}(t)$ provides a powerful tool to verify that the assumptions made in the kick model (e.g., mode amplitude and kick probability) are indeed consistent with the modeled F_{EP} evolution. The computed values of $P_{EP,j}$ for the dominant $n_{tor} = 2$, 4, 6 TAEs used in the simulations discussed in Sections III and IV are shown in Fig. 10. A first observation is that *dominant* modes can be identified. In this case, the $n_{tor} = 4$ TAE accounts for most of the fast ion power going to the modes, followed by the $n_{tor} = 6$ mode. The contribution of the $n_{tor} = 2$ mode appears negligible. Overall, all three modes are characterized by a net positive power, except for short time windows. This indicates that their mode structure and amplitude, combined with the calculated kick probabilities $p(\Delta E, \Delta P_{\zeta})$, are consistent with the modeled scenario.

It is important to note that consistency encompasses all the modes, rather than a single mode. For example, Fig. 10(b) compares the values of $P_{EP,j}$ for the same $n_{tor} = 4$ mode when all other modes are included (referred to as *multi-mode* case) with values obtained excluding the other

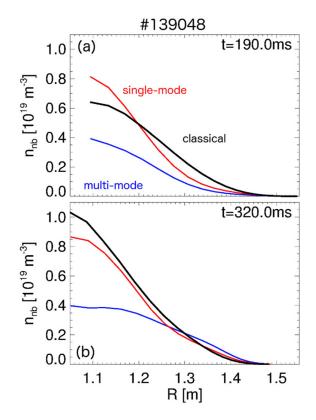


FIG. 11. Fast ion density profile at two different times from multi-vs single-mode simulations, cf. Fig. 10(b). Note the steepening of the density at $t \approx 190\,\mathrm{ms}$ for the single-mode case, which indicates inconsistency of the assumptions on mode amplitude at that time if all other modes are removed from the simulation.

modes (single-mode case), i.e., A_{mode} is set to 0 for all other modes. It can be seen that $P_{EP,j}$ varies considerably for the two cases, which indicates that there exist mutual effects between the selected modes. This can be understood in terms of phase-space overlap of resonances from different modes, which cause instabilities to interact with the same groups of particles. The effects are not negligible and can result in significant variations in the computed profiles. Figure 11 shows the radial fast ion profiles at two different times for the multi- and single-mode cases. When only the $n_{tor} = 4$ TAE is retained, depletion in the fast ion density is much reduced. In some cases (e.g., at $t \approx 190 \,\mathrm{ms}$ in the figure), the absence of other instabilities causes $P_{EP,j}$ to become negative, which results in an unphysical steepening of the profile. Removing the synergy with the other TAEs, the $n_{tor} = 4$ mode would, therefore, be *stable* at that time.

V. SUMMARY AND CONCLUSIONS

Results on EP distribution function predictions through integrated simulations from two fast ion transport models have been compared for a NSTX discharge featuring robust Alfvénic activity. It is concluded that retaining phase space effects does indeed lead to significant variations in the EP distribution and its temporal evolution, which are not captured by simple diffusive models.

Effects of the distortion of the EP distribution can propagate to integral quantities such as radial EP density, loss rate, and power damped to the plasma through thermalization. Although the radial profile of those quantities is quite model-dependent, differences between the two models eventually reduce as further integrations over minor radius are performed, e.g., to compute the total neutron rate or stored energy.

Clearly, more sophisticated transport models, such as the kick model discussed herein, can provide valuable information to assess the consistency of the simulations. For example, the computed power exchanged between fast ions and instabilities can be used as an indicator to verify the initial assumptions on the modes responsible for enhanced EP transport and the overall consistency of the simulations.

The improved treatment of EP transport by instabilities can make integrated simulations more reliable, at the expenses of increased complexity of the underlying analysis of instabilities and their stability properties. The final choice between EP transport models used in the simulations should therefore be based on the expectations for simulation's output—in short, whether *global* performance indicators (e.g., neutron rate, stored energy, or overall NB-CD efficiency) or more details on quantities such as EP distribution function and/or radial profiles of EP-related quantities are required.

On a longer term, an important goal for physics-driven reduced models integrated in codes such as TRANSP is to improve the fidelity of predictions for future scenarios, in addition to the analysis of present discharges. The implementation of diagnostics in NUBEAM/TRANSP to infer the power exchanged between fast ion and instabilities provides a means to enable such predictions. Assuming the damping rate of each mode is known, mode amplitudes can be computed from Eq. (2) combined with the condition in Eq. (3).

This method is being tested by iteratively computing the amplitudes vs time based on the power balance. If successful, a similar scheme may be implemented directly in TRANSP to update the mode amplitudes during the simulation. Results will be presented in a dedicated publication.

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- ¹A. Fasoli, C. Gormezano, H. L. Berk, B. N. Breizman, S. Briguglio, D. S. Darrow, N. N. Gorelenkov, W. W. Heidbrink, A. Jaun, S. V. Konovalov, R. Nazikian, J.-M. Noterdaeme, S. Sharapov, K. Shinoara, D. Testa, K. Tobita, Y. Todo, G. Vlad, and F. Zonca, Nucl. Fusion 47, S264 (2007).
- W. W. Heidbrink and G. J. Sadler, Nucl. Fusion 35, 243 (1995).
 N. N. Gorelenkov, S. D. Pinches, and K. Toi, Nucl. Fusion 54, 125001
- (2014).
- ⁴D. A. Spong, B. A. Carreras, and C. L. Hedrick, Phys. Fluids B 4, 3316 (1992).
- ⁵S. Briguglio, G. Vlad, F. Zonca, and C. Kar, Phys. Plasmas 2, 3711 (1995).
- ⁶W. Park, E. V. Belova, G. Y. Fu, X. Z. Tang, H. R. Strauss, and L. E. Sugiyama, Phys. Plasmas **6**, 1796 (1999).
- ⁷J. Lang, Y. Chen, S. E. Parker, and G. Fu, Phys. Plasmas **16**, 102101 (2009).
- ⁸Y. Todo, K. Shinohara, M. Takechi, and M. Ishikawa, Phys. Plasmas 12, 012503 (2005).
- ⁹M. Schneller, P. Lauber, R. Bilato, M. Garcia-Munoz, M. Brudgam, S. Gunter, and ASDEX Upgrade Team, Nucl. Fusion **53**, 123003 (2013).
- ¹⁰K. Ghantous, N. N. Gorelenkov, H. L. Berk, W. W. Heidbrink, and M. A. Van Zeeland, Phys. Plasmas 19, 092511 (2012).
- ¹¹W. W. Heidbrink, M. A. Van Zeeland, M. E. Austin, E. M. Bass, K. Ghantous, N. N. Gorelenkov, B. A. Grierson, D. A. Spong, and B. J. Tobias, Nucl. Fusion 53, 093006 (2013).
- ¹²R. E. Waltz and E. M. Bass, in 13th IAEA-TM EP Meeting, Beijing, China, 2013.
- ¹³N. N. Gorelenkov, W. W. Heidbrink, G. J. Kramer, J. Lestz, M. Podestà, M. A. Van Zeeland, and R. B. White, in 14th IAEA Technical Meeting on Energetic Particles in magnetic confinement systems, O-06, Vienna, Austria, 2015.
- ¹⁴M. Ono, S. M. Kaye, Y.-K. M. Peng, G. Barnes, W. Blanchard, M. D. Carter, J. Chrzanowski, L. Dudek, R. Ewig, D. Gates, R. E. Hatcher, T. Jarboe, S. C. Jardin, D. Johnson, R. Kaita, M. Kalish, C. E. Kessel, H. Kugel, R. Maingi, R. Majeski, J. Manickam, B. McCormack, J. Menard, D. Mueller, B. Nelson, B. Nelson, C. Neumeyer, G. Oliaro, F. Paoletti, R.

- Parsells, E. Perry, N. Pomphrey, S. Ramakrishnan, R. Raman, G. Rewoldt, J. Robinson, A. L. Roquemore, P. Ryan, S. Sabbagh, D. Swain, E. J. Synakowski, M. Viola, M. Williams, J. R. Wilson, and NSTX Team, Nucl. Fusion 40, 557 (2000).
- ¹⁵J. E. Menard, S. Gerhardt, M. Bell, J. Bialek, A. Brooks, J. Canik, J. Chrzanowski, M. Denault, L. Dudek, D. A. Gates, N. Gorelenkov, W. Guttenfelder, R. Hatcher, J. Hosea, R. Kaita, S. Kaye, C. Kessel, E. Kolemen, H. Kugel, R. Maingi, M. Mardenfeld, D. Mueller, B. Nelson, C. Neumeyer, M. Ono, E. Perry, R. Ramakrishnan, R. Raman, Y. Ren, S. Sabbagh, M. Smith, V. Soukhanovskii, T. Stevenson, R. Strykowsky, D. Stutman, G. Taylor, P. Titus, K. Tresemer, K. Tritz, M. Viola, M. Williams, R. Woolley, H. Yuh, H. Zhang, Y. Zhai, A. Zolfaghari, and NSTX Team, Nucl. Fusion 52, 083015 (2012).
- ¹⁶R. Hawryluk, in *Physics of plasmas close to thermonuclear conditions* (Proceedings from the International School of Plasma Physics, Villa Monastero, Varenna, Italy, 1979).
- ¹⁷See http://w3.pppl.gov/~pshare/help/transp.htm for "TRANSP webpage," for more details on the TRANSP code.
- ¹⁸R. J. Goldston, D. C. McCune, H. H. Towner, S. L. Davis, R. J. Hawryluk, and G. L. Schmidt, J. Comput. Phys. 43, 61 (1981).
- ¹⁹A. Pankin, D. McCune, R. Andre, G. Bateman, and A. Kritz, Comput. Phys. Commun. **159**, 157 (2004).
- ²⁰M. Podestà, M. Gorelenkova, and R. B. White, Plasma Phys. Controlled Fusion 56, 055003 (2014).
- ²¹R. B. White and M. S. Chance, Phys. Fluids **27**, 2455 (1984).
- ²²C. Z. Cheng, Phys. Rep. **211**, 1–51 (1992).
- ²³G. Y. Fu, C. Z. Cheng, and K. L. Wong, Phys. Fluids B 5, 4040 (1993).
- ²⁴N. N. Gorelenkov, C. Z. Cheng, and G. Y. Fu, Phys. Plasmas 6, 2802 (1999).
- ²⁵R. B. White, *The Theory of Toroidally Confined Plasmas*, 3rd ed. (Imperial College Press, London, UK, 2014).
- ²⁶S. S. Medley and A. L. Roquemore, Rev. Sci. Instrum. **75**, 3625 (2004).
- ²⁷S. Kubota, X. V. Nguyen, W. A. Peebles, L. Zeng, and E. J. Doyle, Rev. Sci. Instrum. 72, 348 (2001).
- ²⁸N. A. Crocker, W. A. Peebles, S. Kubota, J. Zhang, R. E. Bell, E. D. Fredrickson, N. N. Gorelenkov, B. P. LeBlanc, J. E. Menard, M. Podestà, S. A. Sabbagh, K. Tritz, and H. Yuh, Plasma Phys. Controlled Fusion 53, 105001 (2011).
- ²⁹E. D. Fredrickson, N. A. Crocker, R. E. Bell, D. S. Darrow, N. N. Gorelenkov, G. J. Kramer, S. Kubota, F. M. Levinton, D. Liu, S. S. Medley, M. Podestà, K. Tritz, R. B. White, and H. Yuh, Phys. Plasmas 16, 122505 (2009).
- ³⁰E. D. Fredrickson, N. A. Crocker, D. S. Darrow, N. N. Gorelenkov, G. J. Kramer, S. Kubota, M. Podestà, R. B. White, A. Bortolon, S. P. Gerhardt, R. E. Bell, A. Diallo, B. P. LeBlanc, F. M. Levinton, and H. Yuh, Nucl. Fusion 53, 013006 (2013).
- ³¹E. M. Carolipio, W. W. Heidbrink, C. B. Forest, and R. B. White, Nucl. Fusion 42, 853 (2002).
- ³²R. B. White, N. N. Gorelenkov, W. W. Heidbrink, and M. A. V. Zeeland, Plasma Phys. Controlled Fusion **52**, 045012 (2010).
- ³³M. Podestá, M. Gorelenkova, E. D. Fredrickson, N. N. Gorelenkov, and R. B. White, "Effects of energetic particle phase space modifications by instabilities on integrated modeling," Nucl. Fusion (to be published).