Flowing Liquid Lithium (FLILI) System

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Conceptual design of the FLiLi system. (a) Example of FLiLi system as a limiter for a tokamak with a circular cross section (e.g., HT-7). (b) Assembly of distributor, feeding pipe, guide plate with LiLi flow (green), heat sink, collector and exhaust mechanism. (c) Separate parts of FLiLi system. (d) Two FLiLi systems for NSTX-U



Toroidal tray and FLiLi on HT-7 (August 2012)



(a) HT-7 plasma over a toroidal tray with a pool of LiLi. FLiLi is located at the low field side of the tray. (b) HT-7 toroidal tray with a pool of LiLi and the FLiLi system with poloidal flow of LiLi (green color). (c) Backside view on FLiLi. (d) FLiLi as a limiter for shifted HT-7 plasma, (e) Poloidal cross-section of FLiLi (f) 3-D elements of FLiLi.

There are no technical obstacles to adjust a pair of FLiLi systems for NSTX-U as well

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Flow parameters and key properties

$$egin{aligned} V_{cm/s} &= 0.2 - 1, \quad Q_{cm^3/s} = 1 - 2, \quad H^{filt}_{mm} = 1, \quad L^{filt}_{cm} = 2 - 10 \ \Delta p^{filt}_{Pa} &= 1.6 \cdot 10^2 V^{filt}_{cm/s} L_{cm} B^2_T, \quad \Delta p^{dist}_{Pa} = 1.6 \cdot 10^2 V^{dist}_{cm/s} L_{cm} rac{d}{w} B^2_ot \end{aligned}$$



Design requirement $\Delta p_{Pa}^{filt} > \Delta p_{Pa}^{dist}$ Good properties are countless:

- 1. FLiLi solves the problem of the Li surface contamination: open loop during machine operation, close loop overnight
- 2. Flow rate is under external control by pressure in the feeding pipes.
- 3. The system is scalable in both poloidal and toroidal direction, and from a laboratory test chamber to a real tokamak device.
- 4. Minimal in-vessel inventory of LiLi. LiLi is supplied from outside and is exhausted to outside the VV.
- 5. The bulk of LiLi is protected from plasma disruptions by the filter layer.
- 6. No side walls for LiLi flow, no leading edges.
- 7. Simple for maintenance, at the end of the campaign can be flushed out by argon and then by vinegar.
- 8. Is insensitive to yet unknown $\mathbf{j}\times\mathbf{B}$ forces.
- 9. Filter channel geometry and orientation is flexible.

FLiLi is compatible with any tokamak, including NSTX-U. The plasma regime, it can provide, is beyond the dreams.

FLiLi is simply amazing.



Flow parameters and key properties

The most fundamental aspect (and a huge problem) of magnetic fusion is that its physics and technology cannot be presented as a set of independent virtual "bricks".

Everything in the tokamak plasma regime (confinement, stability, fueling, power extraction, wall erosion) is interlinked in unbreakable fashion.

All plasma physics tendencies in the current non-LiWF are bad.

In integrated tokamak plasma physics, the plasma edge and PFC represent the most sensitive element. This understanding should be the guidance for strategy.

The Table of consistency of NSTX-U Li suggestions with the tokamak physics and technology.

Name	$ au_E$	CoreSt	EdgeSt	Disr	$Q rac{MW}{m^2}$	Li MHD	T_{surf}^{LiLi}	h_{mm}	$V \frac{cm}{s}$	$(VS)\frac{cm^3}{s}$	HT-7	NSTX-U	NxtStep
FLiLi	BP	BP	BP	BP	BP	BP	$< 400^{o}$ C	0.1	<1	2	Yes	Yes	Yes
Pool	BP	BP	BP	BP	< 5	TBD	$< 400^{o}$ C	1-3	$\simeq 1$	0	TBD	Yes	No
Trench	BP	BP	BP	BP	< 5	Ignored	$< 400^{o}$ C	1	??	??	TBD	Yes	No
CPS	gmbl	gmbl	gmbl	gmbl	TBD	TBD	Any	1-5	$\simeq 1$	TBD	TBD	Yes	No
BP		- the best possible											
TBD		- to be determined											
gmbl		- gambling with the problem											

In fact, except of FLiLi there is no reasonable concept for NSTX-U.

The RDM is a substitution of the real problem, intended to drain the research time and taxpayers money.

Instead of "bricks", like RDM, the phases of implementation of FLiLi should be adopted

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