Advancing Non-Solenoidal Startup on the Pegasus ST

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Local Helicity Injection (LHI) is a Promising Non-Solenoidal Startup Technique



- Compact, modular washer-stack arc sources inject edge current
- Unstable current streams form tokamak-like state via magnetic reconnection, Taylor relaxation
- Physics and engineering tradeoffs strongly coupled to injector location

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0-D Power Balance Model Used to Explore Projections for NSTX-U Startup

• $I_p(t)$ from 0-D power balance model: $I_p[V_{LHI} + V_{IR} + V_{IND}] = 0$



- Helicity dissipation (V_{IR}) dependent on T_e
- Importance of V_{LHI}, V_{IND} depends on injector geometry, plasma growth scenario
 - Final plasma depends strongly on full time evolution
- LFS mid-plane injection: V_{LHI} early, V_{IND} late
- HFS divertor injection mainly V_{LHI}

• Need to explore plasma evolution with different dominant drive terms



On Pegasus: Utilize Different Injector Geometry to Emphasize Different Drive Mechanisms

• $I_p(t)$ from 0-D power balance model:

$$I_p \left[V_{LHI} + V_{IR} + V_{IND} \right] = 0$$



- Vary relative drive ratios to inform predictive model
- Future: Test scaling to high I_p in both LFS and HFS injection



New Helicity Injectors in the Divertor Region of Pegasus are Installed and Operating

- HFS → 3-4x increased helicity input
 Access to higher I_p startup
- Static geometry \rightarrow low V_{IND}
- Injector operation at longer pulse, high-B_{TF}
- Initial HFS injector campaign in progress

Divertor Injectors after Installation



Centerstack

Lower div Strike Plate



In the Process of Mitigating PMI in the Divertor Injector Geometry

PMI on injector surfaces



PMI on lower divertor plate

- PMI: Cathode spots, arc-back, divertor plate interaction
- Consequence: Impurity injection, reduced drive, loss of reproducibility
- PMI mitigated: Operation at low B_{TF}, improved alignment and local limiters
- PMI is much reduced, but still optimizing





Initial Results from HFS Injector Campaign are Promising

- Larger area injectors (2 cm² → 4 cm²) function well at full field (0.25 T)
- Demonstrated relaxation, current growth at full field
 - First milestone achieved
- $I_p > 100 \text{ kA to date}$
- Producing attractive handoff targets - $<T_e> \sim 100$ eV, $\overline{n_e} \sim 10^{19}$ m⁻³
- Plasmas are highly elongated

 $-\kappa > 2.6$



Fully V_{LHI} Driven Discharges at Low B_{TF} Achieved

- Full growth and sustainment dominantly driven by LHI
- MHD markedly different from LFS LHI
 - Initial phase: current drive from large scale reconnection of helical streams
 - MHD abruptly drops by an order of magnitude





Interim Operation at Low B_{TF} Allows Observation of Sustained High I_{N} High κ Discharge

- Low $I_{TF} \sim 0.6 I_p$
- $I_N = 5A \frac{I_P}{I_{TF}} > 10$
 - Constant or ramped-down B_{TF}
- HFS injector geometry → naturally high elongation
- Ready access to High β_T
 - Aided by anomalous ion heating $(T_i > T_e)$





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LHI Provides Access to High- β_T at A ~ 1 with Non-Solenoidal Sustainment and Anomalous Ion Heating

- Equilibrium reconstructions indicate high β_T (~<P>/B_{T0}²)
- High β_T plasmas often terminated by disruption
 n = 1, low-m precursors
- Expands I_N , β_T space for stability studies at extreme toroidicity









HFS Injection Enables Exploring Varied Drive Mechanisms. Coincidentally: Unique Access to High I_N , β_T Space

- HFS injector operation and relaxation to a tokamak demonstrated at full TF ($B_{TF_{inj}} \sim 0.25T$)
- Completely V_{LHI} driven startup and sustainment realized
- Sharp drop in MHD during I_p ramp suggests change in current drive mechanism
- HFS injection at low B_{TF} enables sustained non-inductive operation at high κ , high I_N, and high β_T
- Present campaign:
 - Optimize HFS injector implementation to mitigate PMI at high B_{TF}
 - Develop high Ip scenarios to test scaling
 - Design CHI system for comparison studies (with PPPL, U. Wash)

