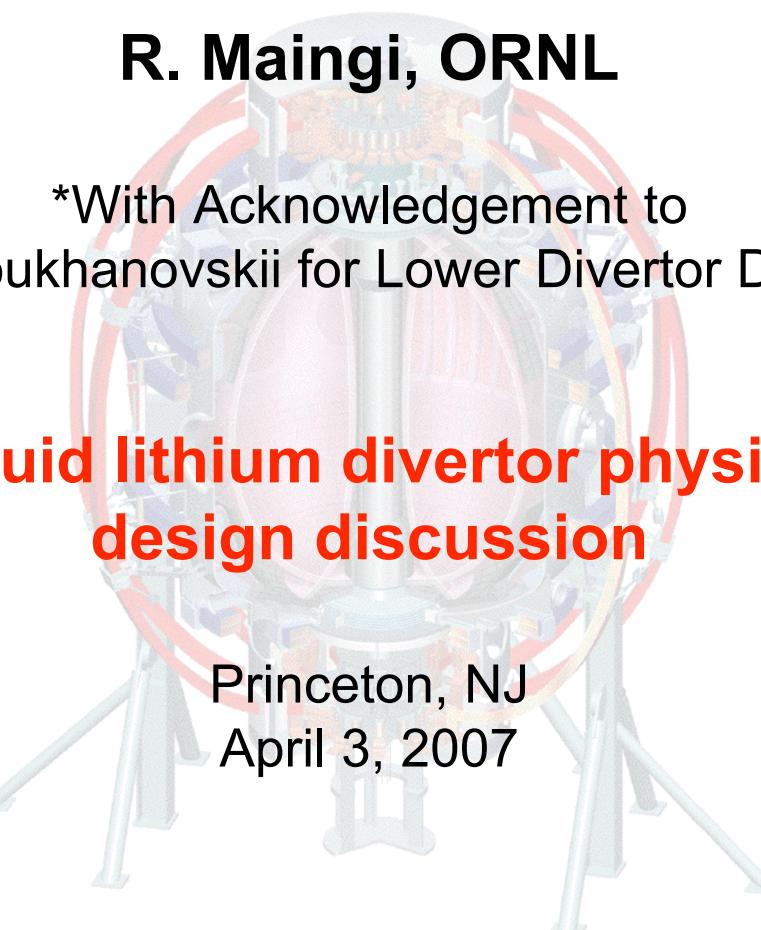


Liquid Lithium Divertor 0-D Pumping Projections and Sensitivities

R. Maingi, ORNL

*With Acknowledgement to
V.A. Soukhanovskii for Lower Divertor D_α data



**Liquid lithium divertor physics
design discussion**

Princeton, NJ
April 3, 2007

Motivation and Technique



- Desire predictive models for effect of pumping on NSTX edge plasma
 - Provide means for comparing density control schemes, e.g. different Lithium tray design parameters (or even in-vessel cryopumping)
 - Should be compared with other experiments and more detailed calculations
- Consider simple recycling model to evaluate examples of each scheme
 - DIII-D data from first cryopump in 1993
 - CDX-U data from liquid Lithium
- Goal: Predict range of reduction in edge density in H-mode

Pumping calculations will help specify the LLD design parameters



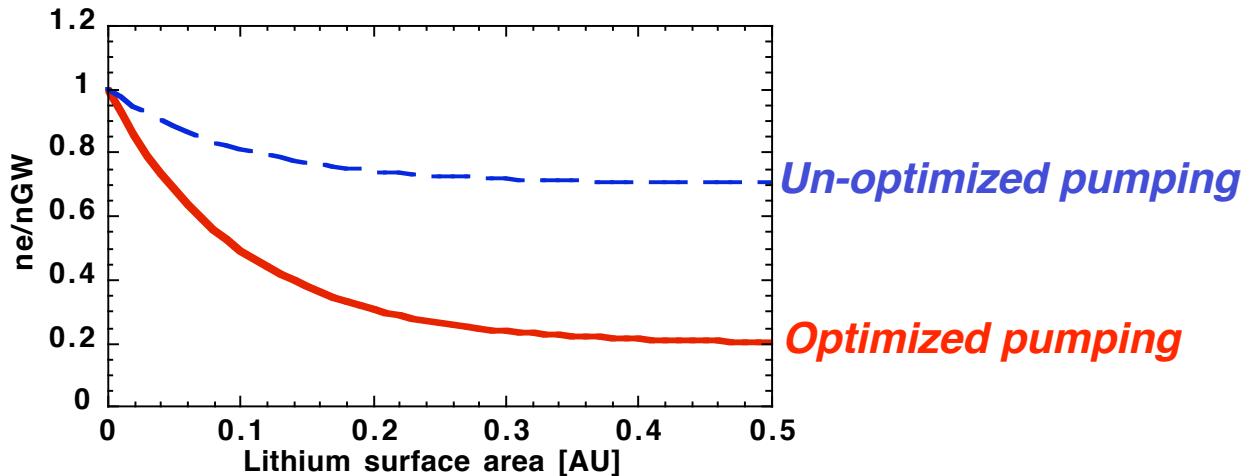
- **0-D calculations presented in this talk:**
 - Parameterized as ratio of pump to core fueling probabilities
 - Requires an assumed relation between pump probability and lithium surface area
- 1-D calculations
 - Onion-skin OEDGE type, requires assessment for NSTX
- 2-D fluid calculations (model)
 - T. Rognlien did NSTX calculations in the past for ALPS/APEX
- 2-D fluid + lithium transport calculations (model)
 - T. Rognlien/J. Brooks did NSTX calcs in the past for ALPS/APEX
- 2-D fluid plasma (data-constrained base case)
 - G. Porter, L. Owen, and R. Maingi have done these for DIII-D
- 2-D fluid plasma + kinetic neutrals (data-constrained base case)
 - L. Owen, M. Rensink, and R. Maingi have done these for DIII-D

Calculations needed for LLD Tray Design Specification



- The following LLD design parameters need to be specified (target: April 15, 2007):
 - 1) Tray Width
 - 2) Tray Major Radius R_{tray}
 - 3) Number of tray segments, gap size(s) between segments, and clocking of segments ($\phi_{\min} - \phi_{\max}$)
- Minimum density will depend on tray-OSP distance

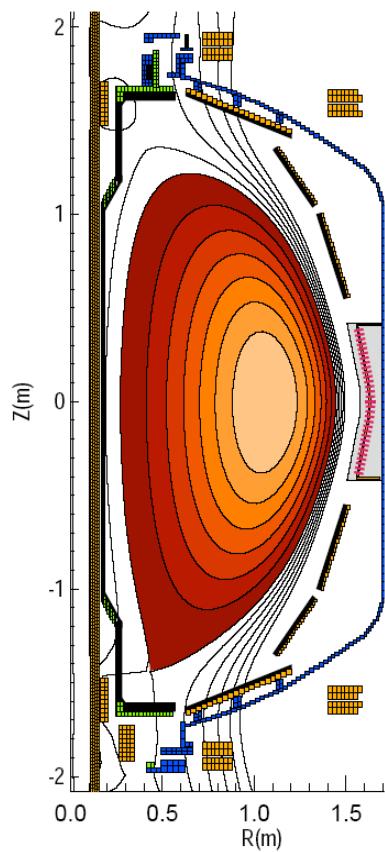
*Large distance between
OSP and LLD radius*



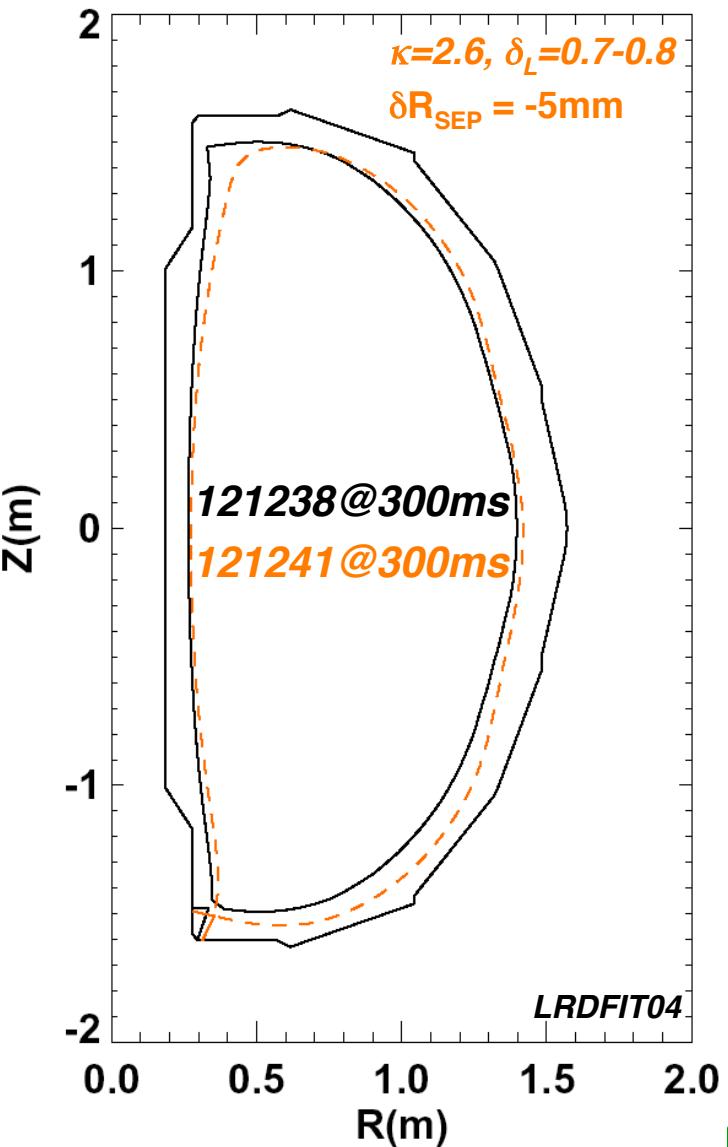
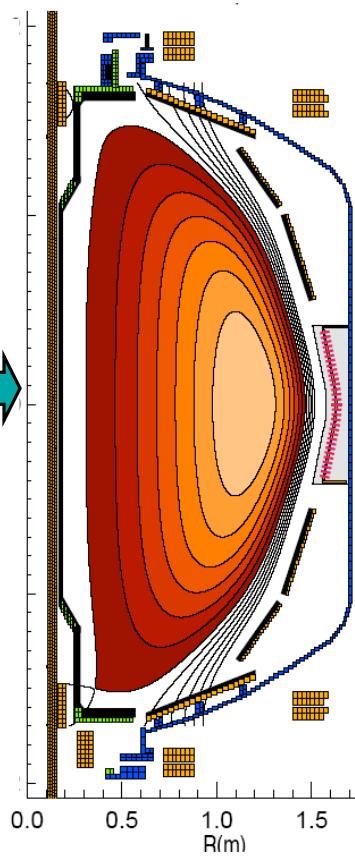
Discharges #116318 @ 0.6 sec and #121238 @ 0.3 sec used for design calculations



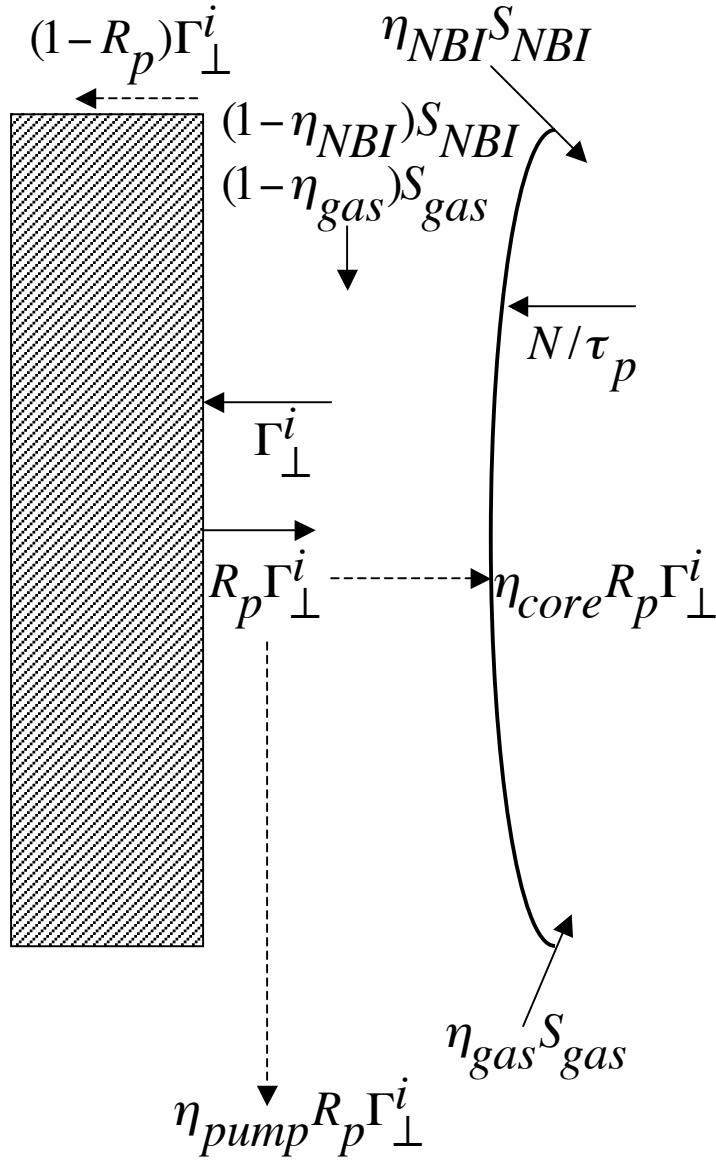
Existing #116313
 $\kappa = 2.3, \delta_{X-L} = 0.75$
 $\delta R_{SEP} = -1\text{cm}$



New target shape
 $\kappa = 2.6, \delta_{X-L} = 0.85$
 $\delta R_{SEP} = -2\text{mm}$



Particle Balance and Recycling Model



- Consider core and SOL particle content equations

$$\frac{dN}{dt} = \eta_{NBI} S_{NBI} + \eta_{gas} S_{gas} - \frac{N}{\tau_p} + \eta_{core} R_p \Gamma_{\perp}^i$$

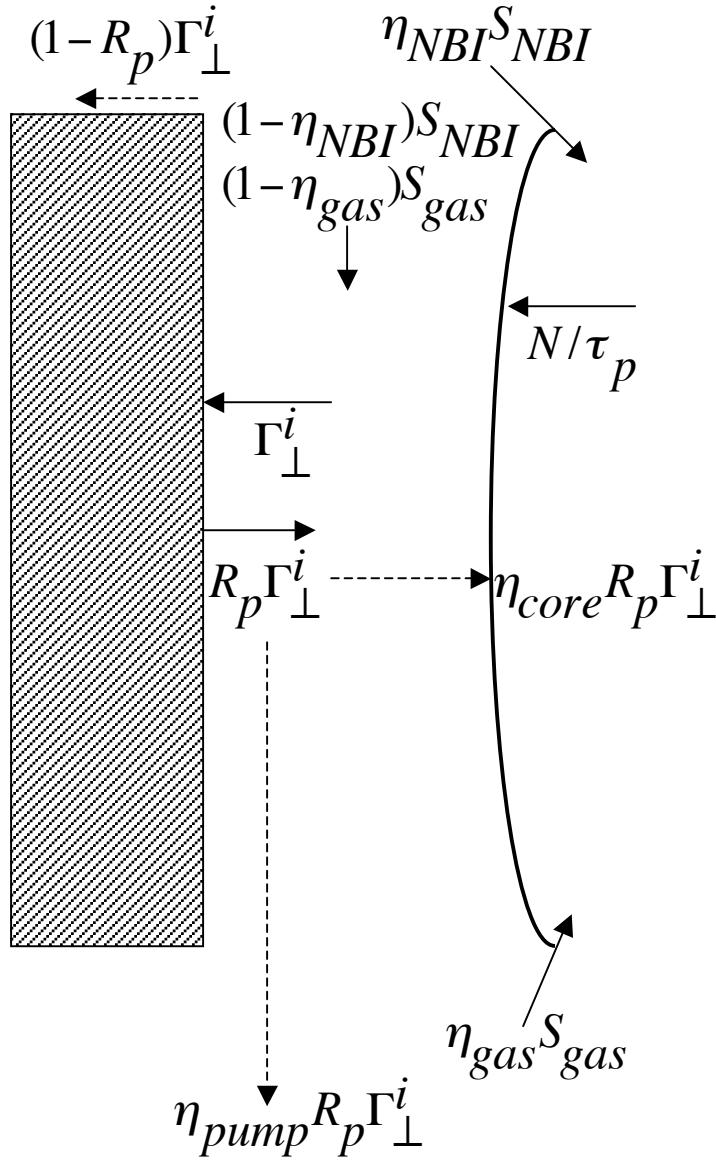
$$\frac{dN_i^{SOL}}{dt} + \frac{dN_0^{SOL}}{dt} = (1 - \eta_{NBI}) S_{NBI} + (1 - \eta_{gas}) S_{gas} + \frac{N}{\tau_p} - (1 - R_p) \Gamma_{\perp}^i - R_p \Gamma_{\perp}^i (\eta_{pump} + \eta_{core})$$

- Assume SOL neutral and ion density in steady state

$$\frac{dN}{dt} = (1 + \beta - \beta \eta_{NBI}) \eta_{NBI} S_{NBI} + (1 + \beta - \beta \eta_{gas}) \eta_{gas} S_{gas} - \frac{N(1 - \beta)}{\tau_p}, \text{ where}$$

$$\beta = R_p \eta_{core} / \left[(1 - R_p) + R_p (\eta_{pump} + \eta_{core}) \right]$$

Simplified Particle Balance and Recycling Model



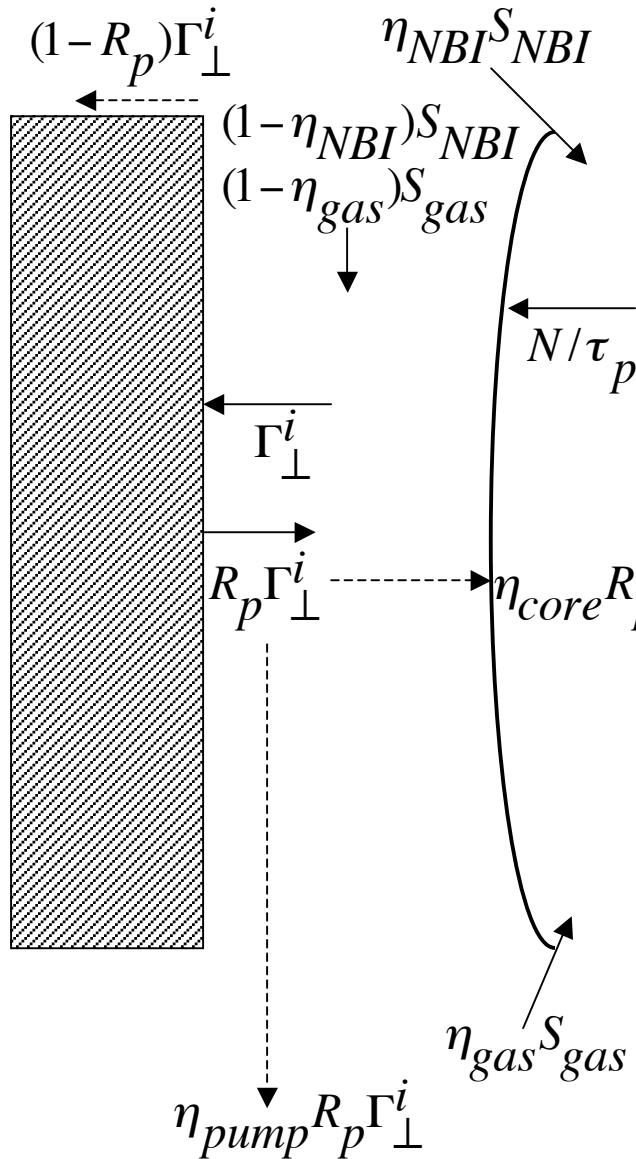
- Define $\tau_p^* = \tau_p/(1-\beta)$
 - Steady state: $\tau_p^* = N/(S_{NBI} + S_{gas})$
- Normal assumptions:
 - $\eta_{NBI} \sim 1$
 - $R_p(\eta_{pump} + \eta_{core}) \gg (1-R_p)$
 - η_{pump}, η_{core} independent of time
- Particle balance equation becomes:
$$\frac{dN}{dt} = S_{NBI} + (1 + \beta(1 - \eta_{gas})) \eta_{gas} S_{gas} - \frac{N}{\tau_p^*}$$

Let $S = S_{NBI} + (1 + \beta(1 - \eta_{gas})) \eta_{gas} S_{gas}$

Solution:

$$N(t) = S \tau_p^{*,1} + S(\tau_p^{*,2} - \tau_p^{*,1}) \exp(-(t/\tau_p^{*,2}))$$
- Has been used to model step change in τ_p (L-H) and pumping ($\eta_{pump} > 0$)

Simplified Particle Balance and Recycling Model



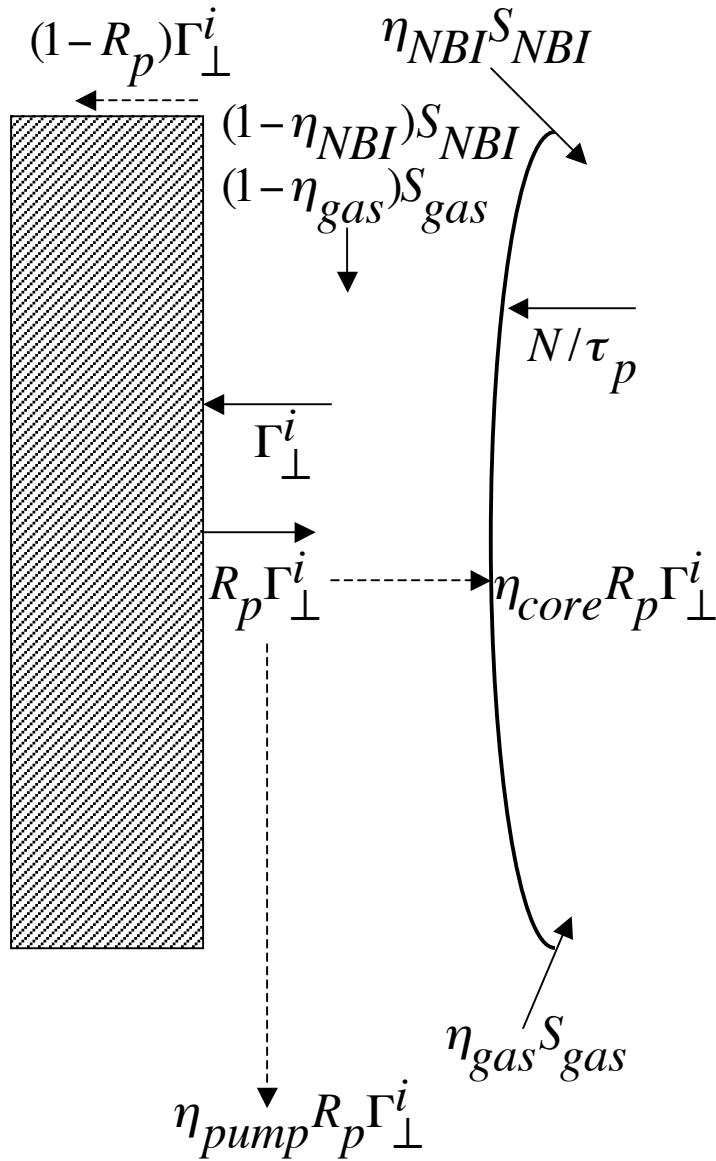
- Density reduction factor

$$n_e^{\text{red}} = \tau_{p,\text{pump}}^* / \tau_{p,\text{nopump}}^*$$

$$= (1-\beta)_{noLi} / (1-\beta)_{Li} \quad \underline{\{\text{constant } \tau_p\}}$$

- $\beta_{noLi} = \eta_{core} R_p / ((1-R_p) + R_p * \eta_{core})$
- $\beta_{Li} = \eta_{core} R_p / ((1-R_p) + R_p * (\eta_{core} + \eta_{pump}))$
- **Need prescription to estimate η_{Li}**
- **Is η_{core} really independent of n_e ?**
- **Is τ_p really independent of n_e ?**

Limits of Particle Balance and Recycling Model



- Note $\tau_p^*/\tau_p = 1/(1-\beta)$
- Pump off: $\tau_p^*/\tau_p \sim 1 + \eta_{core} R_p / (1-R_p)$
 - $\tau_p^*/\tau_p \sim 6$
- Pump on: $\tau_p^*/\tau_p \sim (\eta_{core} + \eta_{pump}) / \eta_{pump}$
 - $\tau_p^*/\tau_p \sim 2$
- n_e should go down by $2/3$ w/pumping
 \Rightarrow *Smaller n_e reduction observed, maybe due to increased core fueling probability at low n_e*
- Input data (from DIII-D studies):
 - $R_p \sim 0.98$ for carbon (reference?)
 - $\eta_{core} \sim 0.1$ (Rensink, PoF B 1993)
 - $\eta_{pump} \sim 0.1$ (Maingi, NF 1999)

Method to Relate 0-D Pump Probability to Divertor Plasma and Lithium tray parameters



Li surface particle sticking
probability - 0.85

$$\eta_{pump} \approx \gamma_{Li}^{sticking} \frac{\int_{R_{min}}^{R_{max,tray}} \Gamma_{\perp}(R) R dR}{\int_{R_{min}}^{R_{max}} \Gamma_{\perp}(R) R dR} \left(\frac{\Gamma_{out}}{\Gamma_{in} + \Gamma_{out}} \right) \left(\frac{\Gamma_{down}}{\Gamma_{up} + \Gamma_{down}} \right) f_{\phi}$$

Impact of R_{tray} , Δ_{tray} , $(R_{OSP} - R_{tray})$
(Γ available from Vlad)

In/out particle flux ratio - 0.8

Tray toroidal coverage - 0.9

Up/down particle flux ratio
0.5 (δ_r^{sep} important)

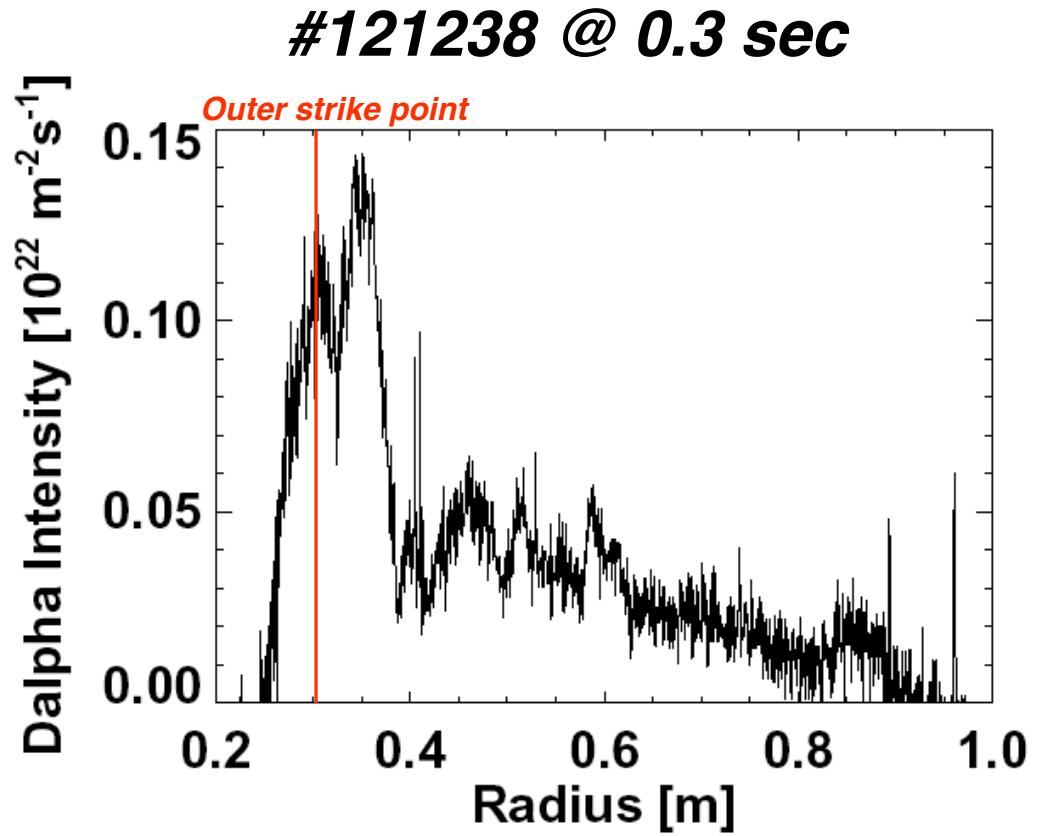
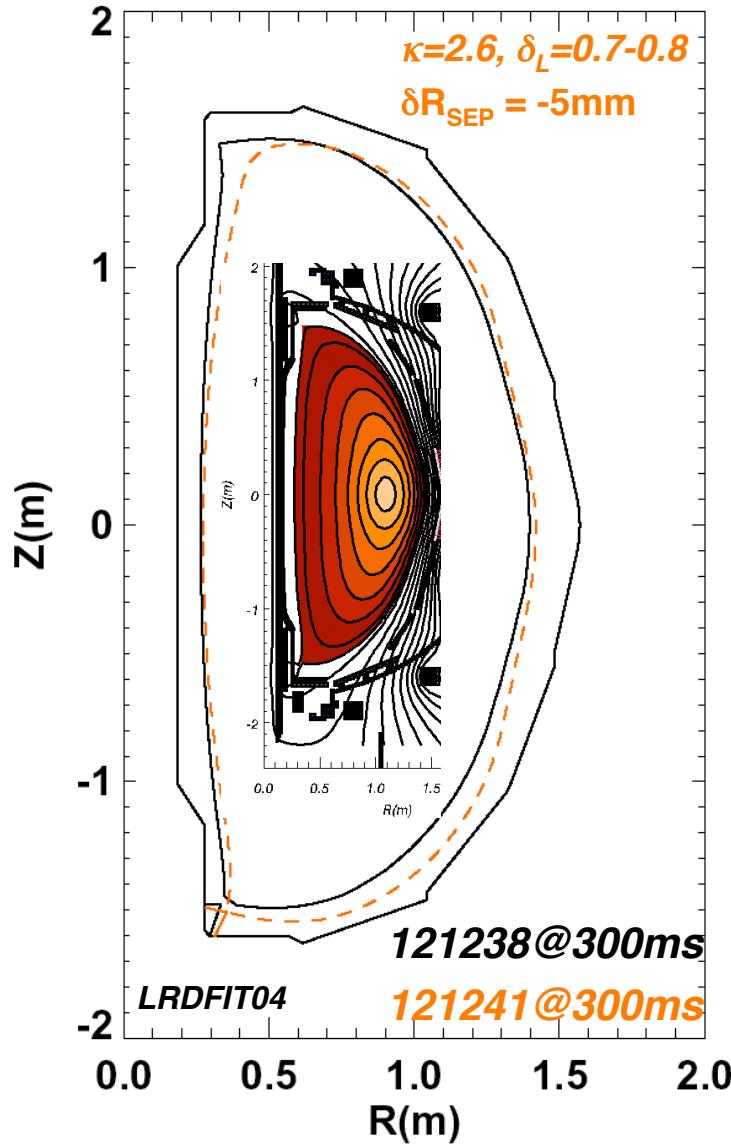
*Red items to be estimated from Vlad's CCD camera data

Procedure



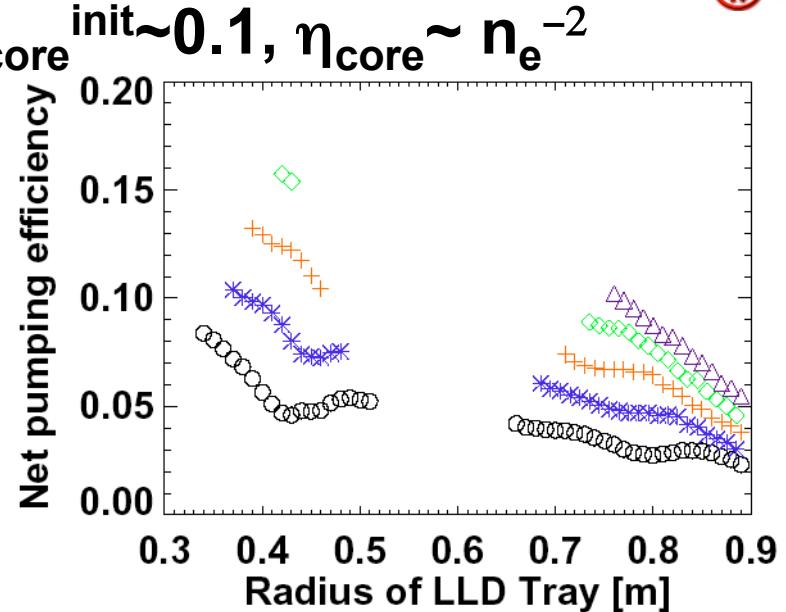
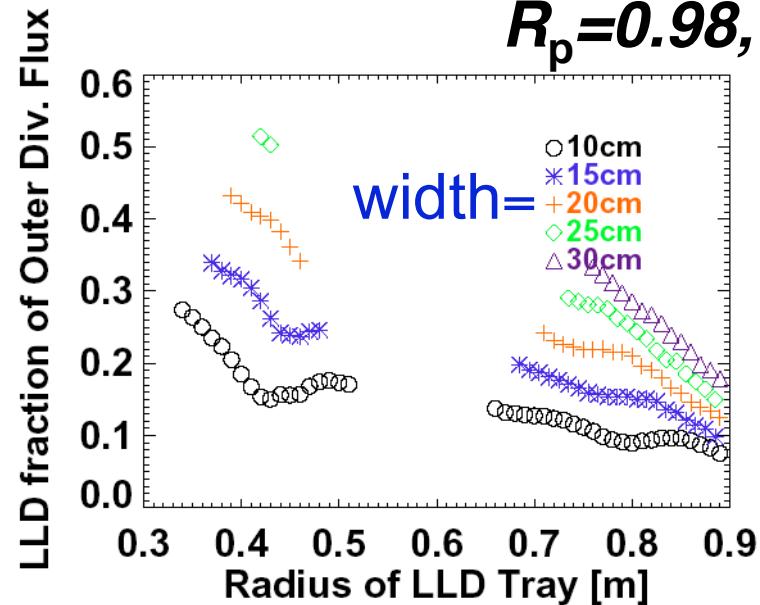
- Convert D_α to particle flux with magic number of 20 ionizations per photon
- Estimate LLD flux intercept fraction from data for a given R_{tray} , W_{tray} , etc. for a given time slice
 - Vary R_{tray} 1 cm at a time
 - R_{tray} starting point a few cm inside of the outer strike point; avoids interpretation of partially detached inner region
 - Avoid covering CHI gap with tray
 - Iterate on $\eta_{\text{core}} \sim 1/n_e^\alpha$ (default: $\alpha=2$)
- Repeat for different W_{tray} , R_p , and other input parameters
- Repeat calculations for different shots with different poloidal flux expansion

Broad SOL D_α profile in high δ (pf1a) #121238

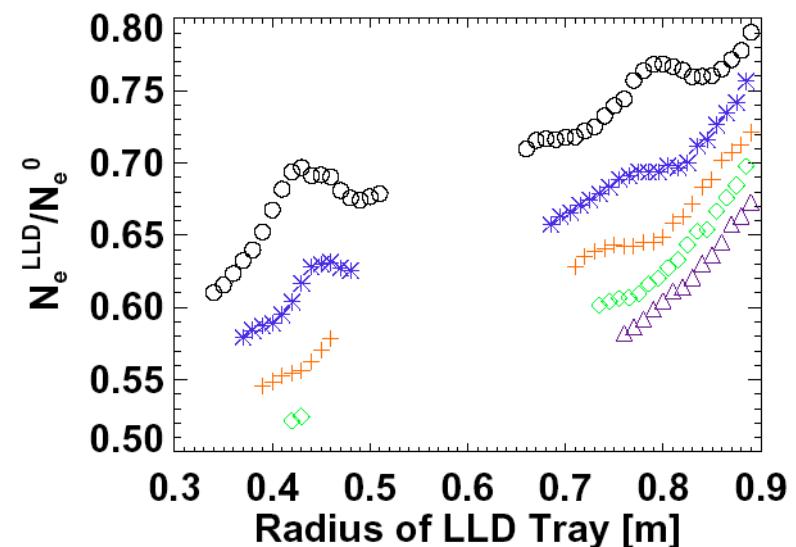
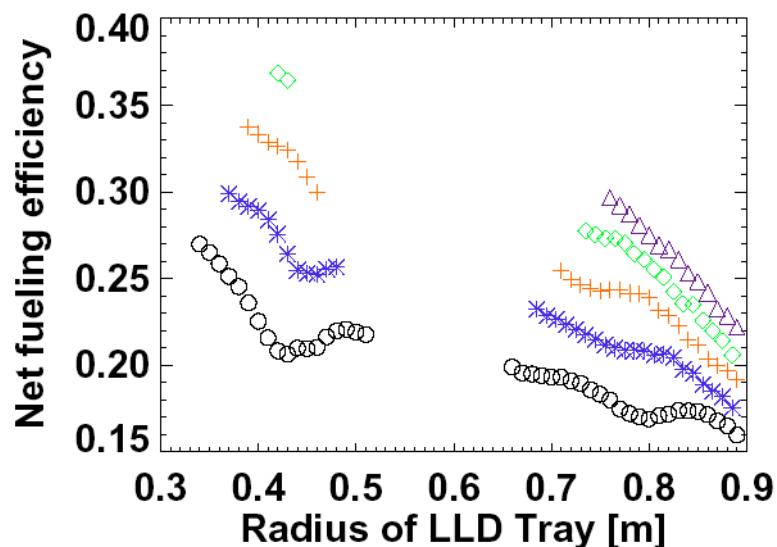


Soukhanovskii

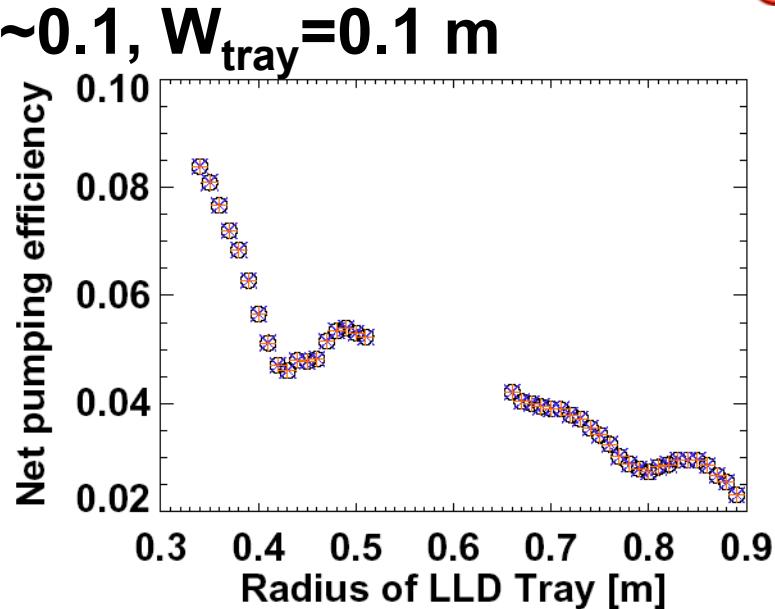
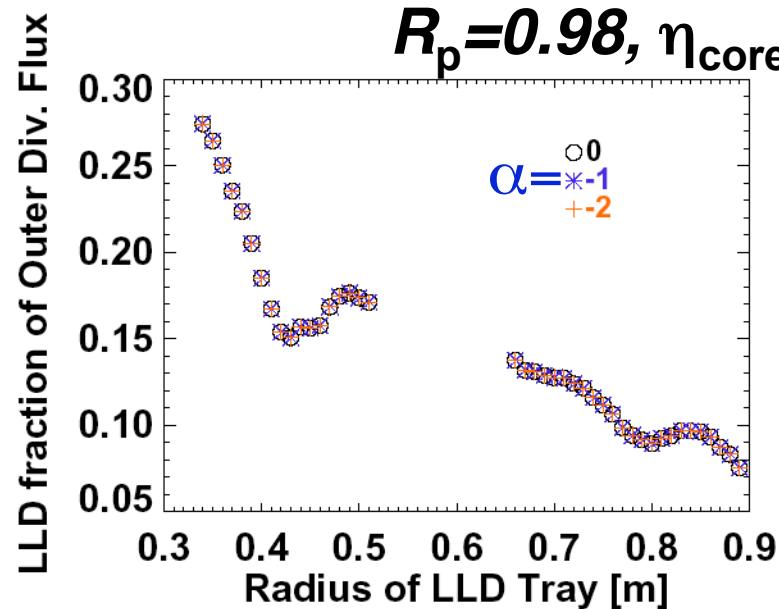
Achievable edge density reduction depends on tray radius and width



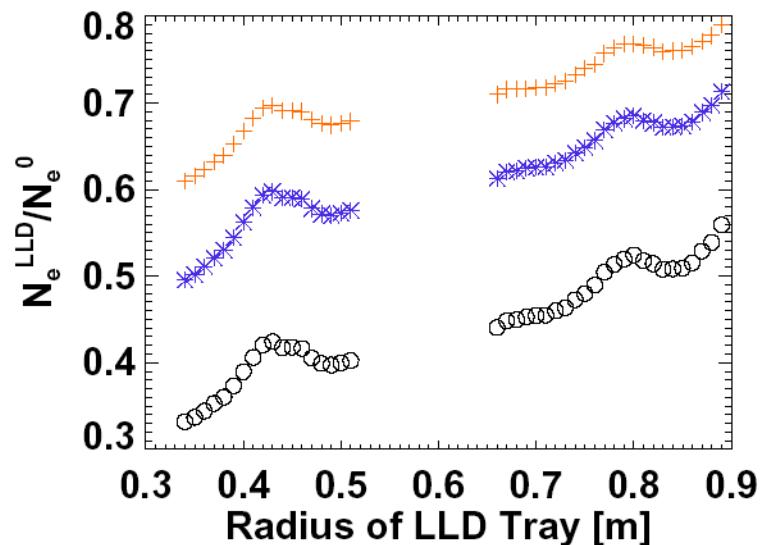
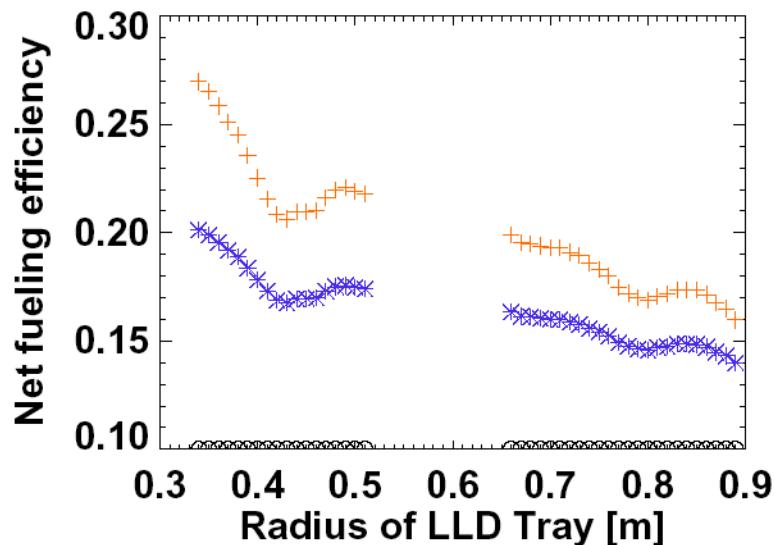
#121238 @ 0.3 sec



Achievable edge density reduction is reduced if core
fueling efficiency $\eta_{\text{core}} \sim n_e^\alpha$

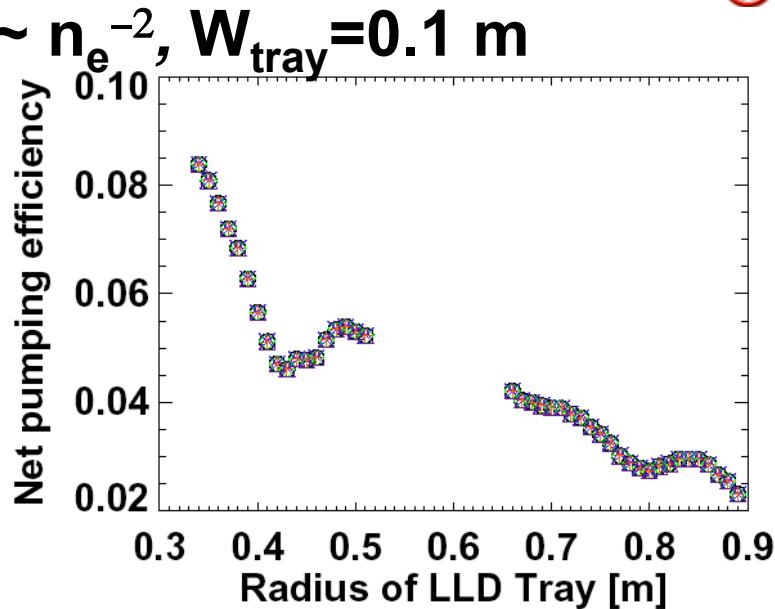
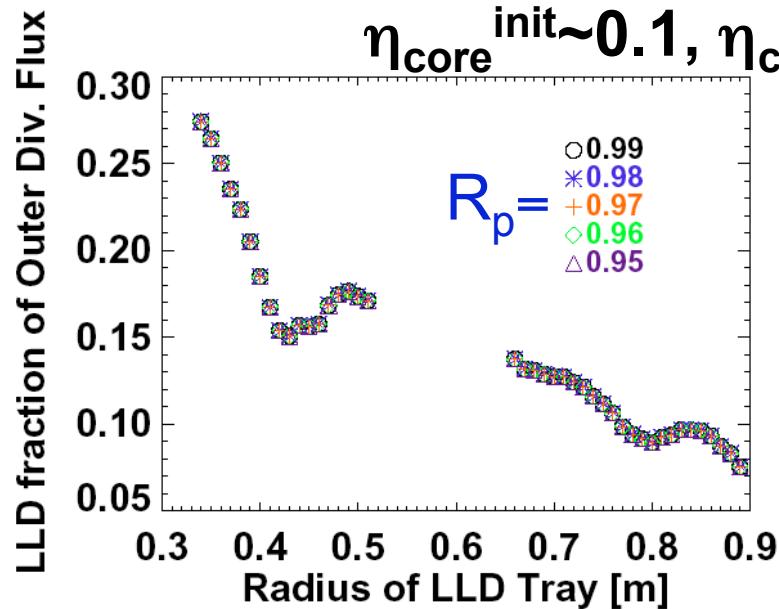


#121238 @ 0.3 sec

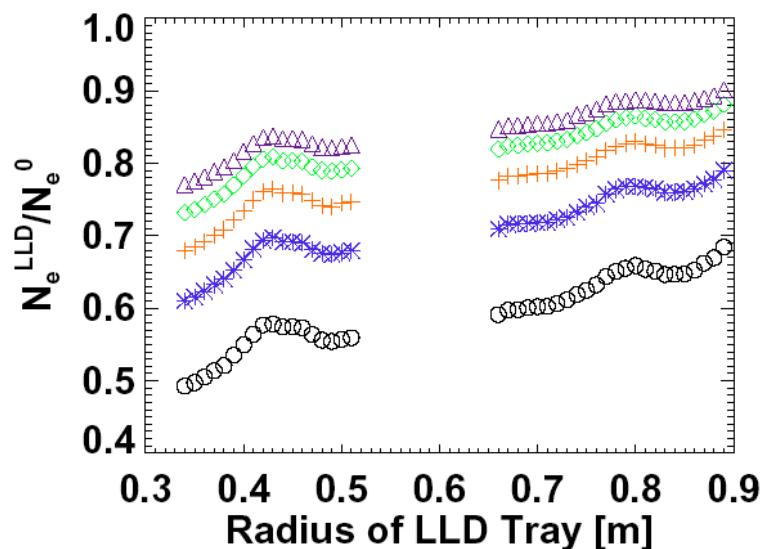
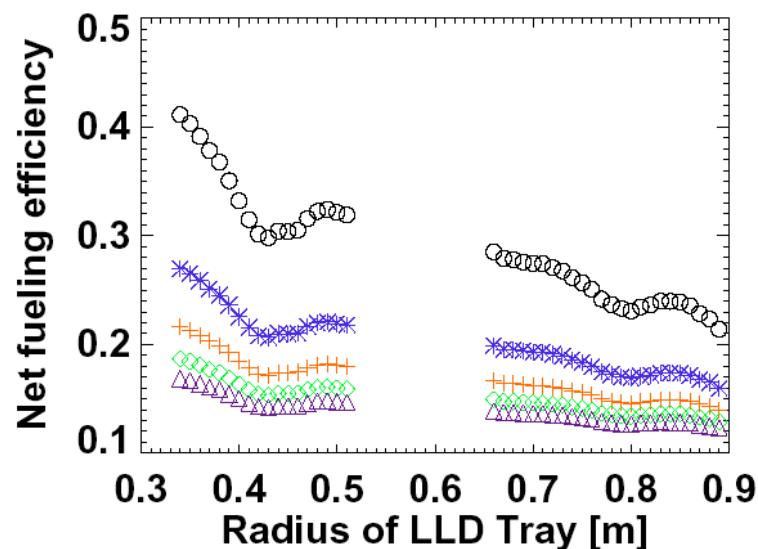


Achievable edge density reduction decreases with assumed initial wall recycling coefficient, R_p

 **NSTX**

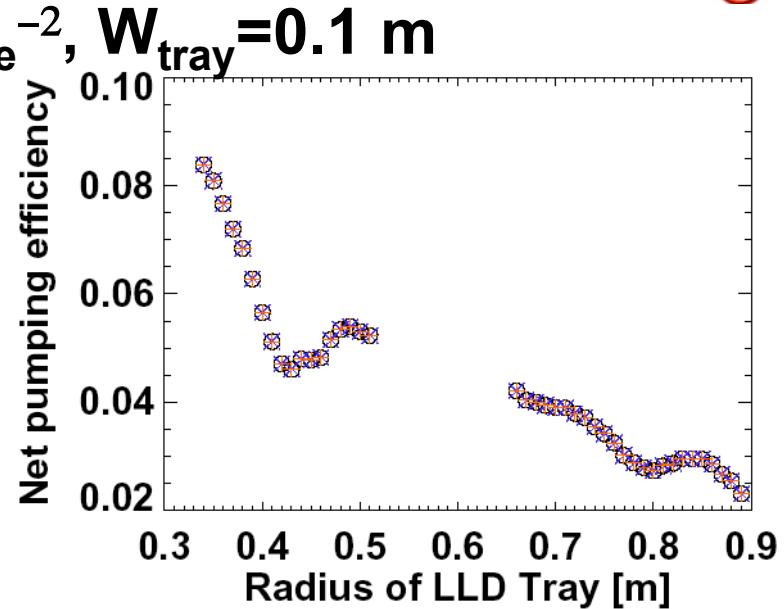
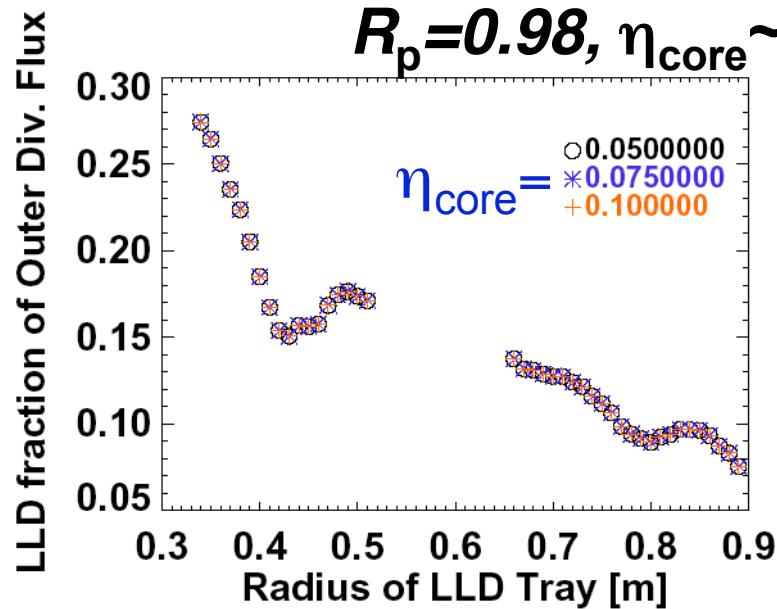


#121238 @ 0.3 sec

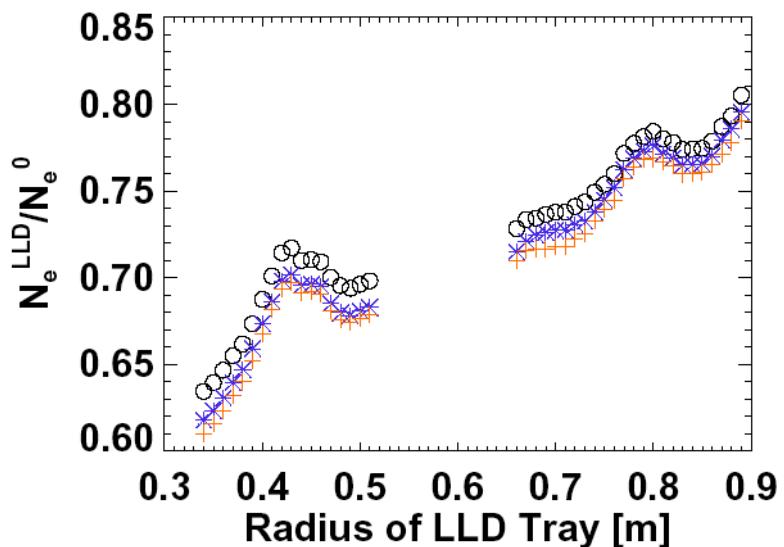
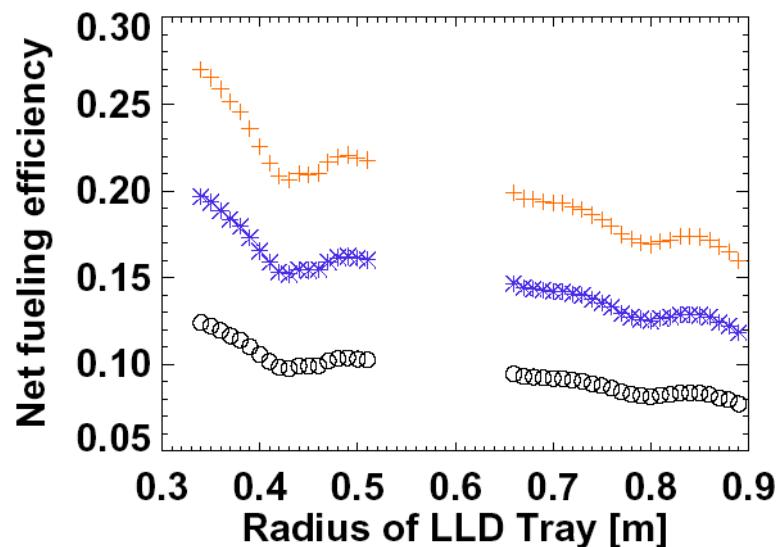


Achievable edge density reduction nearly independent of initial core fueling probability, η_{core}

 NSTX



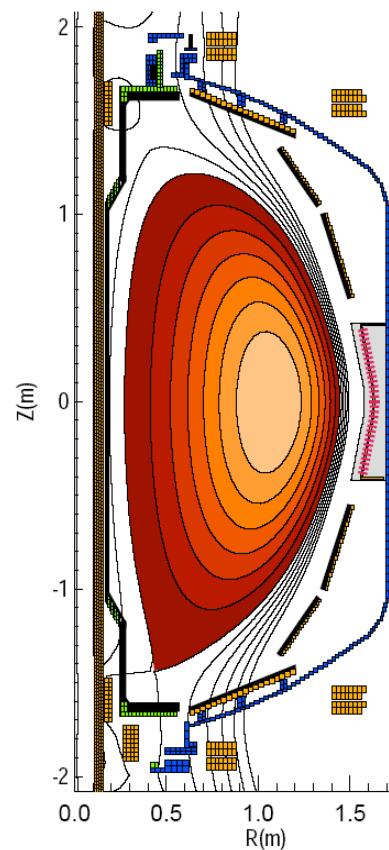
#121238 @ 0.3 sec



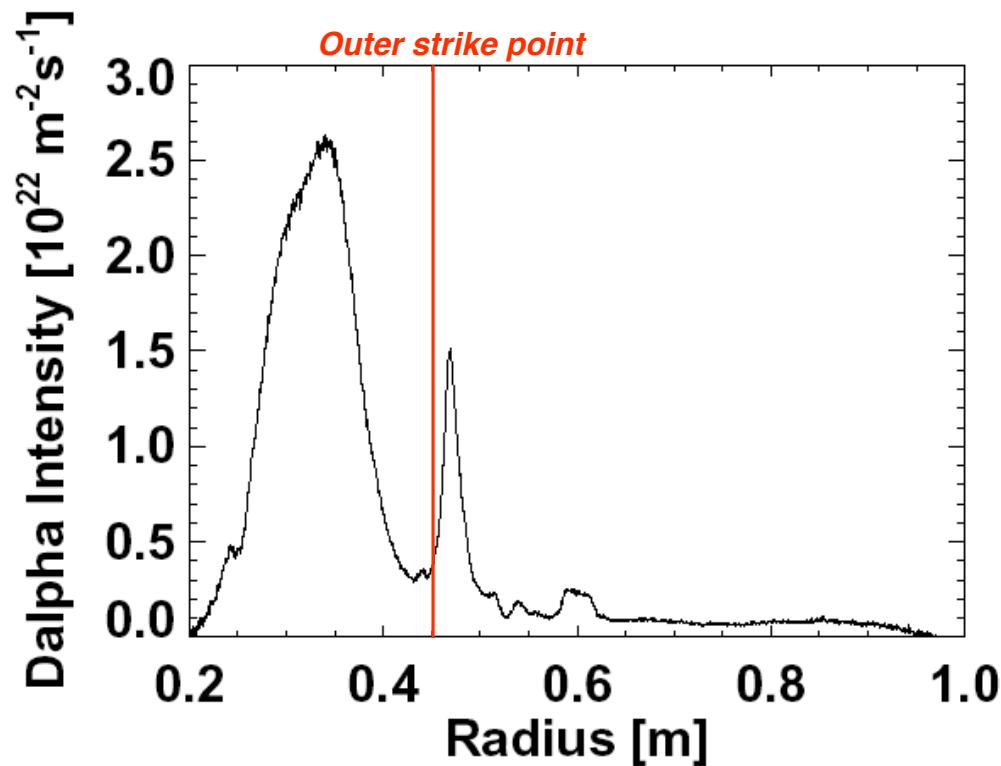
Narrow SOL D _{α} profile in medium δ (pf1b) #116318



Existing #116313
 $\kappa = 2.3$, $\delta_{X-L} = 0.75$
 $\delta R_{SEP} = -1\text{cm}$



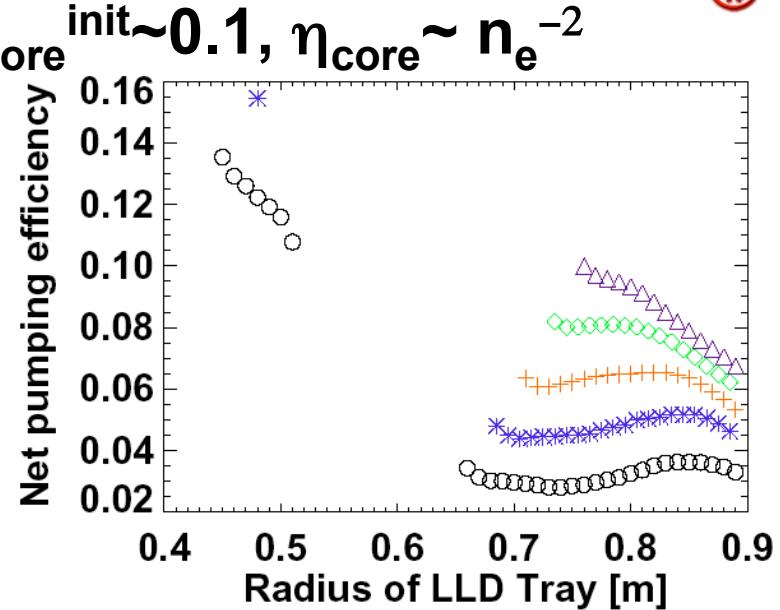
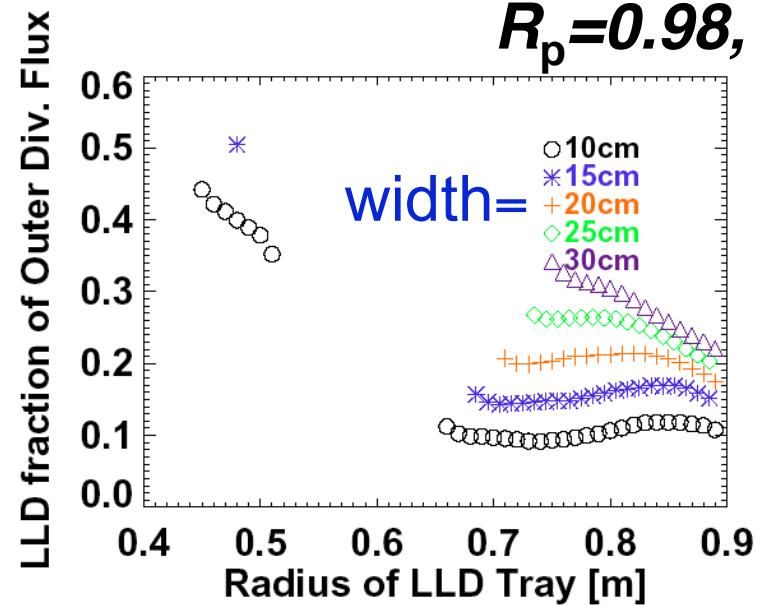
#116318 @ 0.6 sec
(no data on #116313)



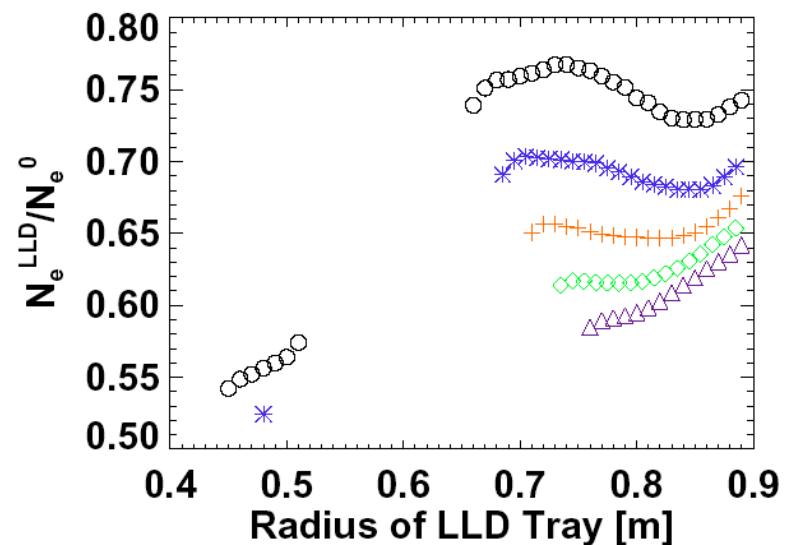
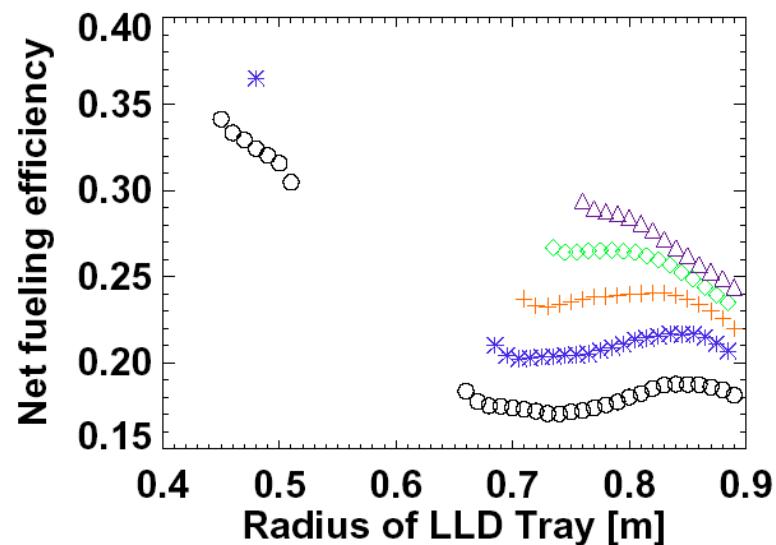
Soukhanovskii

Achievable edge density reduction depends on tray radius and width

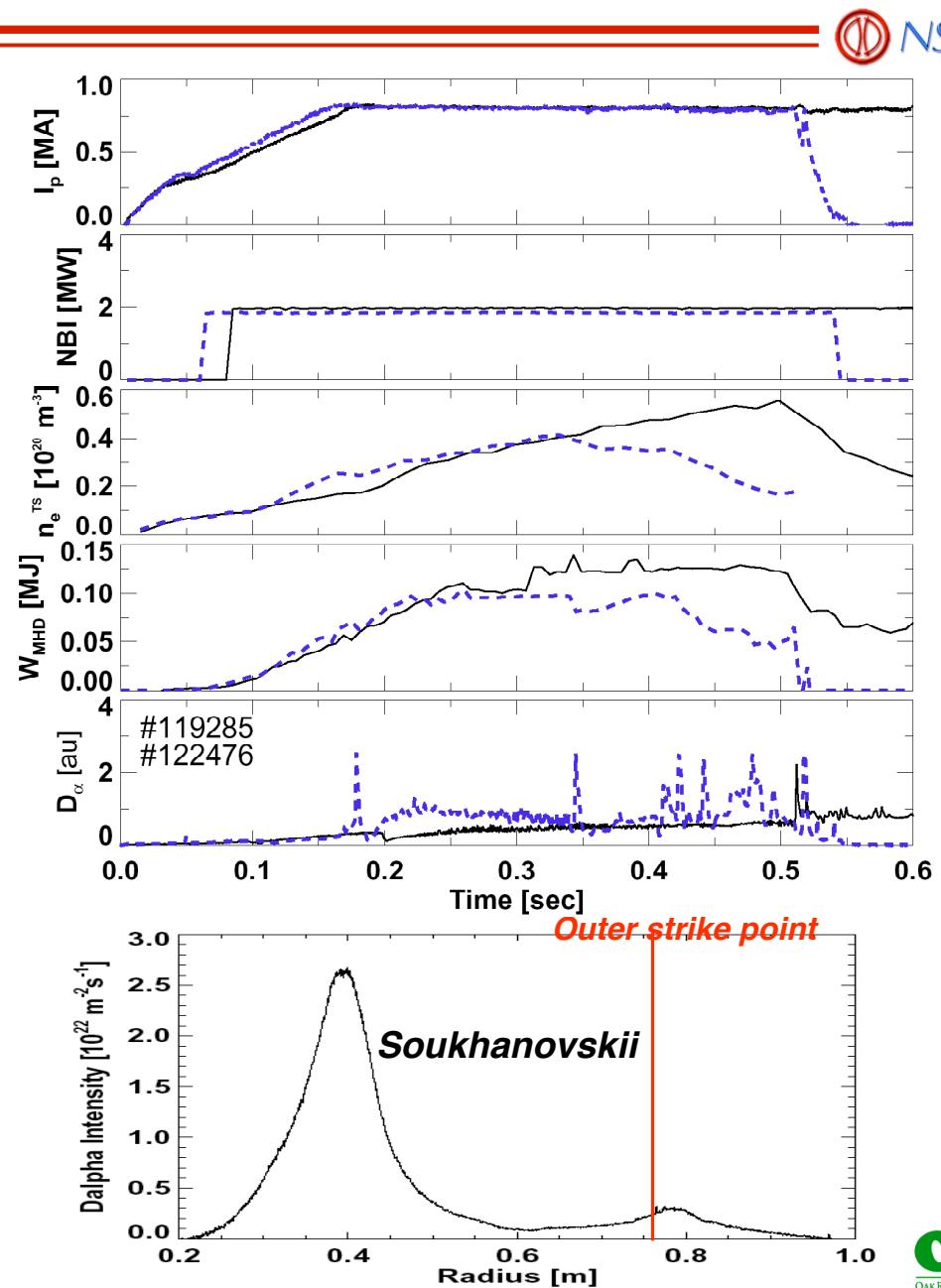
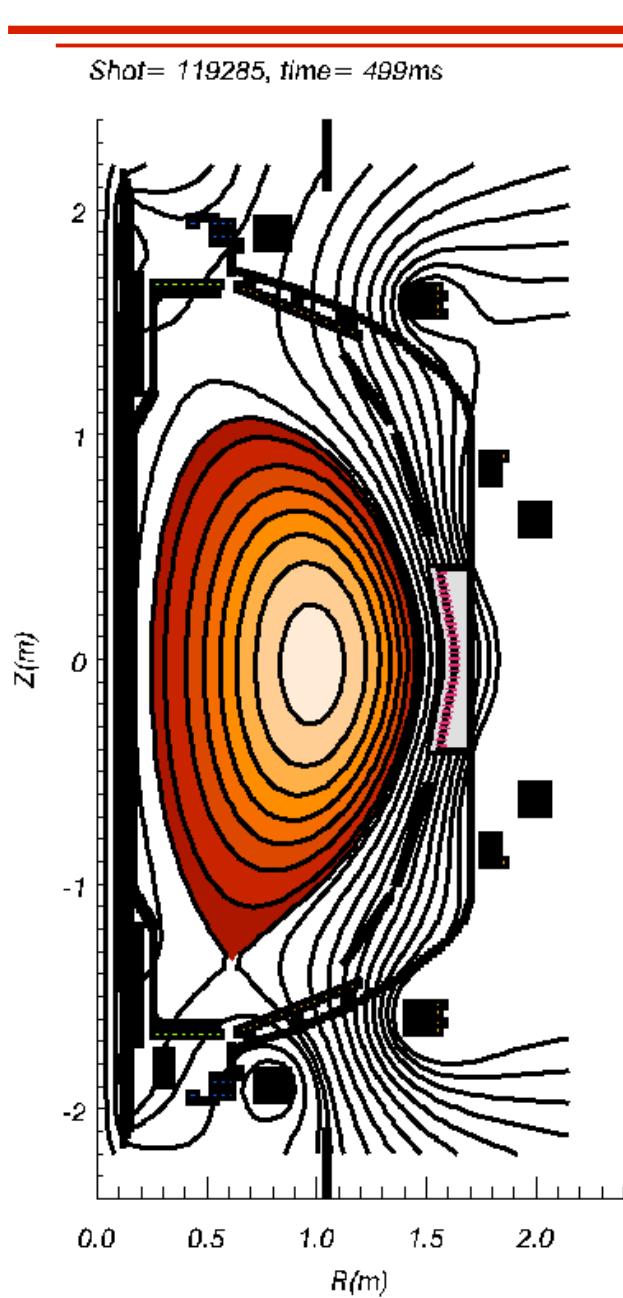
 **NSTX**



#116318 @ 0.6 sec



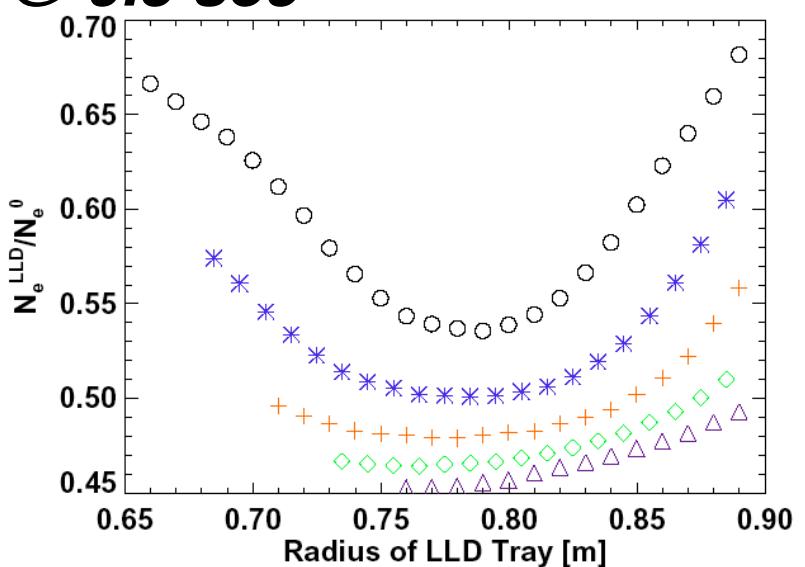
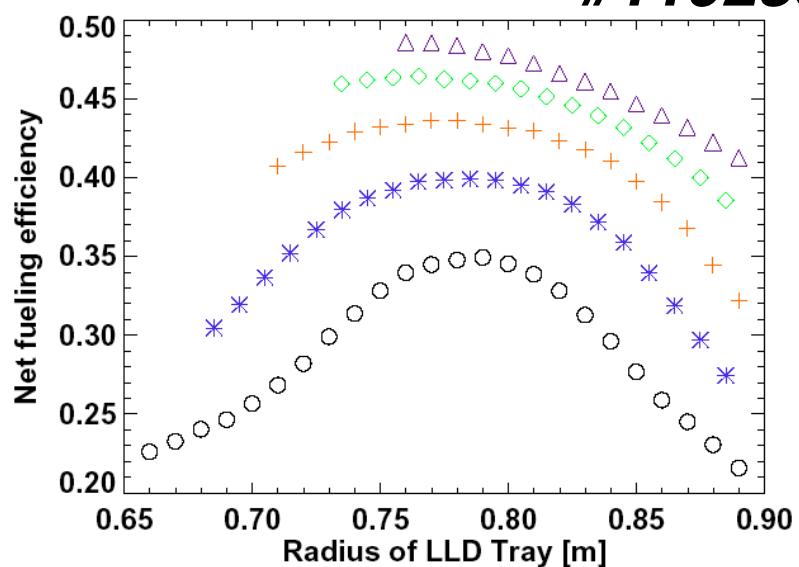
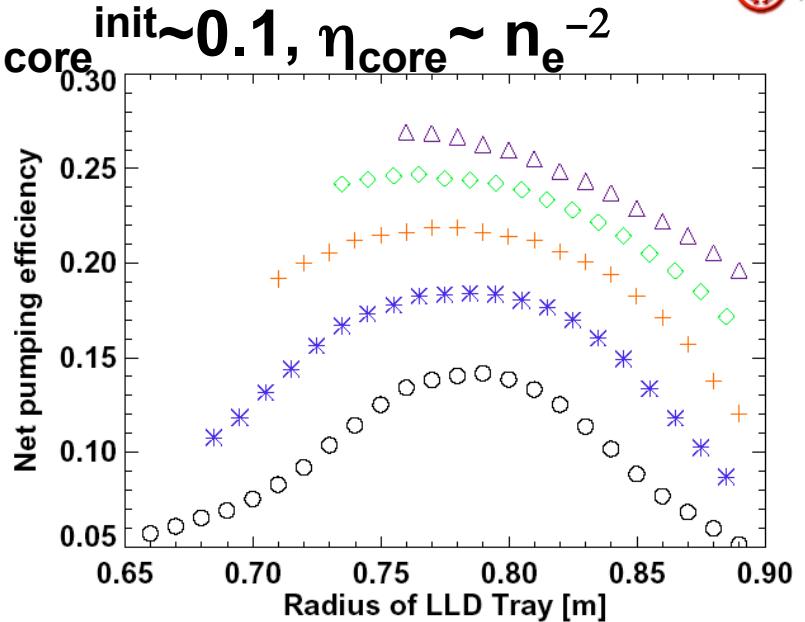
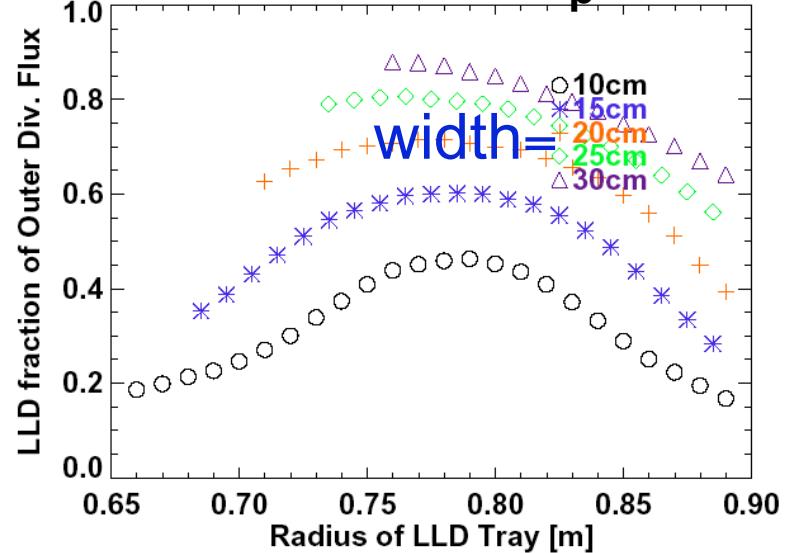
Narrow SOL D_α profile in low δ (pf2) #119285



Achievable edge density reduction depends on tray radius and width



$$R_p = 0.98, \eta_{core}^{\text{init}} \sim 0.1, \eta_{core} \sim n_e^{-2}$$



#119285 @ 0.5 sec

Discussion and Conclusions

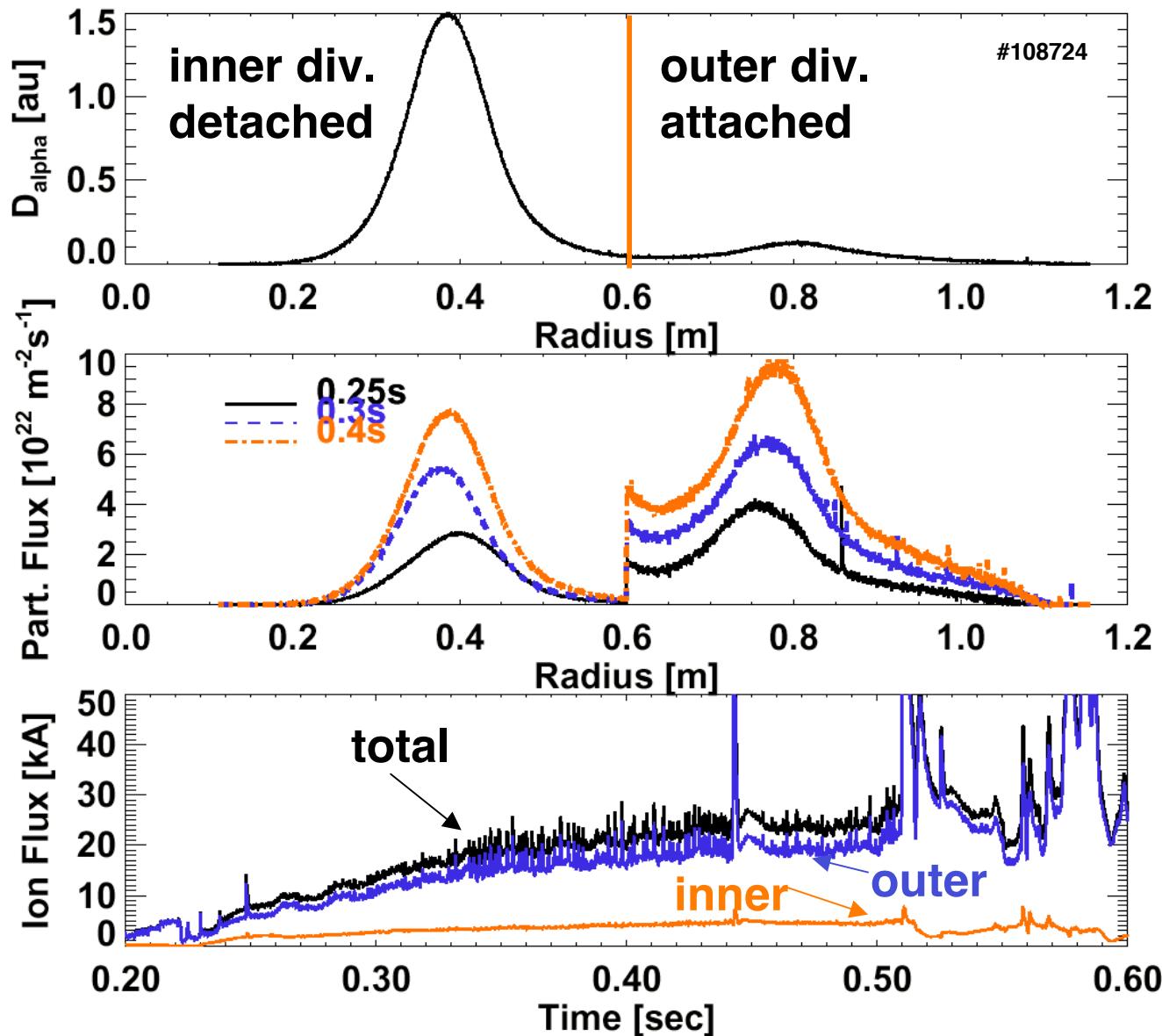


- 20cm wide tray just outboard of the CHI gap likely to provide sufficient density reduction as required for long pulse high non-inductive fraction reported at the Dec. 2006 research forum
- To get a full 50% density reduction will probably require a tray near the outer strike point
 - Inboard of CHI gap for high δ discharges
 - Outboard of CHI gap for low δ discharges
- Actual density reduction factor depend strongly on how quickly core fueling efficiency increases with decreasing density, and the pre-Li global wall recycling coefficient
- Intend to compare with UEDGE calculations, when available

Backup

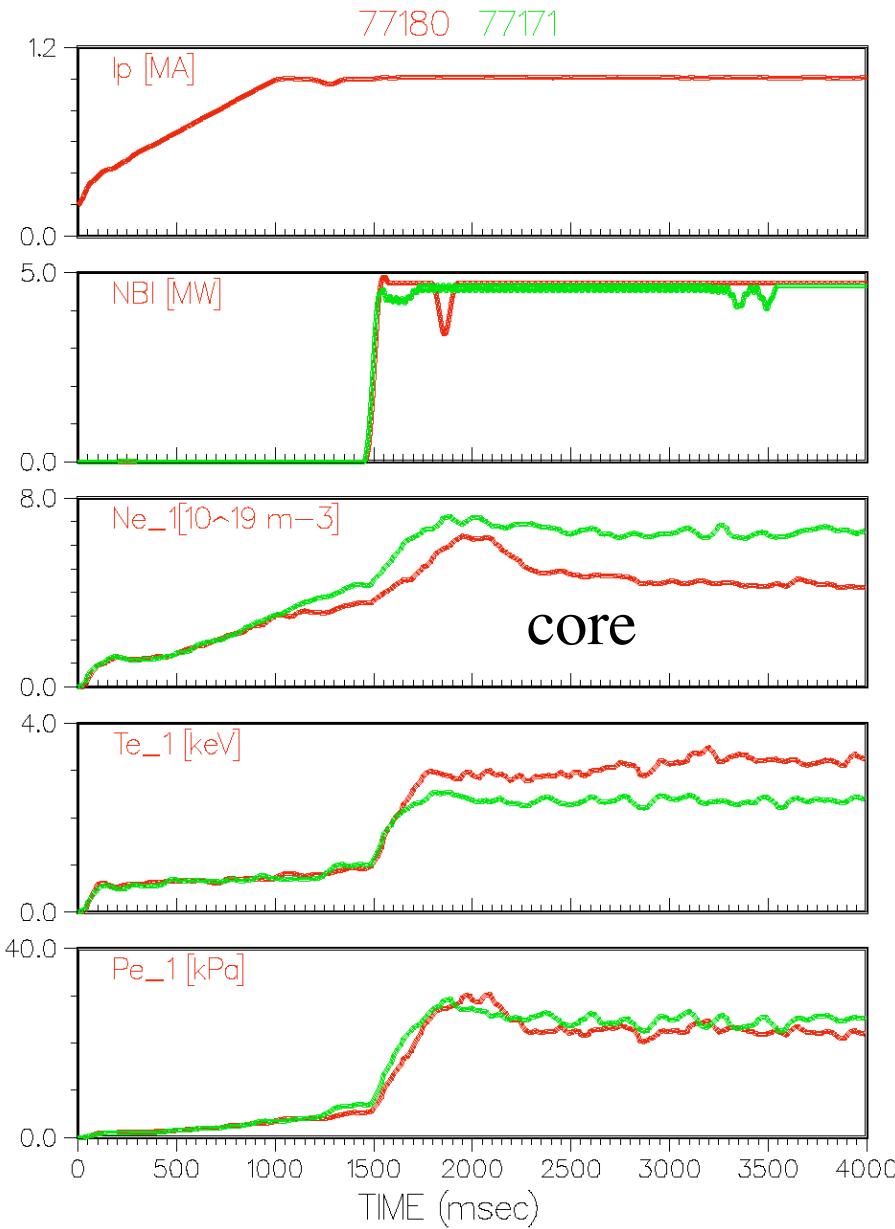


NSTX D_α Peaked on Inboard Side, but Particle Flux Peaked on Outboard side because Inner Divertor is Usually Partially Detached

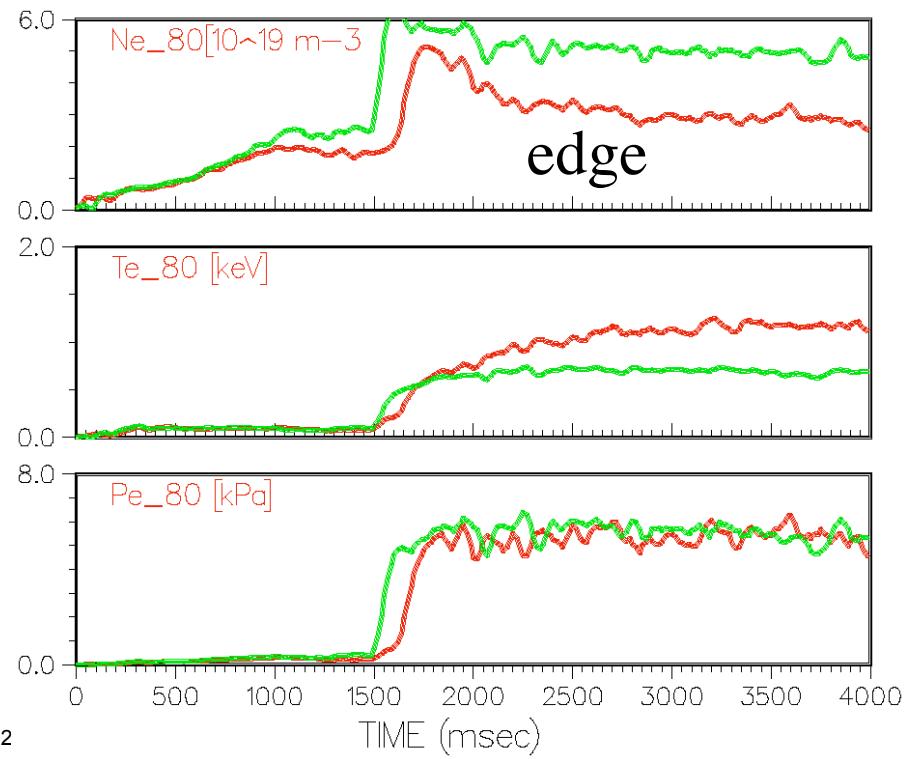


- Inner side detached
- Outer side attached
- Ions/photon =1 (detach)
- Ions/photon =20 (attach)
- Division at $R \sim 0.6\text{m}$
- Out div. has $\sim 4\times$ times current of inner div.

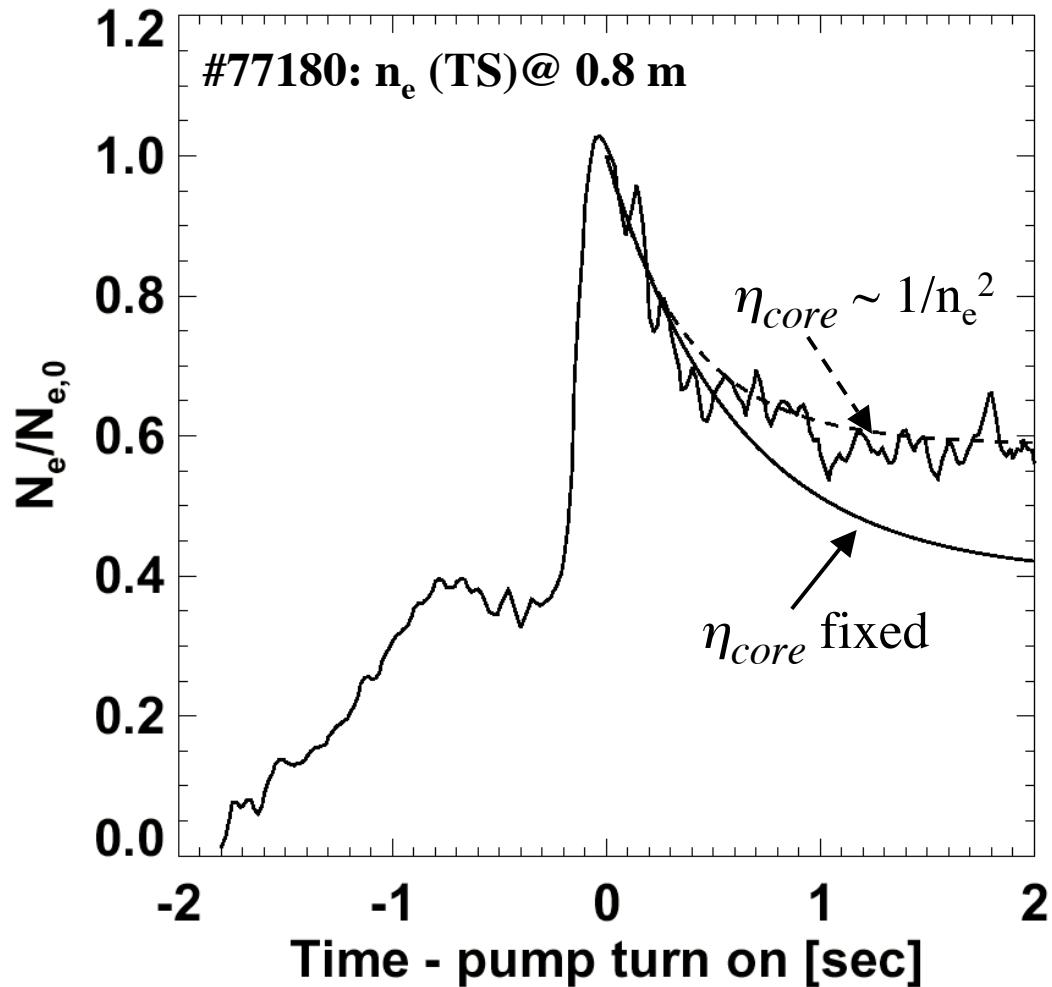
Comparison of Unpumped and Pumped DIII-D Discharges



- Edge electron pressure holds constant as n_e reduced
- Relative change in edge n_e larger than core



Particle Balance and Recycling Model - DIII-D cryopump



- DIII-D specific data:
 - $R_p \sim 0.98$ for carbon (reference?)
 - R_p changes slowly (Maingi, NF 1996)
 - $\eta_{core} \sim 0.05\text{-}0.15$ (Rensink, PoF B 1993)
 - $\eta_{pump} \sim 0.1$ (Maingi, NF 1999)
 - $\eta_{gas} \sim 0.1$ (Maingi, JNM 1997)
 - $\tau_p / \tau_E = 2.5$ (\sim Owen, JNM 1997)
- Solid η_{core} - fixed in time
 - N_e goes down on τ_p^* timescale
- Dashed $\eta_{core} \sim 1/n_e^2$
 - τ_p^* increases with time
 - N_e equilibrates faster than initial τ_p^*