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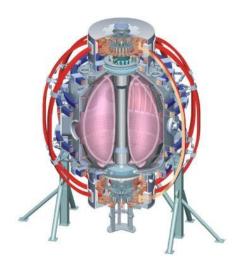


### **Overview of NSTX Liquid Lithium Divertor Performance and Divertor Upgrade Plans**

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**American Physical Society, Division of Plasma Physics Meeting,** 





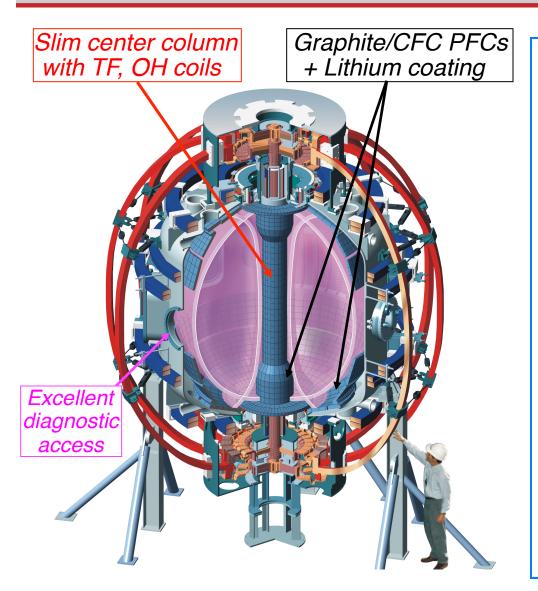
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### NSTX Designed to Study High-Temperature Toroidal Plasmas at Low Aspect-Ratio



Aspect ratio A 1.27 - 1.6

Elongation  $\kappa$  1.8 – 3.0

Triangularity  $\delta$  0.2 – 0.8

Major radius 0.85 m

Toroidal Field  $B_{T0}$  0.4 – 0.55 T

Plasma Current  $I_p$  0.7 – 1.5MA

Auxiliary heating:

NBI (100kV) 7 MW

RF (30MHz) 6 MW

Central temperature 1 – 5 keV

Central density ≤1.2×10<sup>20</sup>m<sup>-3</sup>

Toroidal beta  $\beta_T$  10 – 40 %

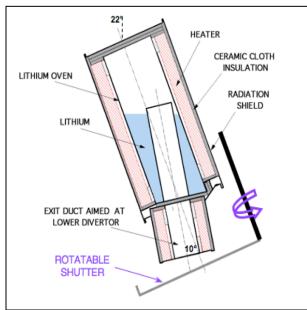
### Liquid has Been Proposed for Providing a Self-healing Plasma Facing Surface for DT Reactors

- NSTX lithium program on <u>diverted H-mode plasmas</u> grew from experience with TFTR limiter plasmas, and is aimed towards using liquid lithium to control density, edge recycling, impurity influxes, and eventually power handling.
  - Edge fueling is reduced as plasma D efflux incident on Li forms LiD
    - Solid lithium provides short pulse capability but has limited LiD capacity
    - Liquid lithium has much higher LiD capacity, and has potential for power handling and self healing
- Over the longer term, NSTX will investigate if liquid lithium can help integrate 4 important potential benefits for fusion
  - Divertor pumping over large surface area compatible with high flux expansion solutions for power exhaust and low collisionality
  - Improved confinement
  - ELM reduction and elimination
  - High-heat flux handling (e.g., via capillary-porous flow,...)

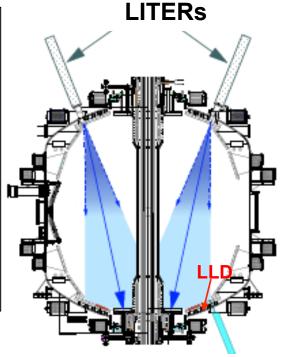


### Dual <u>Lithium Evaporators</u> (LITERs) Are Now Routinely Used To Deposit Lithium Coatings On NSTX Lower Divertor Between Discharges

#### **LITER Oven**



 Rotatable shutter stops lithium when diagnostic window shutters are open.

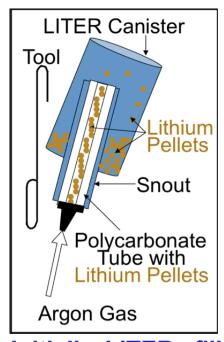


 LITERs aimed toward the graphite divertor.
 Shown are 1/e widths of the emitted distribution.  Lithium transported over broad area by wings of LITER distribution and plasma migration.

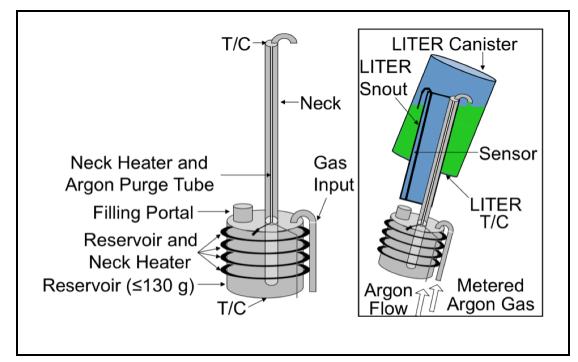


 After exposure to air (2009), 600g Li deposition converts to white lithium carbonate (Li<sub>2</sub>CO<sub>3</sub>).

# LITERs Filled Using Liquid Lithium Fill System for Improved Loading Efficiency and Reduced Impurities



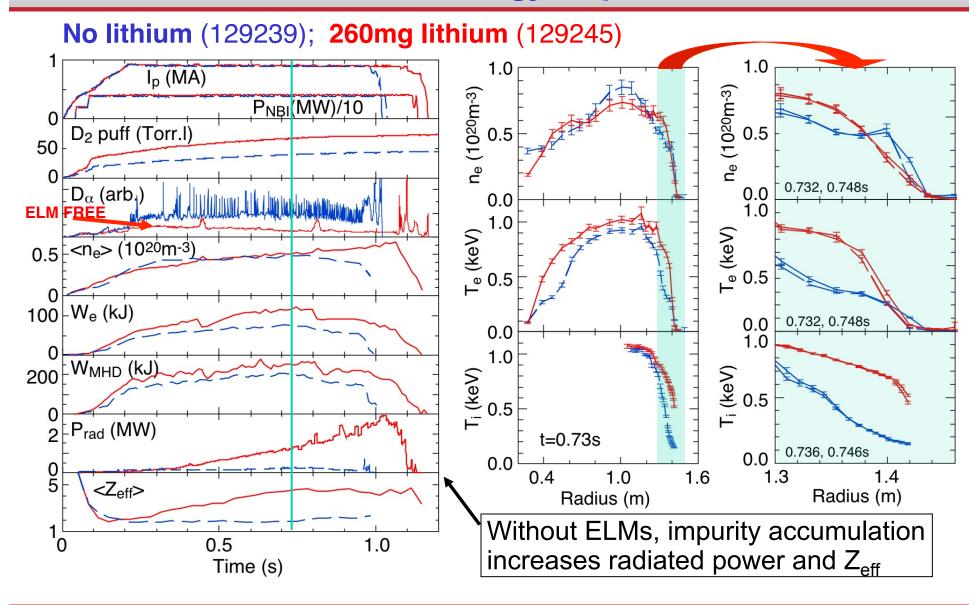
 Initially, LITERs filled using solid Li pellets (50% packing fraction for pellets, 40g max).



 Recently, LITERs filled using liquid lithium injected with argon (100% packing fraction, for liquid, 80g max).

 After lithium filling, prior to installation on NSTX, LITERs are outgassed in vacuum chamber by raising temperature slowly to 600°C to remove any argon and dissolved gases.

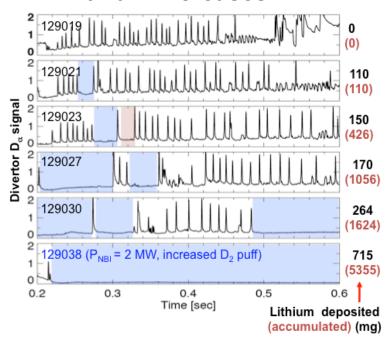
# Lithium Coating Reduces Deuterium Recycling, Suppresses ELMs, Increases Stored Energy, Improves Confinement



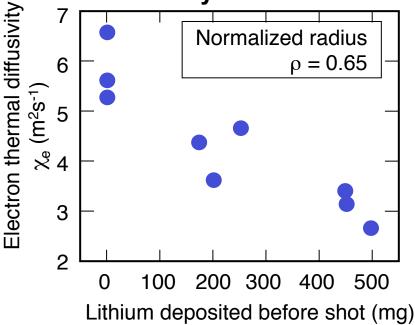


# Lithium Evaporated on Divertor Suppresses ELMs and Improves Confinement

 ELMS Reduced as Lithium Increases

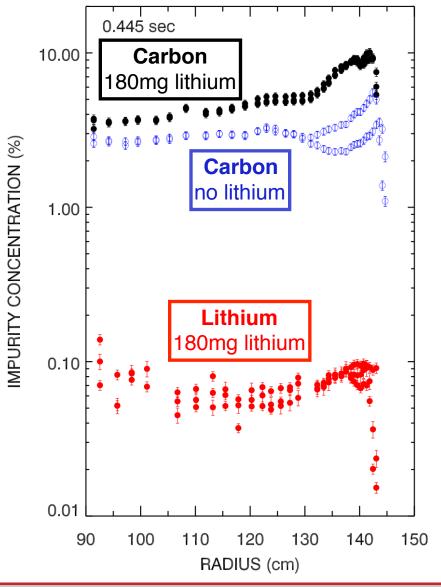


 TRANSP analysis confirms electron thermal transport in outer region progressively reduced by lithium.



- H-mode threshold reduced by lithium coating by up to factor 4
- Fast-ion contribution to total energy (∝T<sub>e</sub><sup>3/2</sup>/n<sub>e</sub>) also increases

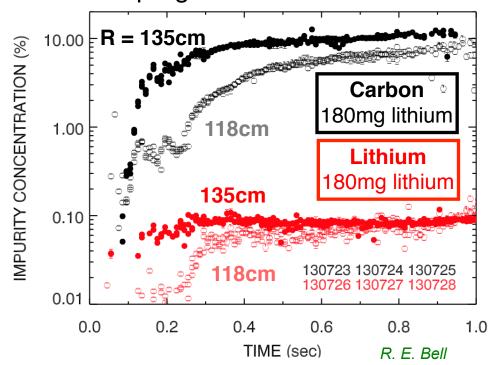
# Lithium Concentration in Plasmas Remains Low but Carbon Concentration Rises with Lithium Coating



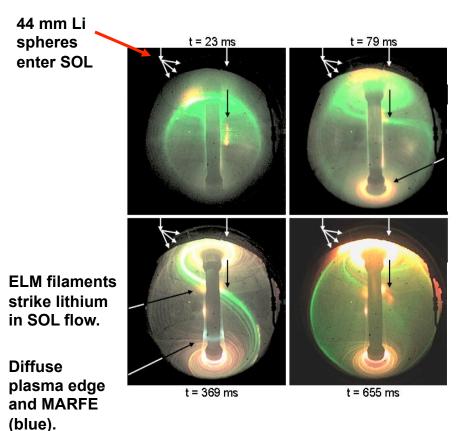
Quantitative measurements of C<sup>6+</sup>, Li<sup>3+</sup> with charge-exchange recombination spectroscopy

$$n_{\rm C}/n_{\rm Li} = 30 - 100$$

Hollow profiles early for both C and Li fill in as time progresses



# Lithium Powder Injection in Progress to Test Increased Rate of Lithium Delivery to Plasma Wetted Areas

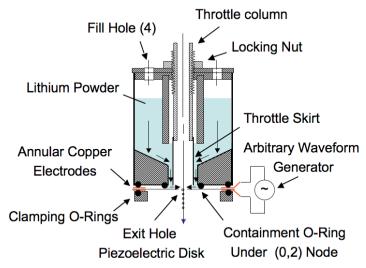


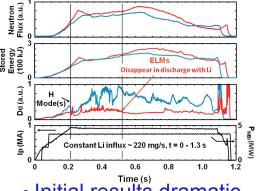
As NBI starts, lithium is swept toroidally in SOL flow

Lower single null strike points evaporate Li before t=0.

ELMS and MARFES vanish by t~500ms. Sharp plasma edge and Li radiative mantle

### Schematic of Lithium Powder Piezoelectric Acoustic Injector

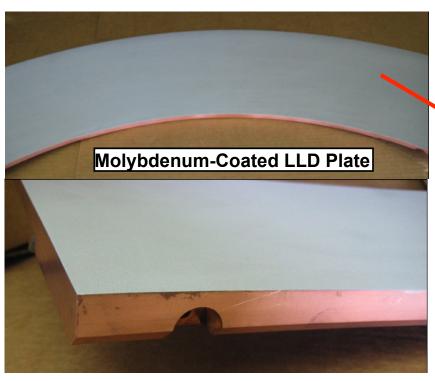




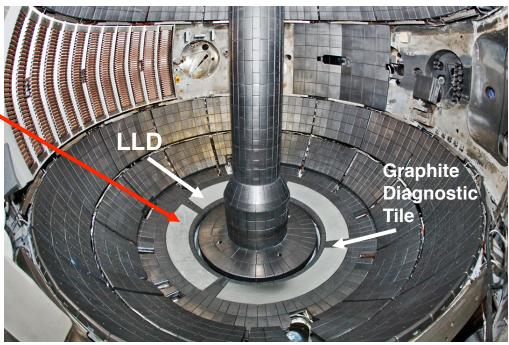
 Initial results dramatic but not yet reproducible

- Lithium particles delivered before and during discharges.
- Particle delivery rates up to 80 mg/s (LITER ~10-20 mg/min per unit)

# Liquid Lithium Divertor (LLD) Installed in NSTX with Porous Molybdenum Face to Hold Lithium



 0.165 mm Mo plasma sprayed with 45% porosity on a 0.25 mm SS barrier brazed to 22.2 mm Cu.



- 4 heated plates (80°each) separated by graphite diagnostic tiles.
- Each toroidal section electrically grounded to vessel at one midsegment location to control eddy currents.

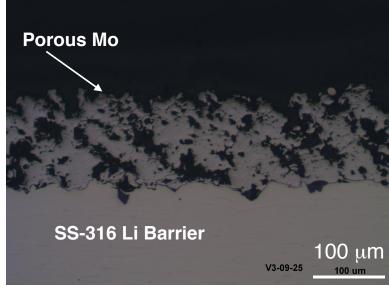
### **LLD Lithium-bearing Surface is Thin Porous Molybdenum**

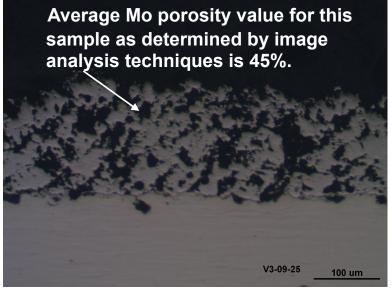
#### Key properties of an acceptable LLD lithium surface

- sufficient surface tension to hold Li in presence of JxB forces
- ability of liquid Li to flow across metal surface (wetting capability)
- minimize temperature rate of rise of Li --> rapid heat transfer from Li to base

Thin plasma sprayed porous Mo, on a thin SS-316 Li barrier, on thick Cu baseplate thermal sink, is highest confidence initial approach

Cross sectional views of plasma sprayed porous molybdenum LLD sample





Longitudinal Transverse

• The average thickness of the deposit on this sample was 0.152 mm



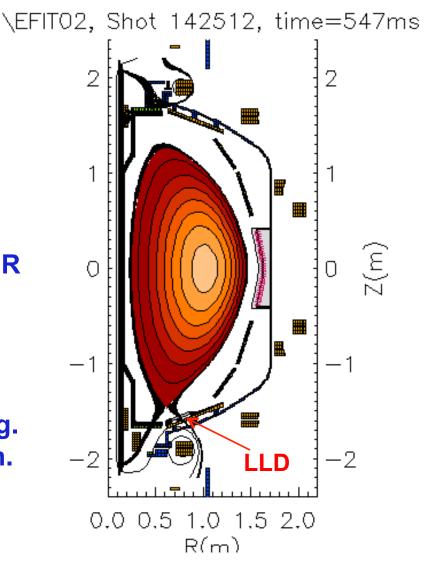
# Discharges Run With Outer Strike Point on Liquid Lithium Divertor

Discharge

$$I_p = 0.8MA$$
,  
 $P_{NB} = 4.0MW$   
 $B_T = 0.48T$   
 $R_{OSP} = 0.78m$ 

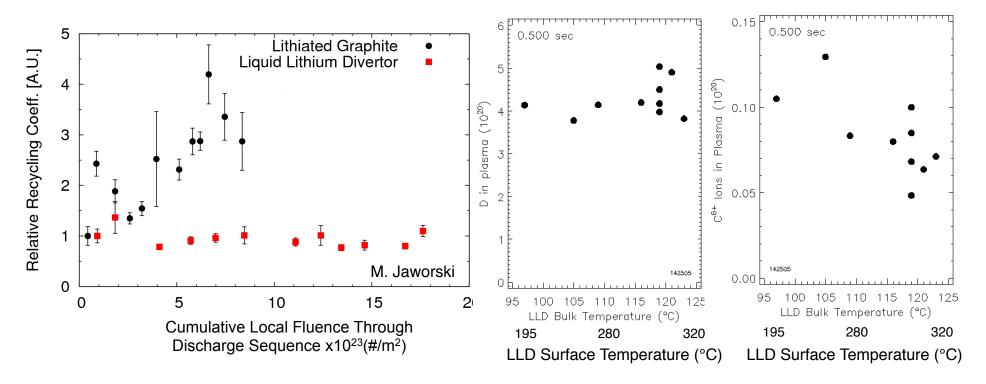
#### LLD Lithium Content

- LLD loaded using one LITER (2.5% eff. at LLD radius)
- 7.5g evaporated from one LITER preceding the exp.
- Estimated fresh Li coating on LLD 0.025 x 7.5g =0.188g.
- Fresh Li thickness ~0.35 $\mu$ m.



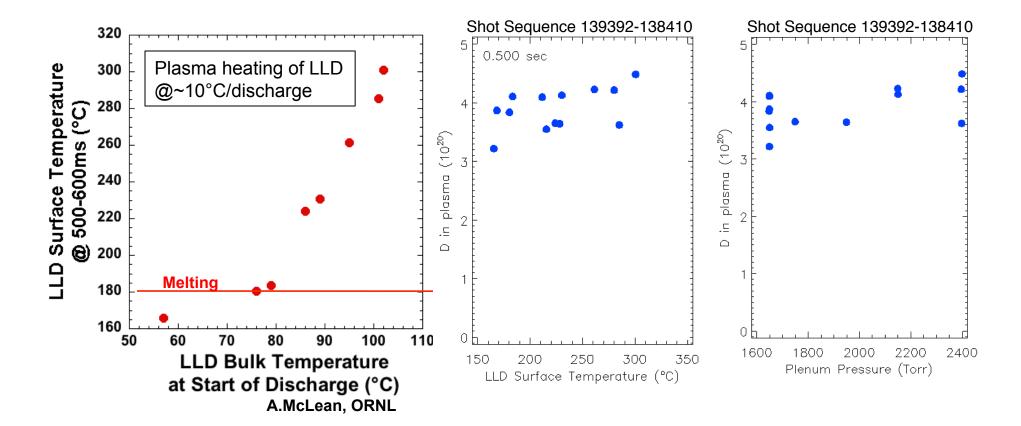


### Constant Fueled Discharges With Outer Strike Point on LLD Exhibit Persistence of Recycling Behavior, Constant Core Particle Content, & Decrease in Core Carbon as LLD Liquefies



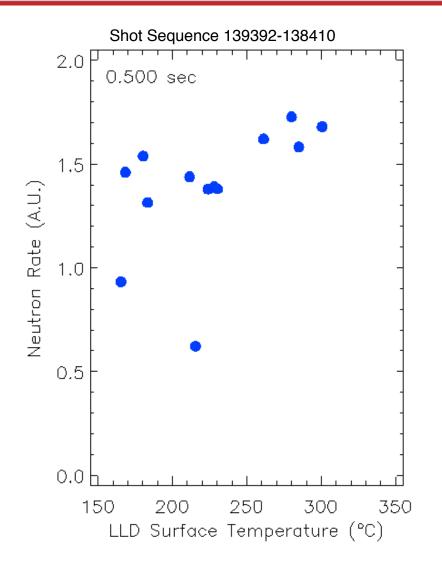
- Systematic rise in relative recycling coefficient on lithiated-graphite, but not on LLD.
   Indicates difference between lithiated-graphite and LLD.
- Multi-shot ion fluence indicates LLD has "reservoir" effect compared to Li-graphite.
- Liquid lithium decreases core carbon.

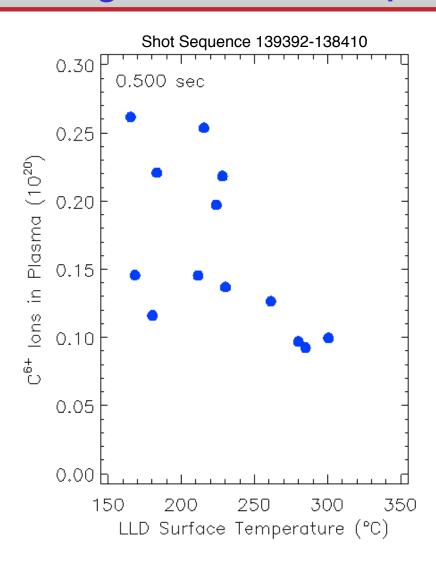
## Discharges With Outer Strike Point on LLD and Increasing Fueling Exhibit Constant Core Particle Content, as LLD Liquefies





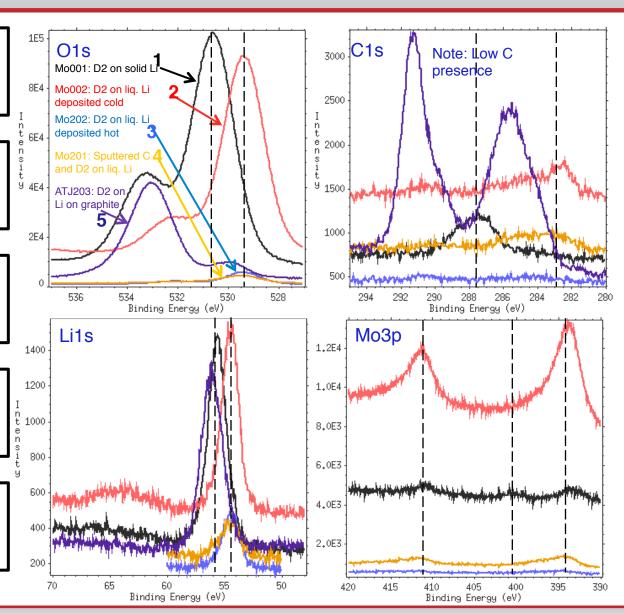
# Discharges With Outer Strike Point on LLD and Increasing Fueling Exhibit Decreasing Carbon and Increasing Neutrons as LLD Liquefies





### LLD Porous Molybdenum Samples Analyzed 5 Laboratory Irradiation Scenarios to Test Various Possible NSTX Conditions

- 1) Mo001: 30 min  $D_2$  irradiation on  $2\mu$ m Li deposited on cold Mo.
- 2) Mo002: 30 min  $D_2$  irradiation on  $2\mu$ m Li at 255° C. Li deposited on cold Mo.
- 3) Mo202: 30 min  $D_2$  irradiation on  $3\mu$ m Li at 200° C. Li deposited on Mo at 250° C.
- 4) Mo203C: Carbon sputtered (via Ar) onto Li-coated Mo. 30 min D<sub>2</sub> irradiation at 200° C.
- 5) ATJ203: 30 min  $D_2$  irradiation on ATJ graphite with  $2\mu$ m Li dose.



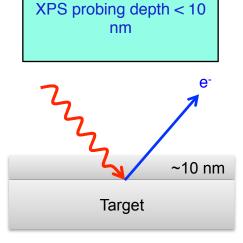
### LLD Porous Molybdenum Samples Analyzed to test Various Possible NSTX Conditions – % Concentrations

 Metal concentrations (neglecting impurities)

Metal and impurity concentrations

	% Li	% Mo			% Li	% Mo	%C	%O
Mo001	99.033	0.967	$\leftarrow$ Li-2 $\mu$ m, T <sub>room</sub> $\rightarrow$	Mo001	85.023	0.830	2.866	11.281
Mo002	98.766	1.234	←Li-2 $\mu$ m, T <sub>room</sub> →	Mo002	86.294	1.078	5.089	7.539
Mo101	98.791	1.209	$\leftarrow$ Li-2 $\mu$ m, T <sub>melt</sub> $\rightarrow$	Mo101	71.669	0.877	4.593	22.860

\*Nominal Li thicknesses

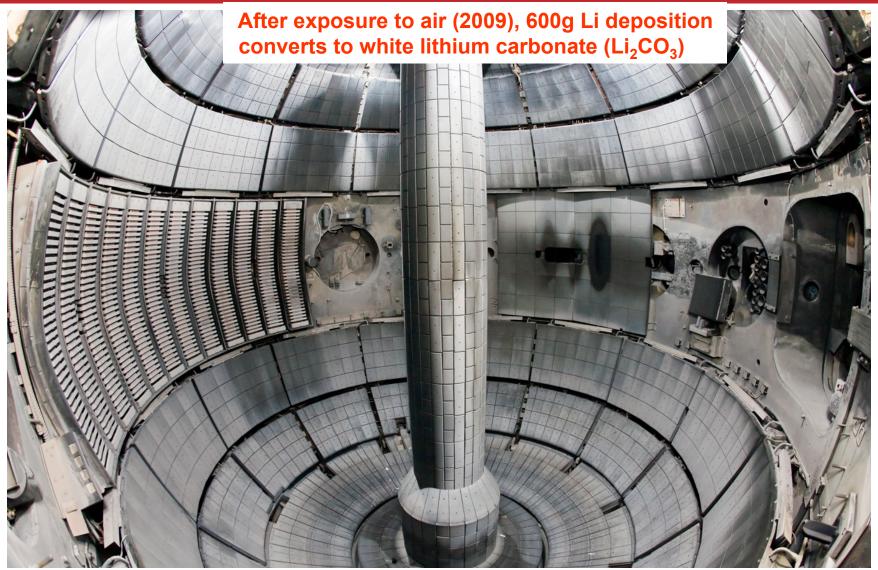




Surface lithium is plentiful and dominant after heating.

Implication: lithium wets rather than pools.

# Lithium Transported Over Broad Area By Wings of LITER Distribution and Plasma Migration



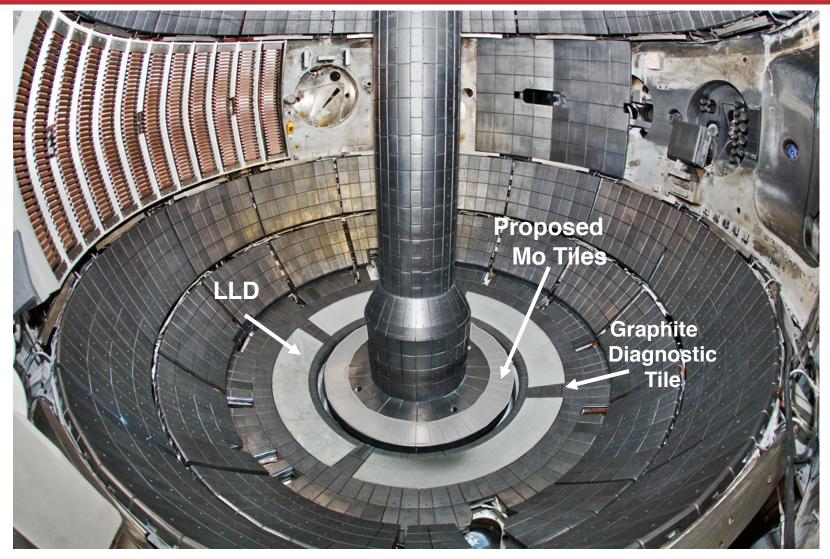
Li<sub>2</sub>CO<sub>3</sub> coatings must be removed prior to evacuation to reduce plasma oxygen levels

### **Vessel Clean Up After Lithium Deposition**

- Under NSTX vacuum conditions, Li reacts with the vacuum residual gases (predominantly H<sub>2</sub>O, CO, CO<sub>2</sub>) to form LiOH, Li<sub>2</sub>O, Li<sub>2</sub>CO<sub>3</sub>
- NSTX is vented with atmosphere and purged with humidified atmosphere for 1 week prior to first personnel entry.
- After inspection, sample collection, and photography, all surfaces are washed with deionized water and a wet lint-free cloth.
- All graphite tiles are then abraded with Scotchbrite® to remove Li<sub>2</sub>CO<sub>3</sub>.
- All other metal surfaces are washed with a 5% solution of acetic acid (common vinegar, CH<sub>3</sub>COOH) to convert hard ceramic Li<sub>2</sub>CO<sub>3</sub> to water soluble lithium acetate (LiC<sub>2</sub>H<sub>3</sub>O<sub>2</sub>) and removal with wet lint-free cloth.



## Location of Molybdenum Tiles Proposed to be Installed on Inner Divertor in 2011



• Allows both Strike Points to be positioned on Mo to test reduction in divertor carbon source term

### **Summary and Conclusions**

- Significant improvements in the performance of NSTX NBI heated divertor discharges resulting from lithium depositions are observed.
  - Lithium evaporated on divertor suppresses ELMs and improves confinement
- Constant fueled discharges with outer strike point on LLD exhibit constant core particle content, decrease in recycling, and decrease in core carbon as LLD liquefies.
- Discharges with outer strike point on LLD and increasing fueling exhibit constant particle content, increasing neutrons, decreasing carbon as LLD liquefies.
- Liquid lithium surface impurities and the complex surface morphology can reduce the formation of LLD and decrease pumping.
- NSTX experiments with solid lithium coatings and a liquid lithium divertor plate contribute toward developing replenishable liquid lithium walls for providing a pumping, impurity flushing, low-Z, self-healing plasma facing surface for long duration discharges.

