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# Halo Current and Current Quench Rate Characteristics During Disruptions in NSTX

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For the

NSTX Macroscopic Stability Topical Science Group

12th Meeting of the ITPA MHD Stability Topical Group

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CRPP, Lausanne, Switzerland

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# Examining disruption eddy currents, halo currents, and $I_p$ current quench in the spherical torus

## □ Goals

- Provide electromagnetic loading data for future ST design
- Examine ST specific characteristics to enhance tokamak knowledge base

## □ Considerations

- High elongation more subject to  $n = 0$  instability during vertical motion (VDE)
  - General issue: disruption  $I_p$  quench drives eddy currents ( $I_c$ ) in nearby conducting structures (C); currents lead to significant  $J \times B$  forces on in-vessel components
    - $\tau_c$  ( $L_c/R_c$  time) long: total flux change matters
    - $\tau_c$  short: instantaneous flux change matters
- Strong  $1/R$   $B_t$  variation in ST makes center column halo currents a large concern
- Halo currents can flow linking the plasma and in-vessel components when plasma comes in contact with plasma facing components
  - Currents are parallel to  $B$  in the plasma edge; distributed to vessel based on inductance/resistance
- Disruptions are faster in the ST vs. conventional aspect ratio
  - Expand studies to wider range of disruption conditions/plasma parameters

$$\frac{dI_c}{dt} + \frac{I_c}{\tau_c} = -\frac{1}{L_c} \frac{d\Phi}{dt}$$



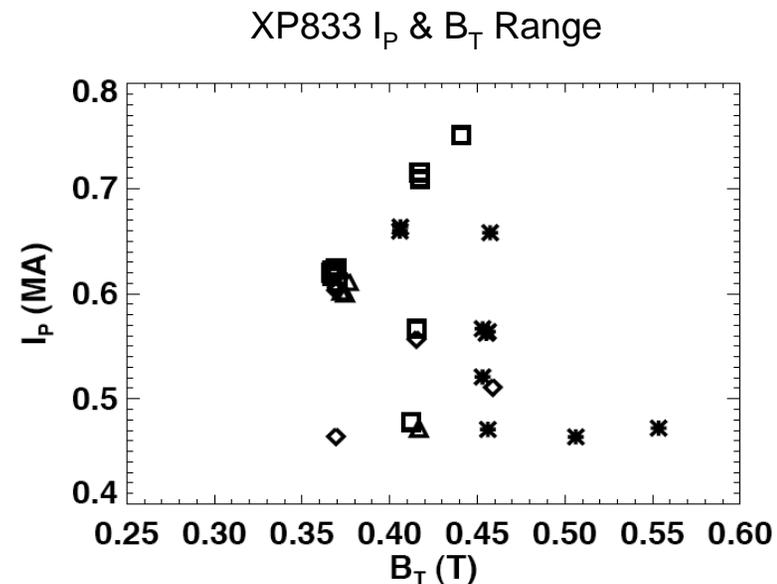
# Disruptions analyzed over a wide range of plasma conditions

## ❑ Disruption database

- ❑ Greater than 900 shots, covering all operational regimes
- ❑ Select shots with fast current quenches, large halo currents
- ❑ Maximum halo current in all measured paths, current quench characteristics

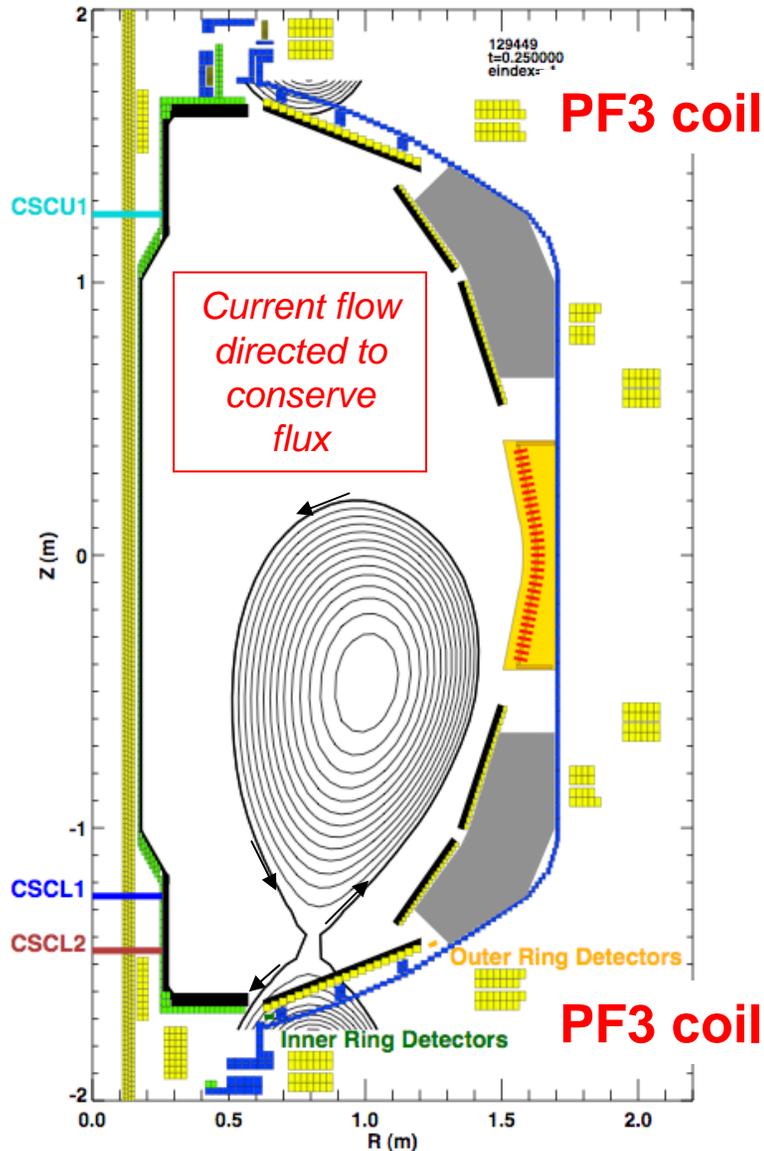
## ❑ Dedicated experiment in 2008

- ❑ Triggered VDEs
- ❑  $I_p$  and  $B_T$  scans at fixed shape, NBI power
- ❑ Detailed tracking of vertical motion
- ❑ More accurate characterizations than in large database analysis

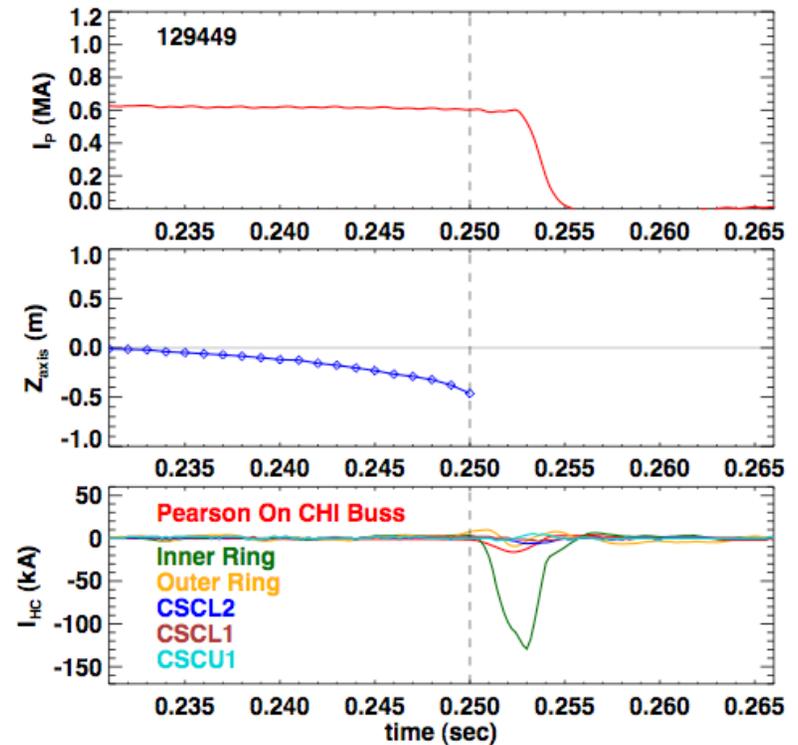




# Downward Going Disruption With Large Halo Currents

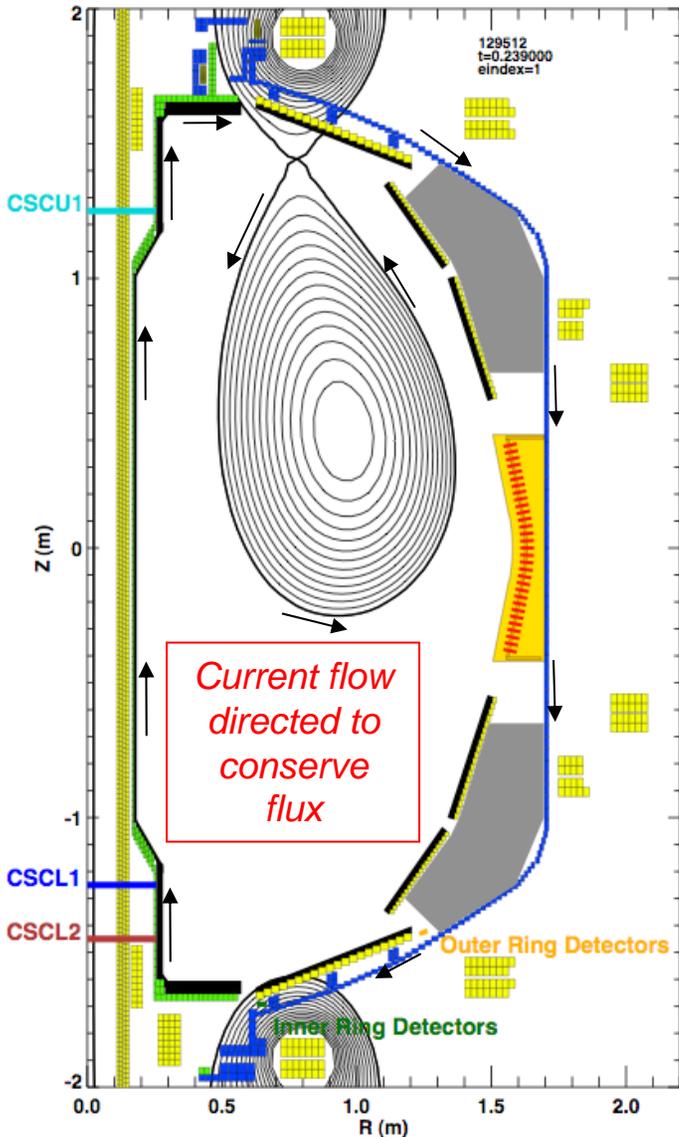


*Shot 129449*  
Downward Going VDE  
(PF3 Voltage Freeze + Offset)  
Current Flows OUT of Divertor Plate

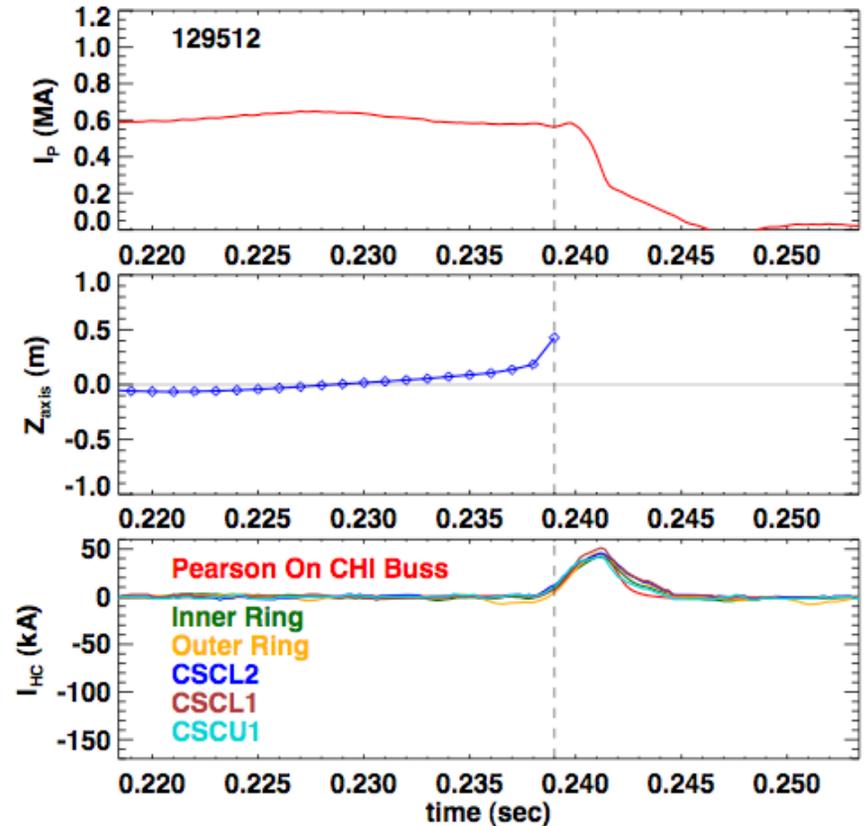


*Typical Pattern for Dedicated VDE Experiment*

# More Limited Measurements for Upward Going VDE



*Shot 129512*  
Upward Going VDE

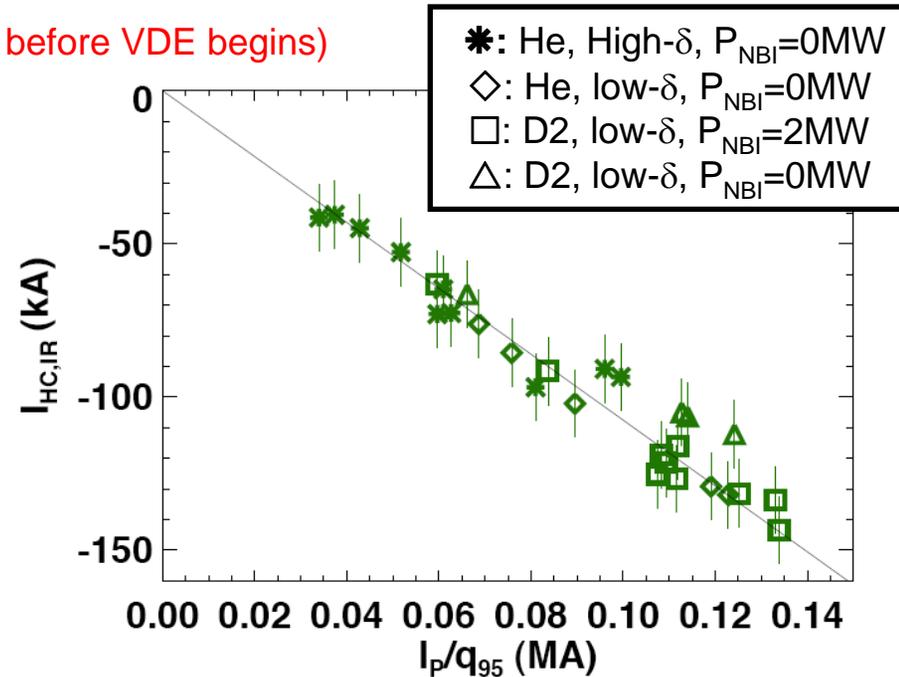
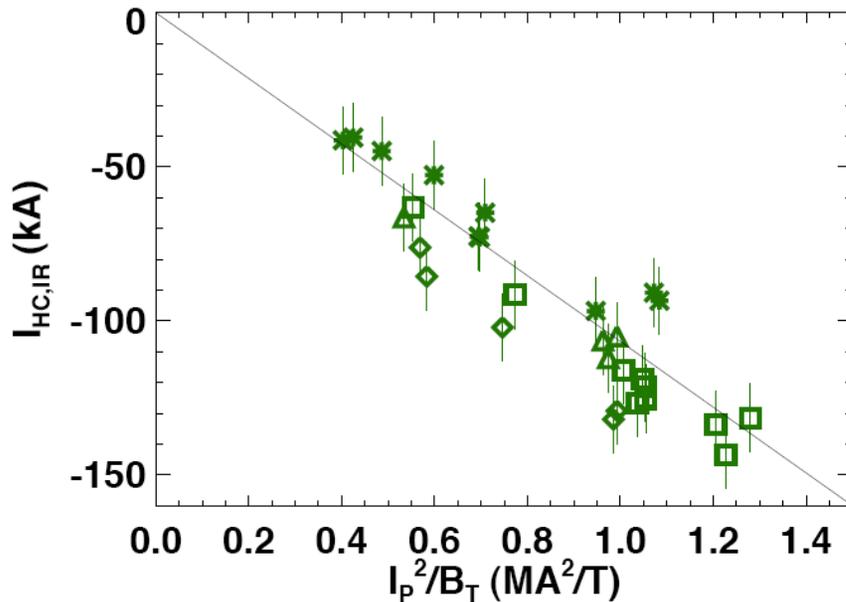


**Leads to Largest Center Stack Casing Currents**



# Halo Currents In Vessel Bottom Scale as $I_p/q$ , Consistent With Simple Models

(Quantities measured just before VDE begins)



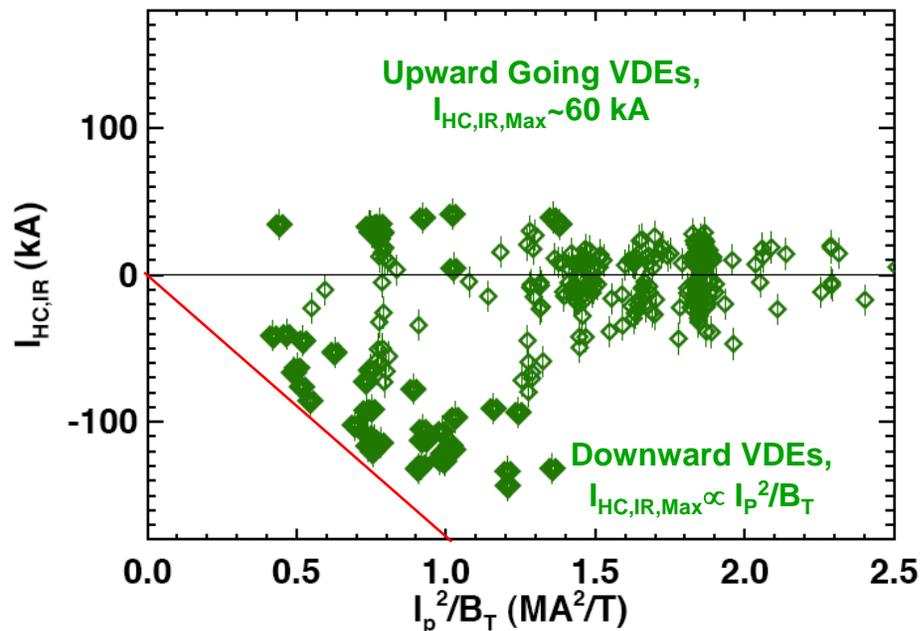
■ Simple explanation of scaling (P.J. Knight, et al., Nucl. Fusion **40** (2000) 325.)

- $I_p$ : Halo currents increase with  $I_p$
- $q$ : halo currents flow parallel to  $B$ , poloidal component increases if  $q$  decreases

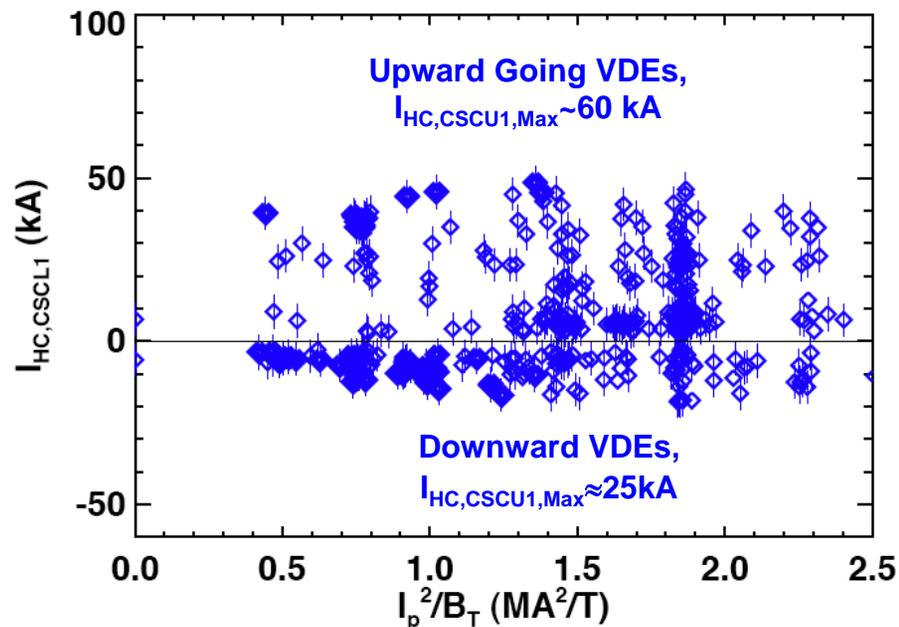
■ Scaling coefficient independent of working gas,  $P_{\text{NBI}}$ , shape

# Vessel Bottom Currents Largest in Triggered VDE Experiments

Vessel Bottom,  $I_{HC}$  vs  $I_p^2/B_T \sim I_p/q$



Lower PF1A Transition,  $I_{HC}$  vs  $I_p^2/B_T$



Solid Symbols: Deliberate VDEs (XP833 & XP811), Open Symbols: All Others

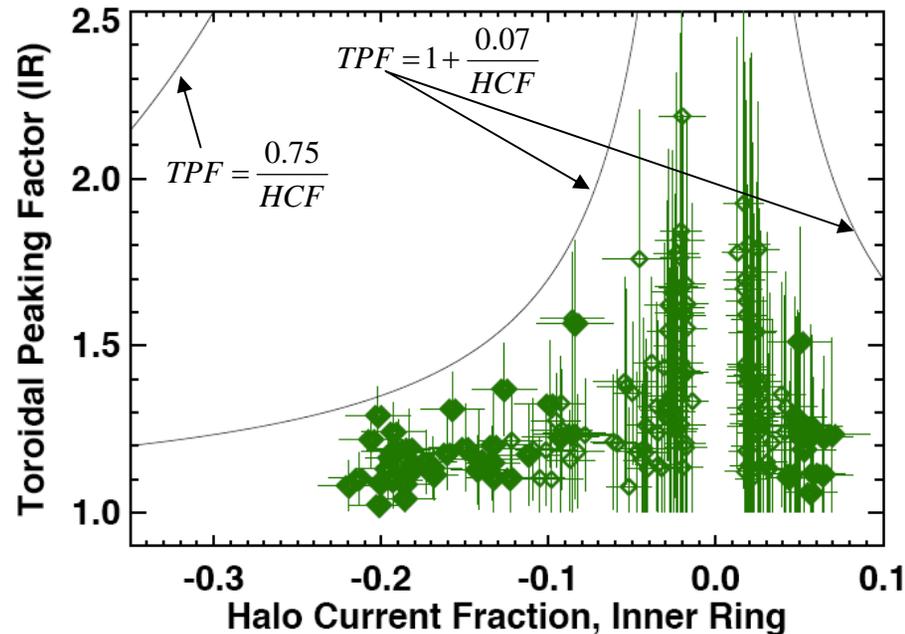
## □ Simplified conclusion

- Upward VDEs: 60 kA max center stack casing current, no observed scaling with  $I_p^2/B_T$ ,  $I_p/B_T$ , or  $I_p$
- Downward VDEs:  $I_p^2/B_T$  scaling for currents into outboard divertor



# Toroidal Peaking Factor Decreases With Halo Current Fraction

## Inner Ring, TPF vs. HCF

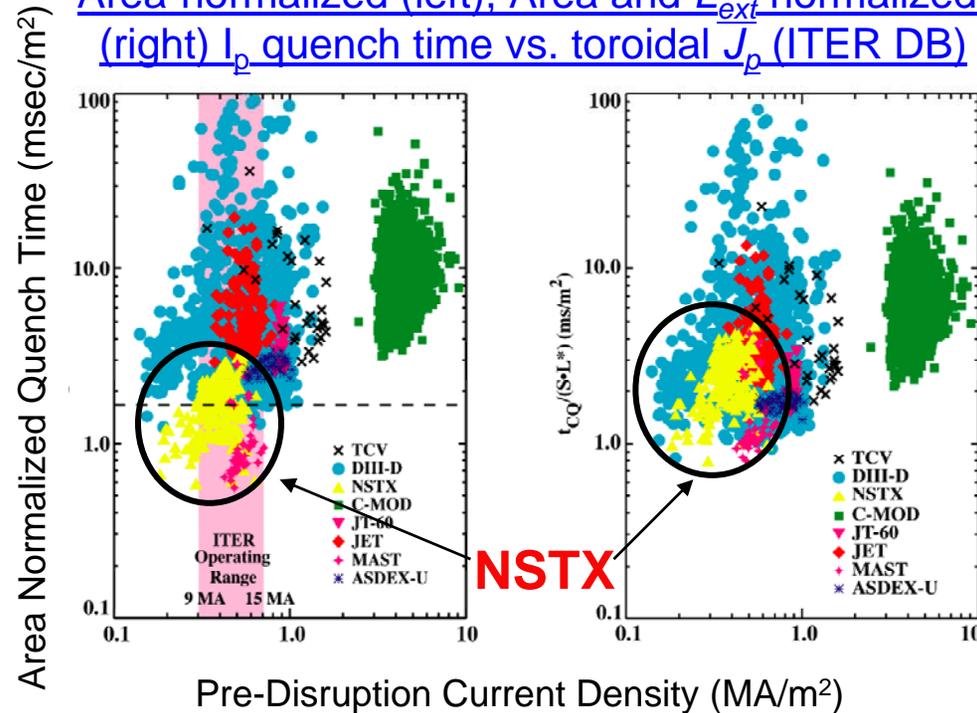


- ❑ Toroidal peaking factor  $TPF = 6 \cdot \max(B_{i=1:6}) / \sum B_i$ 
  - ❑ Uncertainty larger at small halo current fraction (HCF)
- ❑ ITER Assumption:  $TPF \times HCF < 0.75$ 
  - ❑ NSTX Data Well Below This Scaling

# Current Quench Rates Are Fast in the ST

## Past ITPA result

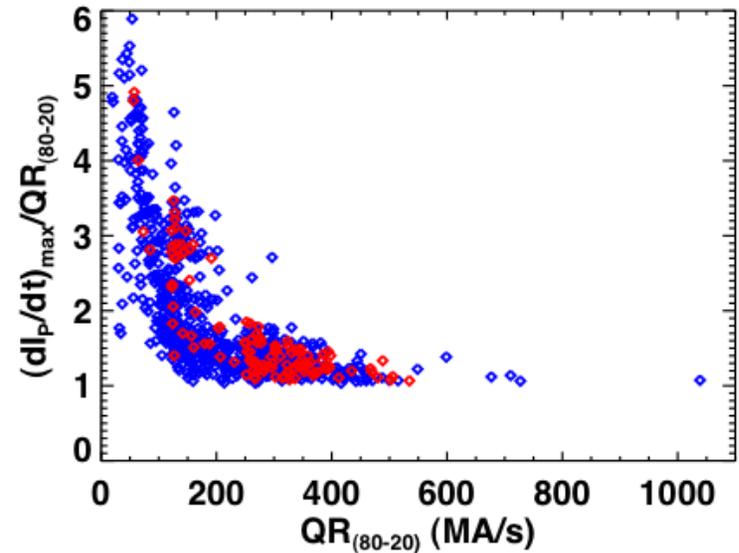
Area-normalized (left), Area and  $L_{ext}$ -normalized (right)  $I_p$  quench time vs. toroidal  $J_p$  (ITER DB)



- ❑ Fastest NSTX disruption quench times of 0.4 ms/m<sup>2</sup>, compared to ITER recommended minimum of 1.7 ms/m<sup>2</sup>
- ❑ Reduced inductance at high- $\kappa$ , low-A explains difference

$$\tau_{L/R} = \frac{\mu_0}{2\pi\eta} \left[ \ln\left(\frac{8}{\sqrt{k\varepsilon}}\right) - \frac{7}{4} \right]$$

## Maximum Quench Rates/Average Quench Rate vs. Average QR

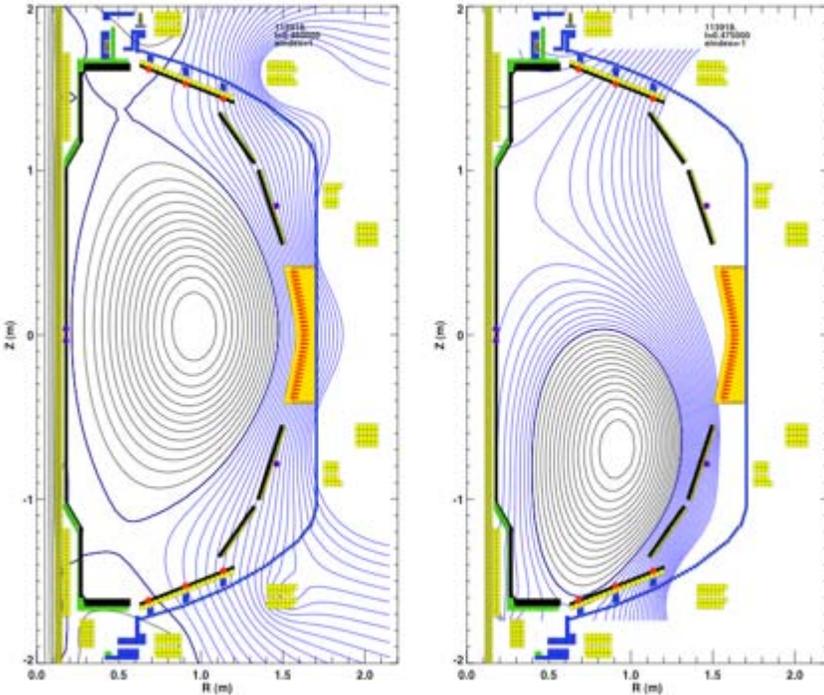


Average Quench Rate:  $QR_{(80-20)} = -\frac{.6 * I_{PD}}{t_{20} - t_{80}}$

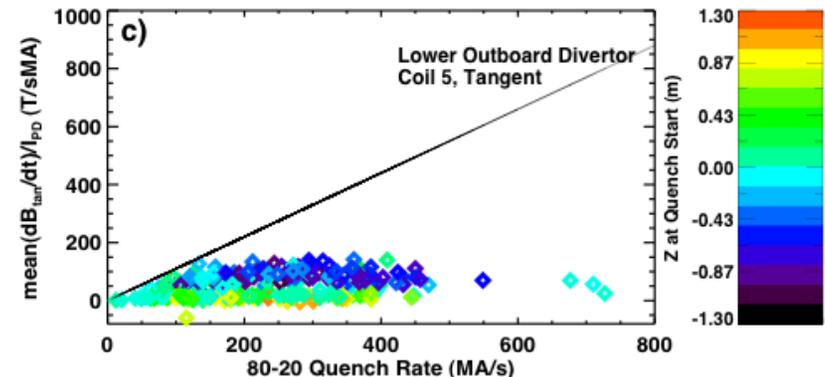
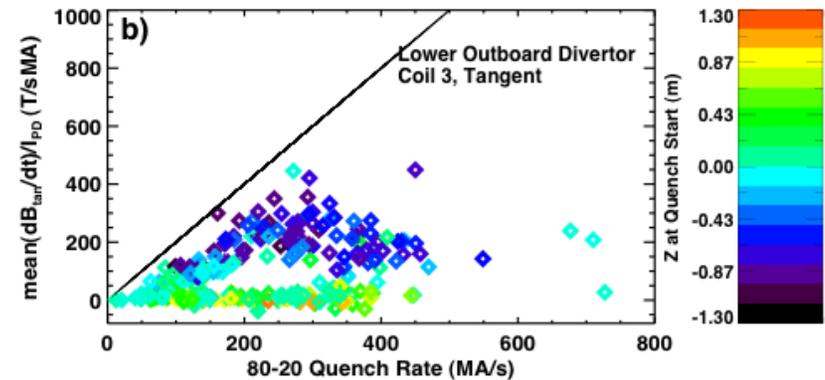
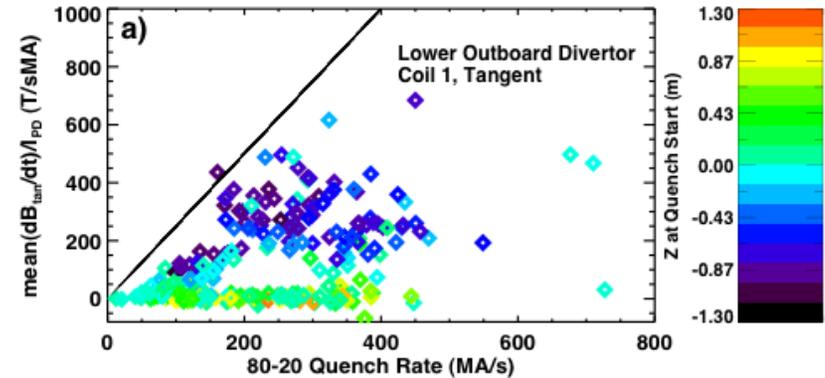
- ❑ Maximum quench rates often larger than average quench rates
  - ❑ Issue for components with short L/R time



# Quench rate important in determining the eddy current drive

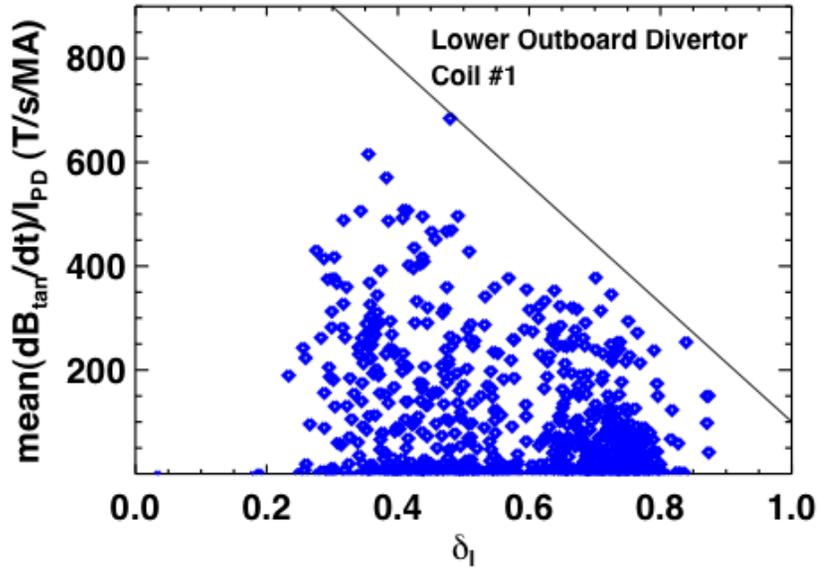


- dB/dt at Lower outboard divertor
  - Points sorted by the axis vertical position just before the disruption
  - Ip-normalized dB/dt increases with quench rate
  - Scatter in data due to details of plasma motion and shape



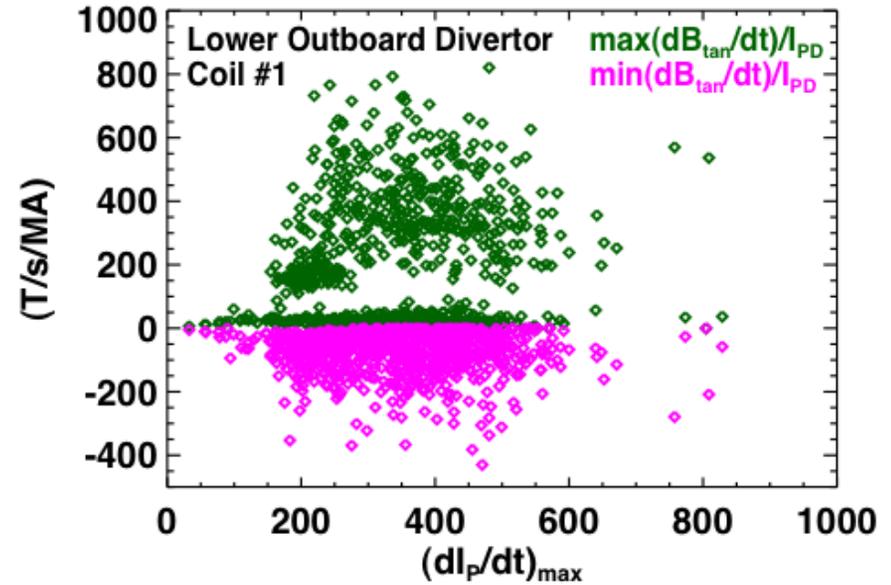
# Geometry and plasma motion affects local field variation

Mean( $dB/dt$ )/ $I_p$  vs Lower Triangularity  
Outboard Divertor



- ❑ At low  $\delta$ , Shaping field coil current attracts plasma toward sensor coil during VDE
  - ❑ maximizes measured  $dB/dt$
- ❑ Same result for upward-going VDE and upper sensor coil

(Max and min)( $dB/dt$ )/ $I_p$  vs ( $dI_p/dt$ )<sub>max</sub>  
Outboard Divertor



- ❑ Large positive values due to large values of  $dI_p/dt$  during quench
- ❑ Large negative values due to rapid downward plasma motion (~constant  $I_p$ )
- ❑ Current quench effect is dominant, but only factor 2 larger

# Halo currents, eddy currents, and current quench rates investigated in ST geometry

- ❑ Large database relevant to disruption EM loading
- ❑ Halo currents up to 150kA measured with new diagnostics,  $I_p/q_{95}$  scaling, in outboard divertor
- ❑ Halo currents up to 60 kA, and no observed scaling, on center stack casing
- ❑ Fastest  $I_p$  quenches of 1GA/s, with instantaneous rates often much faster than the average
- ❑ Current quench rate, plasma geometry, plasma motion important in determining the local eddy current drive

## *Planned for Next Run Campaign*

- Toroidally extended halo current measurements into four liquid lithium divertor (LLD) sectors
  - Toroidally localized halo current measurements at four positions at LLD
    - Fast ( $> 1.5$  kHz) IR thermography for thermal quench studies

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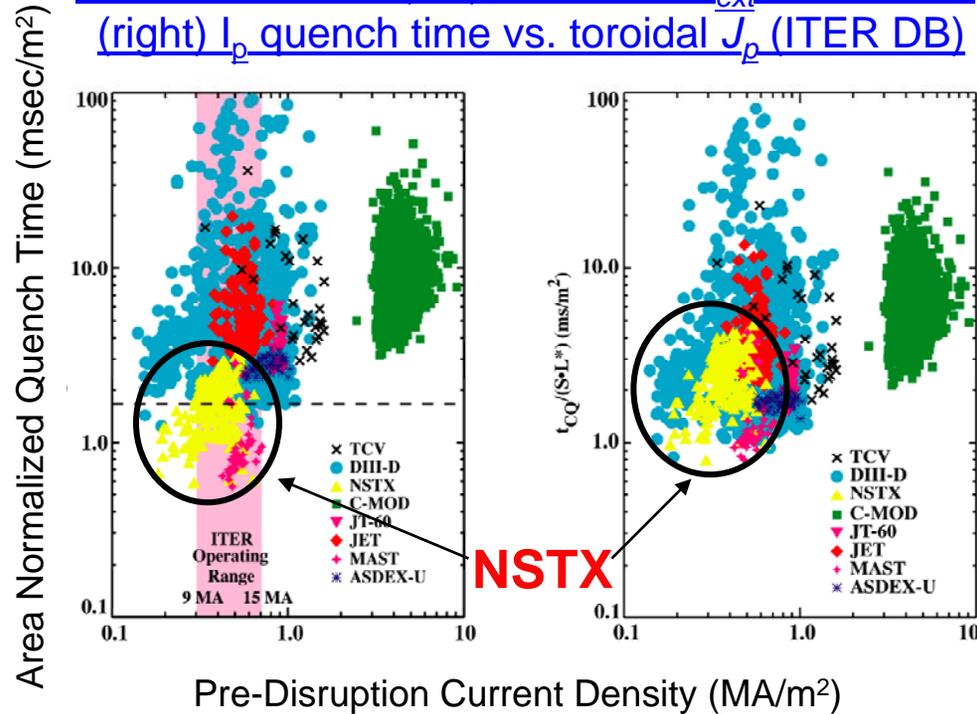
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Backup slides follow



# NSTX Disruption Studies Contribute to ITER, Aim to Predict Disruption Characteristics & Onset For Future Large STs

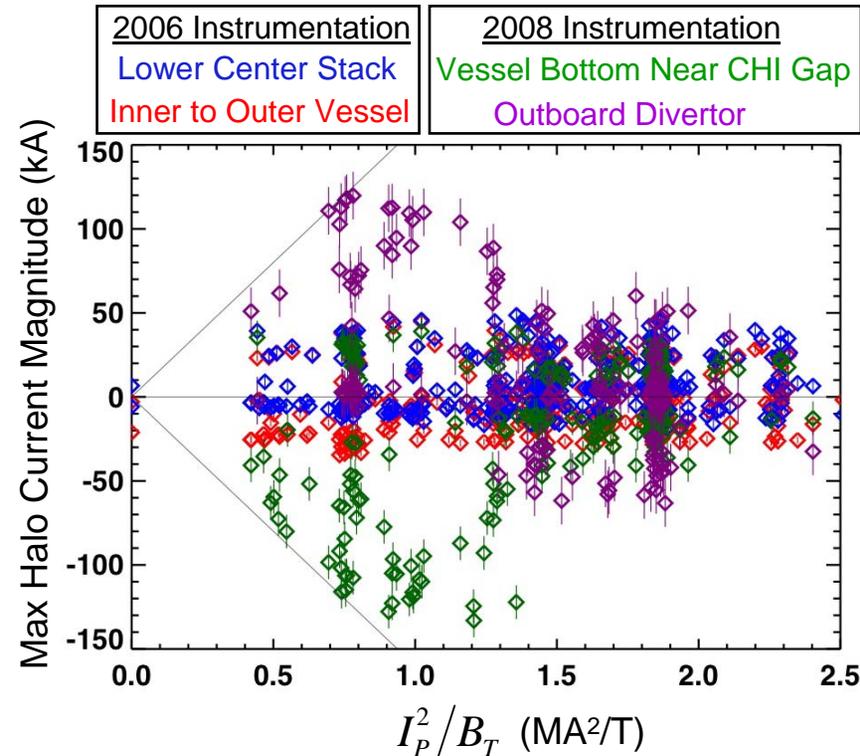
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- Reduced inductance at high- $\kappa$ , low-A explains difference

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## Halo Current Magnitudes and Scaling



- New instrumentation in 2008 yields significant upward revision of halo current fractions (now up to 20%)
  - reveals scaling with  $I_p$  and  $B_T$
  - Mitigating effect: Largest currents for deliberate VDEs
- Toroidal peaking reduced at large halo current fraction

Expand Results For a Complete Characterization of Disruption Dynamics, Including Prediction Methods

