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NSTX

Dissipative divertor experiments in NSTX

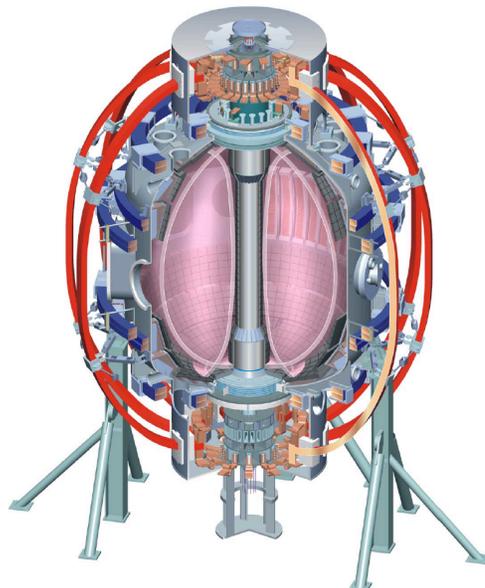
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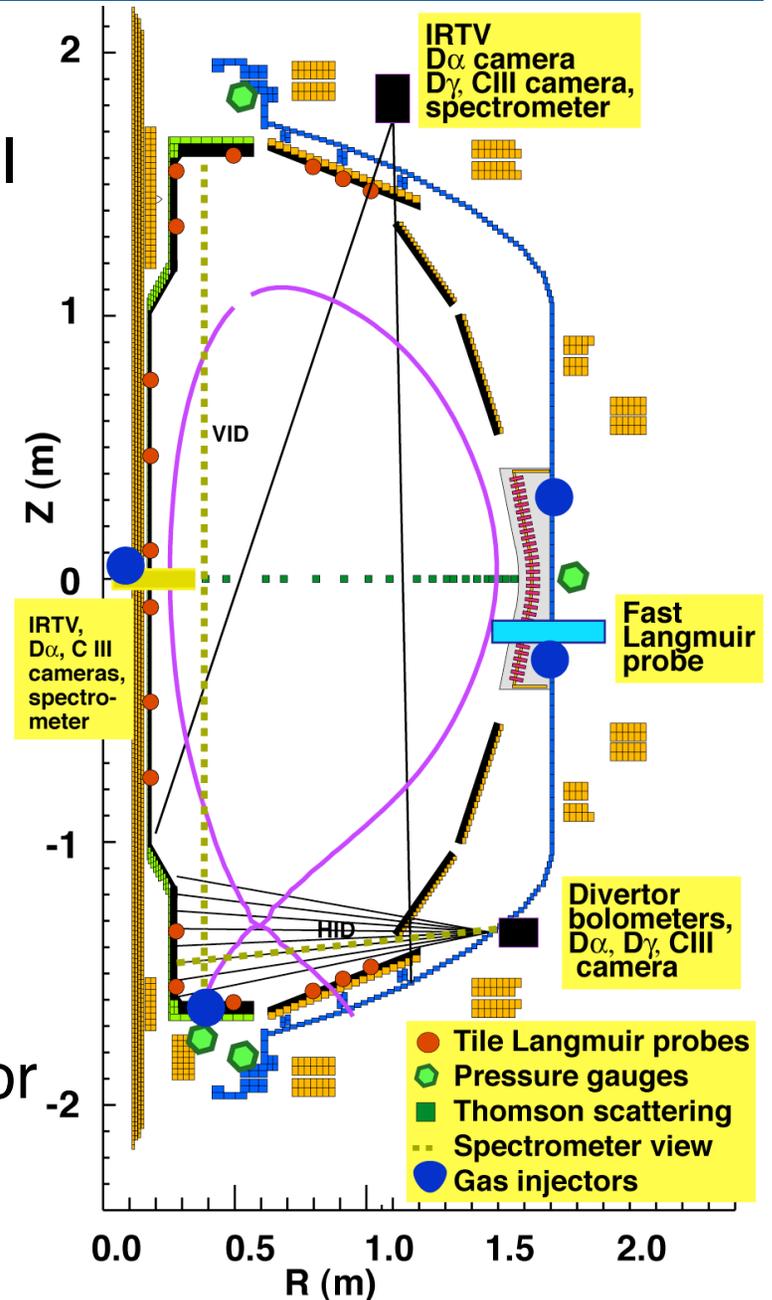
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SOL and divertor physics in ST geometry are studied in NSTX

- Divertor heat load mitigation - one of key Boundary Physics issues in large aspect ratio tokamaks, more so in Spherical Tori (ST)
 - Compact divertor - small divertor volume
 - High edge q (8-20) - smaller PFC plasma-wetted area
 - Typical divertor tile temperature in ~ 1 s NSTX pulses $T < 300$ C. Engineering limit is $T = 1200$ C. Long pulses will require steady-state heat flux mitigation solution
- Present focus is on divertor regime characterization and development of heat flux mitigation scenarios for plasmas with $\kappa = 1.8 - 2.5$, $\delta = 0.4 - 0.9$, at low aspect ratio $A = 1.35$, $q_{\text{out}} < 6$ MW/m², $P_{\text{in}}/R < 9$

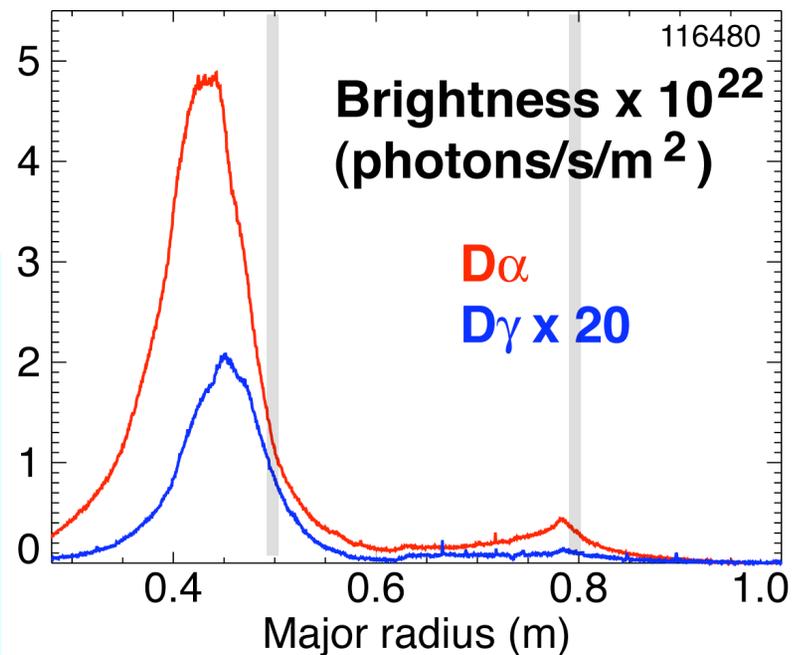
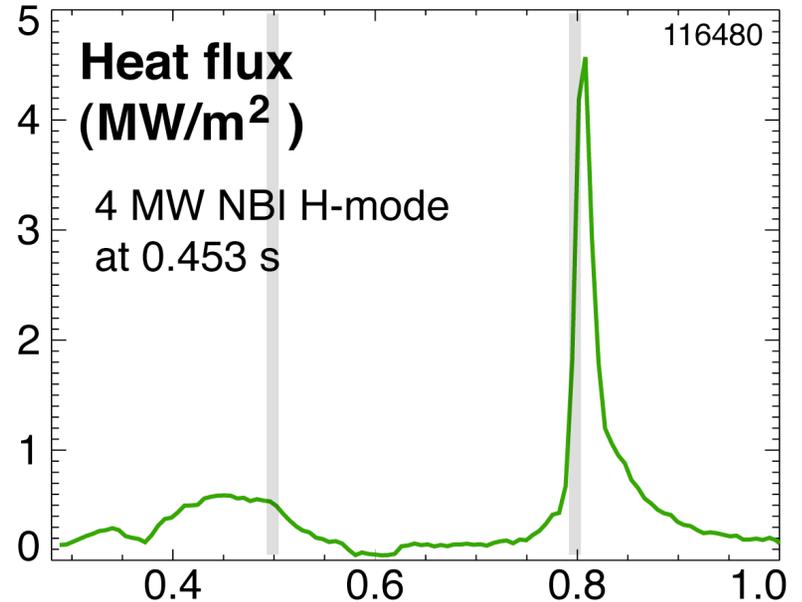
Development of NSTX diagnostic set enabled detailed lower single null divertor studies in FY05

- Divertor IRTV
- Divertor/SOL cameras with D_{α} , D_{γ} , C III filters
- Neutral pressure gauges
- Divertor UV-VIS spectrometer
- Midplane and divertor bolometry
- Divertor and Center Stack Langmuir probes
- Midplane Multi-point Thomson scattering
- Gas Injectors at various poloidal locations - midplane high field side (HFS), low field side (LFS), lower divertor with inj. rates $\Gamma < 10^{22} \text{ s}^{-1}$

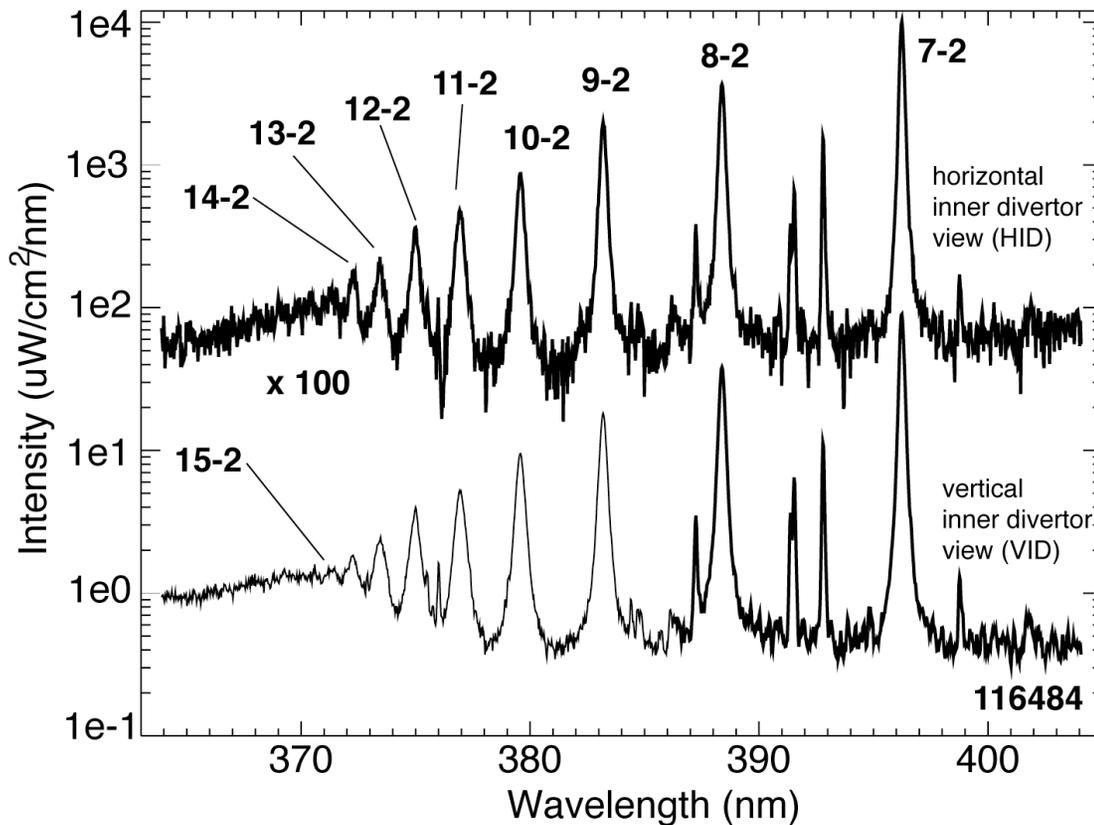


NSTX divertor regimes

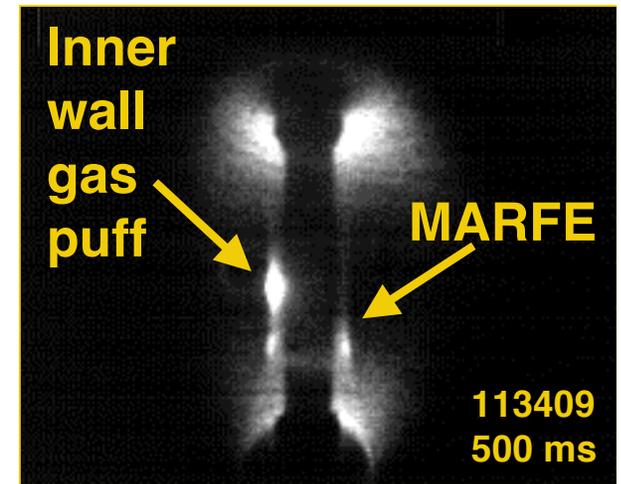
- Observations and measurements:
 - Heat flux asymmetry always $q_{out}/q_{in} > 1$, typically $q_{out}/q_{in} = 2-4$. Typical peak heat flux $q_{in} < 0.5-1.0 \text{ MW/m}^2$, $q_{out} < 2-6 \text{ MW/m}^2$ in 2-6 MW NBI-heated plasmas
 - Recycling in-out asymmetry up to 15 from divertor D_α profiles
 - Divertor D_γ observed in inner divertor only, typical ratio D_γ/D_α about 0.020 - 0.030 - sign of volume recombination
 - High divertor neutral pressure (0.1-0.2 mTorr), neutral compression ratio is 5-10 (open divertor)
- **Inner divertor leg is naturally detached** throughout most of operational space, similarly to conventional tokamak divertors operating w/o pumping. **Outer divertor leg is always attached, being in sheath-limited and high-recycling regime** up to $n_e \sim n_G$



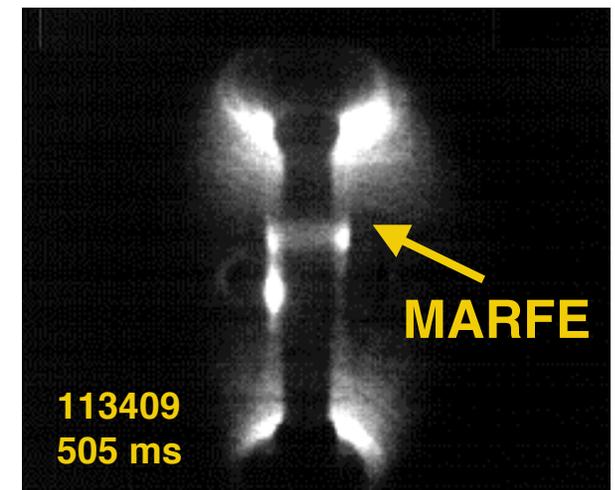
High n_e , low T_e inferred from Balmer series line emission measured in detached inner divertor



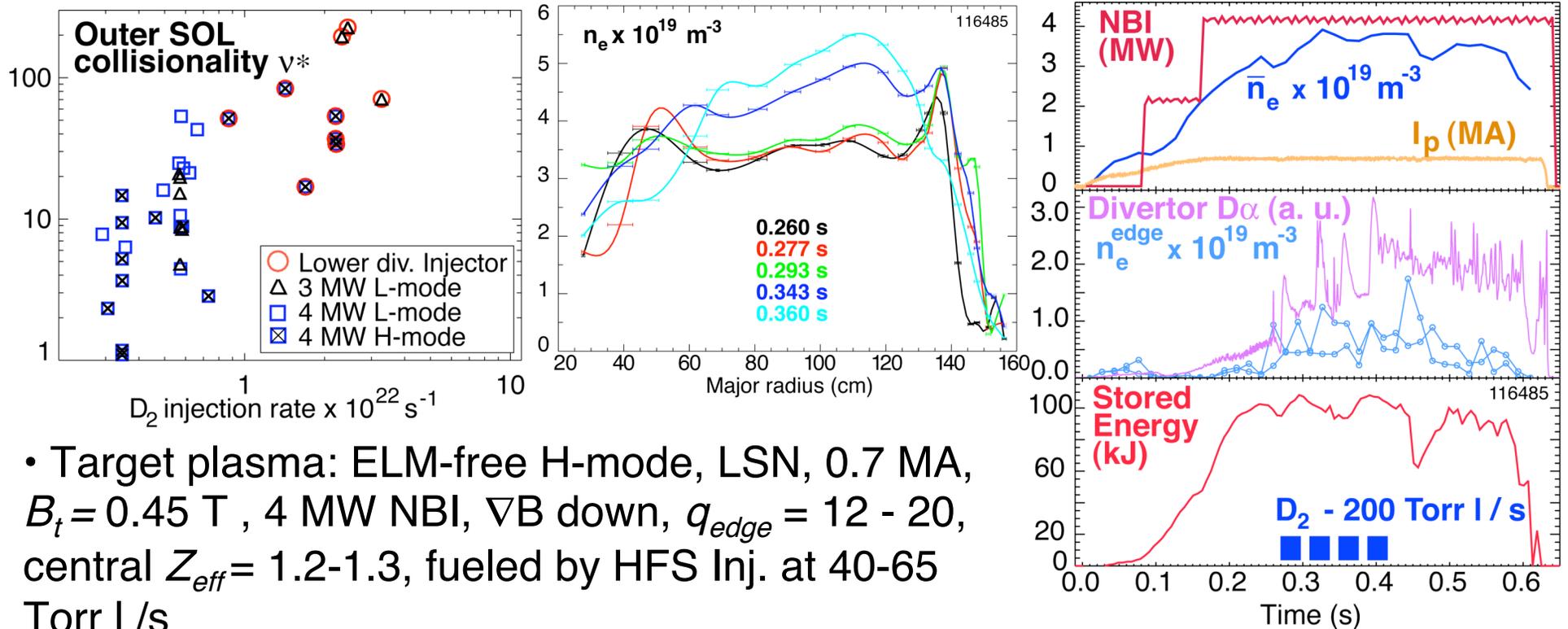
- Stark broadening of 8-2 line yields $n_e = 1-5 \times 10^{20} \text{ m}^{-3}$
- Line intensity ratios (Saha-Boltzman population distribution): $T_e = 0.3-1.0 \text{ eV}$
- Inner SOL MARFE oscillating between midplane and divertor often observed by visible cameras
- Inner divertor region is a cold dense opaque plasma
- challenge for modeling!



Fish-eye view of NSTX plasma



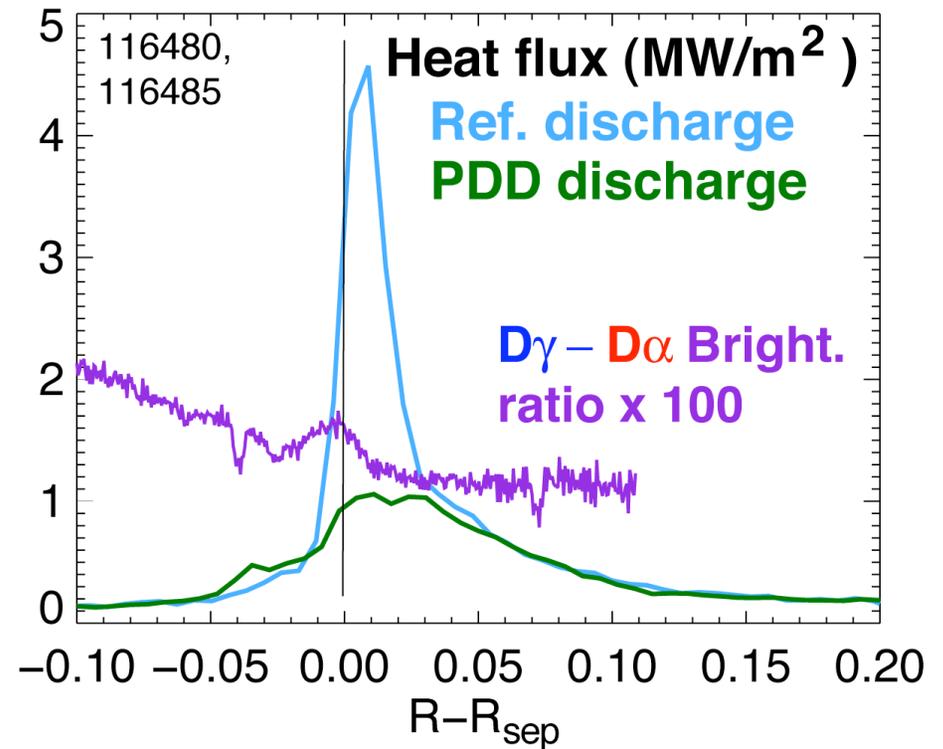
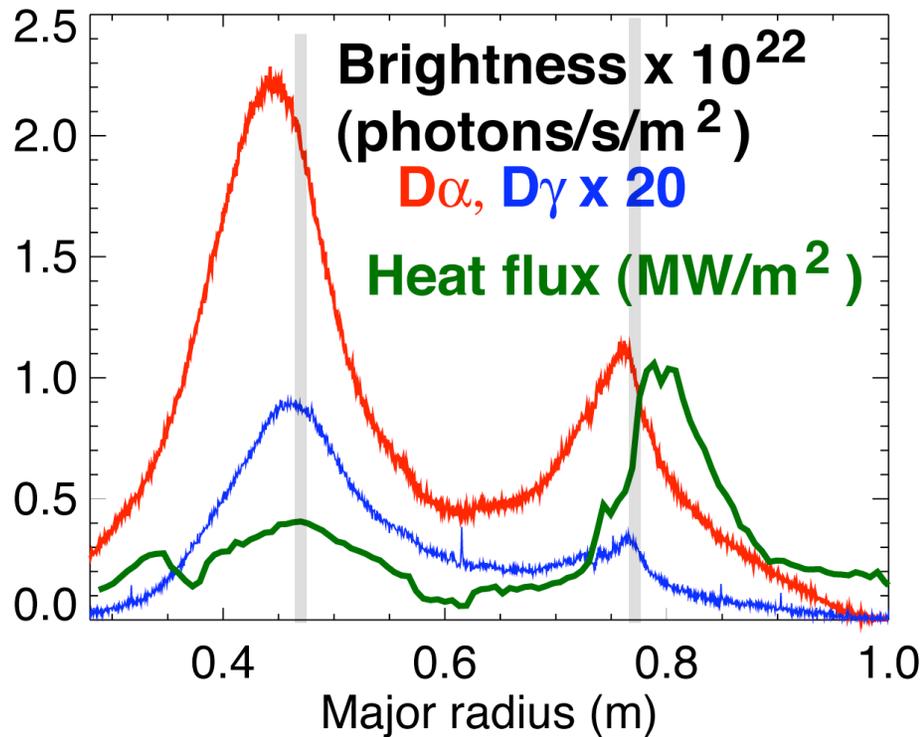
Inject D₂ to detach outer divertor leg



- Target plasma: ELM-free H-mode, LSN, 0.7 MA, $B_t = 0.45$ T, 4 MW NBI, ∇B down, $q_{edge} = 12 - 20$, central $Z_{eff} = 1.2-1.3$, fueled by HFS Inj. at 40-65 Torr l/s

- Injected D₂ at LFS midplane at 20 - 120 Torr l/s and / or lower divertor (LD) at 0 - 200 Torr l/s - found that **D₂ poloidal injection location matters!**
- Retained H-mode for $5 \times \tau_E$ and detached outer leg with LD injection only
- Midplane injections quickly bring plasma to operational limits (confinement degradation, locked modes, large low m, n MHD modes)
- Concluded outer leg *partial* detachment from peak heat flux reduction, radial peak shift, and volume recombination onset (D_γ/D_α ratio)

Outer leg partial detachment evident from peak q_{out} reduction and onset of volume recombination



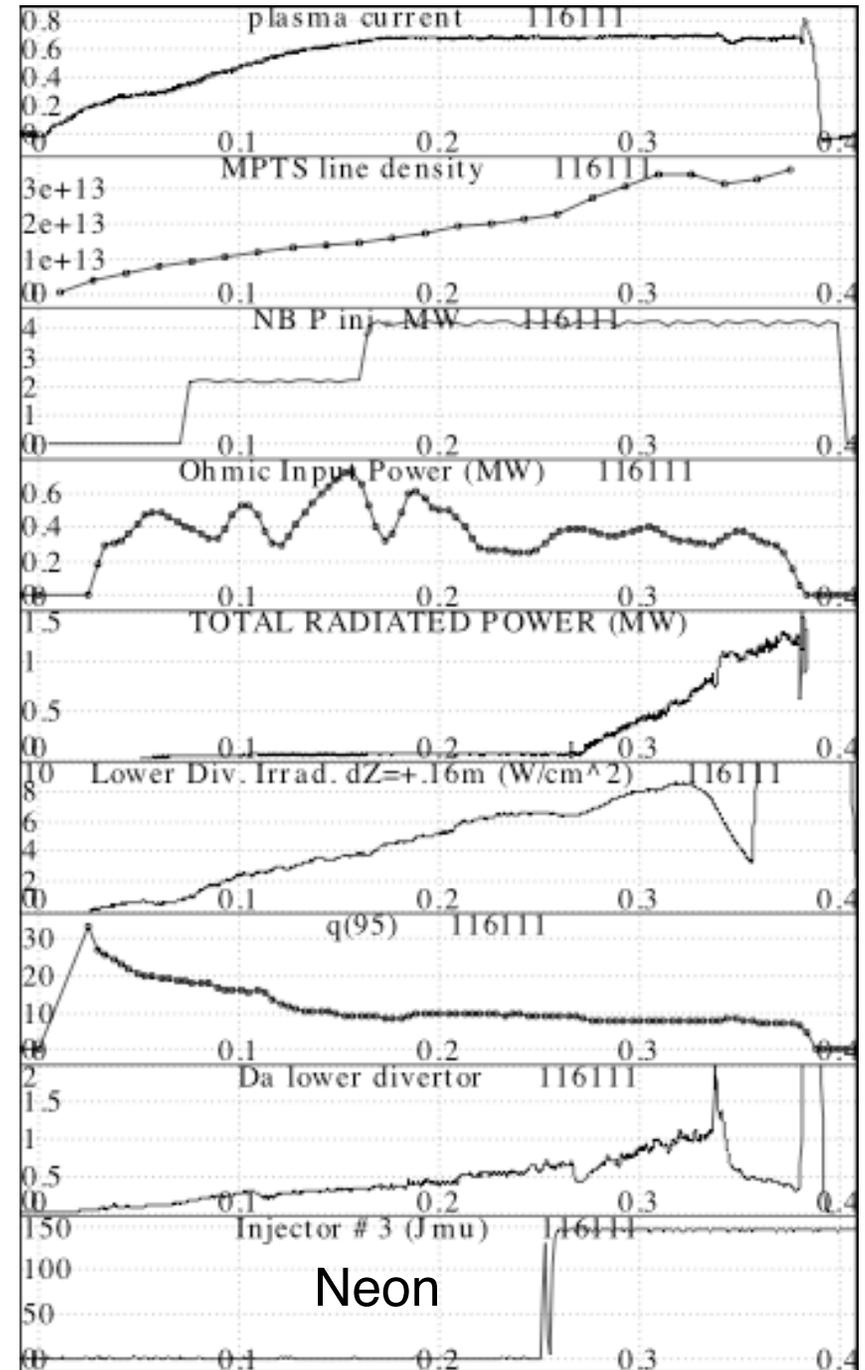
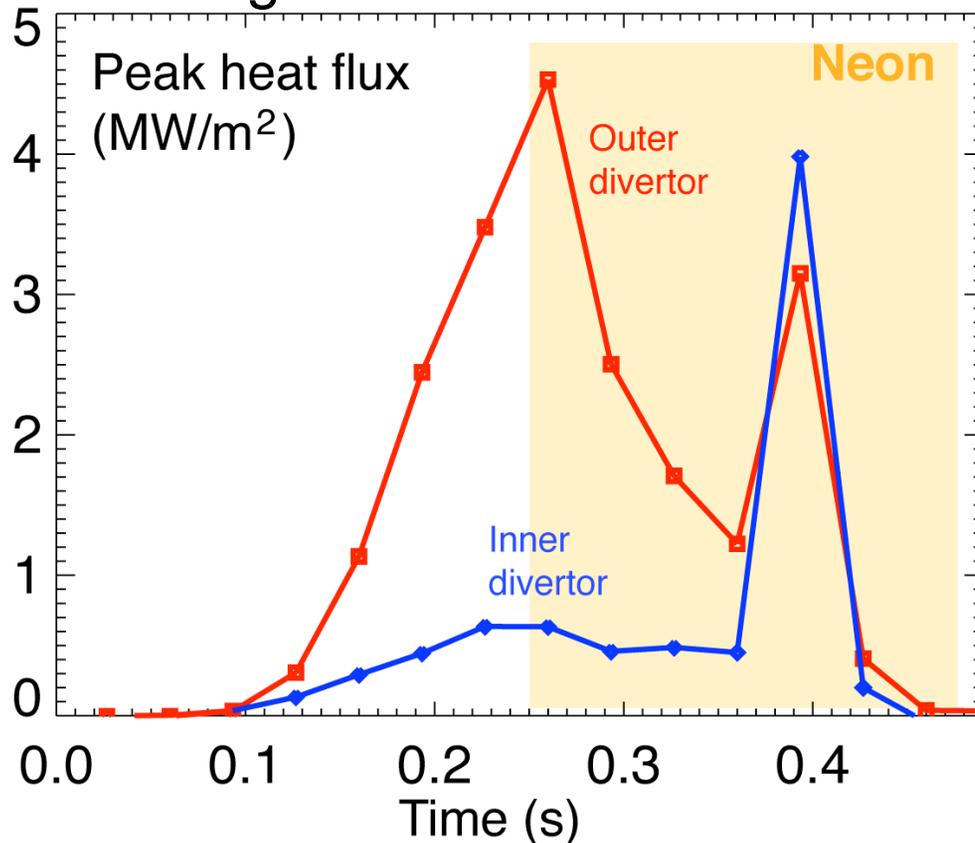
- D_2 puffing from lower divertor
 - Decreases peak q_{out} by 4-5, with peak shift outward by up to 3 cm
 - Broadens D_α and D_γ brightness profiles and increases outer leg D_γ / D_α ratio from typical 0.010-0.015 to 0.015-0.020
 - No change in detached inner divertor - $q_{in} < 0.5$ MW/m², $D_\gamma / D_\alpha = 2-3 \times 10^{-2}$
- Apparently partial detachment
- Separatrix location from LRDFIT

Summary and Future work

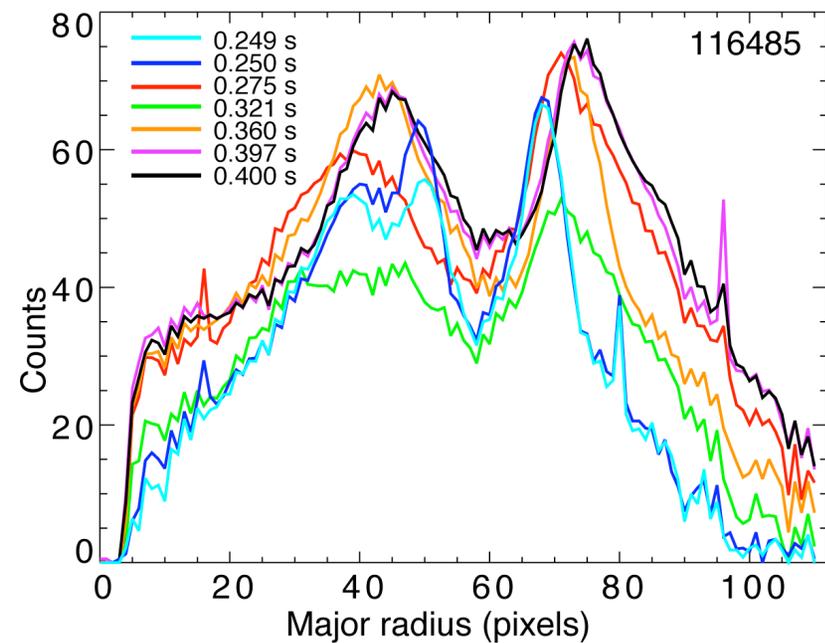
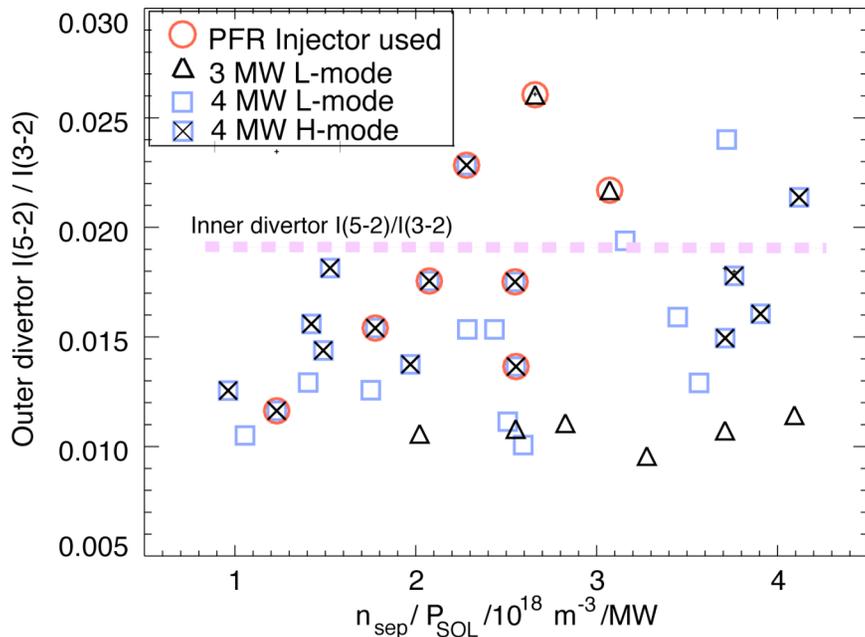
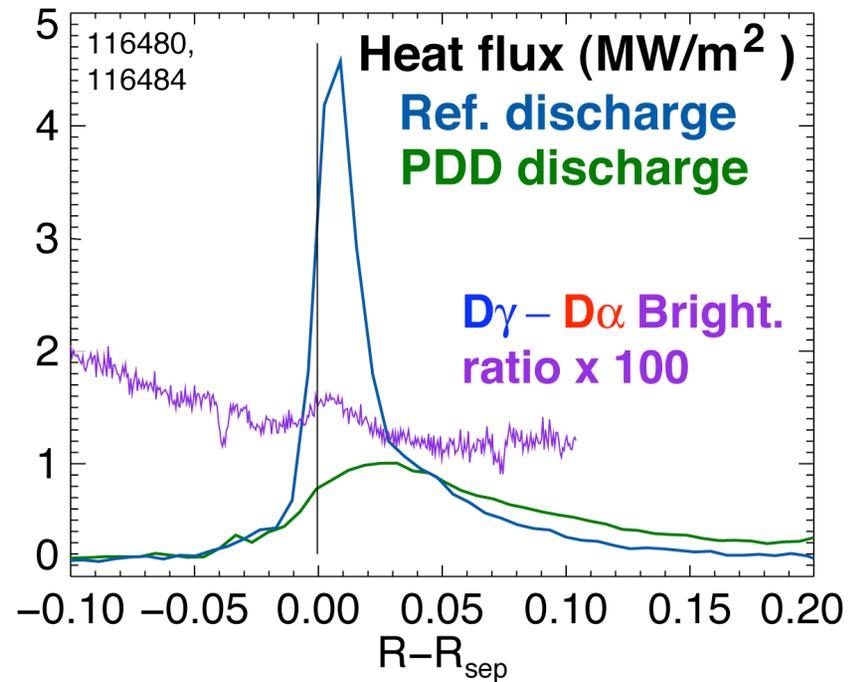
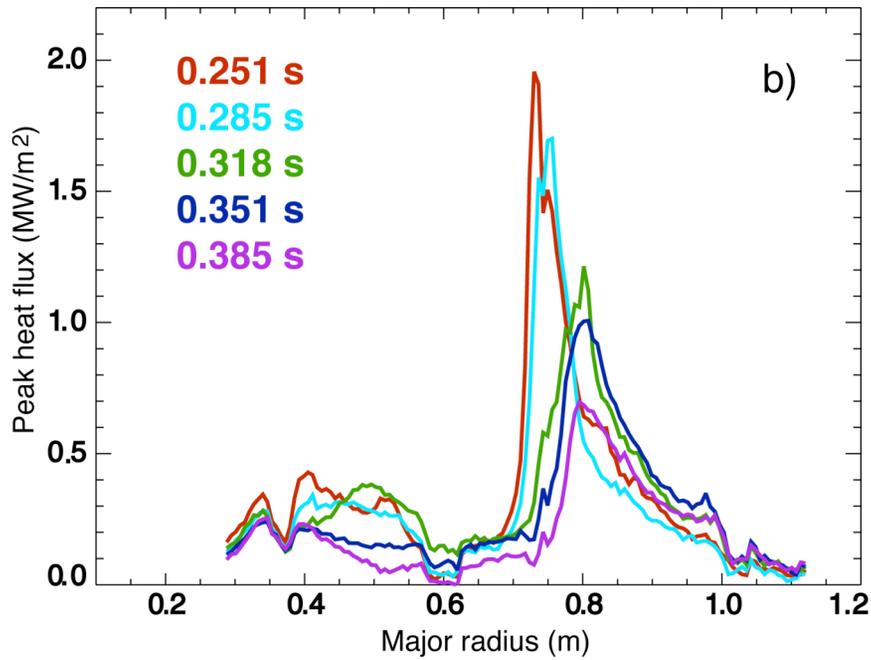
- SOL and divertor physics at low aspect ratio are studied in NSTX
- Longer pulses may cause difficulties in divertor heat flux handling given the naturally occurring NSTX divertor regimes
- D₂ injections were used to detach outer divertor leg in 3-4 MW NBI-heated L- and H-mode plasmas
- Signs of outer leg partial detachment were obtained in 4 MW H-mode plasmas with D₂ injections from lower divertor region: reduction of peak heat flux, shift of heat flux peak, and spectroscopic signatures of volume recombination
- Future work will focus on
 - further characterization of outer leg detachment
 - dissipative divertor scenario development for high δ , κ H-mode plasmas

Neon injections did not detach outer divertor

- Neon puffed into 4 MW H-mode
- Did not cause H-L transition
- Puffing rate $1.5 \times 10^{20} \text{ s}^{-1}$
- $P_{\text{rad}} = 0.3 \times P_{\text{in}}$
- Outer peak heat flux reduced x4, but no sign of volume recombination!



Stronger gas puff leads to stronger detachment



NSTX reference data

NSTX fueling

- Gas injection: low field side (LFS, top + side), high field side (HFS, midplane + shoulder), private flux region. D_2 , He, injected at $S = 20 - 120$ Torr l / s.
- Neutral beam injection system: three beams, 80 - 100 keV, 6 MW, fueling rate: $S < 4$ Torr l / s
- Supersonic gas injection: $S = 60$ Torr l / s

NSTX wall conditioning

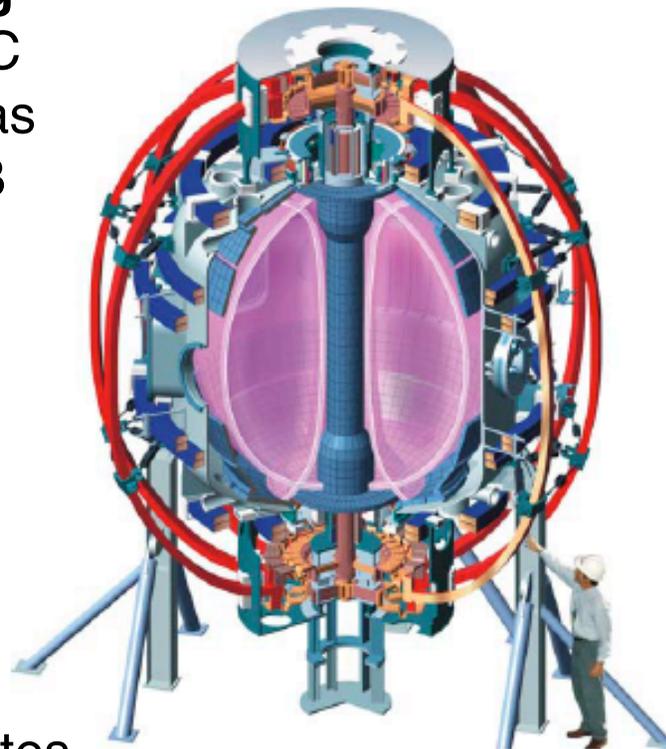
- Between shots He GDC
- He conditioning plasmas
- TMB and Plasma TMB

NSTX pumping

- Turbomolecular pump
(3400 l / s)
- NBI cryopump
(50000 l / s)
- Conditioned walls

PFC

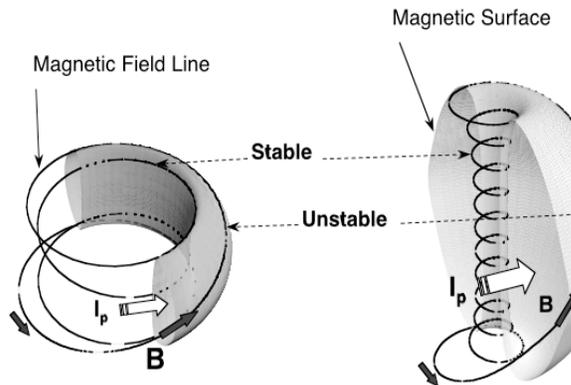
- ATJ graphite tiles on divertor and passive plates
- ATJ and CFC tiles on center stack
- Thickness 1" and 2"



Aspect ratio A	1.27
Elongation κ	2.5
Triangularity δ	0.8
Major radius R_0	0.85m
Plasma Current I_p	1.5MA
Toroidal Field B_{T0}	0.6T
Pulse Length	1s
Auxiliary heating:	
NBI (100kV)	7 MW
RF (30MHz)	6 MW
Central temperature	1 – 3 keV

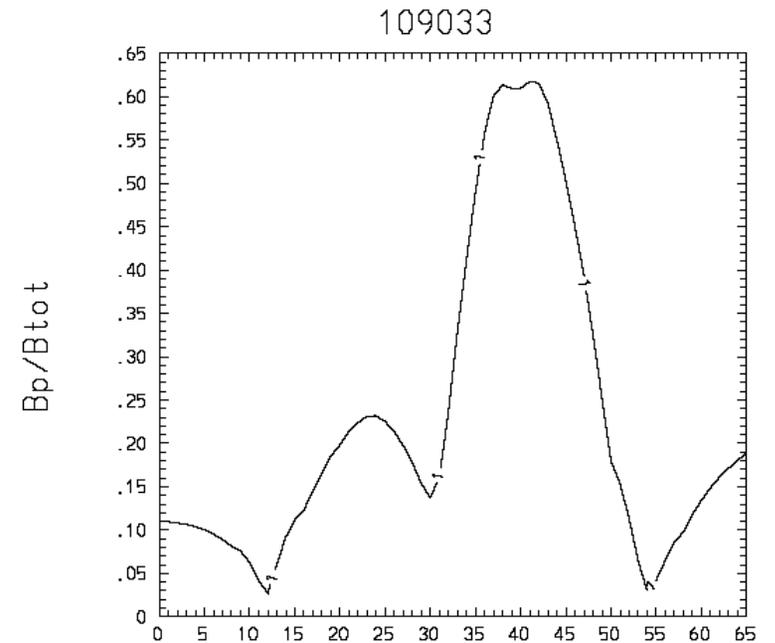
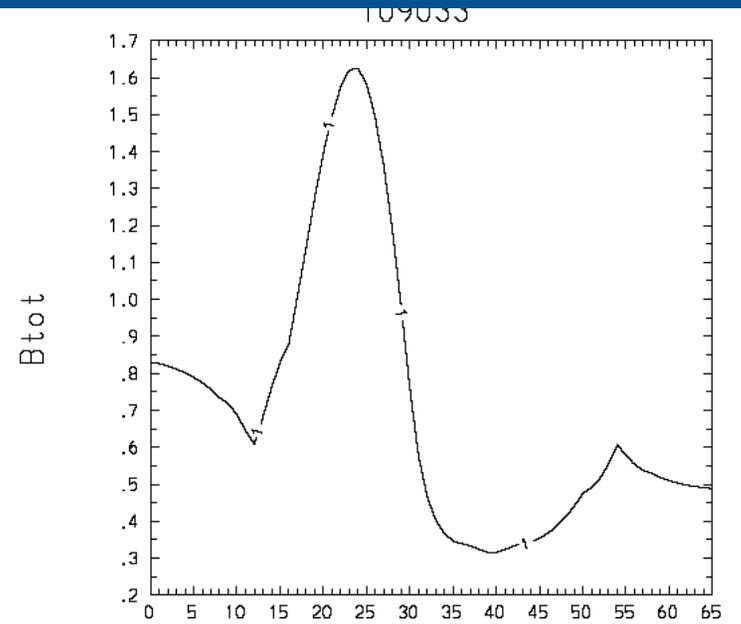
NSTX reference data

κ	1.85
δ	0.4
Drsep (cm)	1.0-1.5
q_{edge}	13
L_{\parallel} , inner (m)	8.3
L_{\parallel} , outer (m)	6.0
M (Mirror ratio)	5.0
f inner (Flux expansion)	2-3
f outer (Flux expansion)	2-3



Tokamak
(safety factor $q = 4$)

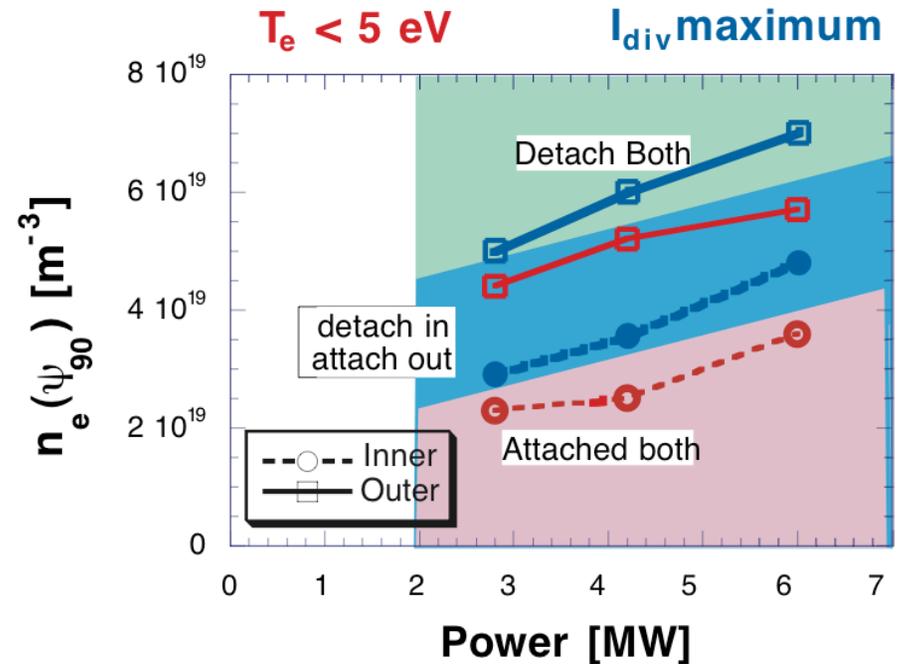
Spherical To
(safety factor $q =$



poloidal index

UEDGE modeling guided detachment experiment

- UEDGE - multifluid 2D edge code based on Braginskii equations
- Model divertor conditions vs P_{in} , n_{edge} for guiding purposes
- H-mode LSN equilibrium used
- diffusive transport model
- Impurities (carbon) included
- Outer midplane n_e , T_e profiles matched, D_α and IRTV not matched



G. Porter, N. Wolf

Attempt to change parallel momentum and power balance:

$$\frac{d}{ds} (m_i n v^2 + p_i + p_e) = -m_i (v_i - v_n) S_{i-n} + m_i v S_R$$

$$\frac{d}{ds} \left(-\kappa T_e^{5/2} \frac{dT_e}{ds} \right) + n v_{||} \left(\frac{5}{2} (T_i + T_e) + \frac{1}{2} m_i v_{||}^2 + I_0 \right) = S_E$$

Development of NSTX diagnostic set enabled detailed lower single null divertor studies in FY05

- **Divertor IRTV**

two Indigo Alpha 160 x 128 pixel microbolometer cameras, 7-13 μm range, 30 ms frame rate

- **Divertor/SOL cameras with D_{α} , D_{γ} , C III filters**

four Dalsa 1 x 2048 pixel CCDs, filter FWHM 10-15 A, frame rate 0.2 - 1 ms

- **Neutral pressure gauges**

four microion gauges on top and at midplane, two Penning gauges in lower and upper divertor, time response 5-10 ms

- **High-resolution divertor UV-VIS spectrometer**

ARC Spectro-Pro 500i, 3 input fibers, time response 15-30 ms, FWHM > 0.6 A

- **Midplane and divertor bolometry**

midplane (AXUV radiometer array), divertor - ASDEX-type four channel bolometer, time response 20 ms

- **Divertor Langmuir probes**

midplane - fast probe, tile LPs - I_{sat} T_e measurements

- **Midplane Multi-point Thomson scattering**

with 2-4 points in SOL

- **Gas Injectors at various poloidal locations, $R < 10^{22} \text{ s}^{-1}$**

- **Enhanced shaping capability - Lower Single Null, $\kappa = 1.8 - 2.5$, $\delta = 0.4 - 0.9$**

