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NSTX Contributions in the USDOE FESAC Fusion Energy Development Plan

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For the NSTX National Team

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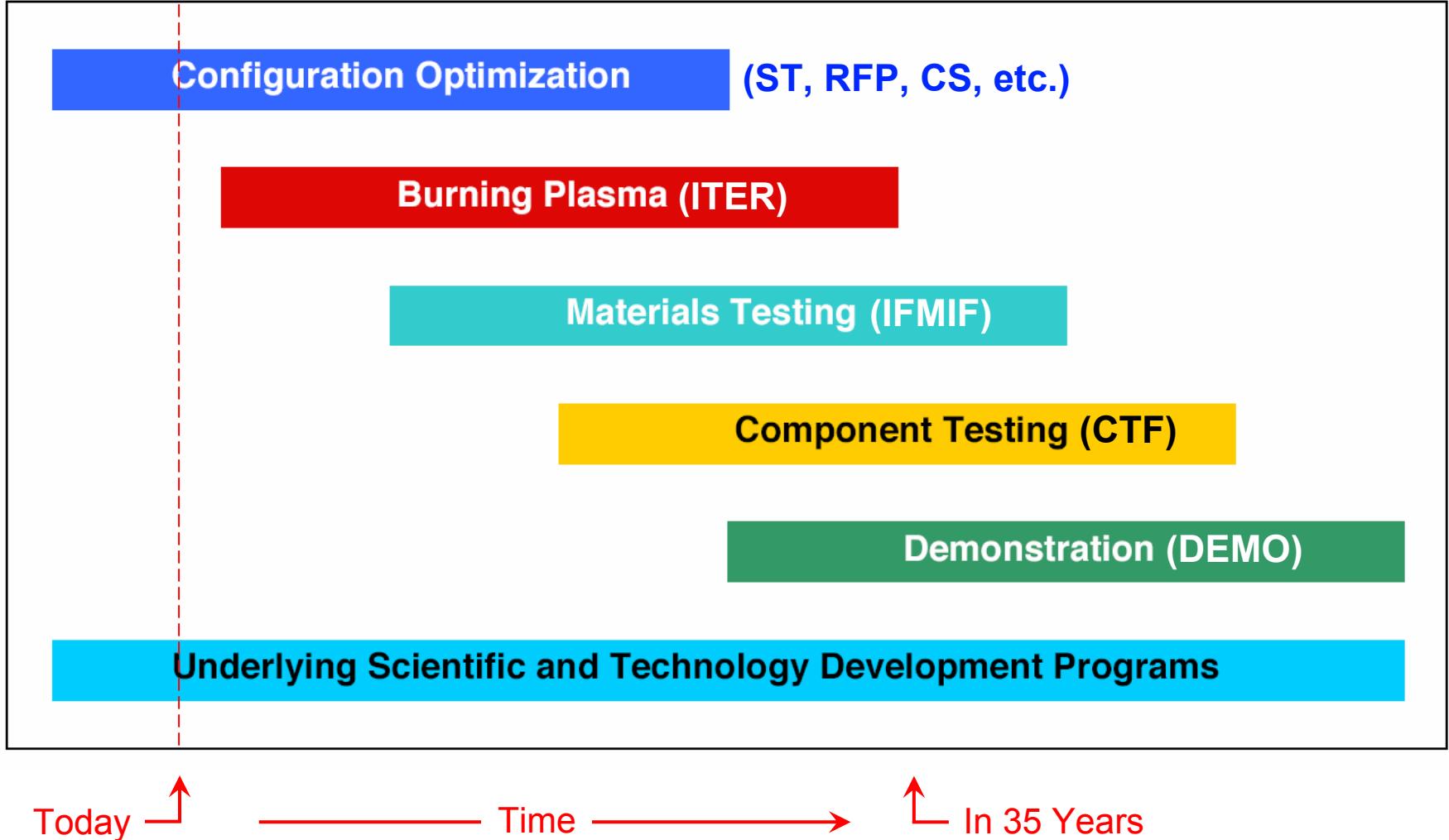
ST Research Offers Exciting Opportunities to Expand Plasma Science & Provide Attractive Next Steps



- FESAC Proposes Fusion Development Plan
- ST research in plasma science for energy development in this plan
- Physics contributions from present experiments
- Additional contributions needed at the 5 – 10 MA level
- NSTX 5-year plan to carry out this exciting research

FESAC Panel Articulates Key Components Needed for Developing Net Fusion Electricity in 35 Years

(“A Plan for the Development of Fusion Energy,” final report to FESAC, March 5, 2003)



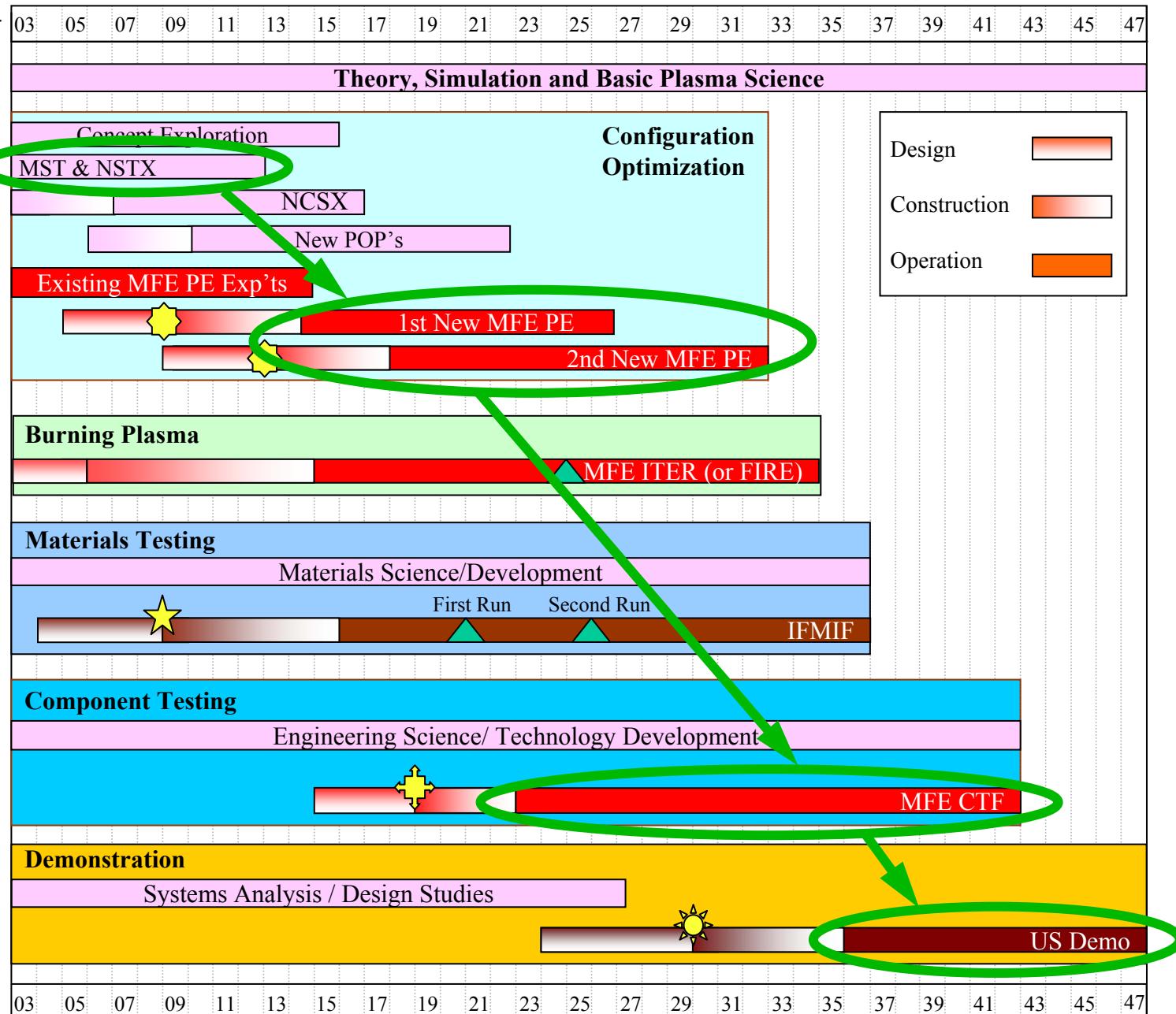
Spherical Torus Is an Integral Part of the Development Plan

Fiscal Year 03 05 07 09 11 13 15 17 19 21 23 25 27 29 31 33 35 37 39 41 43 45 47

MFE Detail and Dependencies

Key Decisions:

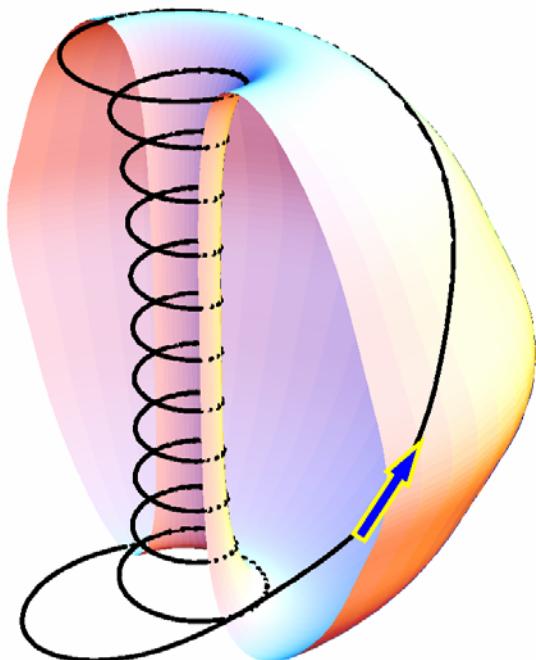
- ◆ MFE PEs
- ★ IFMIF
- ◆ MFE or IFE
- ◆ Demo



Spherical Torus Offers High β Plasmas with Strong Toroidicity & High Safety Factor ($q_{\text{edge}} \sim 10$)



**Spherical Torus provides
Scientifically Interesting
plasmas**



Extended Physics parameter space
available for plasma science:

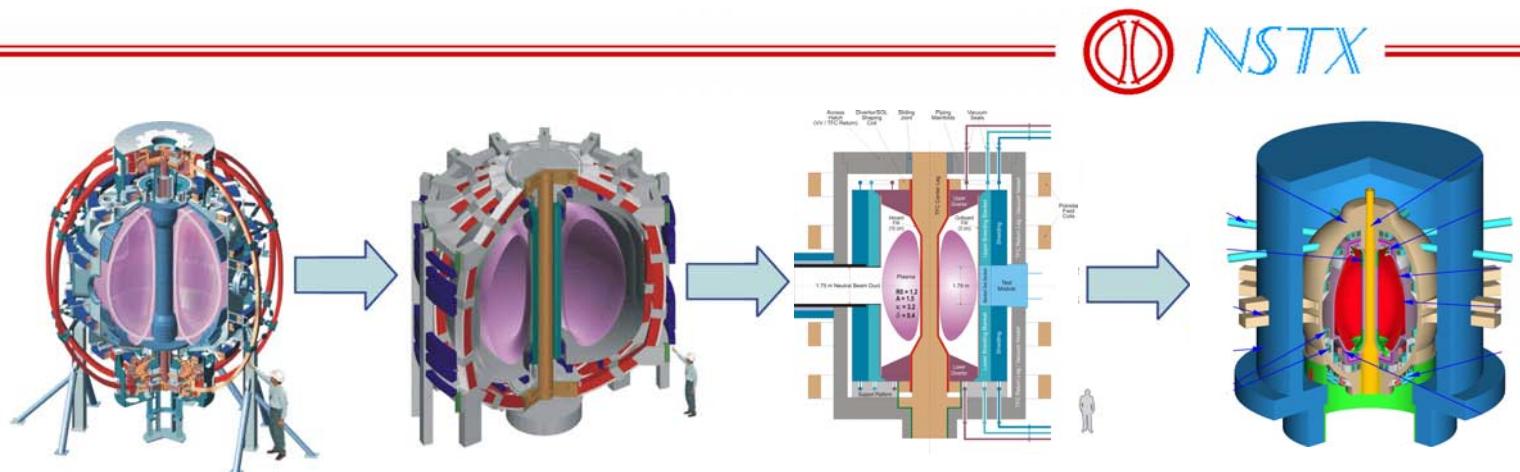
- High β_T ($\leq 40\%$) & central β_0 ($\sim 100\%$)
- Strong plasma shaping & self fields
($A \geq 1.27$, $\kappa \leq 2.5$, $B_p/B_t \sim 1$, $q_{\text{edge}} \sim 10$)
- Small plasma size relative to gyro-radius
($a/\rho_i \sim 30-50$)
- Large mirror in core & edge B field ($f_T \rightarrow 1$)
- Large plasma flow ($M_A = V_{\text{rotation}}/V_A \leq 0.3$)
- Large flow shearing rate ($\gamma_{ExB} \leq 10^6/\text{s}$)
- Supra-Alfvénic fast ions ($V_{\text{fast}}/V_A \sim 4-5$)
- High dielectric constant ($\epsilon = \omega_{pe}^2/\omega_{ce}^2 \sim 50$)

ST Research Studies the Expanded Physics Basis to Optimize Future Steps in This Plan



Plasma Science of Expanded Parameter Space	⇒	Optimized Toroidal Fusion Steps
1) Solenoid-free Startup	⇒	Simplified design, reduced operating cost
2) Reduced turbulence	⇒	Smaller unit size for sustained fusion burn
3) Stable high β_T & β_0	⇒	Lowered magnetic field and device costs
4) Effective wave-energetic particle-plasma interaction	⇒	Efficient fusion α particle, neutral beam, & RF heating
5) Dispersed plasma fluxes	⇒	Survivable plasma facing components
6) Attractive sustained burning plasma properties	⇒	Steady state fusion power source

ST May Lead to Cost-Effective Steps to Fusion Energy



Device	NSTX		NSST		CTF		DEMO
Mission	Proof of Principle		Performance Extension		Energy Development, Component Testing		Practicality of Fusion Electricity
R (m)	0.85		1.5		~1.2		~3.4
a (m)	0.65		0.9		~0.8		~2.4
κ, δ	2.5, 0.8		2.7, 0.6		~3, ~0.4		~3.2, ~0.5
I_p (MA)	1.5	1	10	5	~11		~30
B_T (T)	0.6	0.3	2.6	1.1	~2.2		~1.8
Pulse (s)	1	5	5	50	Steady state		Steady state
P_{fusion} (MW)	–		50	10	~70	~280	~3000
W_L (MW/m ²)	–		–		~1	~4	~4
TF coil	Multi-turn		Multi-turn		Single-turn		Single-turn

Physics Basis for ℓ_i , n_G , β 's, & High Rational q Can be Tested at Relevant Levels at ~1 MA



	<i>NSTX</i>	<i>NSST</i>	<i>CTF</i>	<i>DEMO</i>
Solenoid-free startup				
Internal inductance, ℓ_i	0.5	0.25 – 0.5	~0.25	~0.13
Internal poloidal flux $\propto \ell_i R_0 I_p$ (m-MA)	0.43	1.9 – 7.5	~3.6	~10
Poloidal field energy $\propto \ell_i R_0 I_p^2$ (m-MA ²)	0.43	9.4 – 75	~43	~150
Stable high β's				
Nominal Greenwald density, n_G	~0.5	~0.5	~0.5	~0.6
Beta normal, β_N	≤ 8	8 – 4	4 – 8	~8
Average toroidal beta, β_T	0.2 – 0.4	0.4 – 0.2	0.2 – 0.4	~0.5
Beta gradient, β_T' (/m)	0.25 – 0.5	0.26 – 0.13	0.13 – 0.26	~0.06
Aligned bootstrap current fraction, f_{BS}	0.7	0.8 – 0.2	0.5 – 0.8	~0.9
Resonant field errors / B_T (%)	~ 0.1	~ 0.03	~ 0.01	< 0.01
Rational q values in plasma	$\geq 1 – 3$	$\geq 1 – 3$	> 2 – 3	> 3

Physics Basis for Poloidal Flux & Energy, Bootstrap Current, & Error Fields Will Require Larger Progress

Physics Basis for Collisionality, Ion τ_E , M_{Alfvén}, & Flow Shear Can be Tested at Relevant Levels



	<i>NSTX</i>	<i>NSST</i>	<i>CTF</i>	<i>DEMO</i>
Reduced turbulence & improved confinement				
Average temperature (keV)	~1	4 – 8	~10	~20
Average collisionality, v*	0.05 – 0.2	0.03	~0.02	~0.02
Thermal ion minor radius, a/p _i	40	80 – 120	~100	~150
Ion confinement neoclassical factor, H _{Neoc}	~1	~1	~1	~1
Electron confinement H-mode factor, H _{98e}	~0.7	~1	~1	~1
Alfvén Mach number, M _A = V _{Plasma} /V _{Alfvén}	0.3	~0.3	~0.3	~0.1
Flow shearing rate (10 ⁵ /s)	1 – 10	1 – 10	1 – 10	0.3 – 3

Physics Basis for High Temperature, Normalized Radius, & Electron τ_E Will Require Larger Progress

Physics Basis for High $\omega_{pe}^2/\omega_{ce}^2$, Normalized Fast Ion Radius & Velocity Can be Tested at Relevant Levels



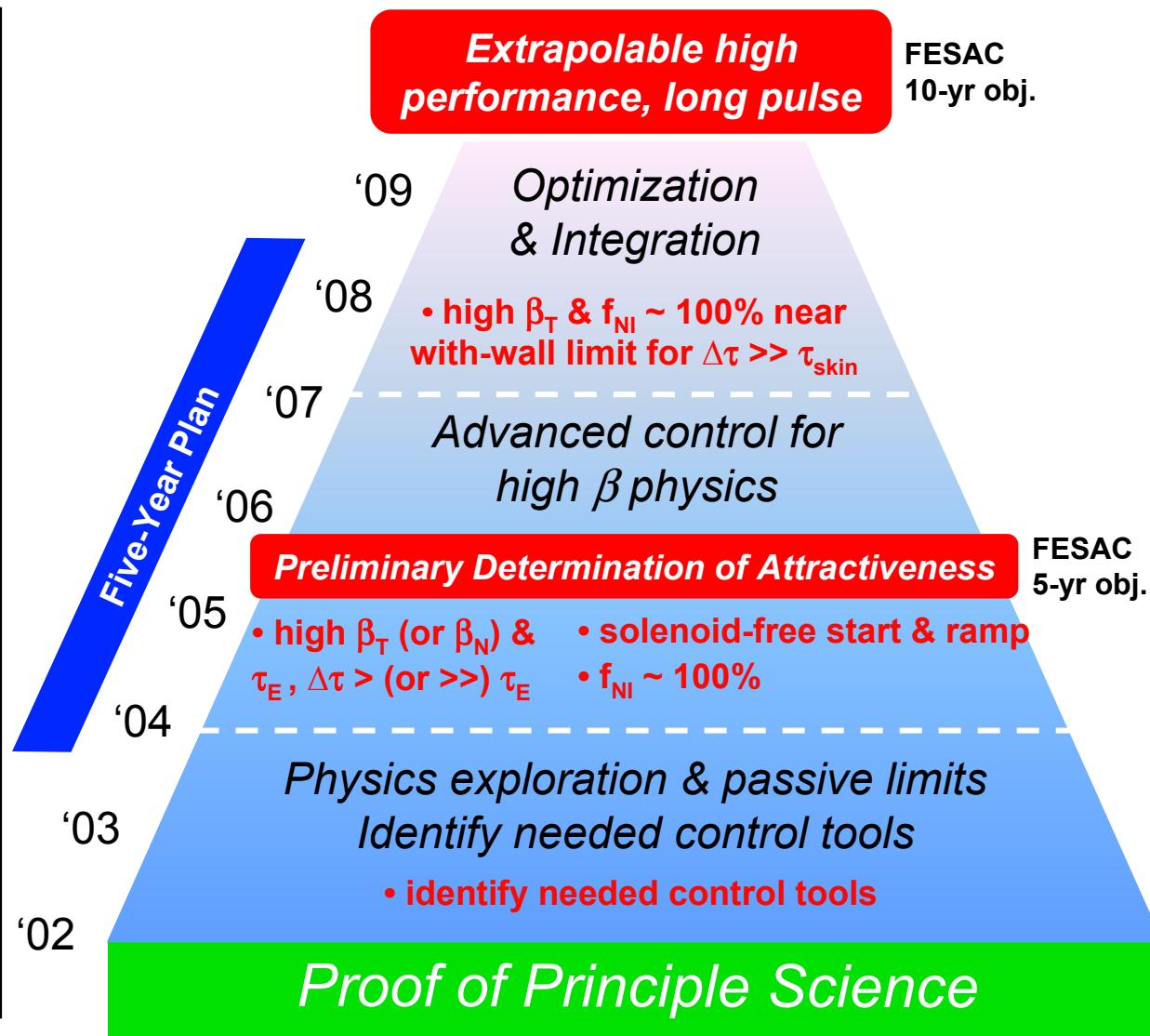
	<i>NSTX</i>	<i>NSST</i>	<i>CTF</i>	<i>DEMO</i>
Effective heating and sustainment				
$\omega_{pe}^2/\omega_{ce}^2$	50	50 – 20	~ 20	~ 25
Beam ion minor radius, a/ρ_{Beam}	5	15 – 22	~ 22	~ 34
Fusion α minor radius, a/ρ_{α}	–	N/A – 6	~ 6	~ 14
$V_{\text{Beam}}/V_{\text{Alfvén}}$	4	1	~ 1	~ 1
$V_{\alpha}/V_{\text{Alfvén}}$	–	N/A – 4	~ 4.5	~ 5
Dispersed plasma fluxes				
P/R (MW/m), $f_{\text{rad}} = 0.5$	8	13 – 20	20 – 40	87
Integrated attractive operations				
$\tau_{\text{pulse}}/\tau_{\text{skin}}$	~ 3	~ 10 – 0.5	→ ∞	→ ∞

Physics Basis for High Heat Flux & Integrated Steady State Operations Will Require Larger Progress

Proposed 5-Year Research Aims to Demonstrate Long Pulse, High Performance Plasma Operations



- **5-year goals**
 - Determine attractiveness
 - Establish science basis for extrapolable high performance and long pulse
 - Database for next PE step (NSST), and in turn for CTF & DEMO
- **Supporting**
 - Implement new key diagnostics
 - Advance control tools & facility upgrades
 - Carry out theory, analyses & modeling



Exciting Diagnostic and Facility Upgrades are Proposed to Support Research



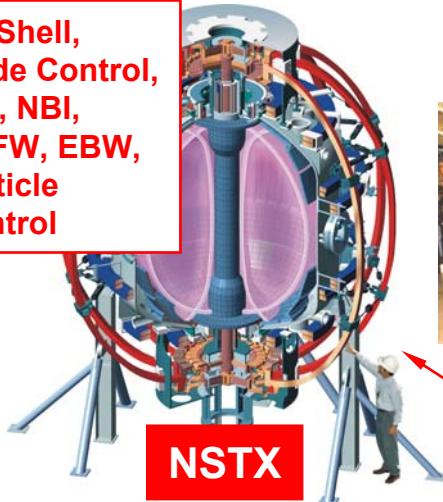
Diagnostics	Facility
MHD <ul style="list-style-type: none">– EBW radiometer, fast ΔT_e– MSE/CIF, LIF polarimeter [Nova]	Very High β <ul style="list-style-type: none">– Ex-vessel field and mode control coils [CU]– Modification of PF1A ($k=2.6$, $\delta=0.6$)– Active mode control systems [CU]
Transport & Turbulence <ul style="list-style-type: none">– High & low-k μ-wave scattering [UCLA, UCD]– μ-wave imaging reflectometer [UCD]– GPI – Planar LIF edge fluctuations [C-Mod, DIII-D, Nova, PSI, SBIR]	CD, MHD, Integrated Scenarios <ul style="list-style-type: none">– EBW (1→4 MW source power) [VLT, MIT, ORNL]
Edge & Divertor <ul style="list-style-type: none">– Divertor laser Thomson scattering	Startup <ul style="list-style-type: none">– EBW– CHI absorber control coils– Outboard PF-only induction
Astrophysics & Diagnostic Development <ul style="list-style-type: none">– X-ray imaging crystal spectrometer [LLNL, Chandra, C-mod, KSTAR, Adv. Diagnostics Program]	Particle & Edge Plasma Control <ul style="list-style-type: none">– Cryopumps– Lithium pellets, coating, flowing surface module [VLT-PFC, CDX-U]

Worldwide Collaboration is a Hallmark of ST Research

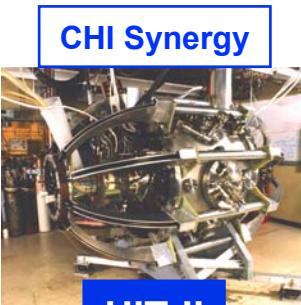


① Concept Exploration (~0.3 MA)

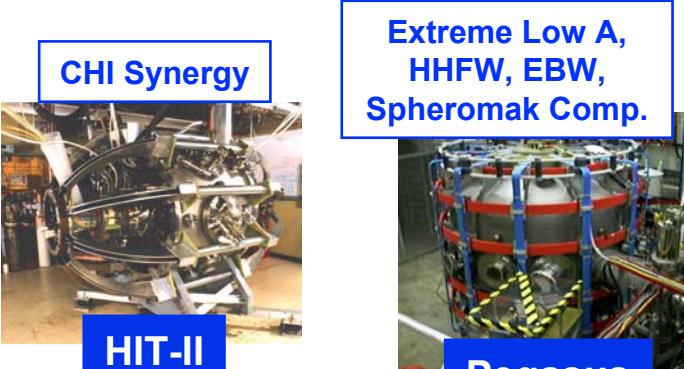
Cu Shell,
Mode Control,
CHI, NBI,
HHFW, EBW,
Particle
Control



NSTX



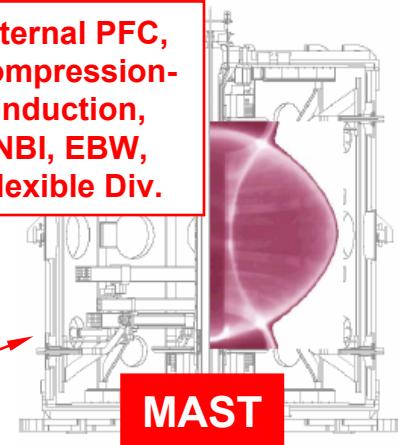
HIT-II



② Proof of Principle (~1 MA)

Extreme Low A,
HHFW, EBW,
Spheromak Comp.

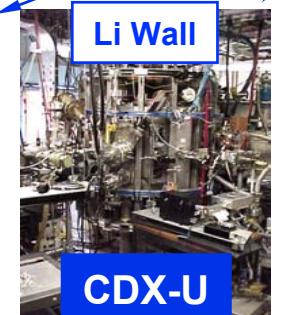
Internal PFC,
Compression-
Induction,
NBI, EBW,
Flexible Div.



MAST



SUNLIST

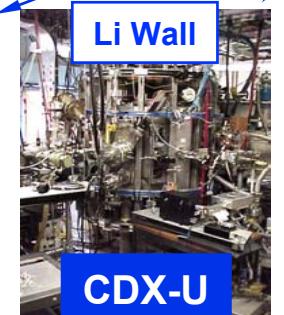


CDX-U



Advanced
Diagnostics

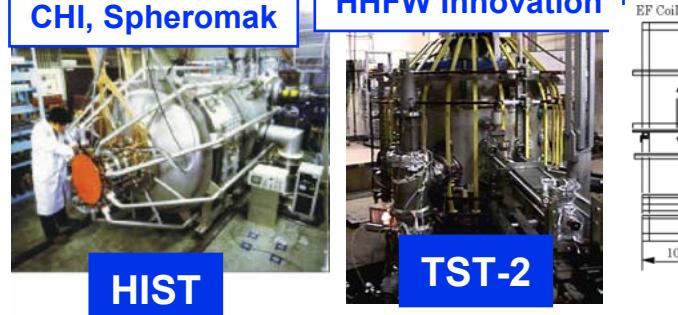
Li Wall



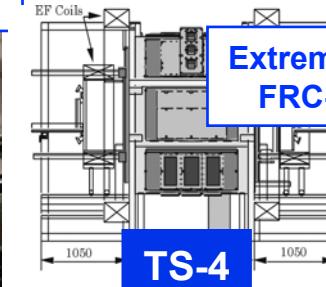
HIST



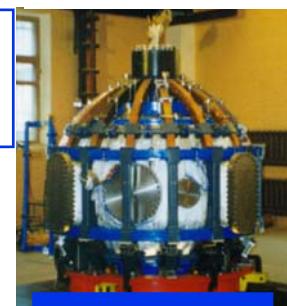
ECH startup,
HHFW Innovation



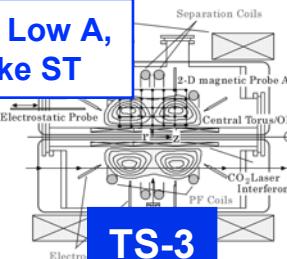
TST-2



TS-4



Globus-M



TS-3

ST Research Offers Exciting Opportunities to Expand Plasma Science & Provide Attractive Next Steps



- FESAC of USDOE articulated a plan to deliver net fusion electricity in 35 years, in which ST is integral part
- ST research studies an expanded physics basis to optimize future steps in this plan: NSST, CTF
- Physics basis for many key topics can be tested in present ST experiments
- Physics basis for additional key topics will require tests at the 5 – 10 MA level
- The NSTX national team is part of worldwide ST community to carry out this exciting research

NSTX National Team & Contributors

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