

# Long-wavelength MHD stability at high pressure required for ITER and other next-step devices

- Motivation

- The resistive wall mode (RWM) is a primary cause of plasma disruption at high  $\beta$

- Understanding passive stabilization physics determining RWM stability is critical to extrapolate stability requirements for future devices
- Active control of RWM required when profile transients cause instability

- Passive stability: Very brief history

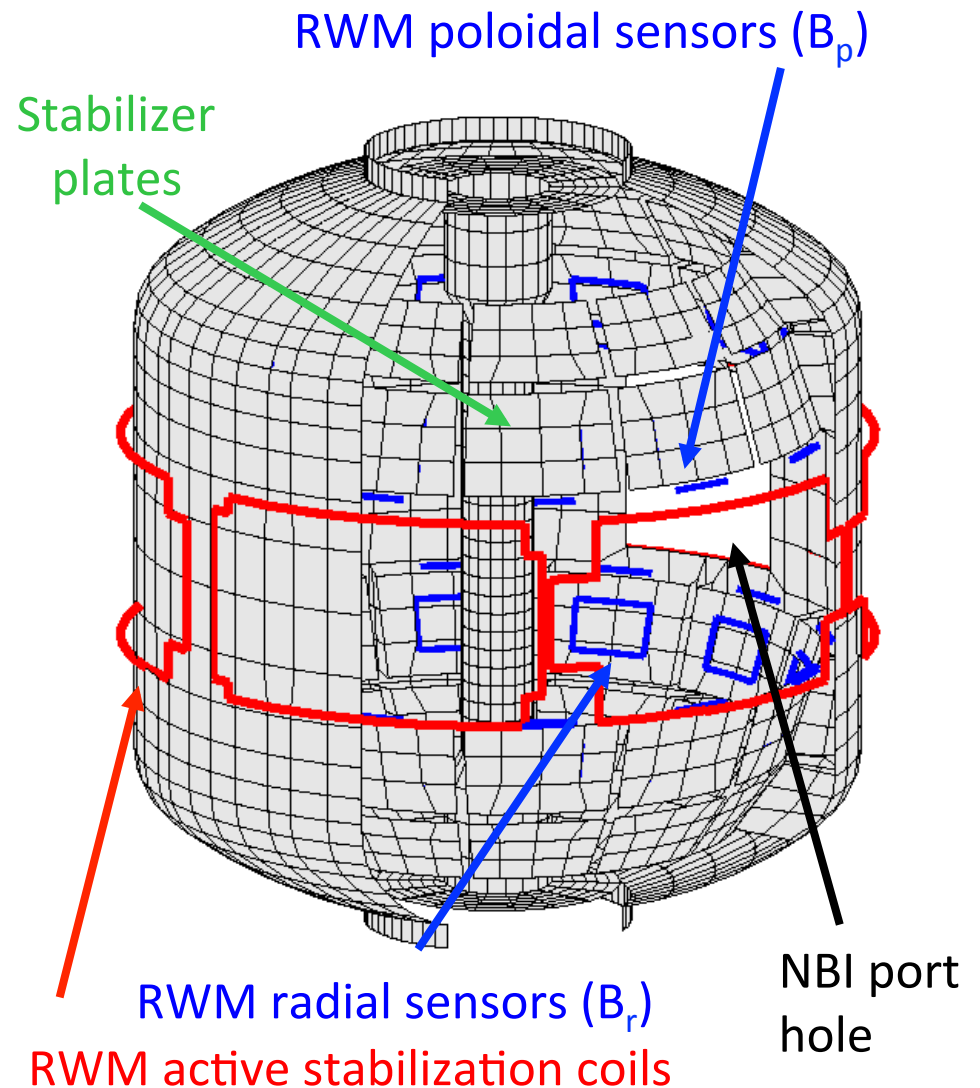
- Early theory: RWM can be stabilized by sufficient plasma rotation
- Critical  $\omega_\phi$  for passive stability assessed ( $\Omega_{\text{crit}}$ )
- Low levels of  $\Omega_{\text{crit}}$  ( $< 0.5\%$  Alfvén at  $q = 2$ ) suggested
- RWMs found to be unstable at relatively high  $\omega_\phi$ , and stability depends on profile, not simple scalar value – **no simple, low  $\Omega_{\text{crit}}$ !**
- Stability model including kinetic effects evaluated (NSTX) - can explain greater complexity of experimental RWM marginal stability

# Understanding plasma stability gradients vs. key profiles is essential for all operational states in devices such as ITER

- Current research focuses on:
  - greater understanding of the stabilization physics
  - projection to future devices
  - quantitative comparison to experiment
  - demonstration of improved active control techniques that can reduce resonant field amplification (RFA) or disruptions
- Outline
  - NSTX active feedback
    - Dual field component active control
    - Model-based state space controller
  - NSTX resonant field amplification experiments
    - Comparisons with kinetic theory: resonances and collisionality
  - ITER analysis with alpha particles and internal transport barriers

# NSTX is a spherical torus equipped to study passive and active global MHD control, rotation variation by 3D fields

- High beta, low aspect ratio
  - $R = 0.86 \text{ m}$ ,  $A > 1.27$
  - $I_p < 1.5 \text{ MA}$ ,  $B_t = 5.5 \text{ kG}$
  - $\beta_t < 40\%$ ,  $\beta_N > 7$
- Copper **stabilizer plates** for kink mode stabilization
- Midplane **control coils**
  - $n = 1 - 3$  field correction, magnetic braking of  $\omega_\phi$  by NTV
  - $n = 1$  RWM control
- Combined sensor sets now used for RWM feedback
  - 48 upper/lower  $B_p$ ,  $B_r$



# Kinetic terms in the RWM dispersion relation enable stabilization; theory consistent with experimental results

Dissipation ( $\text{Im}(\delta W_K)$ ) and restoring force ( $\text{Re}(\delta W_K)$ ) from kinetic term enables stabilization of the RWM:

$$(\gamma - i\omega_r) \tau_w = -\frac{\delta W_\infty + \delta W_K}{\delta W_b + \delta W_K}$$

[B. Hu *et al.*, *Phys. Plasmas* **12**, 057301 (2005)]

$$\delta W_K = \sum_j \sum_{l=-\infty}^{\infty} 2\sqrt{2}\pi^2 \int \int \int \left[ |\langle H/\hat{\epsilon} \rangle|^2 \frac{(\omega - n\omega_E) \frac{\partial f_j}{\partial \epsilon} - \frac{n}{Z_j e} \frac{\partial f_j}{\partial \Psi}}{n\langle \omega_D^j \rangle + l\omega_b^j - i\nu_{\text{eff}}^j + n\omega_E - \omega} \right] \frac{\hat{\tau}}{m_j^{3/2} B} |\chi| \hat{\epsilon}^{5/2} d\hat{\epsilon} d\chi d\Psi, \quad \chi = v_{\parallel}/v$$

$v\tau_w$  contours  
vs.  $v$  and  $\omega_\phi$

Precession Drift

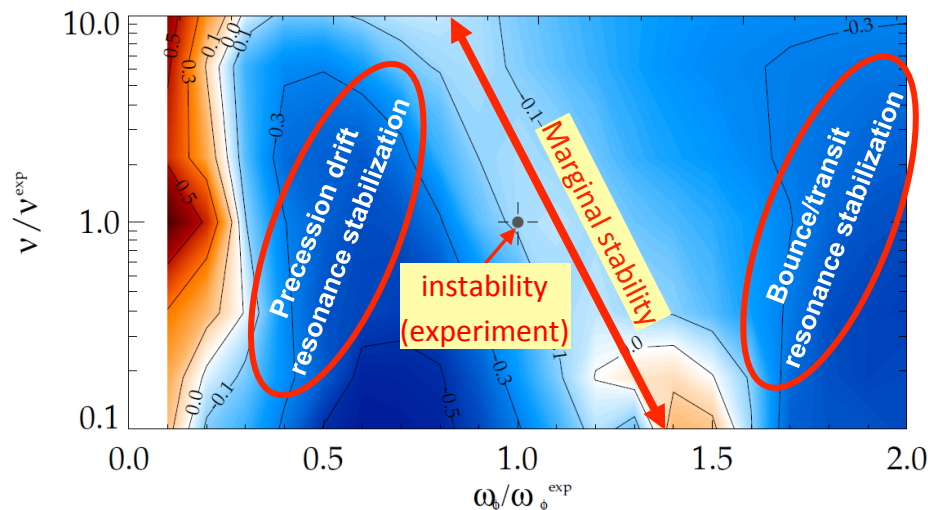
Bounce

~ Plasma Rotation:

$$\omega_\phi \approx \omega_E + \omega_{*i}$$

Collisionality

**MISK Code**



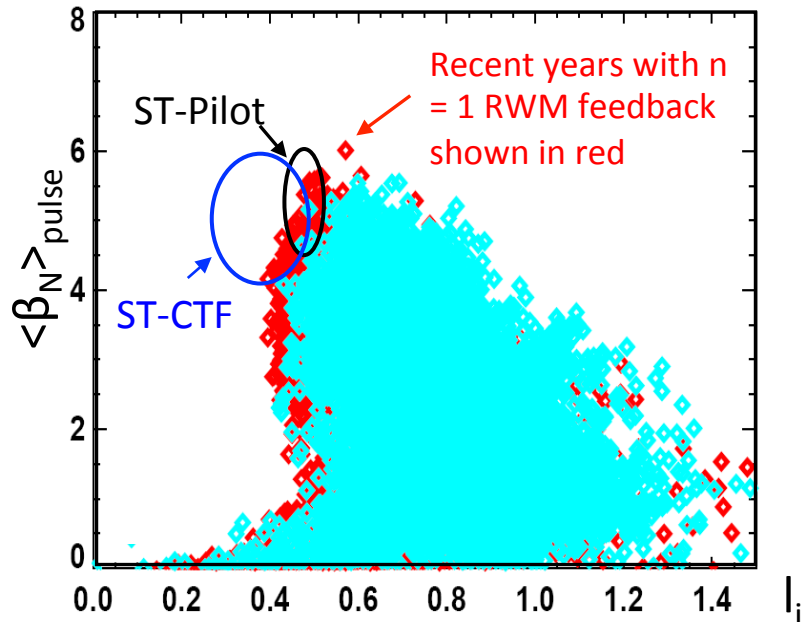
[S. Sabbagh *et al.*, IAEA 2010, EXS/5-5]

- MISK calculations are consistent with RWM instability at intermediate plasma rotation in NSTX
- Instability appears between precession drift resonance at low  $\omega_\phi$ , bounce/transit resonance at high  $\omega_\phi$

[S. Sabbagh *et al.*, *Nucl. Fusion* **50**, 025020 (2010)]

[J. Berkery *et al.*, *Phys. Rev. Lett.* **104**, 035003 (2010)]

# NSTX reaches high $\beta_N$ , low $I_i$ ; RWM stability investigated in unstable plasmas, active control applied

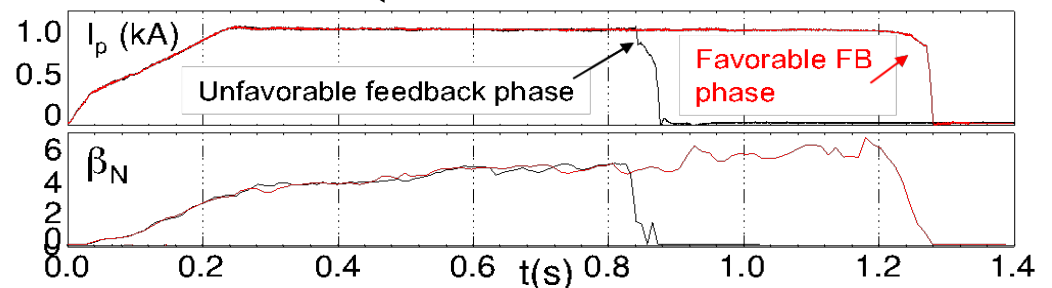
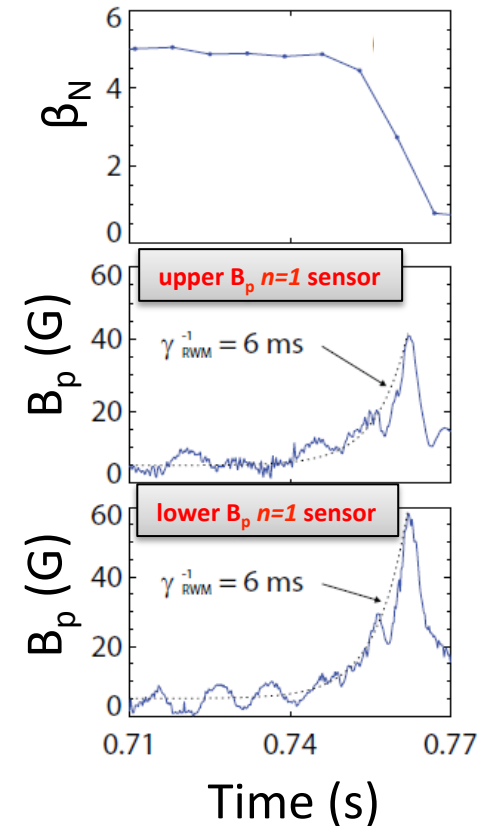


- NSTX plasmas have begun to reach low  $I_i$  and high  $\langle \beta_N \rangle_{\text{pulse}}$  suitable for next-step ST fusion devices
  - Some parameters (e.g. elongation  $> 3$ ) still need to be reached self-consistently
  - Broad current profile  $\rightarrow$  low  $I_i = \langle B_p^2 \rangle / \langle B_p \rangle_\psi^2$ , has global mode stability implications

- Unstable plasma
  - Causes  $\beta$  collapse,  $I_p$  disruption
  - Correlate marginal stability point with kinetic theory MISK calculations

*or*

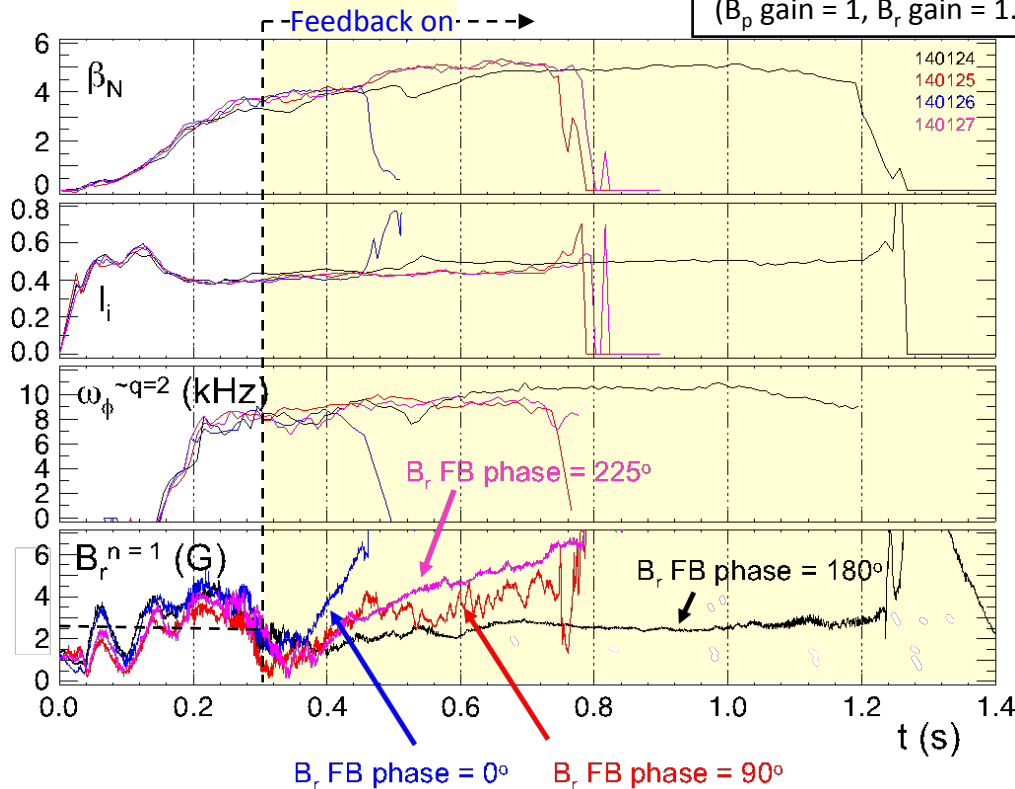
  - Use active control to stabilize



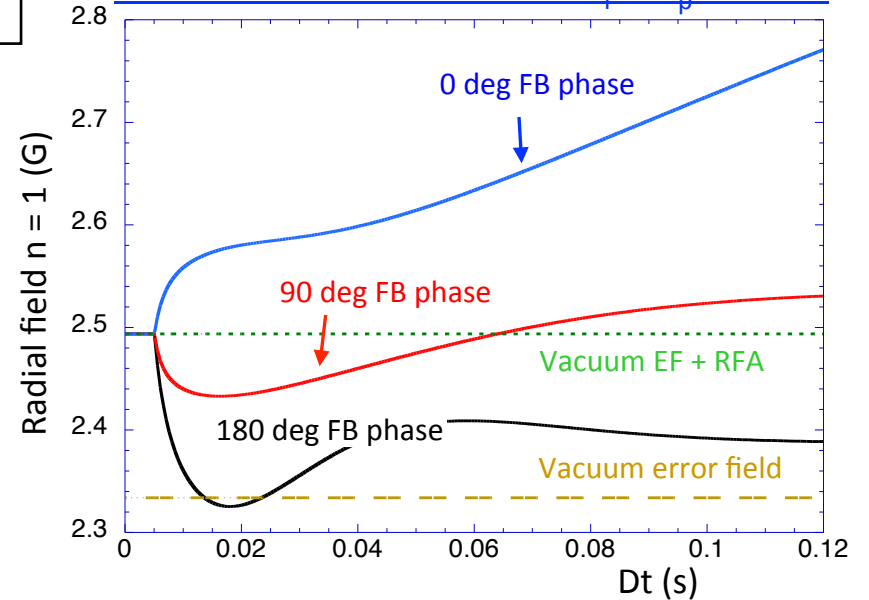
# RWM $B_r$ sensor $n = 1$ feedback phase variation shows superior settings when combined w/ $B_p$ sensors; good agreement w/theory so far

## NSTX Experiments: $B_p + B_r$ feedback

$n = 1 B_r + B_p$  feedback  
( $B_p$  gain = 1,  $B_r$  gain = 1.5)

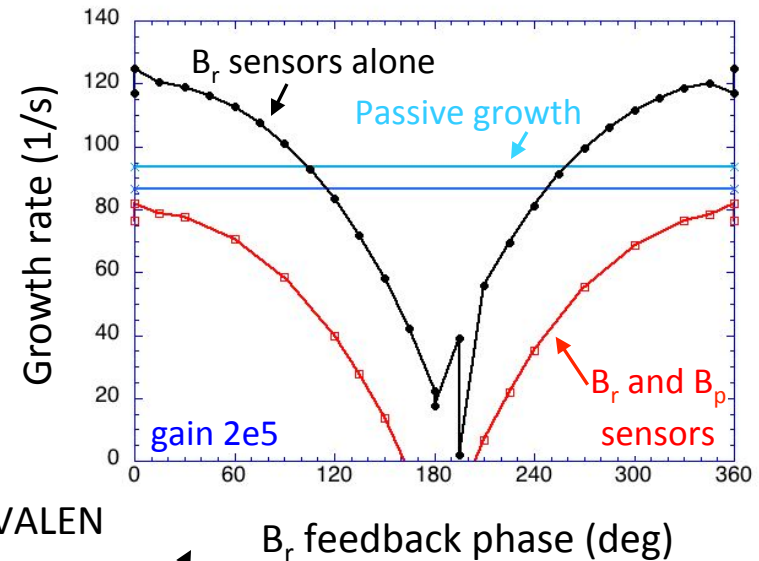
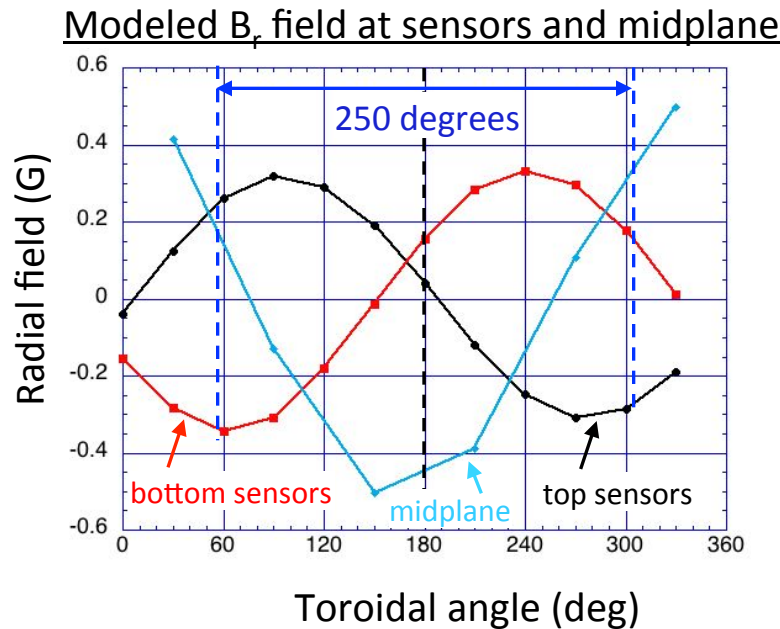


## VALEN calculation of NSTX $B_r + B_p$ control

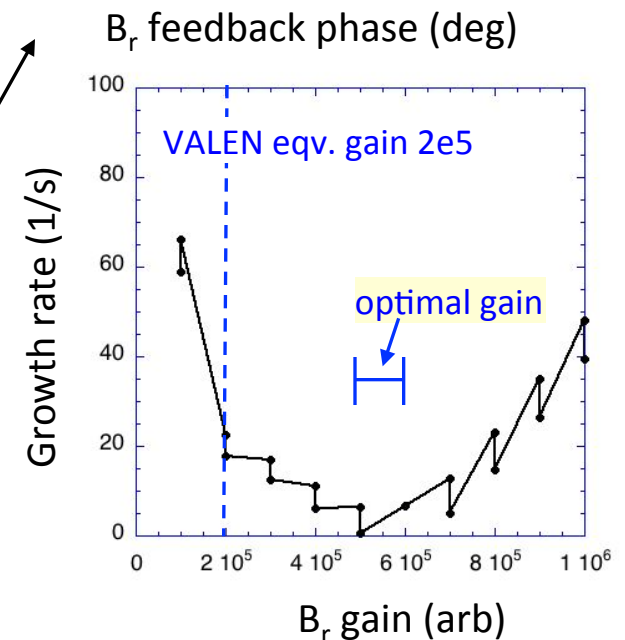


- VALEN calculation of  $B_r + B_p$  feedback follows experiment
  - stable plasma (negative “s”)
  - Now examining plasma response model variation
    - impact of “s”, and diff. rotation (“alpha”) on results

# RWM feedback using upper/lower $B_p$ and $B_r$ sensors shows good agreement with $B_r$ feedback phase; gain not optimized



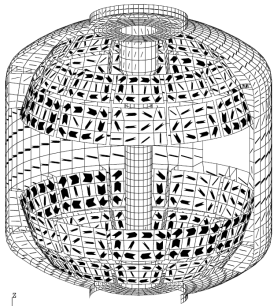
- Both  $B_r$ ,  $B_p$  feedback contribute to active control
  - $B_r$  mode structure and optimal feedback phase agrees with parameters used in experiment
  - $B_r$  feedback alone provides stabilization for growth times down to  $\sim 10$  ms with optimal gain
  - Physics of best feedback phase for  $B_p$  sensors in low  $I_i$  plasmas under investigation



# New RWM state space controller implemented to best sustain high $\beta_N$

Full 3-D model

~3000+ states

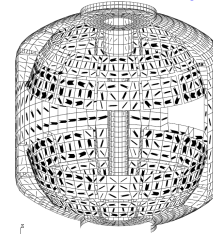


Balancing transformation

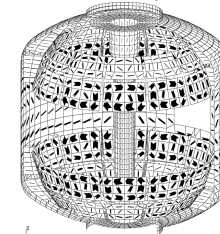
State reduction (< 20 states)

RWM eigenfunction (2 phases, 2 states)

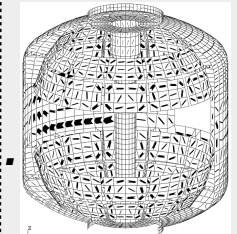
$(\hat{x}_1, \hat{x}_2)$



$\hat{x}_3$

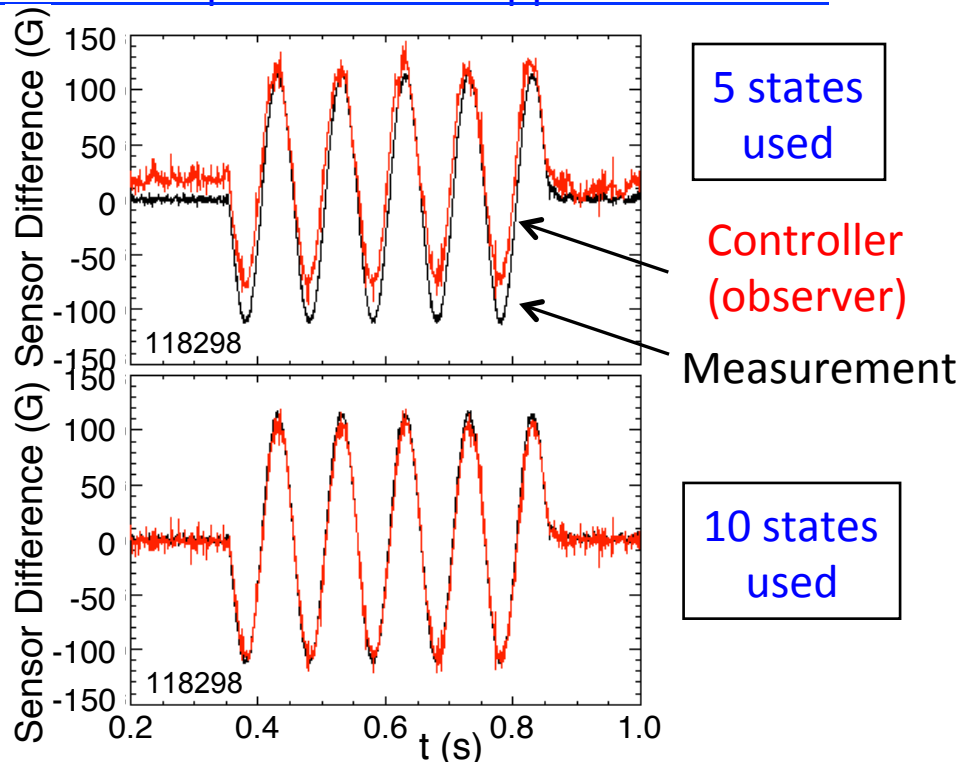


$\hat{x}_4$



$\hat{x}_N$

Controller reproduction of applied  $n = 1$  field



- Controller models, can compensate for wall currents
  - Including mode-induced current
- Potential to allow more flexible control coil positioning
  - May allow control coils to be moved further from plasma, and be shielded (e.g. for ITER)

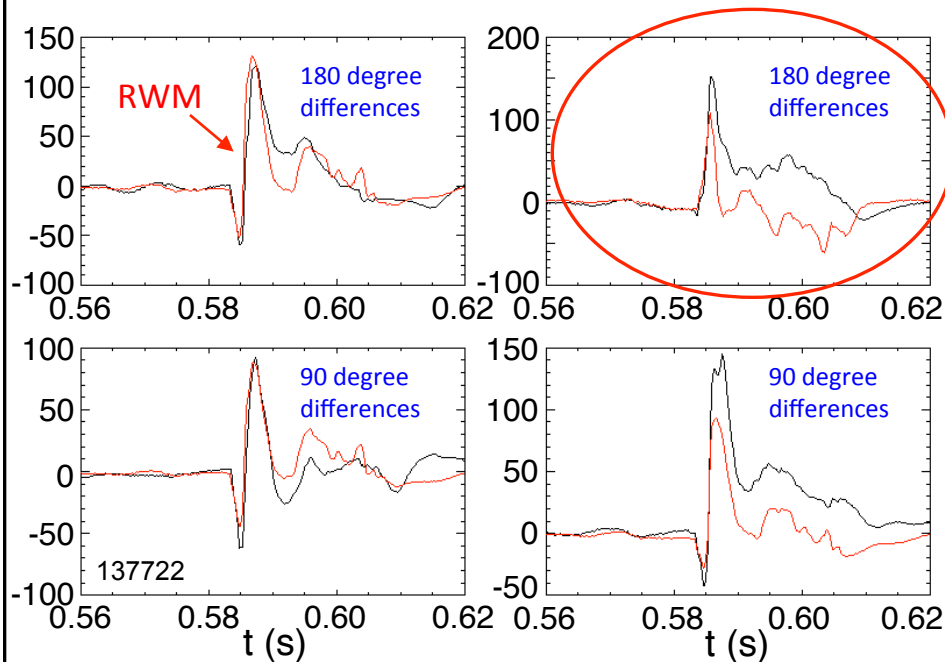
[O. Katsuro-Hopkins *et al.*, Nucl. Fusion **47**, 1157 (2007)]

- Straightforward inclusion of multiple modes ( $n > 1$ ) in feedback



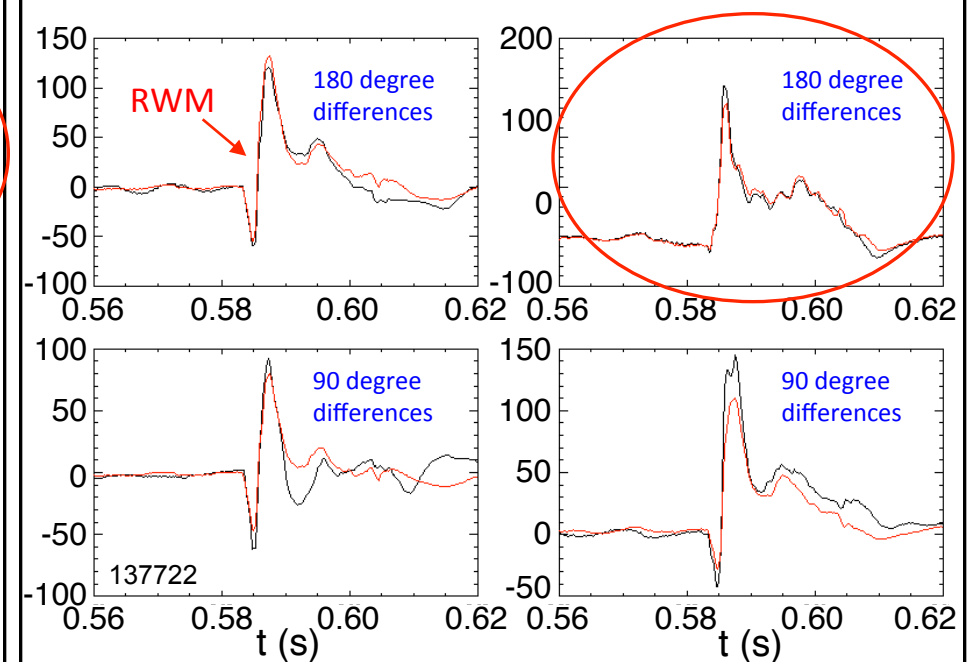
# Increased number of states in RWM state space controller improves match to sensors over entire mode evolution

RWM Upper  $B_p$  Sensor Differences (G) – 2 States



- Reasonable match to all  $B_p$  sensors during RWM onset, large differences later in evolution

RWM Upper  $B_p$  Sensor Differences (G) – 7 States

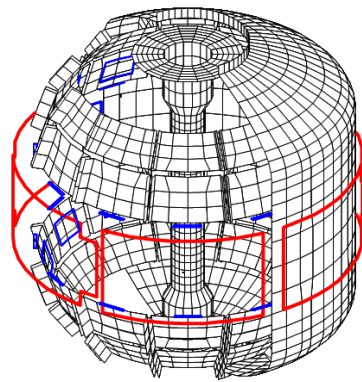


- Some 90 degree differences not as well matched
  - May indicate the need for an  $n = 2$  eigenfunction state

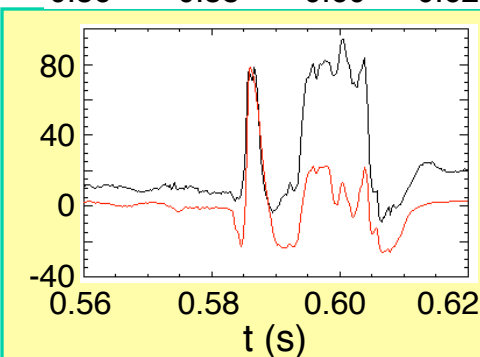
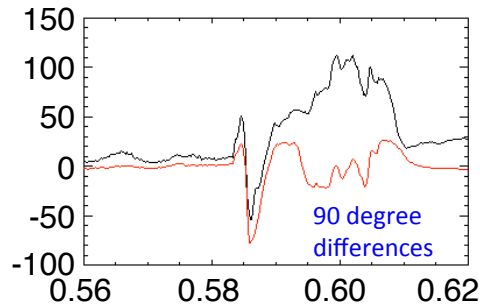
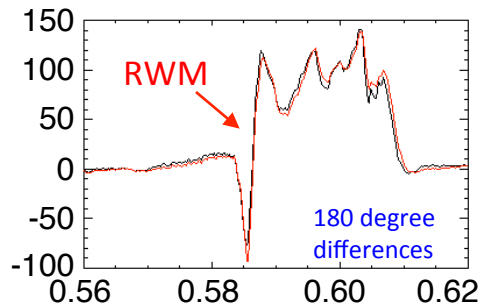
Black: experiment   Red: offline RWM state space controller

# 3-D conducting structure detail can improve RWM state space controller match to sensors

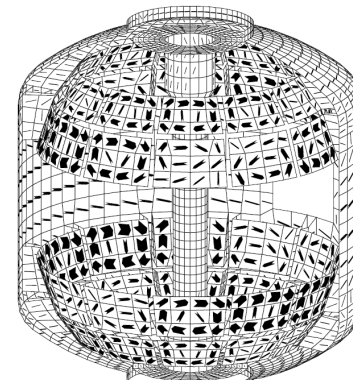
RWM Lower  $B_p$  Sensor Differences (G) – NO PORT



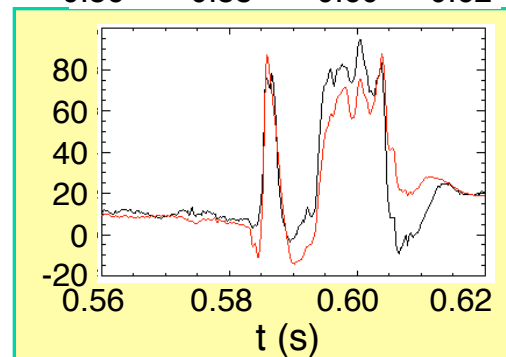
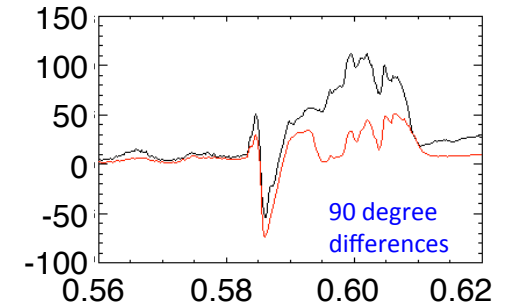
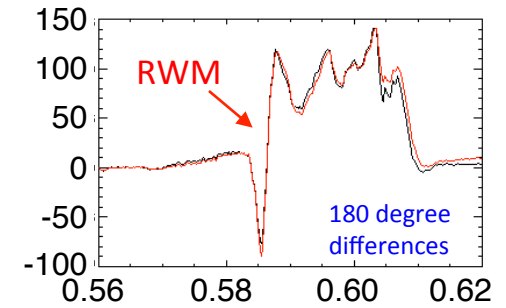
- Some 90 degree differences not well matched



RWM Lower  $B_p$  Sensor Differences (G) – NBI PORT



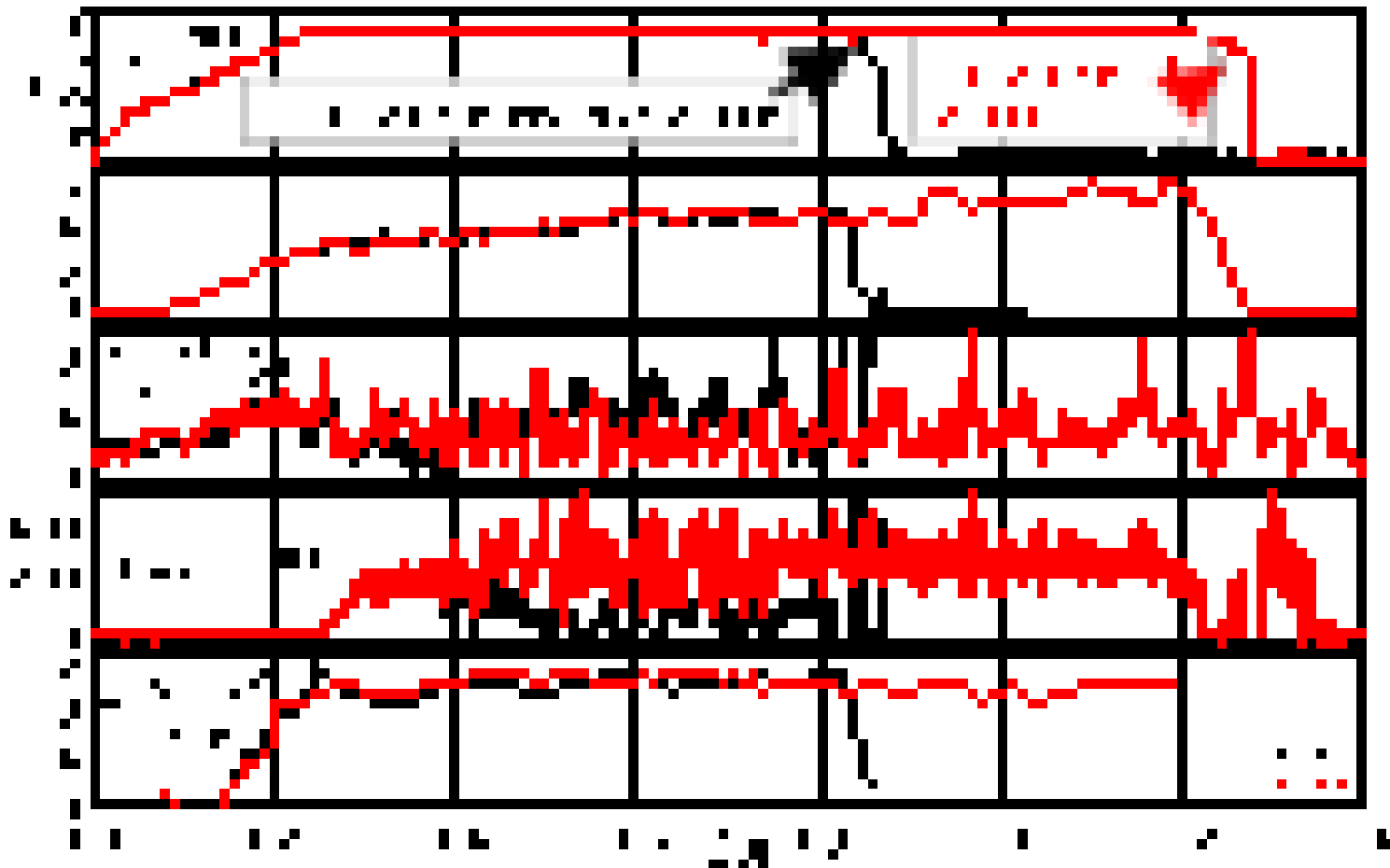
- Adding NBI port leads to greater match on some sensors



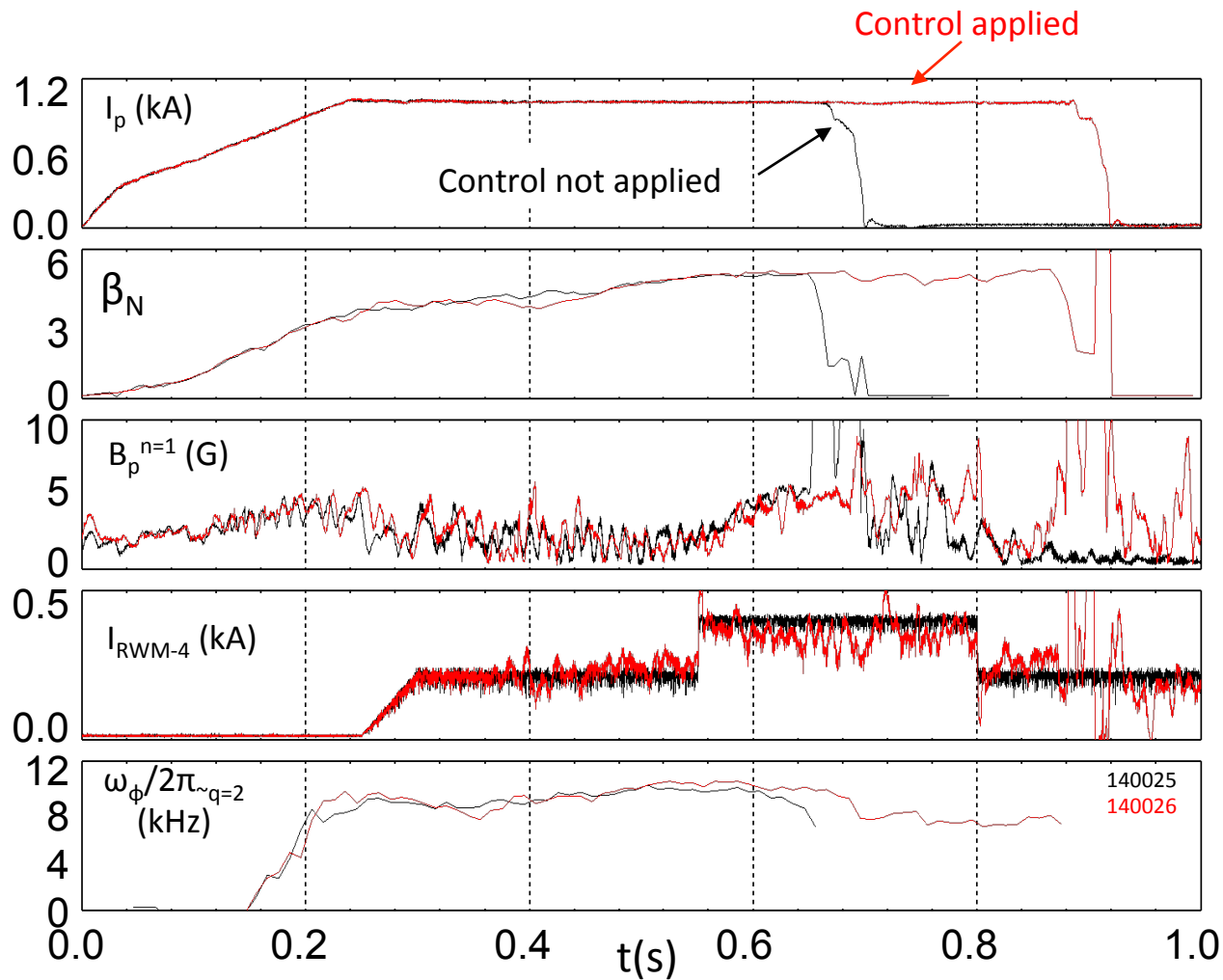
Black: experiment Red: offline RWM state space controller

# New RWM state space controller sustains high $\beta_N$ , low $I_i$ plasma

- RWM state space (12 states) feedback phase scan
  - Best feedback phase produced long pulse,  $\beta_N = 6.4$ ,  $\beta_N/I_i = 13$



# RWM state space controller sustains otherwise disrupted plasma caused by DC n = 1 applied field



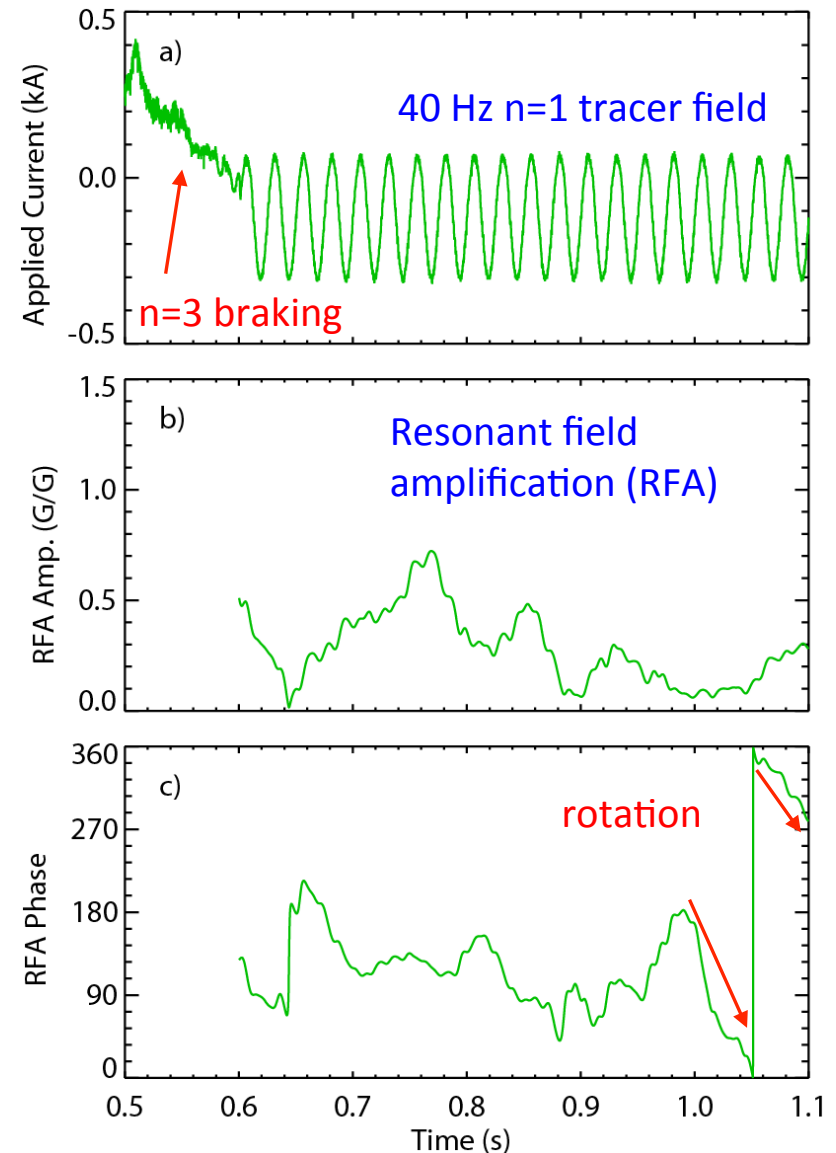
- n = 1 DC applied field
  - Simple method to generate resonant field application
  - Can lead to mode onset, disruption
- RWM state space controller sustains discharge
  - With control, plasma survives n = 1 pulse
  - n = 1 DC field reduced
  - Transients controlled and do not lead to disruption
  - **NOTE: initial run – gains NOT optimized**

# Active MHD spectroscopy is used to probe plasma stability

- Active MHD spectroscopy used as a proxy for RWM stability when modes are stable
  - Resonant field amplification (RFA) of an n=1 applied AC field is measured.
  - Increased RFA indicates decreased stability

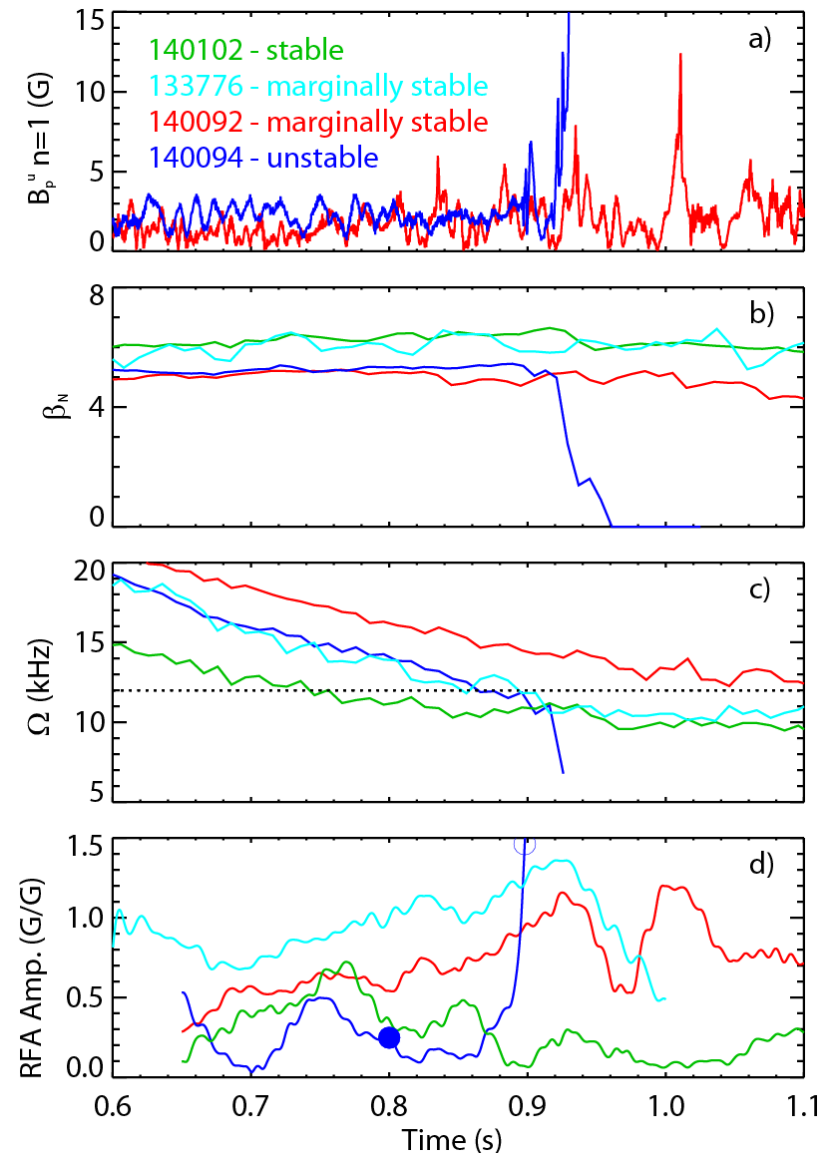
$$RFA = \frac{B_{plasma}}{B_{applied}}$$

[H. Reimerdes *et al.*, Phys. Rev. Lett. **93**, 135002 (2004)]

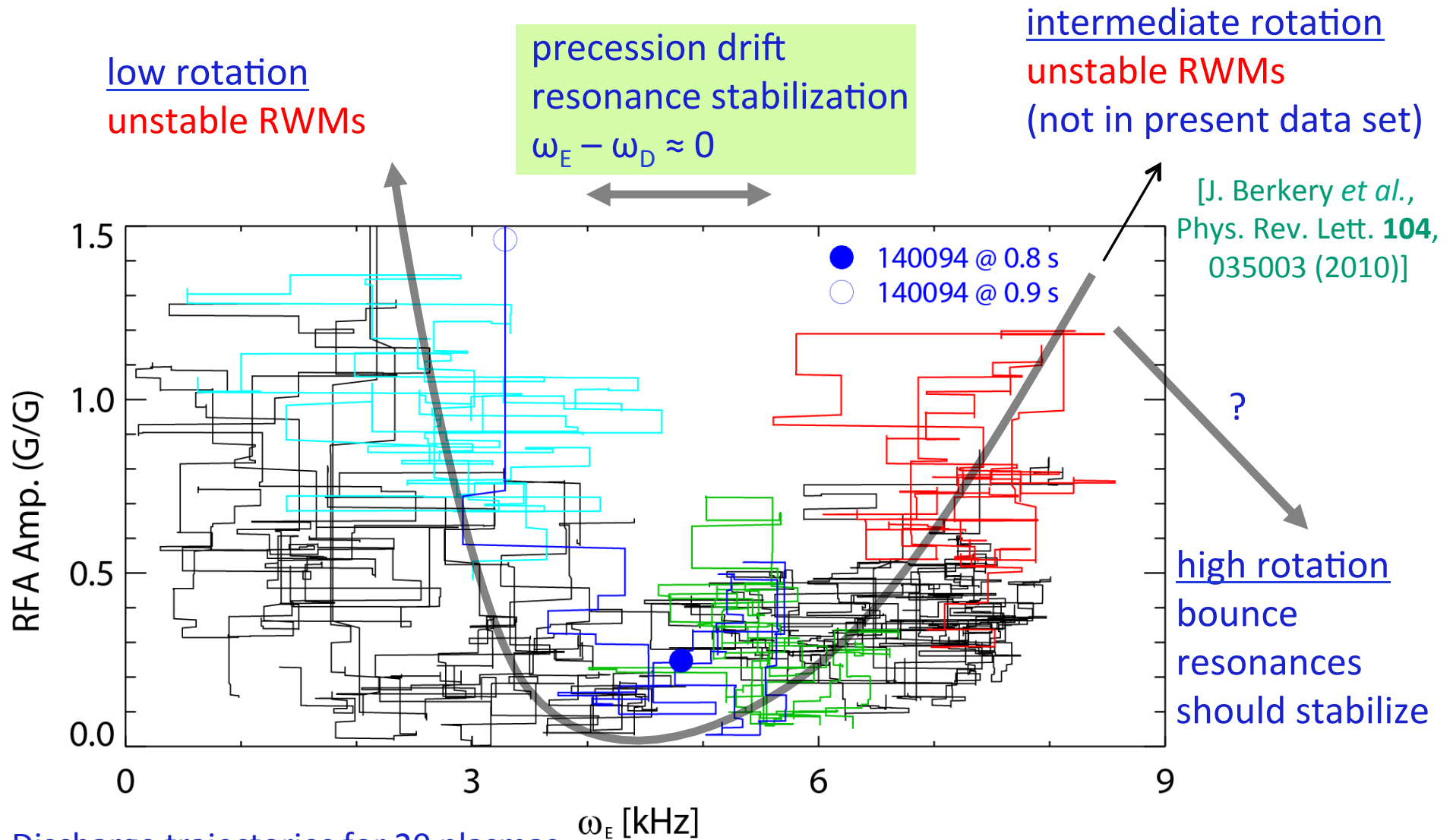


# Resonant field amplification experiments in NSTX gauge the stability of plasmas to compare to kinetic stability theory

- Experiments in NSTX measured RFA of high beta plasmas with rotation slowed by  $n=3$  magnetic braking.
  - 140094: unstable at 0.9 s
  - 140092: same  $\beta$ , higher rotation: remains stable (though approaches marginal twice)
  - 133776: higher  $\beta$ , same rotation: marginally stable
  - 140102: higher  $\beta$ , lower rotation, but stable! Counter-intuitive without invoking kinetic effects



# RFA measurements add additional support to established theory of RWM stability through kinetic resonances

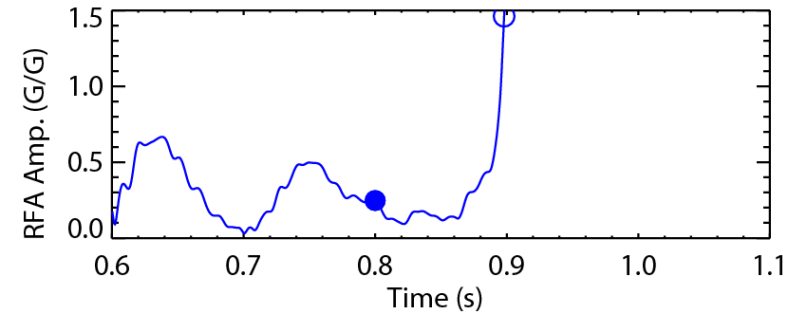


Discharge trajectories for 20 plasmas

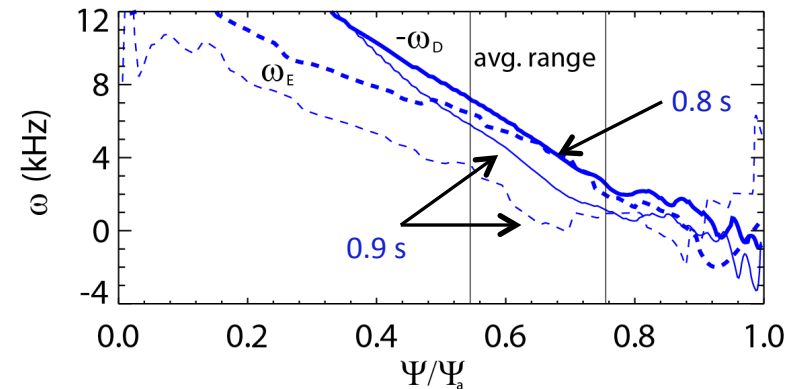
# Experimental instability can be explained by kinetic theory and MISK calculation

- Earlier time (0.8 s, ●):
  - $\omega_E \approx \omega_D$
  - Experiment: stable
  - Theory: stabilizing
  - Calculation: stable
- Later time (0.9 s, ○):
  - $\omega_E < \omega_D$
  - Experiment: unstable
  - Theory: destabilizing
  - Calculation: unstable (at 10% lower rotation)

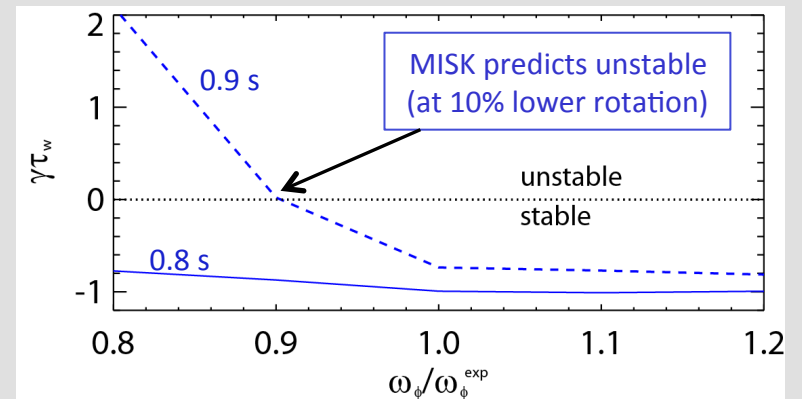
Experiment  
(RFA)



Profiles  
(Theory)



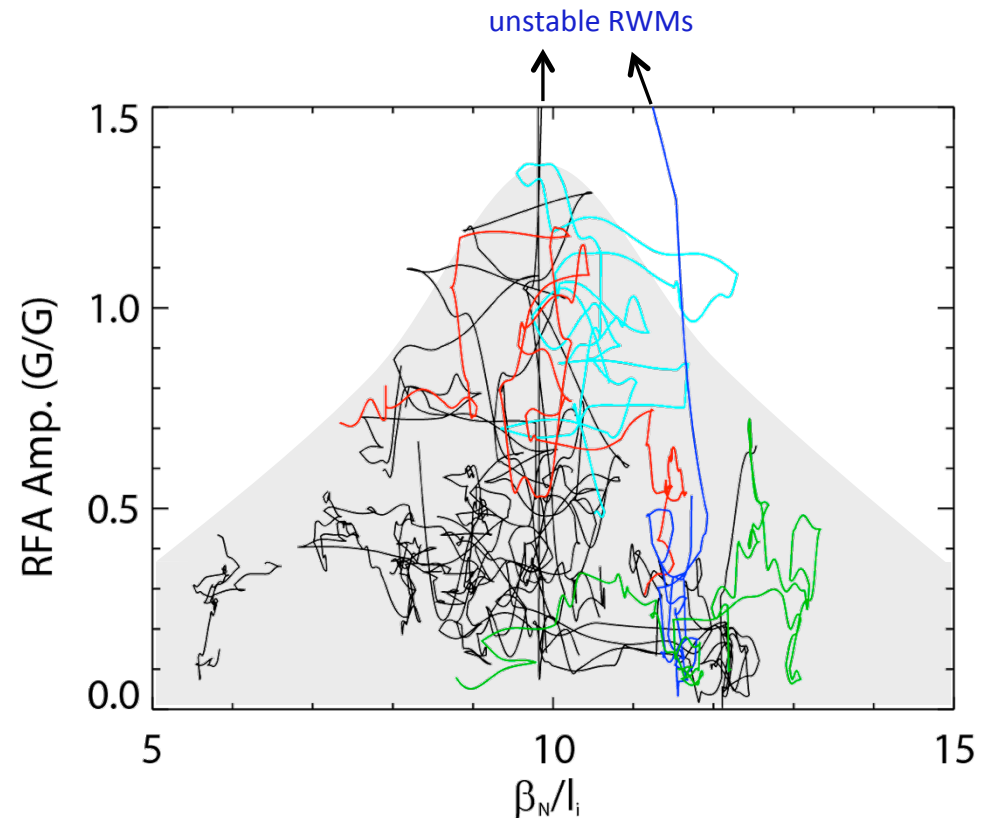
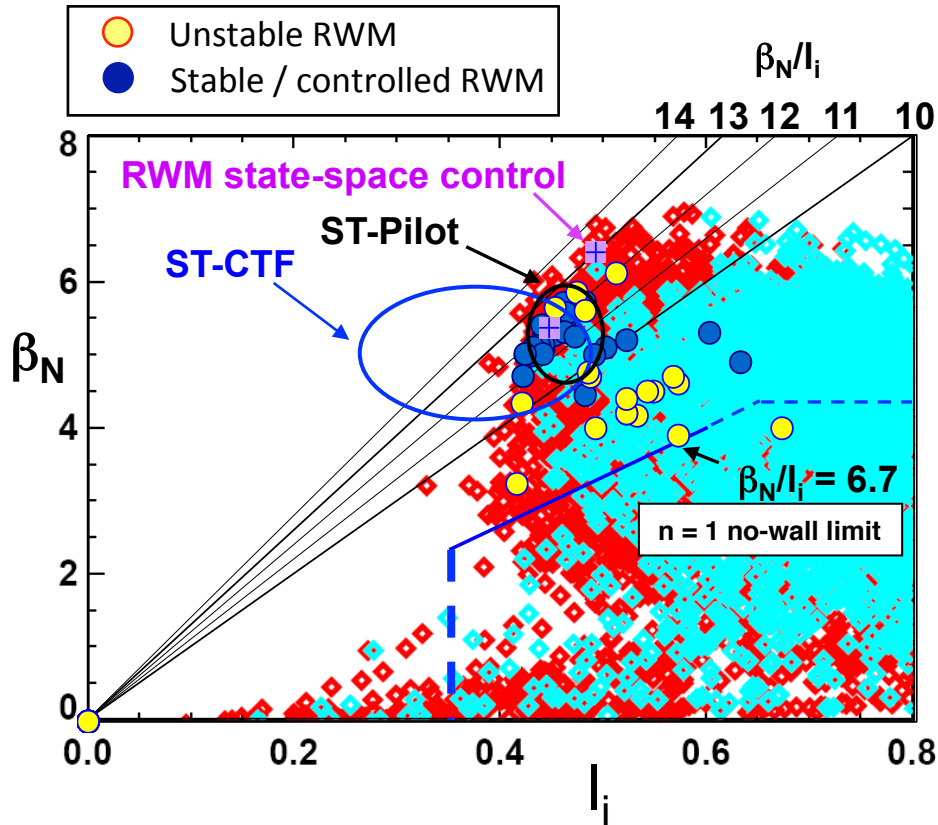
MISK  
Calculation



Notes:  $\omega_D$  shown is for zero pitch angle and  $\epsilon/T = 2.5$  (higher than usual). MISK calculation doesn't include EPs



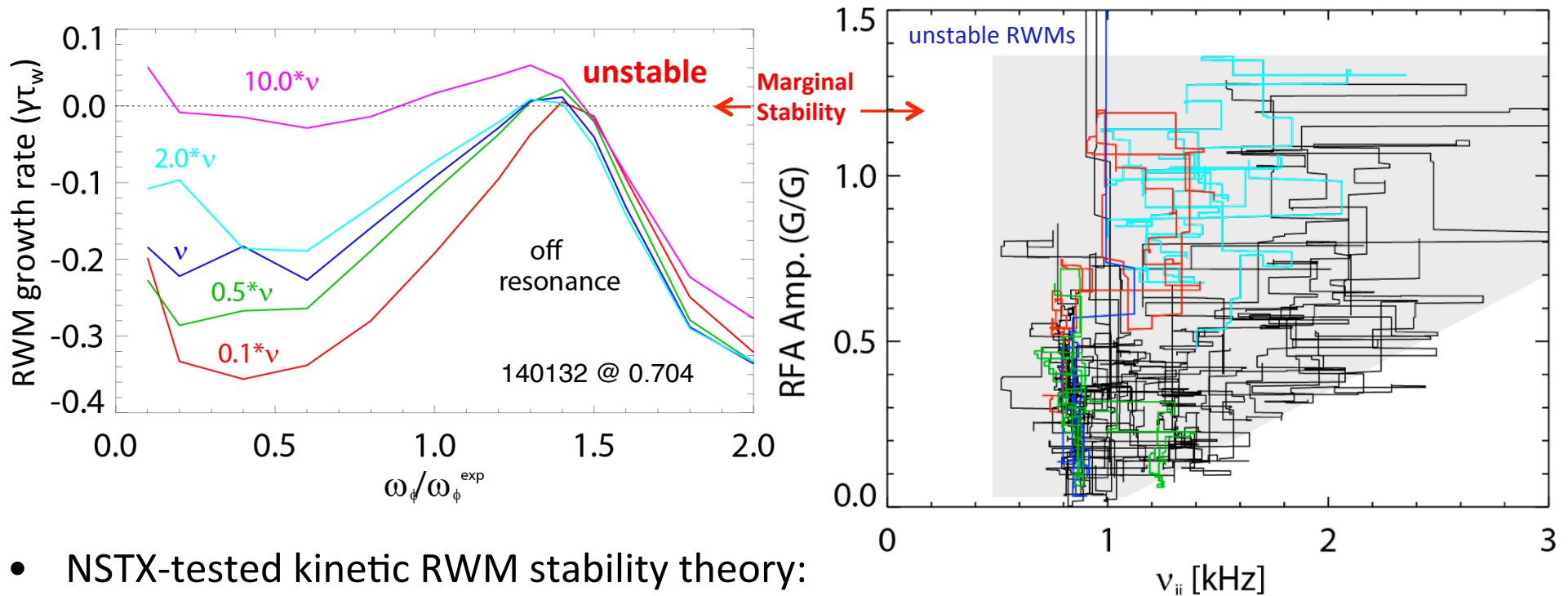
# RFA measurements confirm previous NSTX result that the highest $\beta_N/I_i$ is not the least stable



- NSTX can reach high  $\beta$ , low  $I_i$  range where next-step STs aim to operate
  - Active control experiments reduced disruption probability from 48% to 14%, but mostly in high  $\beta_N/I_i$
- RFA amplitude from 20-shot database also peaks at intermediate  $\beta_N/I_i$ 
  - Increased stability at high  $\beta_N/I_i$  due to kinetic stabilization from resonances

# Theory: Reduced $\nu$ is stabilizing near kinetic resonances

## Experimental Confirmation: Reduced $\nu \rightarrow$ reduced low RFA



- NSTX-tested kinetic RWM stability theory:
  - Stabilizing collisional dissipation reduced (expected from early theory)
  - Stabilizing resonant kinetic effects enhanced (contrasts early RWM theory)
- RFA amplitude reduced at lower  $\nu$  for low RFA (stable) plasmas, little effect on higher RFA (marginal) plasmas
- Expectations in NSTX-U, tokamaks at lower  $\nu$  (ITER)
  - Stronger stabilization near  $\omega_\phi$  resonances; almost no effect off-resonance

[J. Berkery *et al.*, Phys. Rev. Lett. **106**, 075004 (2011)]

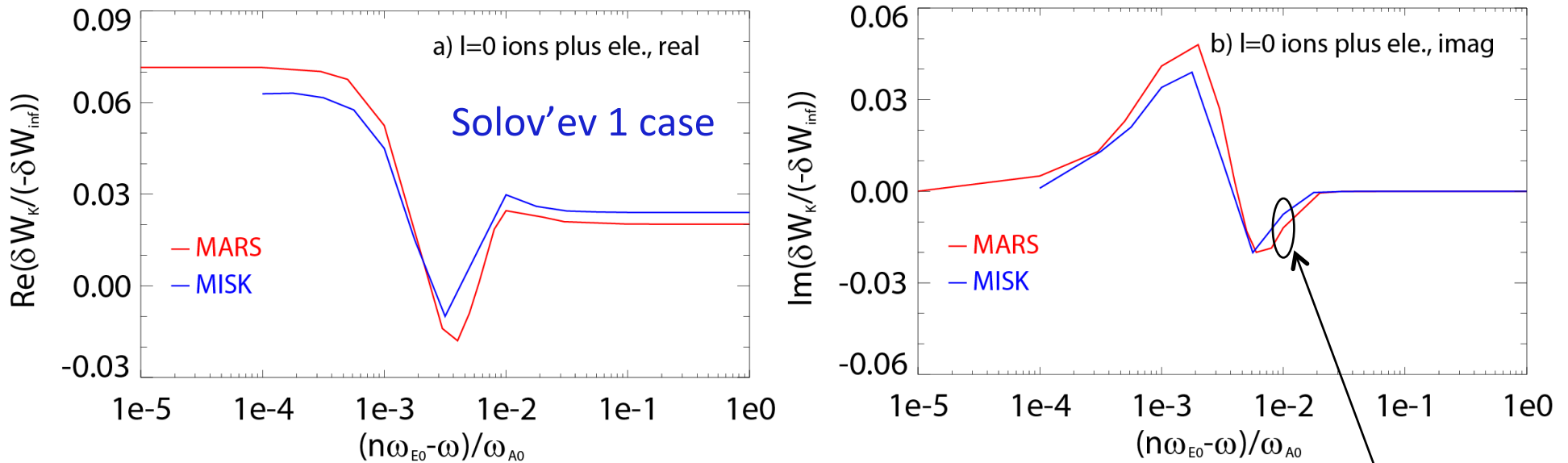
# Agreement achieved between MISK and MARS-K under ITPA MHD Stability Group MDC-2 Benchmarking

Work in progress!

	$r_{\text{wall}}/a$	Ideal $\delta W / -\delta W_{\infty}$	$\text{Re}(\delta W_k) / \delta W_{\infty}$	$\text{Im}(\delta W_k) / (\delta W_{\infty})$	$\gamma \tau_{\text{wall}}$	$\omega \tau_{\text{wall}}$	$\delta W_k / -\delta W_{\infty}$ ( $\omega_E = \infty$ )
<u>Solov'ev 1</u> (MARS-K) (MISK)	1.15	1.187 1.122	0.0256 0.0271	-0.0121 -0.0077	0.804 0.847	-0.0180 -0.0124	0.157 0.153
<u>Solov'ev 3</u> (MARS-K) (MISK)	1.10	1.830 2.337	0.209 0.219	-0.342 -0.128	0.350 0.302	-0.228 -0.065	4.51 0.898
<u>ITER</u> (MARS-K) (MISK)	1.50	0.682 0.677	-4.51 0.653	-0.445 -0.746	-1.43 -0.041	-0.050 -0.538	229 8.46

- Calculations from MISK, and **MARS-K** (perturbative)
  - The relevant frequencies and eigenfunctions now match between codes for both analytical Solov'ev and ITER equilibria.
  - Numerical approach to the frequency resonance fraction energy integral taken in MISK is equivalent to analytical limits computed in MARS-K.

# Agreement achieved between MISK and MARS-K under ITPA MHD Stability Group MDC-2 Benchmarking



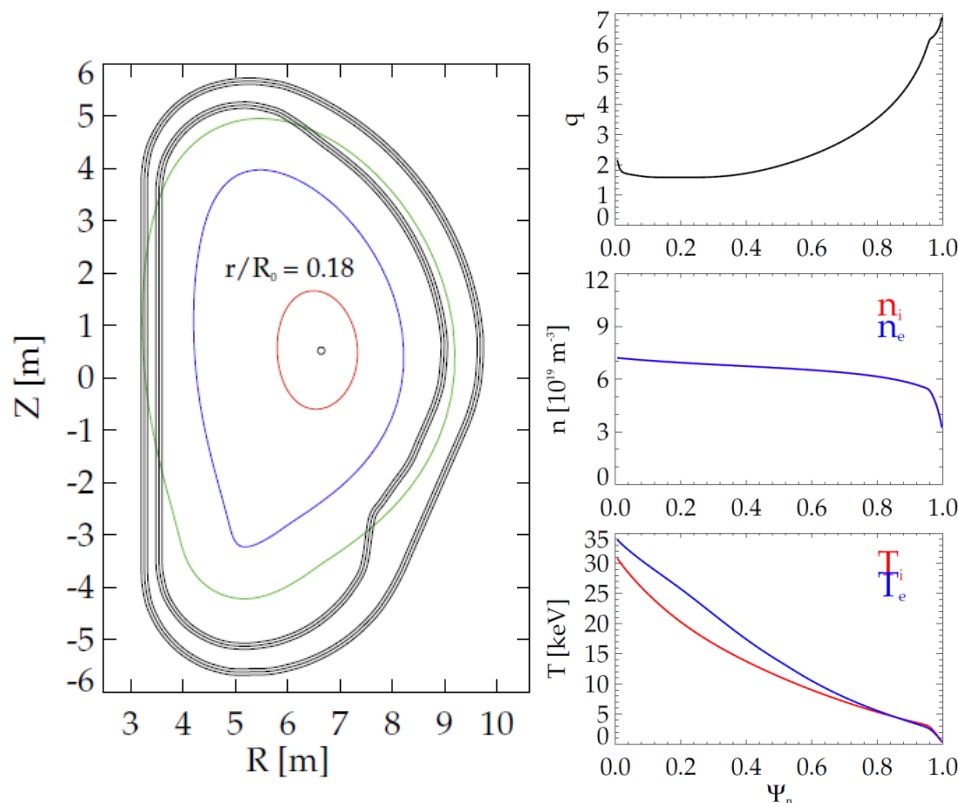
- Comparing  $\delta W_K$  vs.  $\omega_E$  scan (rather than single point ( $\omega_E = 1e-2$ ) in chart)
  - Good agreement for precession drift resonance ( $l=0$  trapped particles) and circulating particles
  - “Light green” in chart can be deceiving: really OK agreement

$\text{Im}(\delta W_K) / (\delta W_{\infty})$

-0.0121  
-0.0077

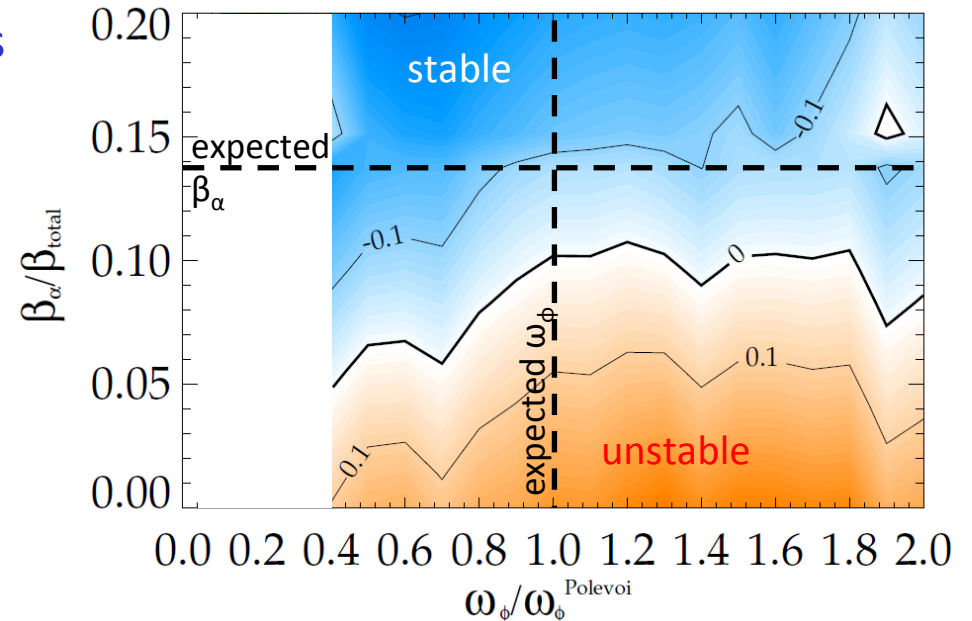
# ITER advanced scenario requires alpha particles for RWM stability across all rotation values

In a previously analyzed case [J.W. Berkery *et al.*, Phys. Plasmas 17, 082504 (2010)],  $\alpha$ s were required for stability across all  $\omega_\phi$ .



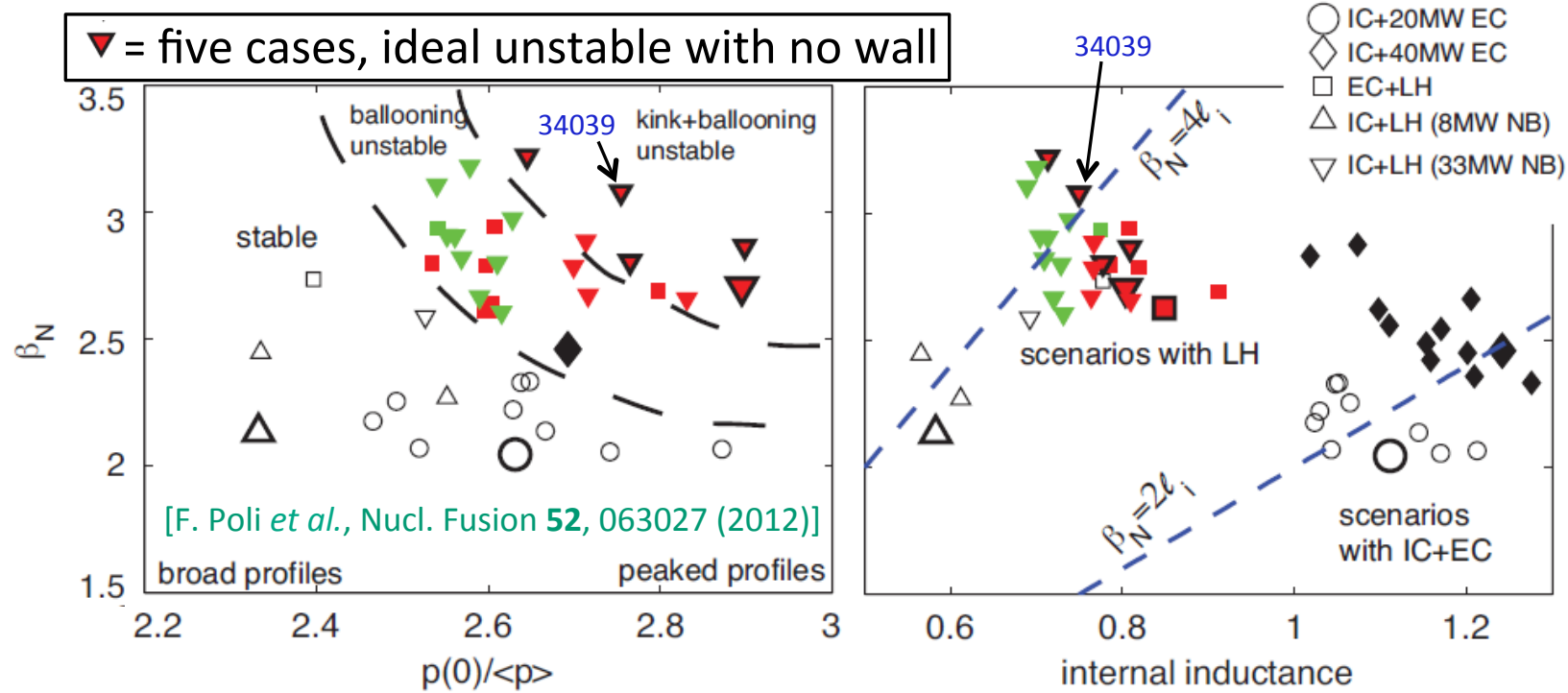
- ITPA MHD WG7 equilibrium
  - $I_p = 9$  MA,  $\beta_N = 2.9$  (7% above no-wall limit)

$\gamma\tau_w$  contours vs.  $\beta_\alpha$  and  $\omega_\phi$



- Calculation revisited with physics improvements (inc. correction to  $\omega_D$  in MISK)
  - Makes calculation somewhat more stable, but generally consistent. Doesn't affect the conclusions.

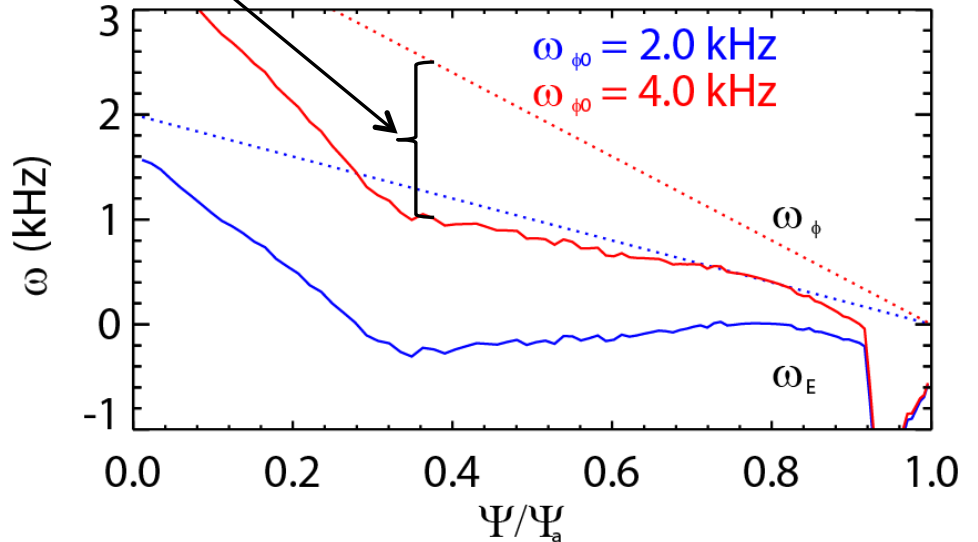
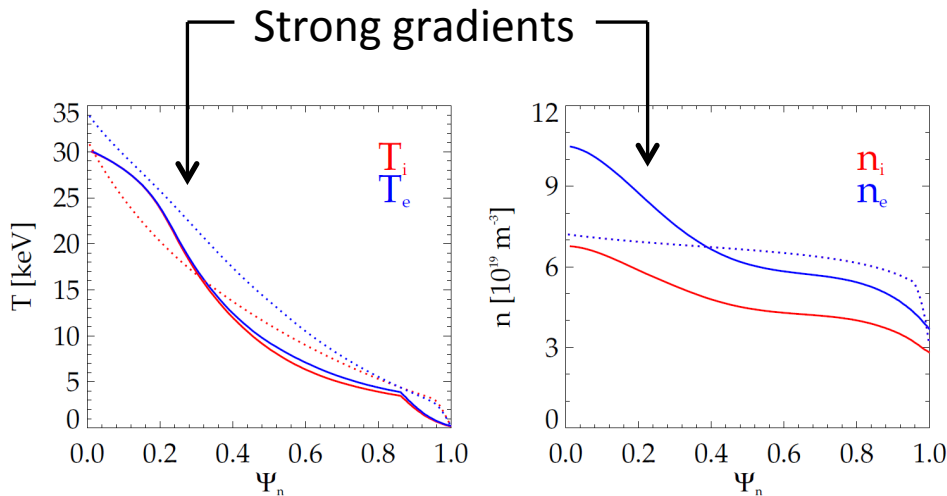
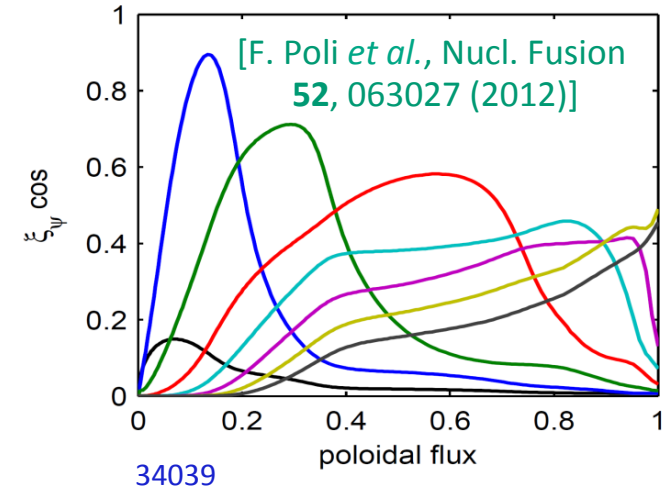
# Kinetic RWM stability analysis started with MISK for a wider database of ITER advanced scenario equilibria



- Five discharges selected; self-consistent variation in parameter space
  - Full discharge evolutions – created by combination of TSC and TRANSP codes
  - Range of  $\beta_N = 2.65 - 3.25$ ; ideal  $n=1$  no-wall unstable
  - Have internal transport barriers
- Include EPs from: 33MW N-NBI (D), 20 MW IC, 40 MW LH
  - Next steps: include anisotropic EP dist.: slowing-down for beams, bi-Max for

# Low rotation, ITBs in ITER can cause stabilizing precession drift resonance in plasma core

- “Infernal” type eigenfunction peaks near core.
- Strong internal gradients create large  $\omega_*$ 
  - Cause difference between  $\omega_\phi$  and  $\omega_{\text{EXB}}$ .
  - Enables resonance with precession drift of trapped thermal ions if  $\omega_\phi$  is low.



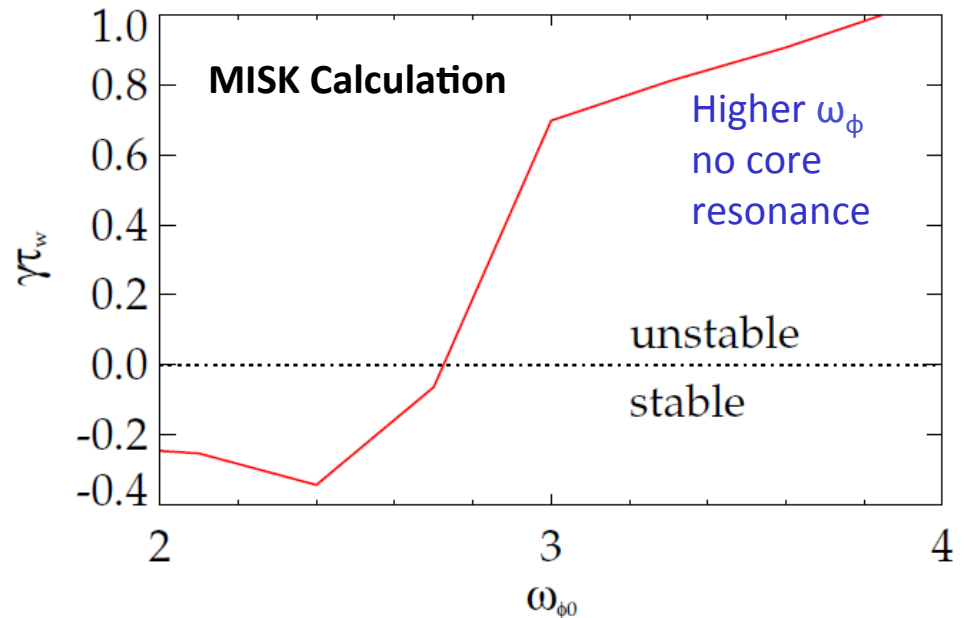
dashed lines = “standard” ITER advanced scenario

$$\omega_\phi \approx \omega_E + \omega_{*i}$$

# Internal transport barriers may be beneficial to RWM stability by lowering the E×B frequency

- Stable region found with no alphas

- At low  $\omega_\phi$ , due to precession resonance of trapped thermal ions, coupled with internal eigenfunction
- Unstable region at higher  $\omega_\phi$  similar to previous results



- Caveat: ITBs can be transient

- This may result in RWM instability if profile dynamics move the plasma off-resonance
- Active RWM control would then be needed during the period when the plasma profiles are away from stabilizing kinetic resonances



# Global mode control and stabilization studies in high- $\beta$ NSTX plasmas aid the goal of disruption avoidance in ITER

- Two active control techniques were used to avoid disruptions
  - Disruption probability was reduced from 48% to 14% in high  $\beta_N/I_i$  plasmas with dual field component (radial and poloidal) active control
  - A model-based state space controller sustained long-pulse, high- $\beta_N$  discharges with
- Dedicated resonant field amplification (RFA) experiments in NSTX revealed key dependencies of stability on plasma parameters
  - RFA measurements add additional support to the established theory of resistive wall mode (RWM) stability through kinetic mode-particle resonances
  - Stability is weakest at intermediate, not the highest, values of  $\beta_N/I_i$ , in agreement with other NSTX active control experiments
  - Relatively stable plasmas appear to benefit from reduced collisionality, in agreement with expectation from kinetic theory
- Application of the model to ITER plasmas indicates
  - Alpha particles may be needed for RWM stability
  - ITBs may be beneficial to RWM stability by lowering the  $E \times B$  frequency

Supported by U.S. Department of Energy Contracts: DE-FG02-99ER54524, DE-AC02-09CH11466, and DE-FG02-93ER54215

# Extra Slides

# Global Mode Control and Stabilization for Disruption Avoidance in High- $\beta$ NSTX Plasmas

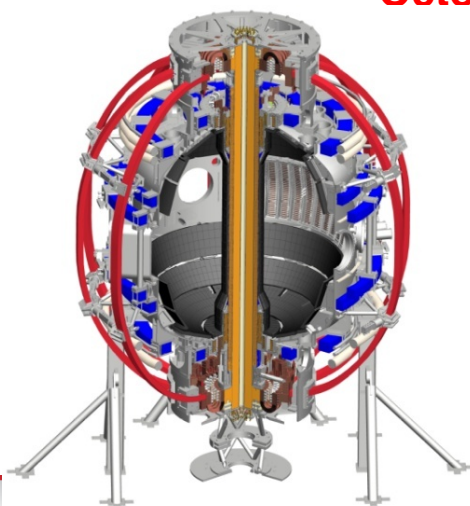
**J.W. Berkery<sup>1</sup>, S.A. Sabbagh<sup>1</sup>, A. Balbaky<sup>1</sup>, R.E. Bell<sup>2</sup>, R. Betti<sup>3</sup>, J.M. Bialek<sup>1</sup>, A. Diallo<sup>2</sup>, D.A. Gates<sup>2</sup>, S.P. Gerhardt<sup>2</sup>, O.N. Katsuro-Hopkins<sup>1</sup>, B.P. LeBlanc<sup>2</sup>, J. Manickam<sup>2</sup>, J.E. Menard<sup>2</sup>, Y.S. Park<sup>1</sup>, M. Podestà<sup>2</sup>, F. Poli<sup>2</sup>, H. Yuh<sup>4</sup>**

<sup>1</sup>Columbia U., <sup>2</sup>PPPL, <sup>3</sup>U. Rochester, <sup>4</sup>Nova Photonics

**24<sup>th</sup> IAEA Fusion Energy Conference  
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Mode Control in NSTX (EX/P8-U7, Berkery)

# State derivative feedback algorithm used for current control

State equations to advance

$$\dot{\vec{x}} = A\vec{x} + B\vec{u} \quad \vec{u} = -K_c\vec{x} = \vec{I}_{cc}$$

$$\vec{y} = C\vec{x} + D\vec{u}$$

Advance discrete state vector

$$\hat{\vec{x}}_t = A\vec{x}_{t-1} + B\vec{u}_{t-1}; \hat{\vec{y}}_t = C\hat{\vec{x}}_t$$

$$\vec{x}_{t+1} = \hat{\vec{x}}_t + A^{-1}K_o(\vec{y}_{sensors(t)} - \hat{\vec{y}}_t) \quad \text{“measurement update”}$$

Control vector,  $u$ ; controller gain,  $K_c$   
 Observer est.,  $y$ ; observer gain,  $K_o$ ;  $D = 0$   
 $K_c$ ,  $K_o$  computed by standard methods  
 (e.g. Kalman filter used for observer)

“time update”

State derivative feedback: superior control approach

$$\dot{\vec{x}} = A\vec{x} + B\vec{u} \quad \vec{u} = -\hat{K}_c\dot{\vec{x}} \longrightarrow \vec{I}_{cc} = -\hat{K}_c\dot{\vec{x}}$$

$$\dot{\vec{x}} = ((I + B\hat{K}_c)^{-1}A)\vec{x}$$

new Ricatti equations to solve to derive control matrices – still  
 “standard” solutions in control theory literature

[T.H.S. Abdelaziz, M. Valasek, Proc. of 16<sup>th</sup> IFAC World Congress (2005)]

# Requests for an electronic copy

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*Supported by U.S. Department of Energy Contracts: DE-FG02-99ER54524, DE-AC02-09CH11466, and DE-FG02-93ER54215*

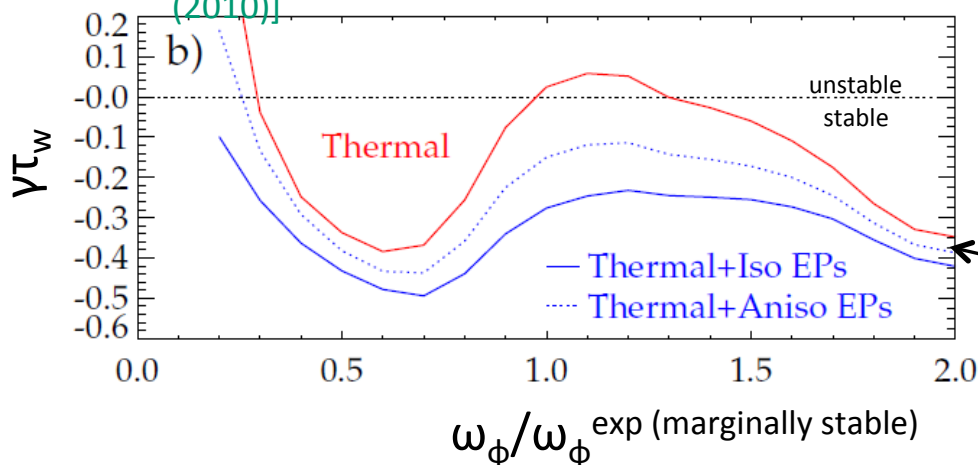
# Improving quantitative agreement consistently: EPs are stabilizing; Anisotropic distribution impacts stability

$$\delta W_K \sim \left[ \frac{1}{\langle \omega_D \rangle + l\omega_b - i\nu_{\text{eff}} + \omega_E} \right]$$

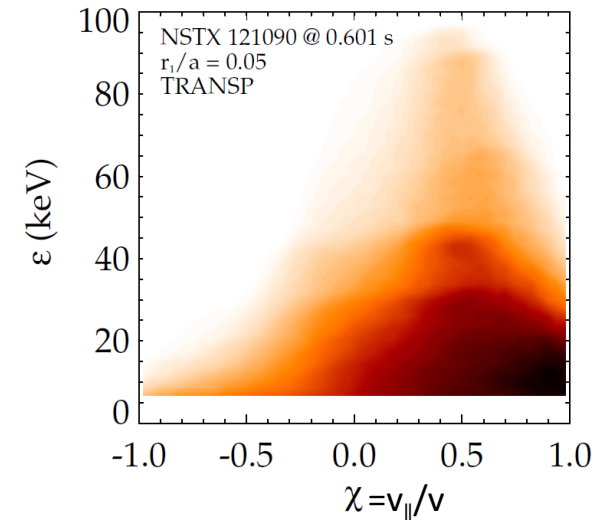
small for Energetic Particles (EPs)

- EPs provide stabilizing force that is nearly independent of  $\omega_\phi$
- EPs generally are not in mode resonance, so the effect is not energy dissipation, but rather a restoring force

[J.W. Berkery *et al.*, Phys. Plasmas **17**, 082504 (2010)]



Beam ions are anisotropic



$$f(\epsilon, \Psi, \chi) = \frac{C(\Psi)}{\epsilon^{\frac{3}{2}} + \epsilon_c^{\frac{3}{2}}} \frac{e^{-(\chi - \chi_0)^2 / \delta\chi^2}}{\delta\chi}$$

Addition of simple anisotropy model ( $\chi_0 = 0.75$ ,  $\delta\chi = 0.25$ ) reduces stabilizing effect, consistent with quantitative comparison to NSTX plasmas

# Pressure anisotropy leads to a modification of the Energy Principle

$$-i\omega\tau_w = -\frac{\delta W_V^\infty + \delta W_F + \delta W_A + \delta W_K}{\delta W_V^b + \delta W_F + \delta W_A + \delta W_K}$$

$\delta W_V$  : usual changes in vacuum potential energy without a wall, and with an ideal wall

$\delta W_F$  : usual isotropic fluid term

$\delta W_A$  : anisotropic fluid correction

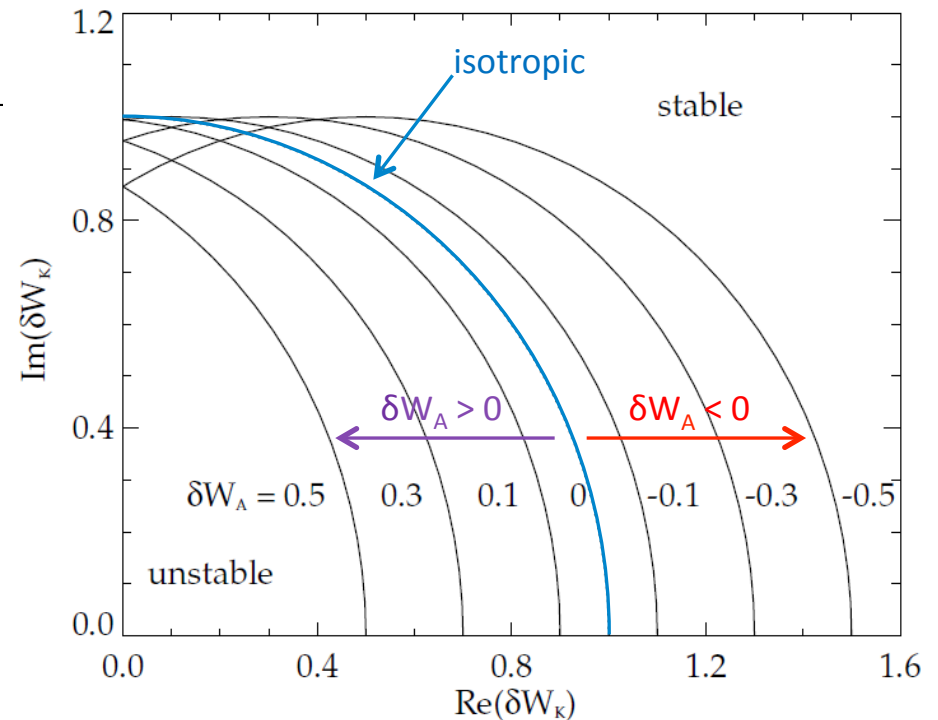
$\delta W_K$  : kinetic term (also modified by anisotropy)

$$\omega = \omega_r + i\gamma$$

$\omega_r$  : real mode rotation frequency

$\gamma$  : RWM growth rate

Positive  $\delta W_A$  shifts the unstable region to the left (destabilizing ballooning term reduced), negative  $\delta W_A$  to the right (**ballooning term increased**).



Stability diagram showing contours of  $\gamma\tau_w = 0$  on  $\text{Re}(\delta W_K)$  vs.  $\text{Im}(\delta W_K)$  with  $\delta W_\infty = -1$  and  $\delta W_b = 1$  [arb.].

# The kinetic approach is used to obtain $\delta W_K$ ; Anisotropic distribution functions affect the kinetic terms

$\delta W$  terms are calculated starting from a plasma force balance:

$$\delta W = \frac{1}{2} \int \boldsymbol{\xi}_{\perp}^* \cdot \left[ \mathbf{j}_0 \times \tilde{\mathbf{B}} + \tilde{\mathbf{j}} \times \mathbf{B}_0 - \nabla \cdot \tilde{\mathbb{P}} \right] d\mathbf{V}$$

with:  $\mathbb{P} = p_{\parallel} \hat{\mathbf{b}}\hat{\mathbf{b}} + p_{\perp} \left( \hat{\mathbf{I}} - \hat{\mathbf{b}}\hat{\mathbf{b}} \right)$

CGL pressures are akin to assumption of fast rotating mode (*not the RWM*), and will not be used. Instead in the kinetic approach, the perturbed pressures are calculated rigorously from the perturbed distribution function:

$$\tilde{f}_j = -\boldsymbol{\xi}_{\perp} \cdot \nabla f_j + Z_j e \frac{\partial f_j}{\partial \varepsilon} \tilde{\Phi} + im_j \left( \omega \frac{\partial f_j}{\partial \varepsilon} - n \frac{\partial f_j}{\partial P_{\phi}} \right) (\mathbf{v} \cdot \boldsymbol{\xi}_{\perp} - \tilde{s}_j) - \frac{m_j}{B} \frac{\partial f_j}{\partial \mu} \left( -i\omega \boldsymbol{\xi}_{\perp} \cdot \mathbf{v}_{\perp} + \frac{\mu}{m_j} \tilde{\mathbf{B}}_{\parallel} + \frac{v_{\parallel}}{B} \mathbf{v}_{\perp} \cdot \tilde{\mathbf{B}} \right)$$

Kinetic effects are modified by inclusion of bi-Maxwellian or anisotropic slowing-down distributions in frequency resonance fraction calculation

$$\delta W_K = \sum_j \sum_{l=-\infty}^{\infty} 2\sqrt{2}\pi^2 \int \int \int \left[ |\langle H/\hat{\varepsilon} \rangle|^2 \frac{(\omega - n\omega_E) \frac{\partial f_j}{\partial \varepsilon} - \frac{n}{Z_j e} \frac{\partial f_j}{\partial \Psi}}{n\langle \omega_D^j \rangle + l\omega_b^j - i\nu_{\text{eff}}^j + n\omega_E - \omega} \right] \frac{\hat{\tau}}{m_j^{\frac{3}{2}} B} |\chi| \hat{\varepsilon}^{\frac{5}{2}} d\hat{\varepsilon} d\chi d\Psi,$$



# Additionally, an anisotropic correction to the fluid term principally modifies the ballooning destabilization term

$$\delta W_F = \frac{1}{2} \int \left\{ \underbrace{\left( -\frac{|\tilde{\mathbf{B}}_{\perp}|^2}{\mu_0} \right)}_{\text{shearAlfvén}} - \underbrace{\frac{B^2}{\mu_0} |\nabla \cdot \boldsymbol{\xi}_{\perp} + 2\boldsymbol{\xi}_{\perp} \cdot \boldsymbol{\kappa}|^2}_{\text{fast magneto-acoustic}} + \underbrace{j_{\parallel} (\boldsymbol{\xi}_{\perp}^* \times \hat{\mathbf{b}}) \cdot \tilde{\mathbf{B}}_{\perp}}_{\text{kink}} \underbrace{+ 2(\boldsymbol{\kappa} \cdot \boldsymbol{\xi}_{\perp}^*) (\boldsymbol{\xi}_{\perp} \cdot \nabla p_{\text{avg}})}_{\text{ballooning}} \right\} dV, \quad (1) \quad (2)$$

$$\delta W_A = \frac{1}{2} \int \left\{ (\sigma - 1) \underbrace{\left( -\frac{|\tilde{\mathbf{B}}_{\perp}|^2}{\mu_0} - \frac{B^2}{\mu_0} |\nabla \cdot \boldsymbol{\xi}_{\perp} + 2\boldsymbol{\xi}_{\perp} \cdot \boldsymbol{\kappa}|^2 + j_{\parallel} (\boldsymbol{\xi}_{\perp}^* \times \hat{\mathbf{b}}) \cdot \tilde{\mathbf{B}}_{\perp} \right)}_{\text{These will be small due to } \sigma \approx 1.} - 2B |\nabla \cdot \boldsymbol{\xi}_{\perp} + \boldsymbol{\kappa} \cdot \boldsymbol{\xi}_{\perp}|^2 \frac{\partial p_{\text{avg}}}{\partial B} \right\} dV, \quad (3) \quad (4)$$

These will be small due to  $\sigma \approx 1$ .

Note: terms 1, 2, and 3 together can be shown to be self-adjoint, so that  $\delta W_F + \delta W_A$  is self-adjoint

$$(4) \rightarrow \delta W_{A2} = \sqrt{2}\pi^2 \int \int \int \frac{1}{m_j^{\frac{3}{2}}} |\nabla \cdot \boldsymbol{\xi}_{\perp} + \boldsymbol{\kappa} \cdot \boldsymbol{\xi}_{\perp}|^2 (\chi^4 - 1) \frac{\partial f_j}{\partial \chi} \frac{\hat{\tau}}{B} \varepsilon^{\frac{3}{2}} d\varepsilon d\chi d\Psi.$$

The anisotropic correction to the ballooning term depends upon the derivative of the distribution function with respect to pitch angle,  $\chi = v_{\parallel}/v$ .

# Two anisotropic distributions are considered: bi-Maxwellian for thermal particles, slowing-down for energetic particles

A bi-Maxwellian distribution with different temperatures  $\perp$  and  $\parallel$  to the magnetic field:

$$f_j^{bM}(\varepsilon, \Psi, \chi) = n_j \left( \frac{m_j}{2\pi} \right)^{\frac{3}{2}} \frac{1}{T_{j\perp} T_{j\parallel}^{\frac{1}{2}}} e^{-\varepsilon\chi^2/T_{j\parallel}} e^{-\varepsilon(1-\chi^2)/T_{j\perp}}$$

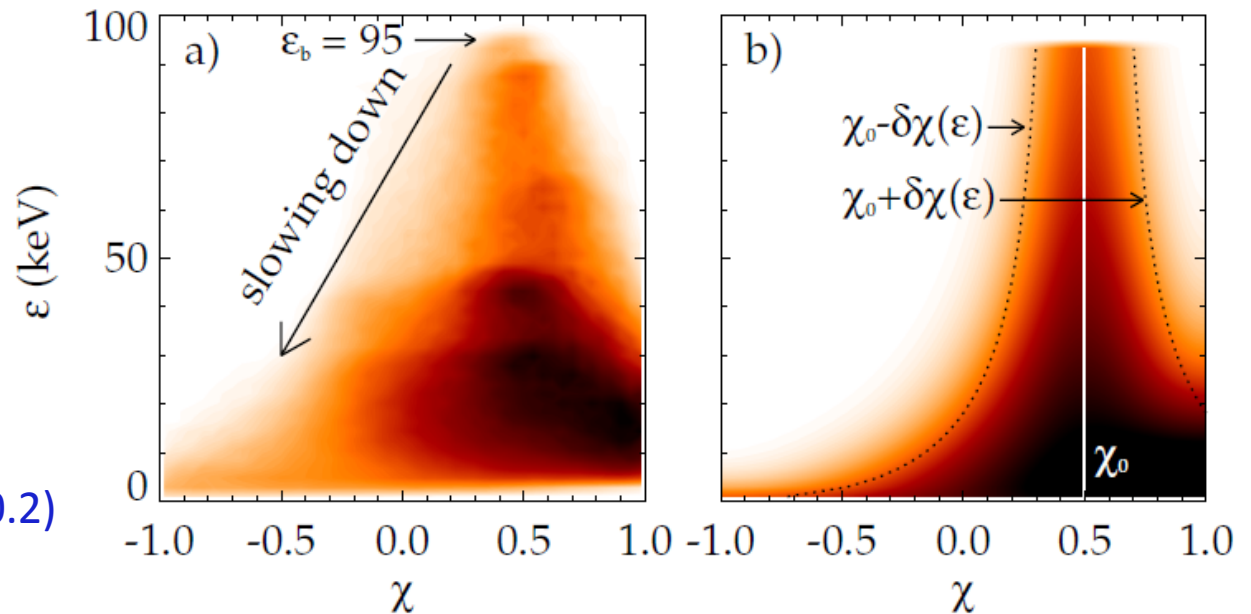
A slowing down distribution function with a Gaussian distribution of particles in  $\chi$ :

$$f_j^b(\varepsilon, \Psi, \chi) = n_j A_b \left( \frac{m_j}{\varepsilon_b} \right)^{\frac{3}{2}} \frac{1}{\hat{\varepsilon}^{\frac{3}{2}} + \hat{\varepsilon}^{\frac{3}{2}}_c} \frac{1}{\delta\chi} \left( \exp \left[ \frac{-(\chi - \chi_0)^2}{\delta\chi^2} \right] + \exp \left[ \frac{-(\chi + 2 + \chi_0)^2}{\delta\chi^2} \right] + \exp \left[ \frac{-(\chi - 2 + \chi_0)^2}{\delta\chi^2} \right] \right)$$

Beam ions are injected with injection pitch angle  $\chi_0$  and initial spread  $\delta\chi_0$

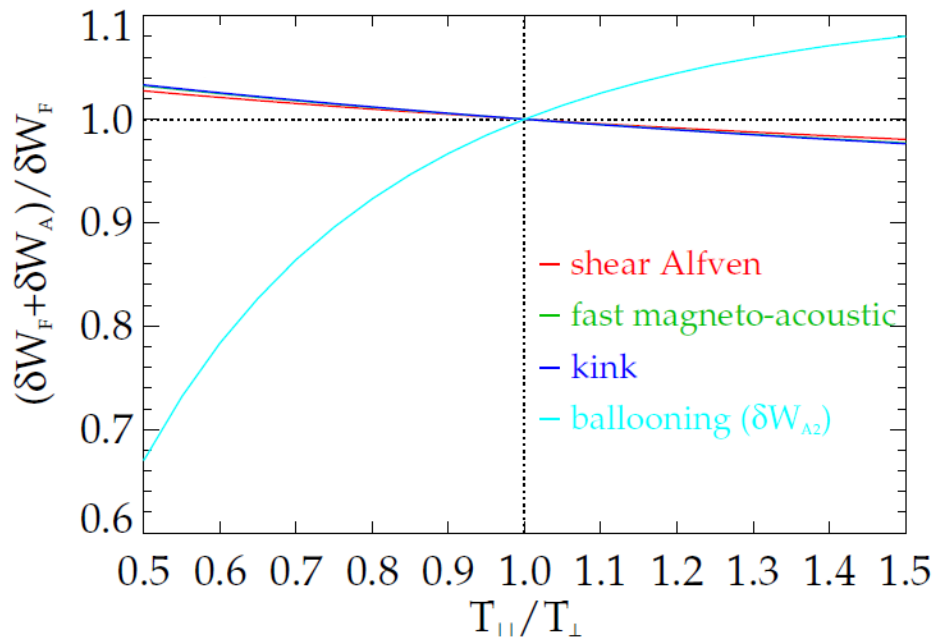
$f(\varepsilon, \chi)$  at a particular  $\Psi$  for an NSTX equilibrium from

- a) TRANSP
- b) model (with  $\chi_0 = 0.5$   $\delta\chi_0 = 0.2$ )

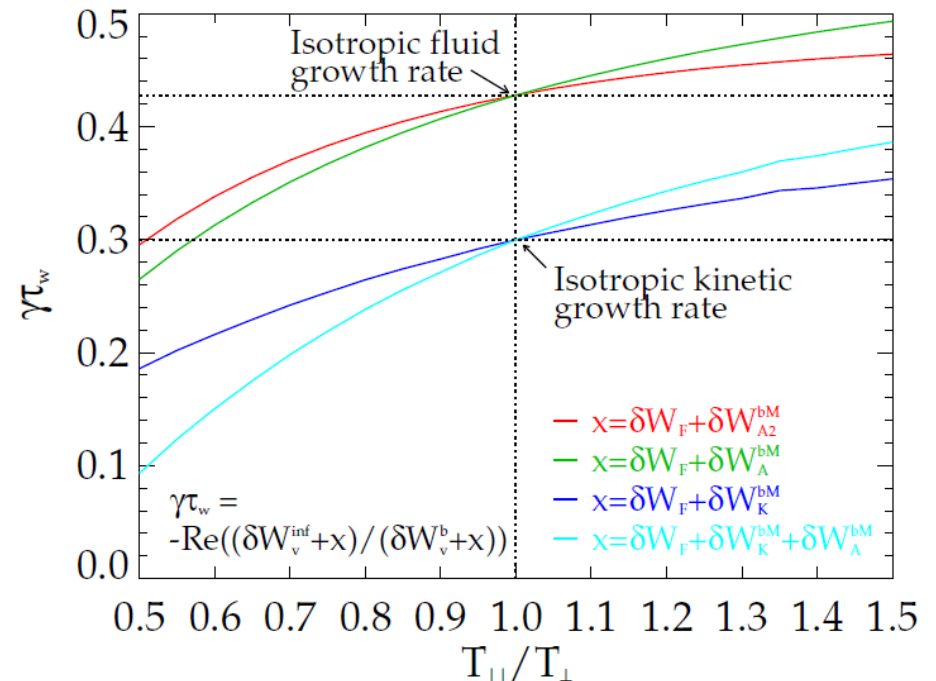


# Anisotropy of thermal particles was explored through $T_{\parallel}/T_{\perp}$ ; could potentially make an impact on the calculated RWM growth rate

## Corrections to each fluid term



## Impact of fluid corrections, and anisotropic kinetic effects on growth rate



Simple Solov'ev test case used to explore the effects.

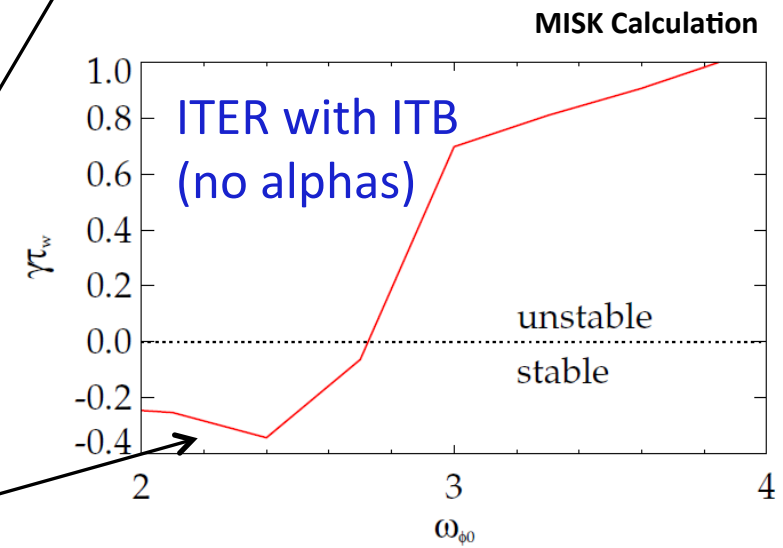
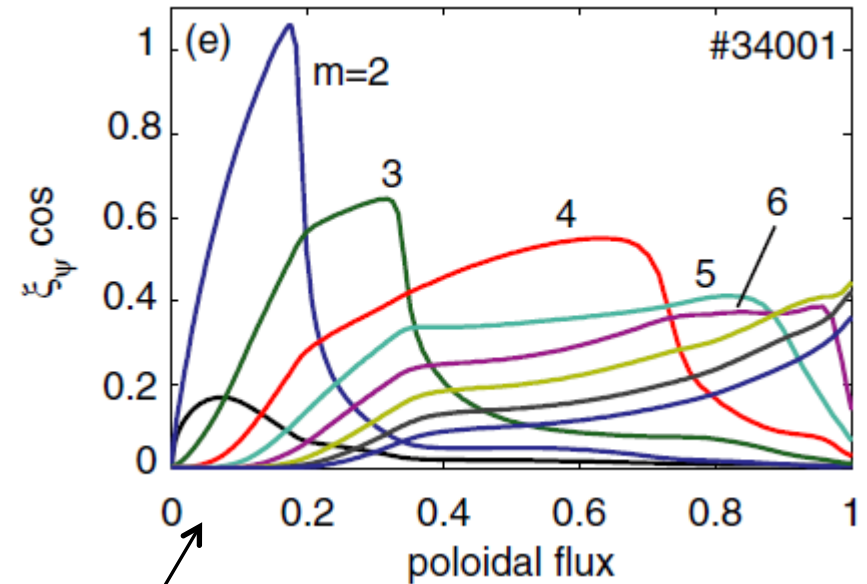
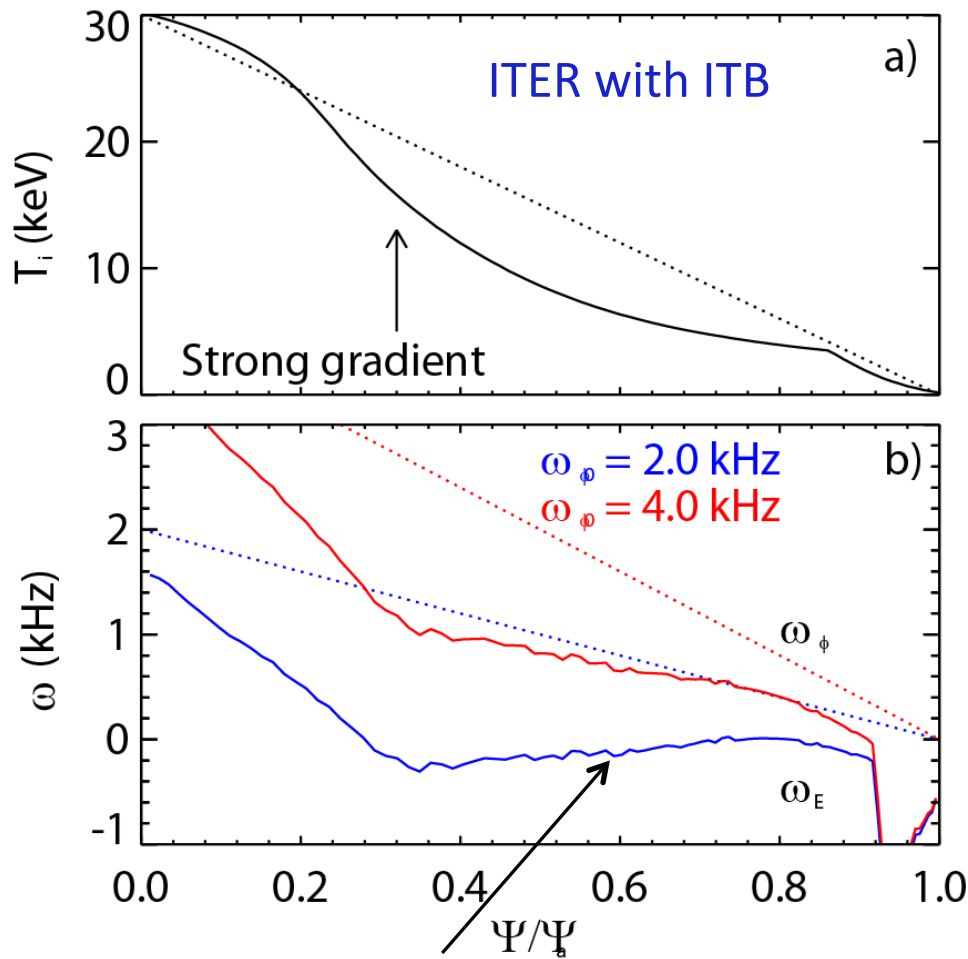
As  $T_{\parallel}/T_{\perp}$  is reduced, the destabilizing ballooning term is reduced.

As  $T_{\parallel}/T_{\perp}$  is reduced, the stabilizing kinetic term is increased.

Most applicable to experimental scenarios with RF heating (ITER).

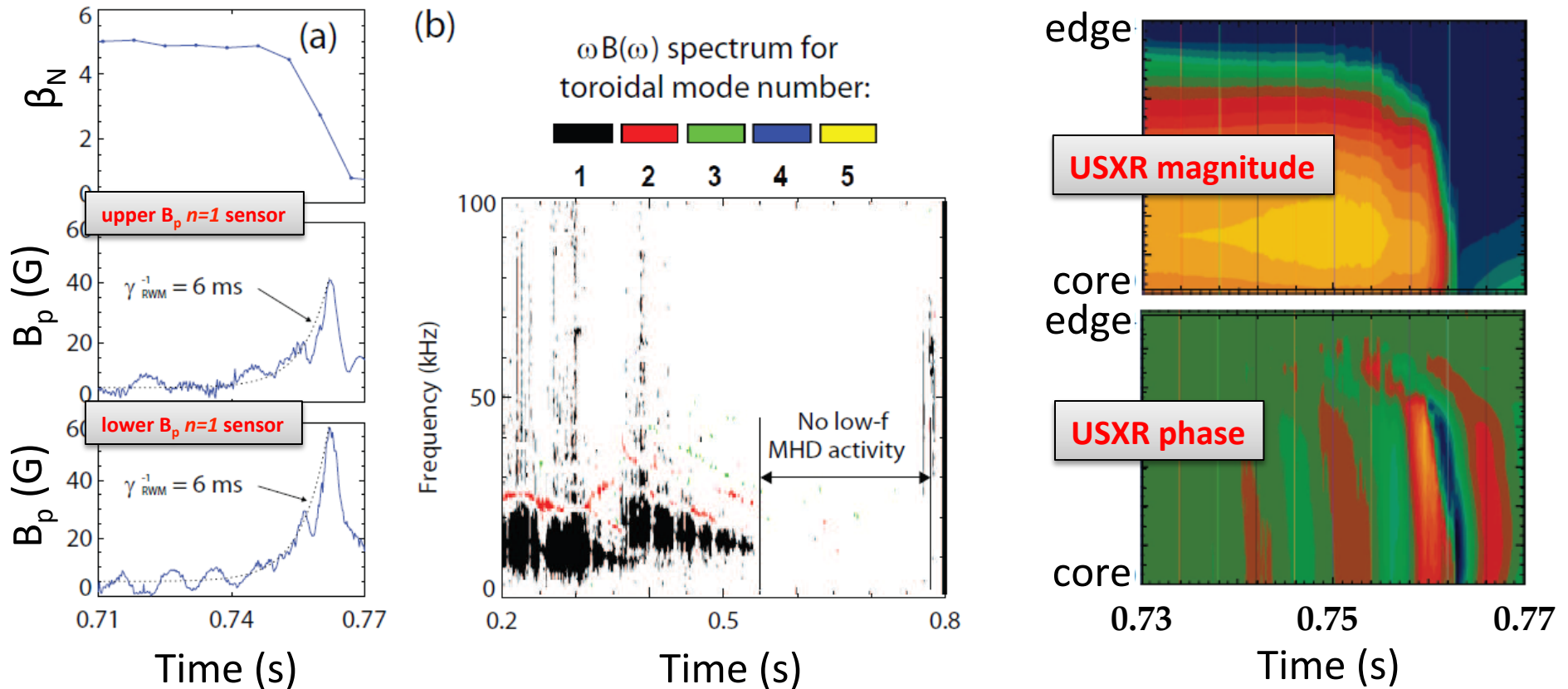
Reduction in RWM fluid growth rate and reduction in RWM kinetic growth rate with both kinetic effects and fluid corrections.

# Internal transport barriers may be beneficial to RWM stability by lowering the E×B frequency



ITB lowers  $\omega_E$  to  $\sim 0$ , combined with "infernal" type eigenfunction: predicted stability at low rotation without alphas

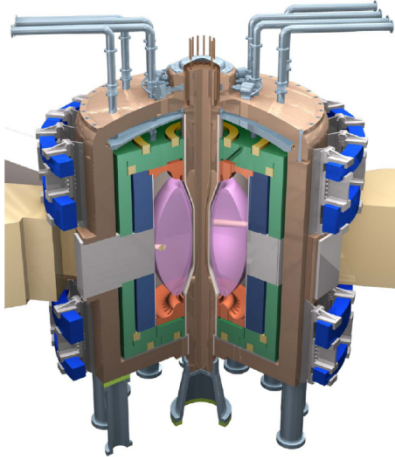
# The RWM is identified in NSTX by a variety of observations



- Growing signal on low frequency poloidal magnetic sensors
- Global collapse in USXR signals, with no clear phase inversion
- Causes a collapse in  $\beta$  and disruption of the plasma

# Future ST fusion applications will have high elongation, broad current profiles, high normalized beta

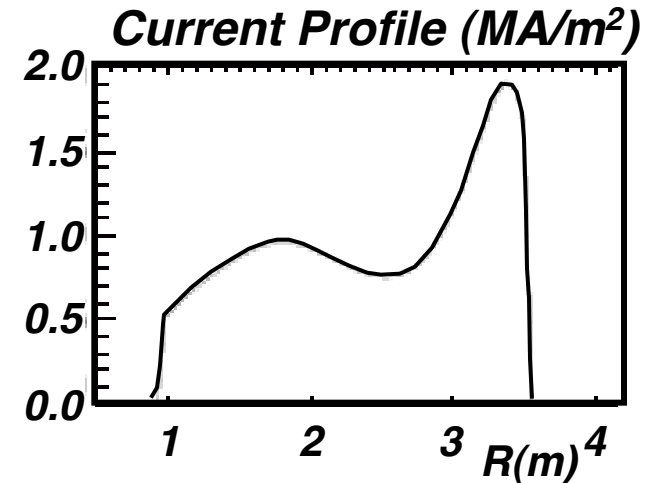
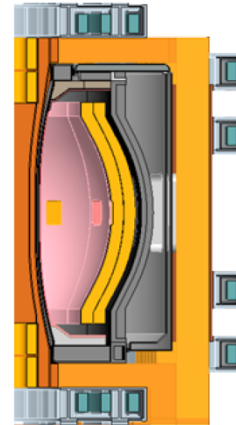
## FNSF / ST-CTF



Y.K.M. Peng, et al., PPCF 47 (2005) B263

## ST-Pilot ( $Q_{eng} = 1$ )

J. Menard, et al., IAEA FEC 2010 Paper FTP/2-2



- Broad current profile  $\rightarrow$  low  $I_i = \langle B_p^2 \rangle / \langle B_p \rangle_\psi^2$ , has global mode stability implications
  - Improved vertical ( $n = 0$ ) and wall-stabilized, rotating kink ( $n = 1$ ) stability
  - Decreased RWM stability, influenced by  $\omega_\phi$
  - “Troyon limit” (ideal, static  $n = 1$  no-wall stability limit)  
 $\beta_N = 10^8 \beta_t a B_t / I_p = \text{constant} \sim 3$ ; variants:  $\beta_N \propto I_i$ ,  $\beta_N \propto 1/(\text{pressure peaking})$

$$I_i = 0.47, \kappa = 3.2$$

$$R = 2.23 \text{ m}, A = 1.7$$

$$I_p = 16 \text{ MA}, B_t = 2.4 \text{ T}$$

$$\beta_N = 5.2, \beta_t = 30\%$$

**Operation at higher  $\beta_N$  possible by passive or active RWM stabilization** ( $\beta_t = 2\mu_0 \langle p \rangle / B_t^2$ )