

# Resistive Wall Mode Active Stabilization in High Beta, Low Rotation Plasmas

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# Experiments in 2006 examined RWM physics and stabilization at ITER-relevant rotation

#### RWM active stabilization

- New control system installed
- RWM control demonstrated
- RWM actively stabilized in slowly rotating plasmas

#### Plasma rotation control

- Sustained rotation by realtime reduction of amplified error field
- Reduced rotation by nonresonant magnetic braking

#### NSTX / ITER RWM control



<u>Advantage</u>: low aspect ratio, high  $\beta$  provides high leverage to uncover key tokamak physics for ITER (e.g. RWM control, momentum dissipation)



## **RWM Active Feedback System Installed on NSTX**



- Stabilizer plates for kink mode stabilization
- External midplane control coil closely coupled to vacuum vessel
  - Similar to ITER port plug designs
- Internal sensors can detect n = 1 − 3 RWM
  - □ Unstable n = 1 − 3 RWMs already observed in NSTX (Sabbagh, et al., NF 46 (2006) 635.)
  - n > 1 RWM studied during n = 1active stabilization

## Dynamic error field correction (DEFC) increases pulse length in strongly rotating plasmas



Open-loop control of error field amplified by stable RWM

- yields higher rotation
- yields longer pulse
- Combination of open + closed loop control yielded best result
  - Rotation increase or saturation at long pulse lengths - first time in NSTX

# RWM stabilized at ITER-relevant rotation for ~ 90/yRWM

(Sabbagh, et al., PRL 97 (2006) 045004.)



- First such demonstration in low A tokamak

  - n = 2 RWM amplitude increases, mode remains stable while n = 1 stabilized
    - <u>Multi-mode</u> research connection to RWM stabilization in RFPs

# RWM stabilized at ITER-relevant rotation for ~ 90/yRWM



(Sabbagh, et al., PRL **97** (2006) 045004.)

# RWM stabilized at ITER-relevant rotation for ~ 90/yRWM



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## Rotation reduced far below RWM critical rotation profile



Rotation typically fast and sufficient for RWM passive stabilization

□ Reached  $\omega_{\phi}/\omega_{A} = 0.48|_{axis}$ 

Non-resonant n = 3 magnetic braking used to slow entire profile

• The 
$$\omega_{\phi}/\Omega_{crit} = 0.2|_{q=2}$$

- $\Box$  The  $\omega_{\phi}/\Omega_{crit} = 0.3|_{axis}$
- Less than  $\frac{1}{2}$  of ITER Advanced Scenario 4  $\omega_{\phi}/\Omega_{crit}$  (Liu, et al., NF 45 (2005) 1131.)
- Rotation profile responsible for passive stabilization, not just single radial location

see paper EX/7-2Rb Sontag

# Observed rotation decrease follows NTV theory



(Zhu, et al., PRL 96 (2006) 225002.)

- First quantitative agreement using full neoclassical toroidal viscosity theory (NTV)
  - Due to plasma flow through non-axisymmetric field
  - Computed using experimental equilibria
  - Trapped particle effects, 3-D field spectrum important
- Viable physics for simulations of plasma rotation in future devices (ITER, CTF, KSTAR)
  - Scales as  $\delta B^2(T/v_i)(1/A)^{1.5}$
  - Low v<sub>i</sub> ITER plasmas will have higher rotation damping

see EX/7-2Rb Sontag

## Varying relative phase shows positive/negative feedback



RWM active feedback on n = 1

Control current relative phase,  $\Delta \phi_f$ 

Phase scan shows superior settings for negative feedback

- Pulse length increases
- □ Internal plasma mode seen at  $\Delta \phi_f$  = 225, damped feedback system response

#### Gain scan also performed

Sufficiently high gain showed feedback loop instability

## RWM may change form and grow during active control



- Poloidal n =1 RWM field decreases to near zero
  - Radial RWM field increasing
- Subsequent growth of poloidal RWM field
  - Asymmetric above/below midplane
- Midplane radial sensor shows RWM bulging
  - Upper/lower radial sensors show decrease, while midplane sensor increases
  - Theory: may be due to stable ideal n = 1 modes becoming less stable (e.g. q evolution)

Future research will assess using combined sensors for optimization

#### Clear differences between RWM and internal plasma mode



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# NSTX begins RWM active stabilization research relevant to ITER and beyond

- **□** First demonstration of RWM active stabilization in high β, low A tokamak plasmas with  $ω_{\phi}$  significantly less than  $Ω_{crit}$ 
  - In the predicted range of ITER
  - Plasma response to feedback control demonstrated
- Stability of n = 2 RWM demonstrated during n = 1 RWM stabilization
  - $\square$  n = 1,2 plasma mode sometimes observed; fast  $\beta$  collapse, recovery
- Plasma rotation reduction by non-resonant applied field; follows NTV theory
  - □ Full NTV calculation yielding quantitative agreement to experiment
  - Key component of RWM stability physics and dynamics; general momentum transport relevance



## Additional slides for poster follow



## Work slides follow



## NSTX begins RWM active stabilization research relevant to ITER, KSTAR, CTF



#### <u>Close connection to present experiments in NSTX impacts</u> <u>the KSTAR stability physics study</u>

- RWM active stabilization demonstrated in low rotation (ITER-relevant) plasmas (Sabbagh, et al., PRL 97 (2006) 045004)
  - □ KSTAR with co-NBI should have rotation control for experiments
- Precise plasma rotation control through neoclassical toroidal viscosity (Zhu, et al., PRL 96 (2006) 225002)
  - n = 2 non-resonant magnetic braking possible rotation control option for KSTAR
- Unstable resistive wall mode with toroidal mode number n > 1 observed (Sabbagh, et al., NF 46 (2006) 635)
  - May need to address n > 1 unstable modes in KSTAR at the highest beta or for certain equilibrium profile shapes
- RWM critical rotation speed (H. Reimerdes, et al., PoP 13 (2006) 056107)
  - Dependence on aspect ratio, Alfven speed, ion collisionality key research topic

