NSTX-U

Supported by U.S. DEPARTMENT OF ENERGY | Office of Science

Overview of Physics Results from the National Spherical Torus Experiment

Coll of Wm & Mary
Columbia U
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U Tulsa
U Washington
U Wisconsin
X Science LLC

S. A. Sabbagh

Columbia University

for the NSTX-U Research Team

24th IAEA Energy Fusion Conference

October 9th, 2012

San Diego, California

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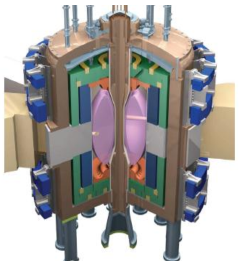
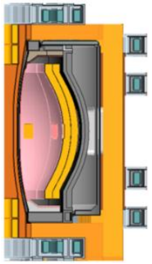
IPP, Garching
ASCR, Czech Rep

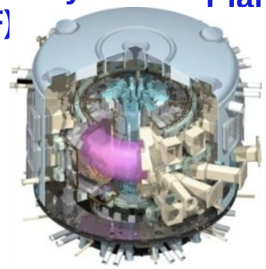
V2.1

NSTX research targets predictive physics understanding needed for fusion energy development facilities

- Enable devices: ST-FNSF, ST-Pilot/DEMO, ITER
  - Leveraging unique ST plasmas provides new understanding for tokamaks, challenges theory


Fusion Nuclear Science Facility (FNSF)

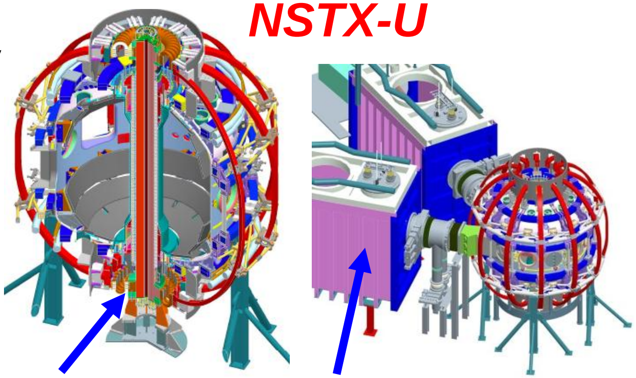

ST Pilot Plant


ITER

Outline

- Develop key physics understanding to be tested in unexplored, hotter ST plasmas
  - Study high beta plasma transport and stability at reduced collisionality, for extended pulse
  - Prototype methods to mitigate very high heat/particle flux
  - Move toward fully non-inductive operation

NSTX-U

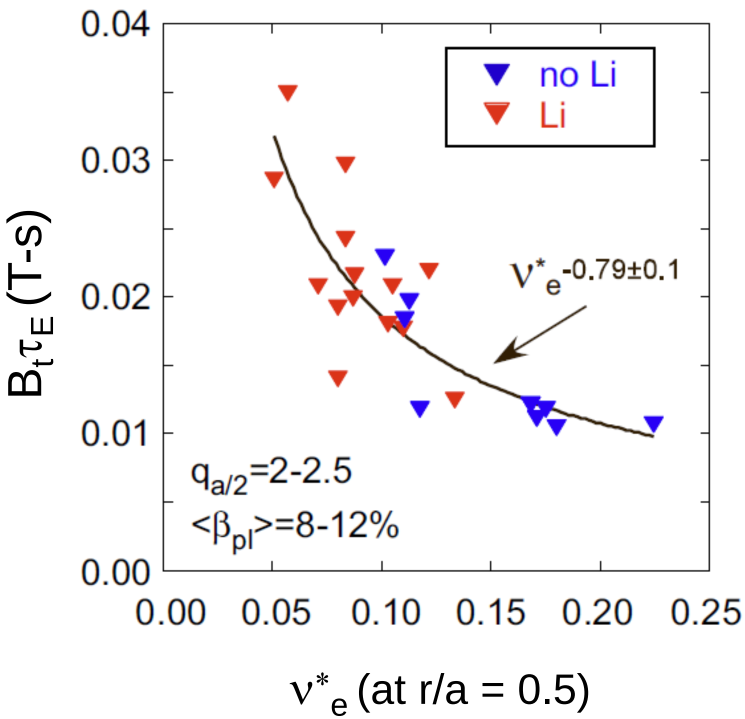

New center-stack   2nd neutral beam

B_T	0.5 → 1 T	P_{NBI}	6 → 12 MW
I_p	1 → 2 MA	pulse	1 → 5 s

3D effects are pervasive in this research

First successful nonlinear microtearing simulations for NSTX predict reduced electron heat transport at lower collisionality

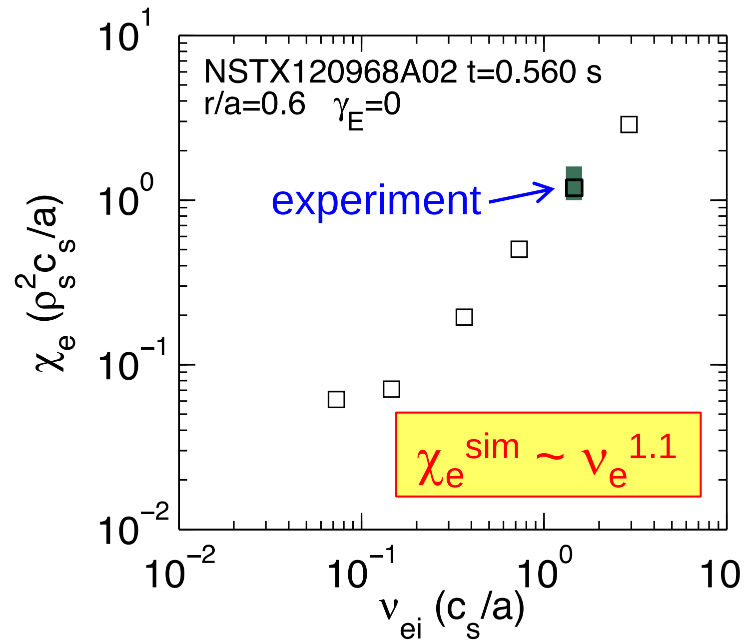
Experiment

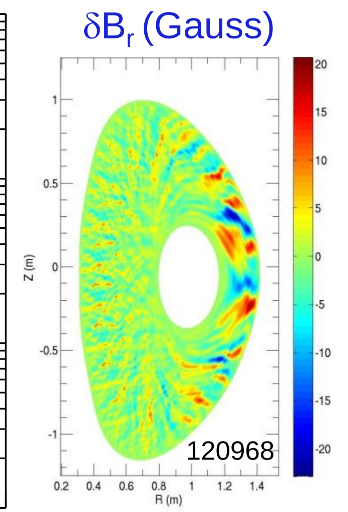


- Increase in τ_E as ν^*_e decreases
- Trend continues when lithium is used

Kaye EX/7-1

Theory





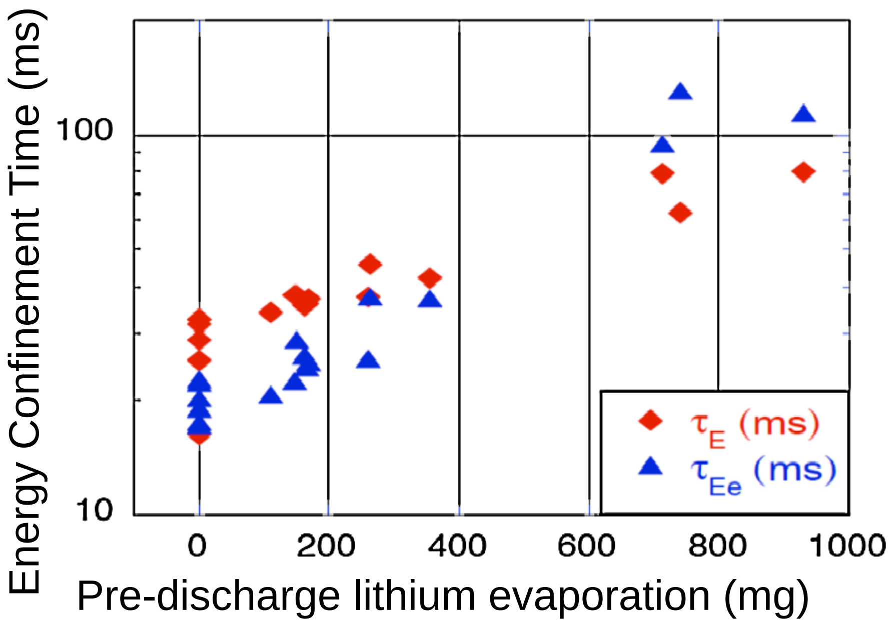
W. Guttenfelder, et al., PRL **106** (2011) 155004

- Predicted χ_e and scaling ~ $\nu_e^{1.1}$ consistent with experiment ($\Omega\tau_E$ ~ $B_t\tau_E$ ~ $\nu^*_e{}^{-0.8}$)
- Transport dominated by magnetic "flutter"
  - Significant $\delta B_r/B$ ~ 0.1%

Guttenfelder TH/6-1

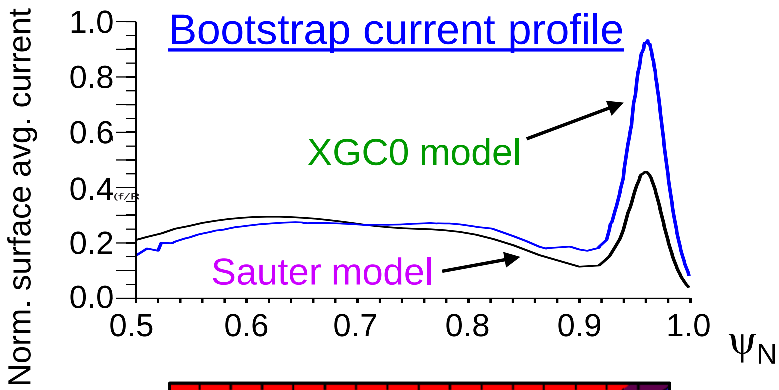
- NSTX-U computed to extend studies down to < 1/4 of present ν^*

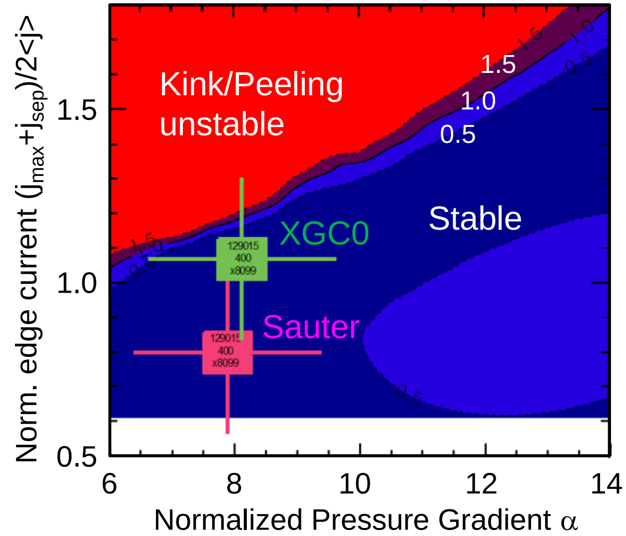
Plasma characteristics change nearly continuously with increasing lithium evaporation; reach kink/peeling limit



R. Maingi, et al., PRL **107** (2011) 145004

- Global parameters generally improve
- ELM frequency declines - to zero
  - ELMs stabilize
- Edge transport declines
  - As lithium evaporation increases, transport barrier widens, pedestal-top χ_e reduced



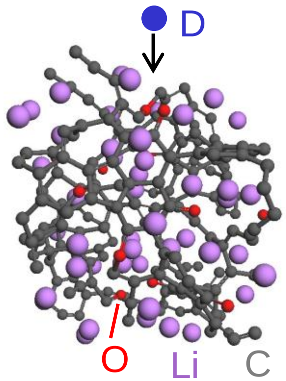


- New bootstrap current calculation (XGC0 code) improves agreement with profile reaching kink/peeling limit

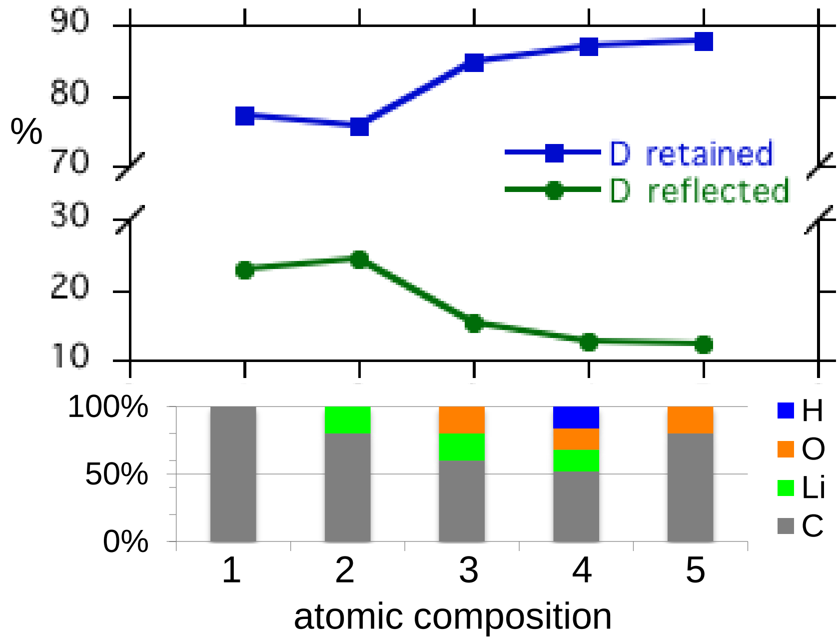
Maingi EX/11-2 **Canik EX/P7-16** **Chang TH/P4-12** **Diallo EX/P4-04**

Simulations and lab results show importance of oxygen in lithium-graphite PMI for pumping deuterium

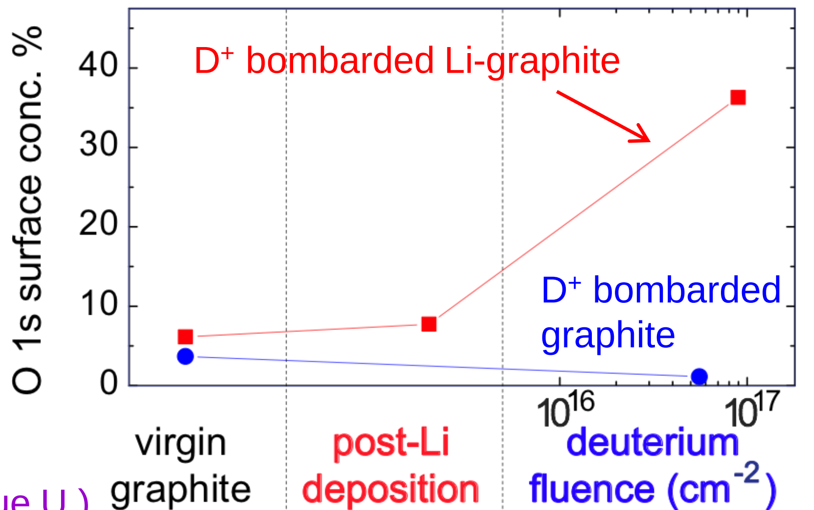
- Quantum-classical atomistic simulations show surface oxygen plays key role in the retention of deuterium in graphite



P. Krstic, sub. to Nature Comm.



- Accordingly, lab results support that Li on graphite can pump D effectively due to O
  - XPS measurements show 2 µm of Li increases surface oxygen content of lithiated graphite to ~10%
  - deuterium ion irradiation of lithiated graphite greatly enhances oxygen content to 20%-40%
    - In stark contrast, D irradiation of graphite *without* Li *decreases* amount of surface O
  - Li acts as an O getter, and the O retains D

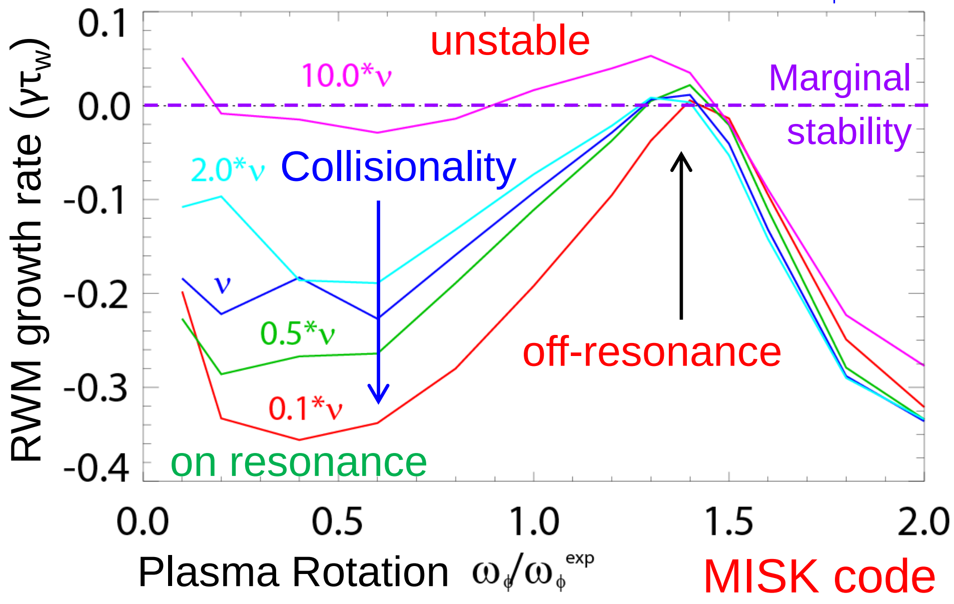


Jaworski EX/P5-31

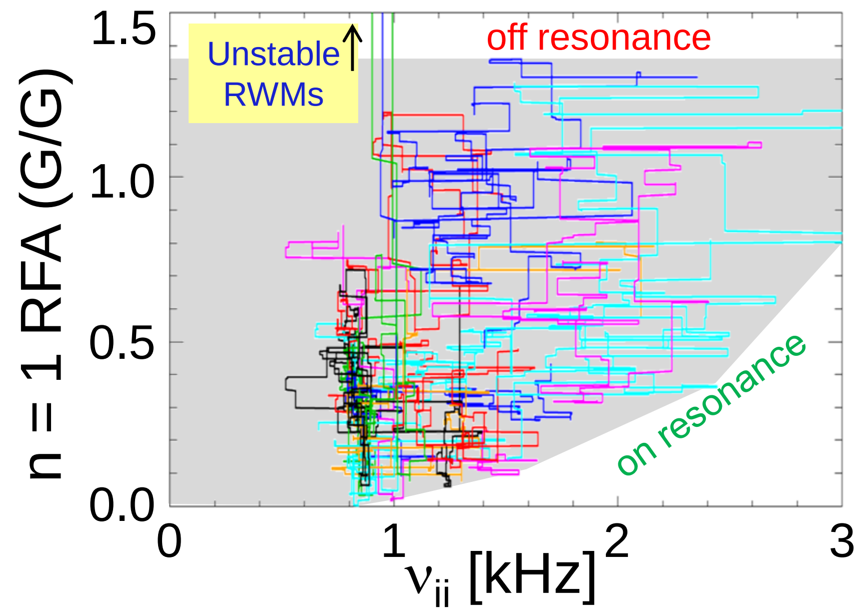
J.P. Allain, Taylor (Purdue U.)

Experiments measuring global stability vs. ν further support kinetic RWM stability theory, provide guidance for NSTX-U

Theory: RWM growth rate vs. ν and ω_ϕ



Exp: Resonant Field Amplification (RFA) vs ν



(trajectories of 20 experimental plasmas)

- Two competing effects at lower ν
  - Collisional dissipation reduced
  - Stabilizing resonant kinetic effects enhanced (contrasts early theory)
- Expectations at lower ν
  - More stabilization near ω_φ resonances; almost no effect off-resonance

J. Berkery et al., PRL **106** (2011) 075004

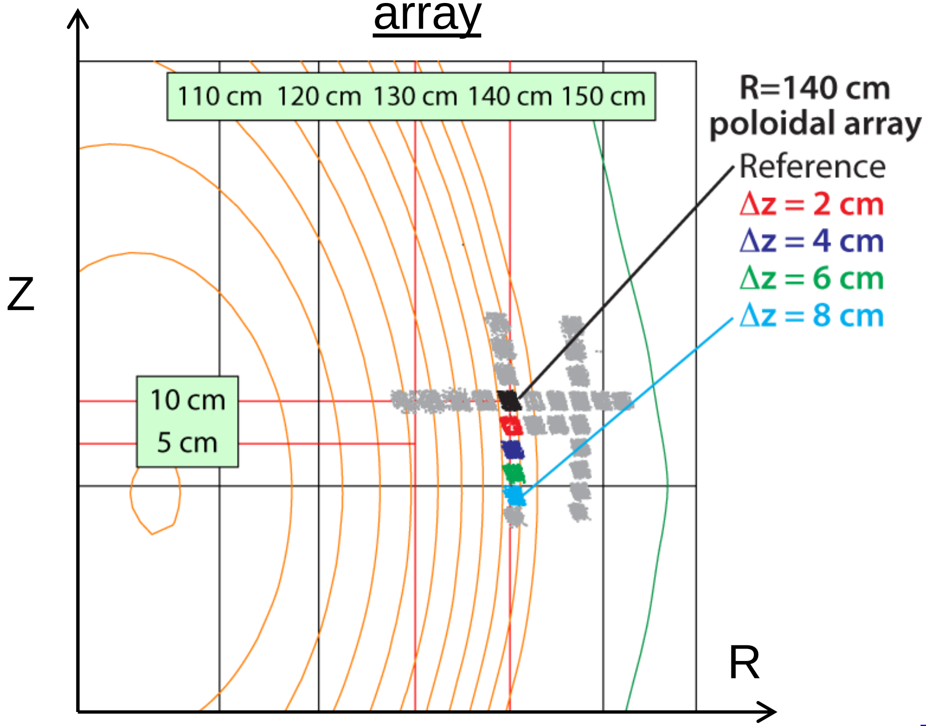
- Mode stability directly measured in experiment using MHD spectroscopy
  - Decreases with ν at lower RFA ("on resonance")
  - Independent of ν at higher RFA ("off resonance")

$$RFA = \frac{B_{plasma}}{B_{applied}}$$

Berkery EX/P8-07

BES measured low-*k* turbulence in ELM-free H-mode pedestal steep gradient region is most consistent with TEMs

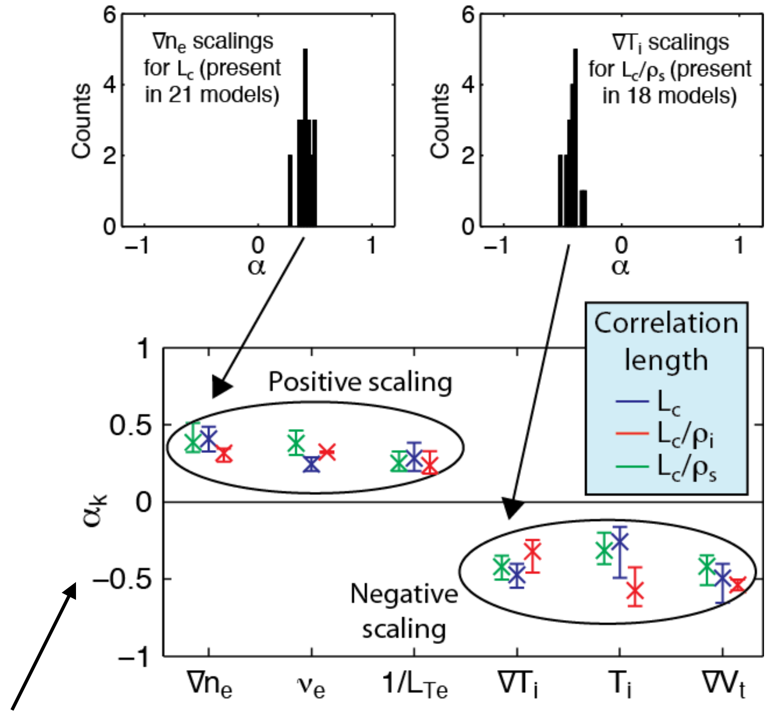
Beam emission spectroscopy (BES) array



- Measurements during MHD quiet periods, in steep gradient region
- Large poloidal correlation lengths
  - $k_\theta \approx 0.2\text{-}0.4\ cm^{-1}$ and $k_\theta \rho_i \approx 0.2$

Smith EX/P7-18

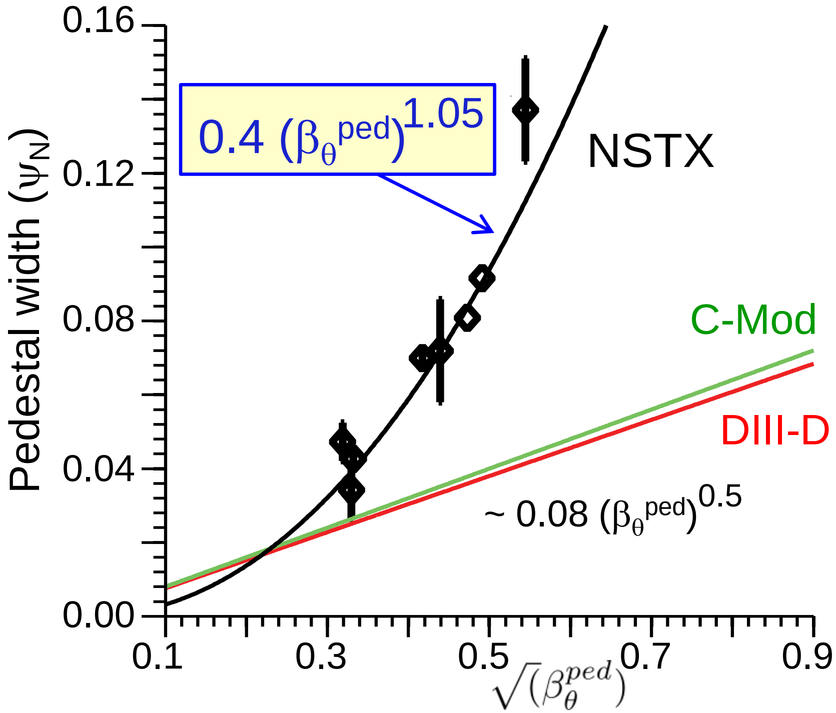
Poloidal Correlation Length vs. Parameters



- Multivariate linear scaling coefficients α_k
- Turbulence measurements in the steep gradient of the pedestal
  - Most consistent with Trapped Electron Modes
  - Partially consistent with KBM and μ-Tearing Modes
  - Least consistent with ITG Modes

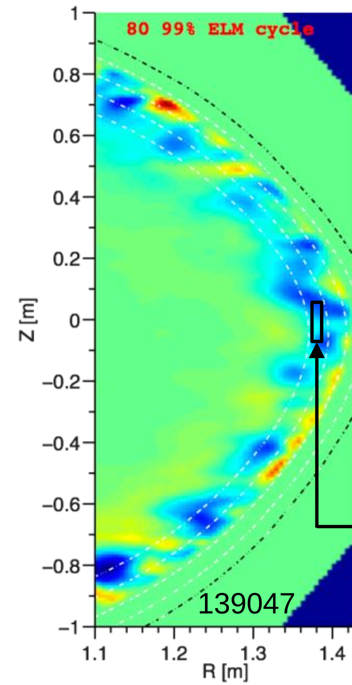
Pedestal width scaling differs from tokamaks; turbulence correlation measurements consistent with theory

Pedestal width scaling



Turbulence correlation lengths

(During inter-ELM period, at pedestal top)



R = 1.38m

	Theory (non-linear XGC1 code)	Experiment
radial	3 cm	2 – 4 cm (reflecto-metery)
poloidal	11 cm	10 – 14 cm (BES)

80% - 99% ELM cycle

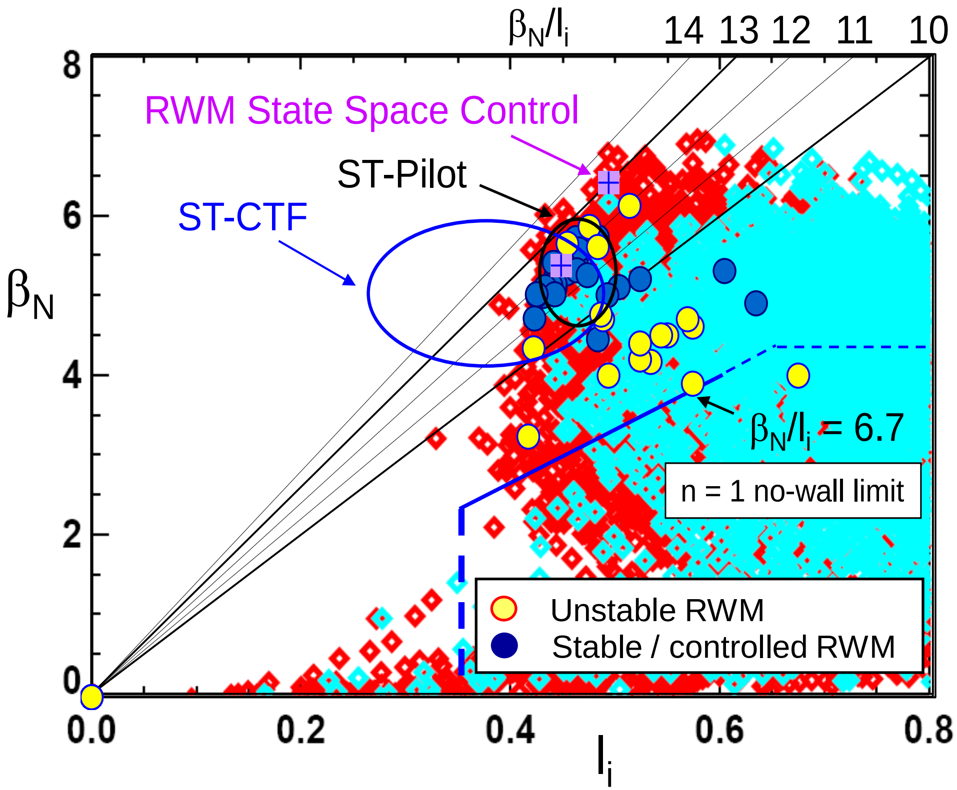
- Pedestal width scaling β_θ^α applies to multiple machines
- In NSTX, observed ped. width is larger
  - Data indicates stronger scaling: β_θ vs. $\beta_\theta^{0.5}$
  - Examining possible aspect ratio effects

- Measured correlation lengths at pedestal top are consistent with theory
  - BES and reflectometry
    - spatial structure exhibits ion-scale microturbulence ($k_\perp \rho_i \sim 0.2 - 0.7$)
    - Compatible with ITG modes and/or KBM

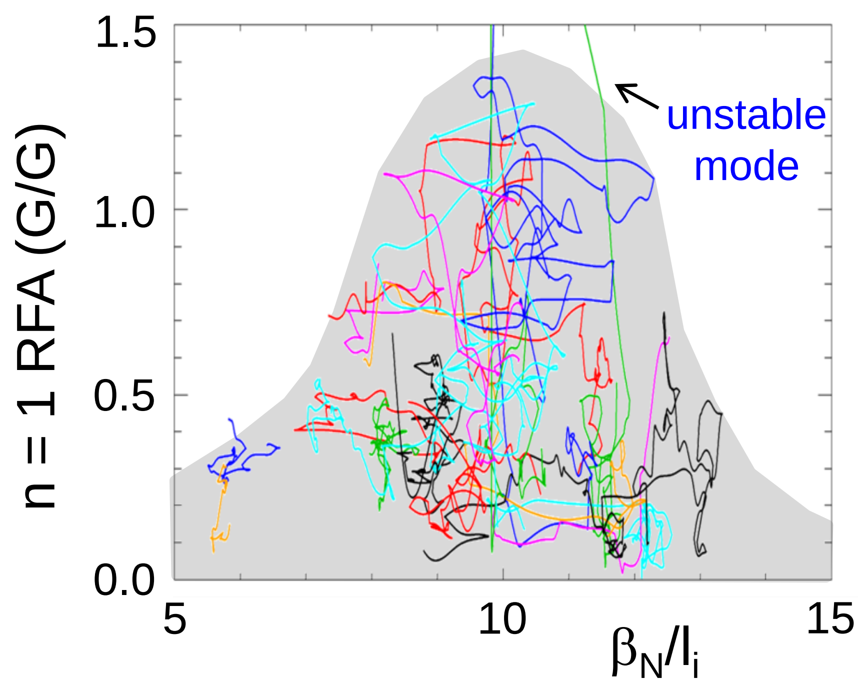
Diallo EX/P4-04

A. Diallo, C.S. Chang, S. Ku (PPPL), D. Smith (UW), S. Kubota (UCLA)

Stability control improvements significantly reduce unstable RWMs at low l_i and high β_N; improved stability at high β_N/l_i



Resonant Field Amplification (RFA) vs. β_N/l_i



- Disruption probability reduced by a factor of 3 on controlled experiments
  - Reached 2 times computed n = 1 no-wall limit of β_N/l_i = 6.7
- Lower probability of unstable RWMs at high β_N/l_i

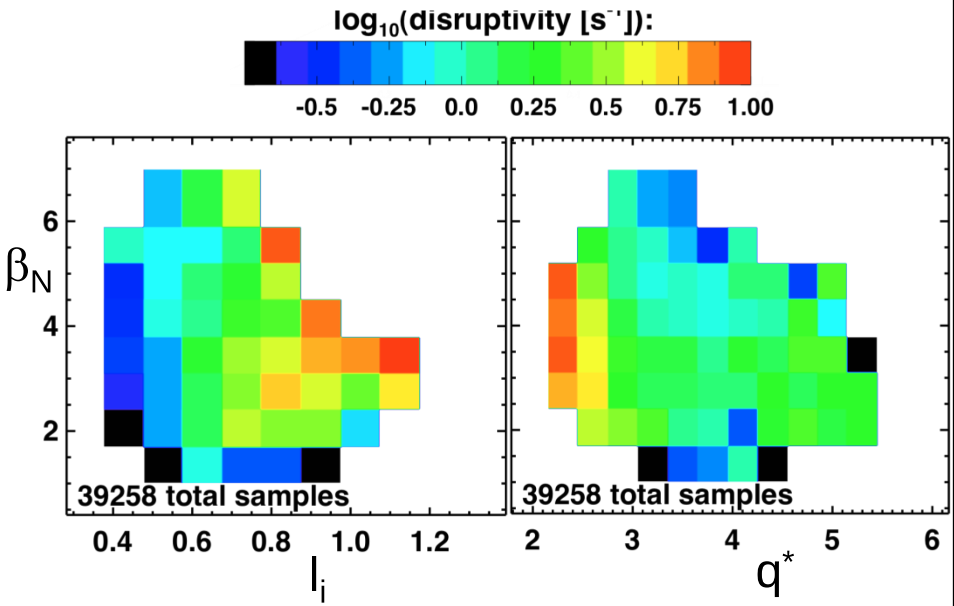
S.A. Sabbagh

- Mode stability directly measured in experiments using MHD spectroscopy
  - Stability decreases up to β_N/l_i = 10
  - Stability increases at higher β_N/l_i
  - Presently analysis indicates consistency with kinetic resonance stabilization

Berkery EX/P8-07

Disruptivity studies and warning analysis of NSTX database are being conducted for disruption avoidance in NSTX-U

Disruptivity

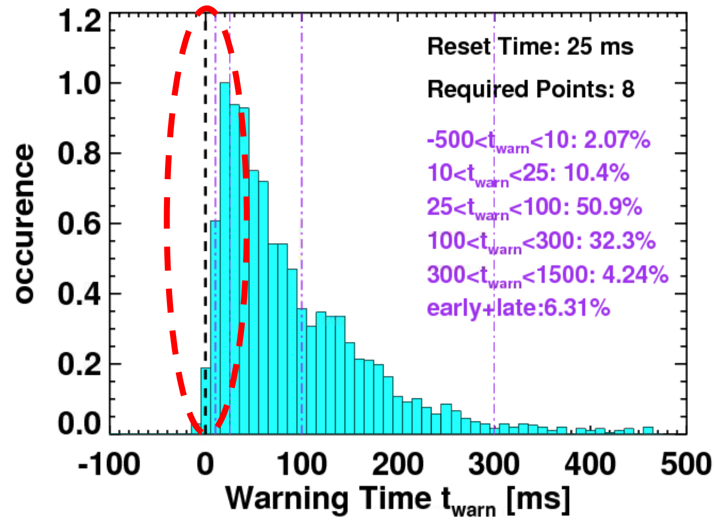


All discharges since 2006

- Physics results
  - Low disruptivity at relatively high β_N ~ 6; $\beta_N / \beta_N^{\text{no-wall(n=1)}}$ ~ 1.3-1.5
    - Consistent with specific disruption control experiments, RFA analysis
  - Strong disruptivity increase for $q^* < 2.5$
  - Strong disruptivity increase for very low rotation

Warning Algorithms

- Disruption warning algorithm shows high probability of success
  - Based on combinations of single threshold based tests

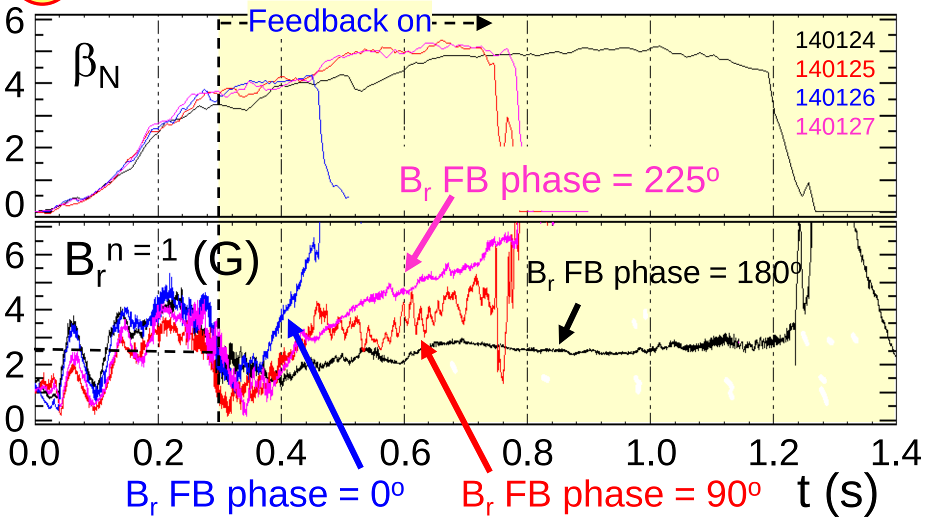


- Results
  - ~ 98% disruptions flagged with at least 10ms warning, ~ 6% false positives
  - False positive count dominated by near-disruptive events

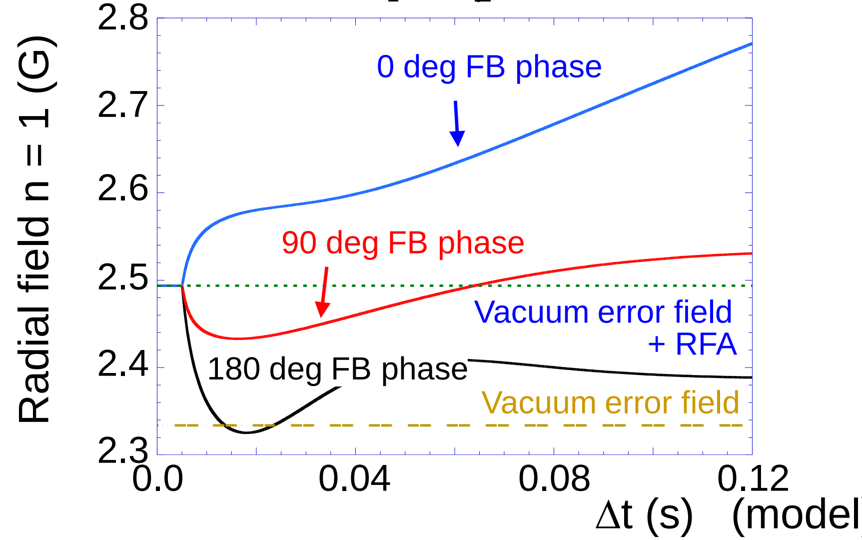
Gerhardt EX/9-3

Improved stability control includes dual field component feedback and state space feedback, improved by 3D effects

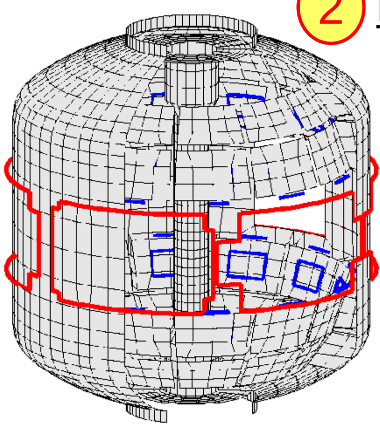
1 Active n = 1 $B_p + B_R$ feedback (FB) control



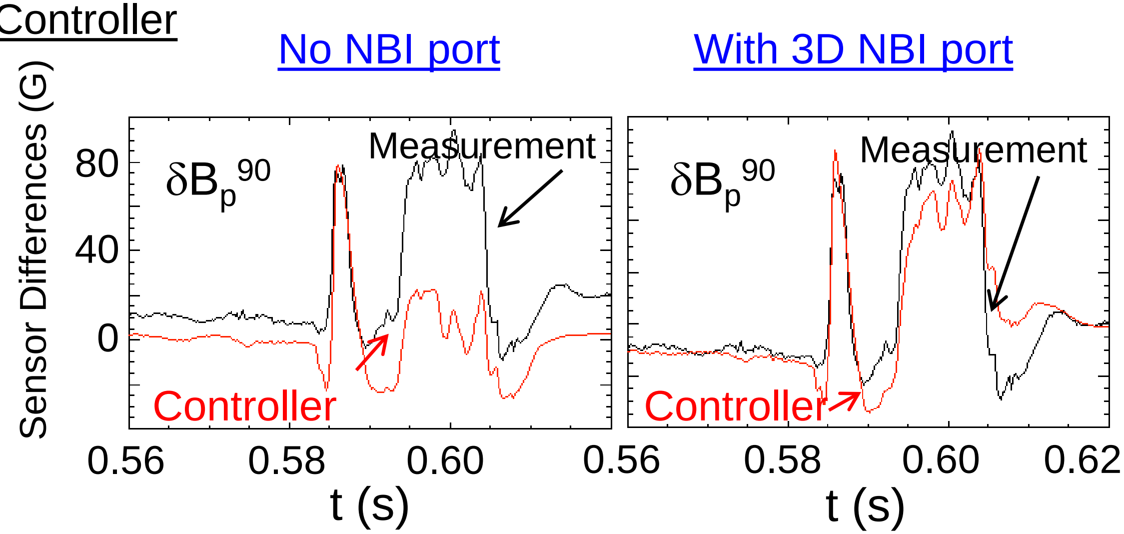
Calculation of $B_r + B_p$ control (VALEN)



2 RWM State Space Controller



3D wall, ports, mode currents

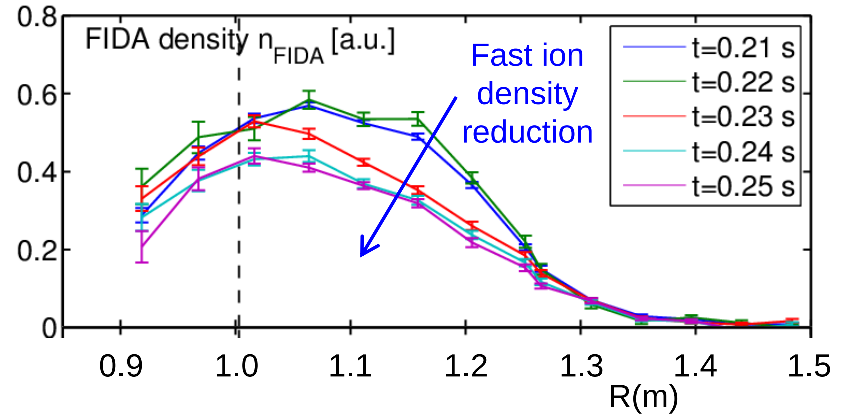


- Inclusion of 3D mode and wall detail improves control

S.A. Sabbagh, O. Katsuro-Hopkins, J.M. Bialek, S.P. Gerhardt

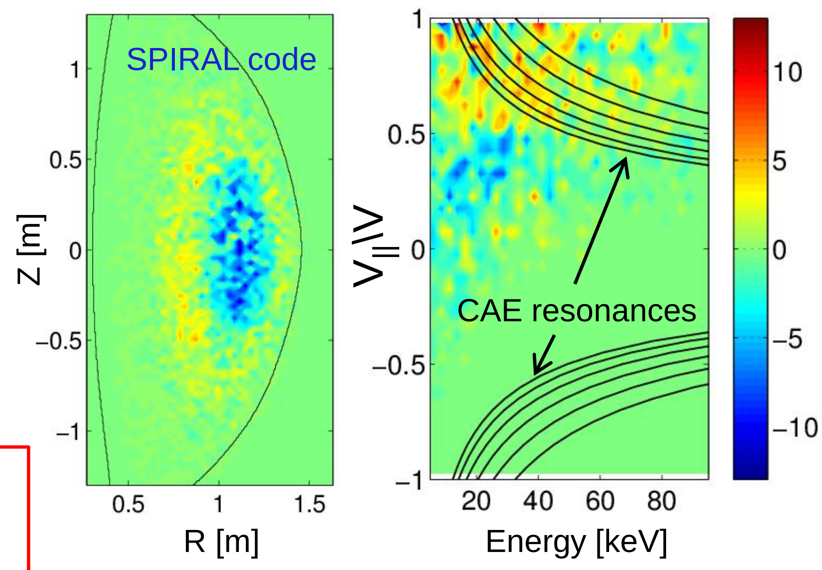
Fast ion redistribution associated with low frequency MHD measured by fast ion D_α (FIDA) diagnostic

- Caused by n = 1 global kink instabilities
- Redistribution can affect stability of *AE, RWMs, other MHD
- Full-orbit code (SPIRAL) shows redistribution in real and velocity space
  - Radial redistribution from core plasma
  - Particles shift towards $V_{||}/V = 1$
- Applied 3D fields alter GAE stability
  - By altered fast ion distribution (SPIRAL)



- Fast ion energy redistribution accounts for neutron rate decrease in H-mode TAE avalanches **Fredrickson EX/P6-05**
- Core localized CAE/GAEs measured in H-mode plasmas (reflectometer) **Crocker EX/P6-02**

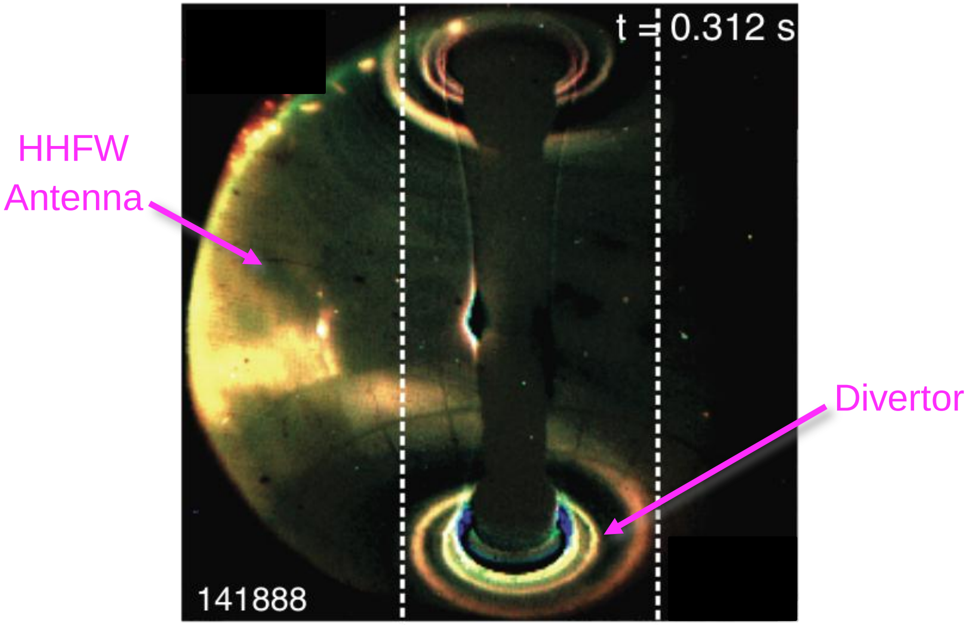
Change in distribution due to kink mode



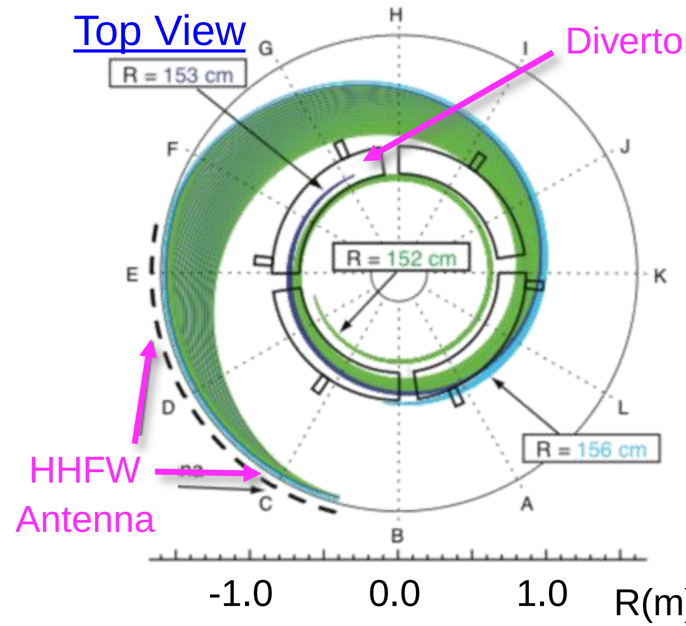
A. Bortolon

Significant fraction of the HHFW power lost in the SOL in front of antenna, flows to the divertor region

Visible camera image of edge RF power flow to divertor



SPIRAL modeling of field lines from antenna to divertor



- RF power couples to field lines across entire SOL width, not just to field lines connected to antenna components
- Shows importance of quantitatively understanding RF power coupling to the SOL for prediction to future devices

R. Perkins, et al., PRL **109** (2012) 045001

Perkins EX/P5-40

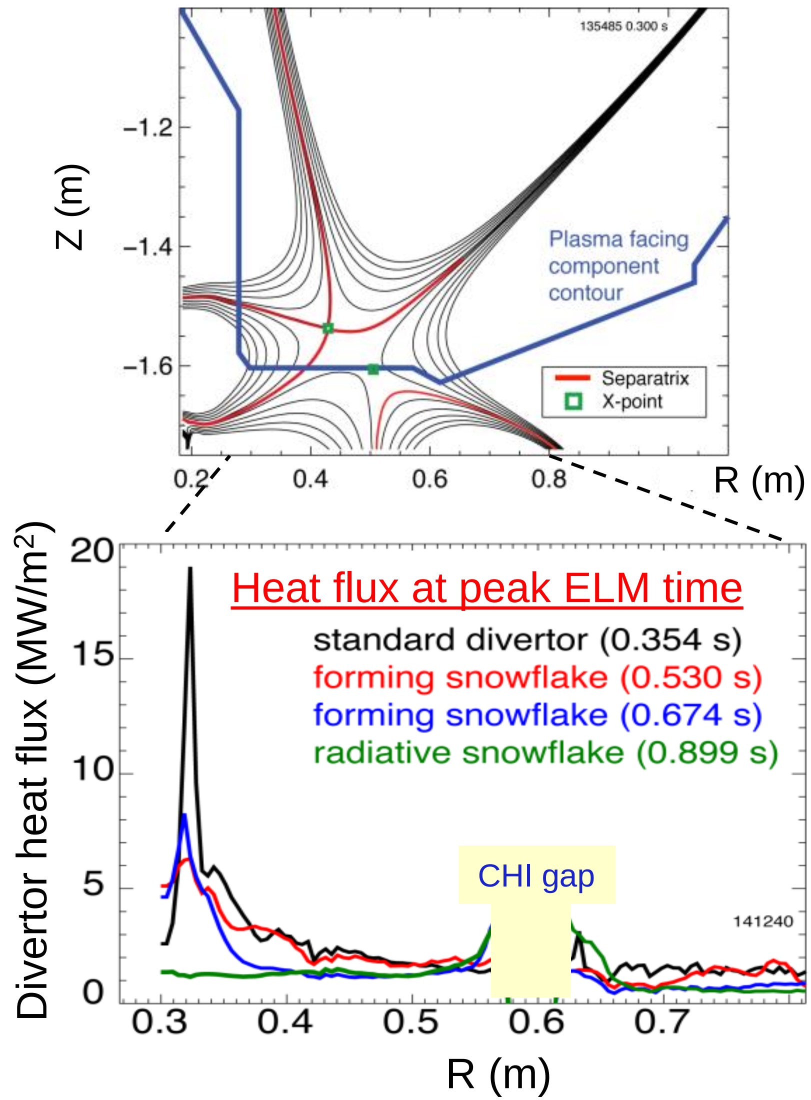
Snowflake divertor experiments provide basis for required divertor heat flux mitigation in NSTX-U

- Needed, as divertor heat flux width strongly decreases as I_p increases

- Snowflake divertor experiments (P_{NBI} = 4 MW, P_{SOL} = 3 MW)
  - Good H-mode τ_E, β_N, sustained during snowflake operation
  - Divertor heat flux significantly reduced both during and between ELMs
    - during ELMs: 19 to ~ 1.5 MW/m^2
    - steady-state: 5-7 to ~ 1 MW/m^2
  - Achieved by a synergistic combination of detachment + radiative snowflake divertor

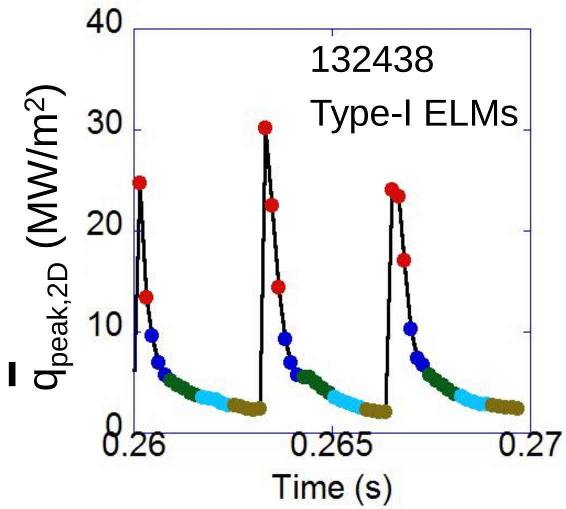
Soukhanovskii EX/P5-21

Snowflake divertor in NSTX

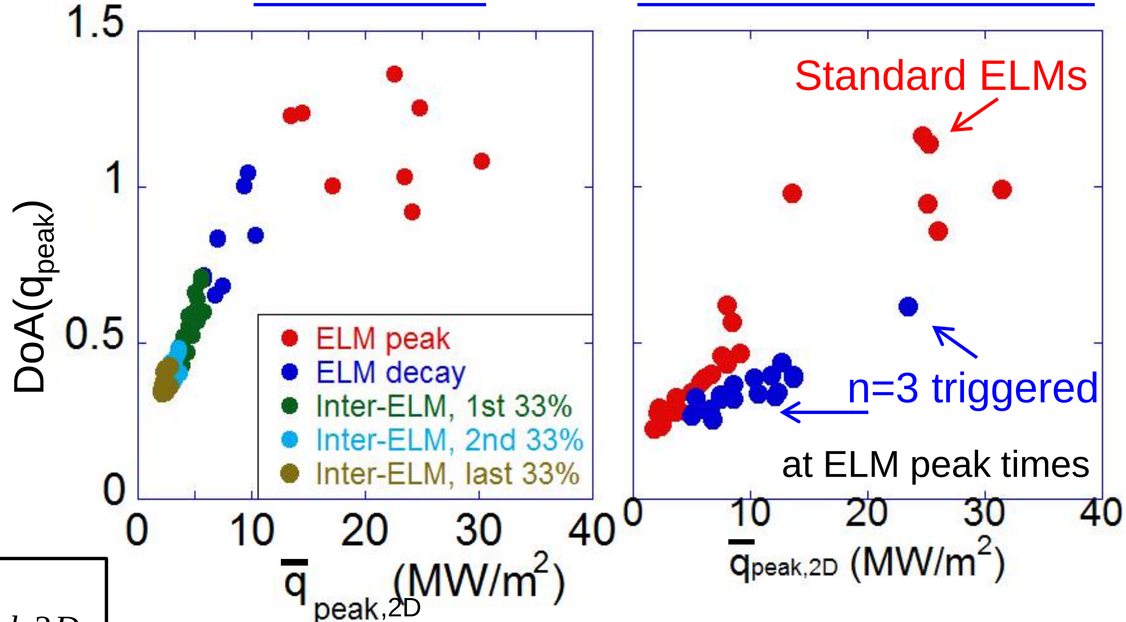


Toroidal asymmetry of heat deposition measured during standard ELMs, but decreases for 3D field-triggered ELMs

Mean Peak 2D Heat Flux During ELMs vs. t



Toroidal Degree of Asymmetry vs. q_{peak}



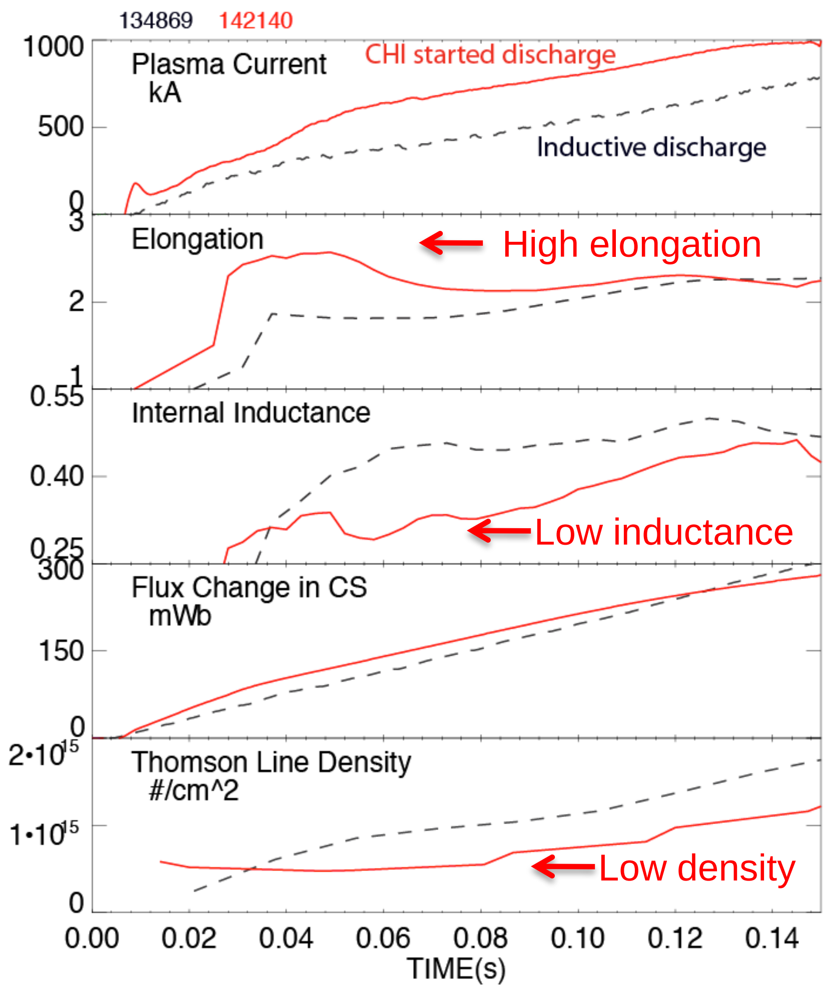
$$DoA(q_{peak}) = \sigma_{qpeak} / \bar{q}_{peak,2D}$$

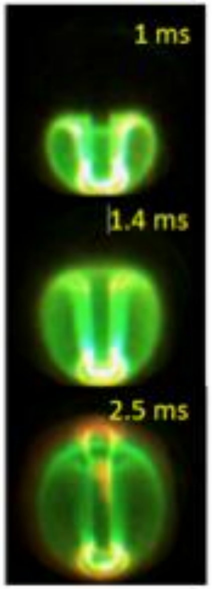
- 2D fast IR camera measurement (6.3kHz), heat flux from TACO code
- Toroidal asymmetry
  - Becomes largest at the peak heat flux for usual Type-I ELMs
  - Reduced by up to 50% in ELMs triggered by n = 3 applied fields

Ahn EX/P5-33

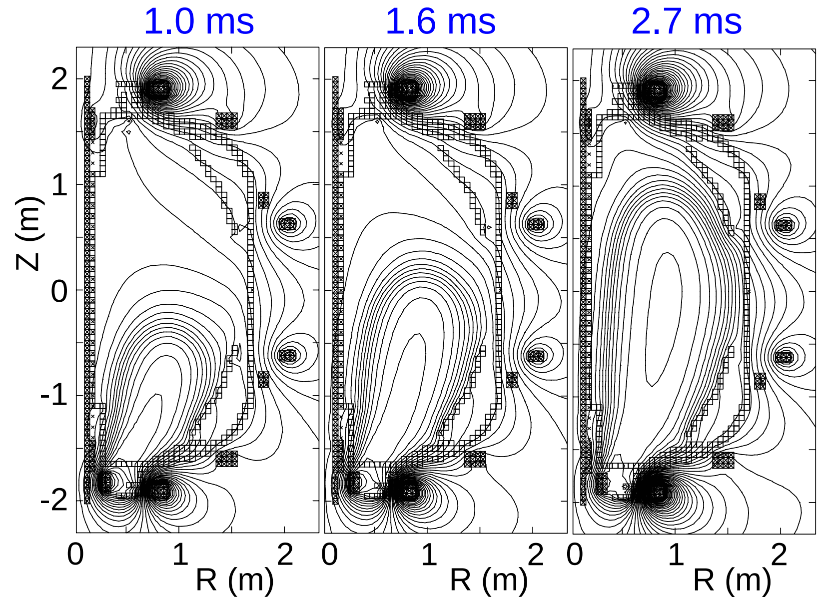
L-mode discharge ramping to 1MA requires 35% less inductive flux when coaxial helicity injection (CHI) is used

CHI assisted startup in NSTX





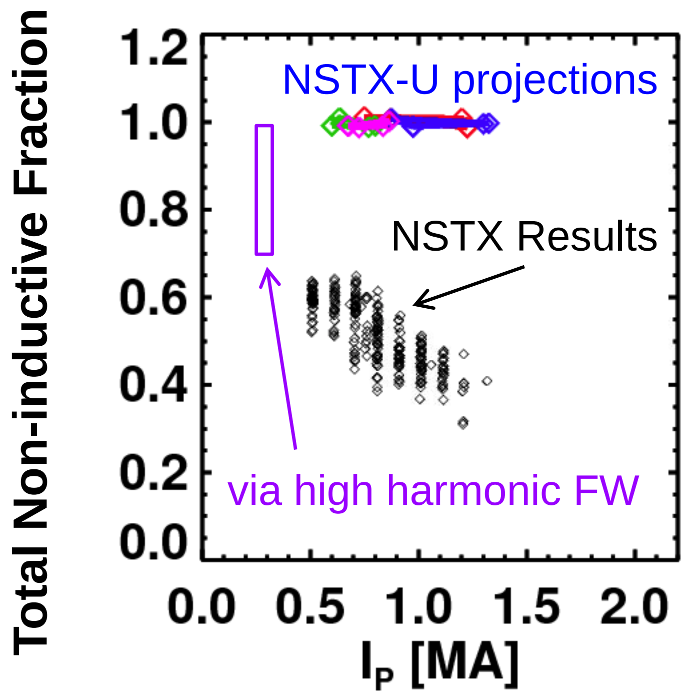
TSC simulation of CHI startup

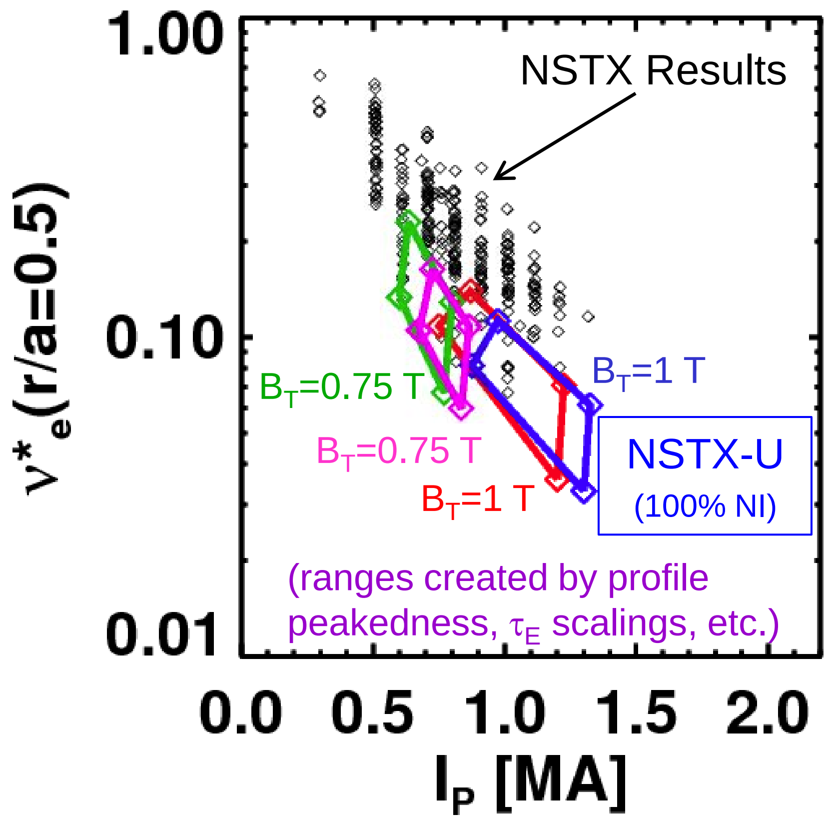


- CHI generates plasmas with high elongation, low l_i and n_e
- TSC now used for full discharge modeling to 1MA
  - CHI start-up + NBI current ramp-up
- Results imply a doubling of closed flux current > 400kA in NSTX-U

Raman EX/P2-10

Non-inductive current fractions of up to 65% sustained in NSTX, >70% transiently; Upgrade projected to achieve 100%





- Maximum sustained non-inductive fractions of 65% w/NBI at I_P = 0.7 MA
- 70- 100% non-inductive reached transiently using HHFW CD

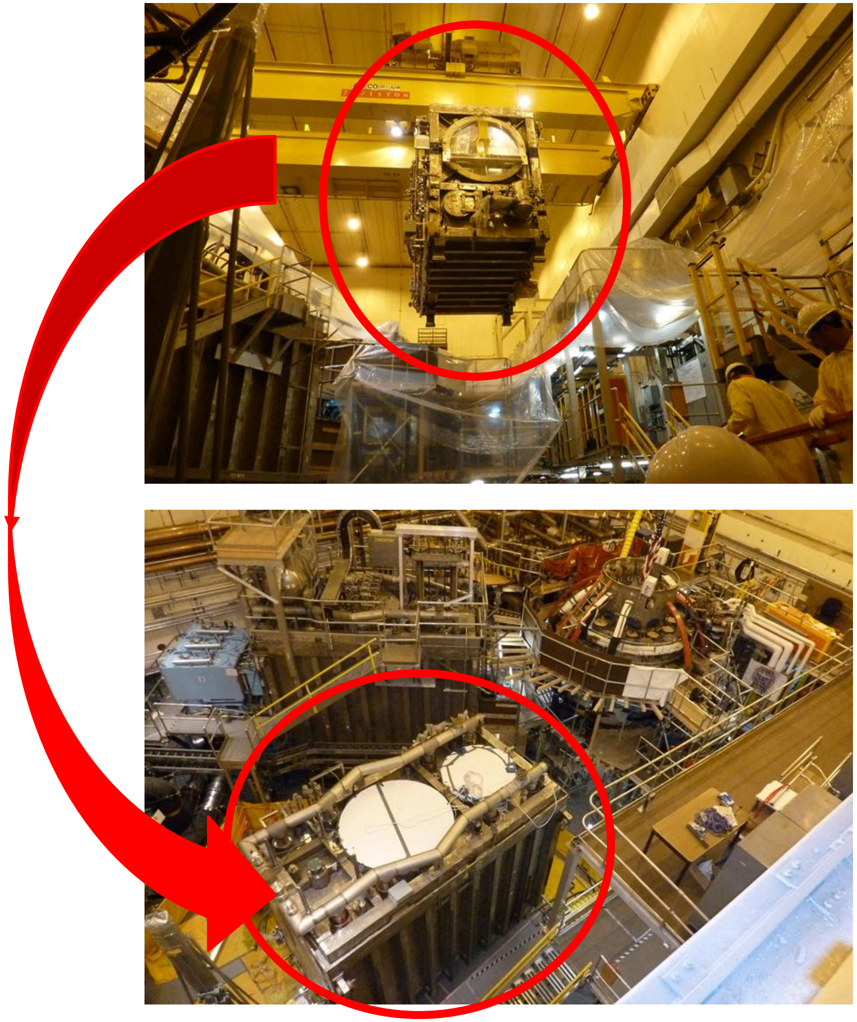
G. Taylor (Phys. Plasmas **19** (2012) 042501)

- 100% non-inductive scenarios found over wide operation range
  - Scenarios at 74% Greenwald density

S. Gerhardt, et al., Nucl. Fusion **52** (2012) 083020

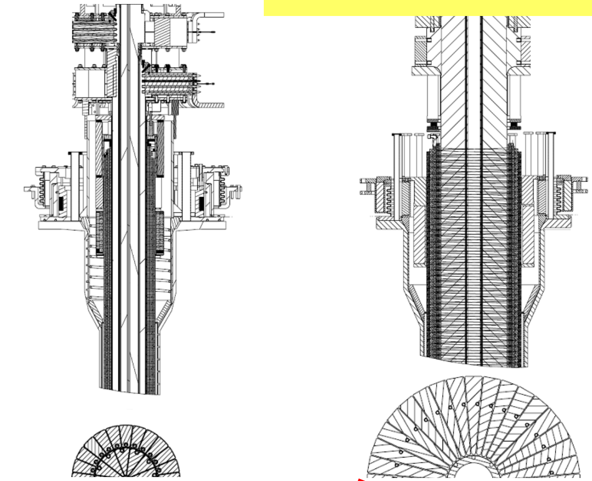
Menard FTP/3-4

Rapid Progress is Being Made on NSTX Upgrade



Old center stack

NEW Center Stack



TF OD = 20cm

TF OD = 40cm

- 2nd neutral beam moved into place
- TF conductors being made

(first plasma anticipated June 2014)

Menard FTP/3-4

Continuing analysis of NSTX data targets a predictive physics understanding required for future fusion devices

- Transport and stability at reduced collisionality
  - τ_E scalings unified by collisionality; non-linear microtearing simulations match experimental χ_e, predict lower χ_e at lower ν_e* shown in experiment
  - Nearly continuous increase of favorable confinement with increased lithium
  - Stabilizing kinetic RWM effects enhanced at lower ν when near resonances
- Pedestal
  - Width scaling stronger than usual $(\beta_p^{ped})^{0.5}$; measured δn_e correlation lengths consistent w/non-linear gyrokinetics at pedestal top
- Pulse sustainment / disruption avoidance
  - Global stability increased + low disruptivity at high β_N/l_i, advanced mode control
  - Disruption detection algorithm shows high (98%) success rate
- Power/particle handling and first wall
  - Large heat flux reduction from synergistic combination of radiative snowflake divertor + detachment, both during, and between ELMs
- Significant upgrade underway (NSTX-U)
  - Doubled B_T, I_p, NBI power; 5x pulse length, projected 100% non-inductive sustainment over broad operating range

NSTX Presentations at the 2012 IAEA FEC

Talks

Thursday

• Progress in Simulating Turbulent Electron Thermal Transport in NSTX	**Guttenfelder**	**TH/6-1**
• The Dependence of H-mode Energy Confinement and Transport on Collisionality in NSTX	**Kaye**	**EX/7-1**

Friday

• Disruptions in the High Beta Spherical Torus NSTX	**Gerhardt**	**EX/9-3**
• Progress on Developing the Spherical Tokamak for Fusion Applications	**Menard**	**FTP/3-4**

Saturday

• The Nearly Continuous Improvement of Discharge Characteristics and Edge Stability with Increasing Lithium Coatings in NSTX	**Maingi**	**EX/11-2**

Posters

Tuesday

Lithium program	**Ono**	**FTP/P1-14**
Co-axial helicity injection	**Raman**	**EX/P2-10**
Particle code NTV simulation	**Kim**	**TH/P2-27**

Wednesday

Bootstrap current XGC	**Chang**	**TH/P4-12**
Pedestal transport	**Diallo**	**EX/P4-04**
Power scrape-off width	**Goldston**	**TH/P4-19**
Vertical stability at low A	**Kolemen**	**EX/P4-28**
Blob dynamics / edge V shear	**Myra**	**TH/P4-23**
EHOs	**Park**	**EX/P4-33**
Core lithium levels	**Podesta**	**EX/P3-02**
C, Li impurity transport	**Scotti**	**EX/P3-34**
Snowflake divertor theory	**Ryutov**	**TH/P4-18**

Thursday

Divertor heat asymmetry	**Ahn**	**EX/P5-33**
L-H power threshold vs. X pt.	**Battaglia**	**EX/P5-28**
NBI-driven GAE simulations	**Belova**	**TH/P6-16**
CAE/GAE structure	**Crocker**	**EX/P6-02**
TAE avalanches in H-mode	**Fredrickson**	**EX/P6-05**
Li deposition / power exhaust	**Gray**	**EX/P5-27**
Liquid lithium divertor results	**Jaworski**	**EX/P5-31**
RF power flow in SOL	**Perkins**	**EX/P5-40**
Snowflake divertor	**Soukhanovksii**	**EX/P5-21**

Friday

Global mode control / physics	**Berkery**	**EX/P8-07**
Edge transport with Li PFCs	**Canik**	**EX/P7-16**
Turbulence near OH L-H trans.	**Kubota**	**EX/P7-21**
ELM triggering by Li in EAST	**Mansfield**	**PD**
Electron-scale turbulence	**Ren**	**EX/P7-02**
Low-k turbulence vs. params.	**Smith**	**EX/P7-18**

Tasks to Complete for this Presentation

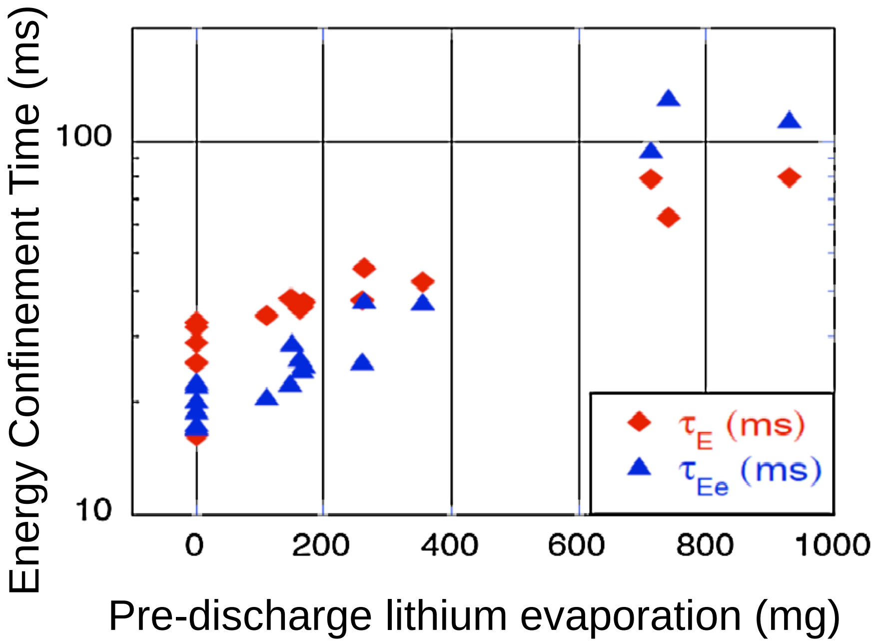
- Additions
  - Add bullet on L-H power threshold – REF Battaglia talk EX/P5-28

- Tasks
  - Shorten talk to 18 slides
  - Poster up to 24 slides

Extra slides for poster follow

Supporting slides follow

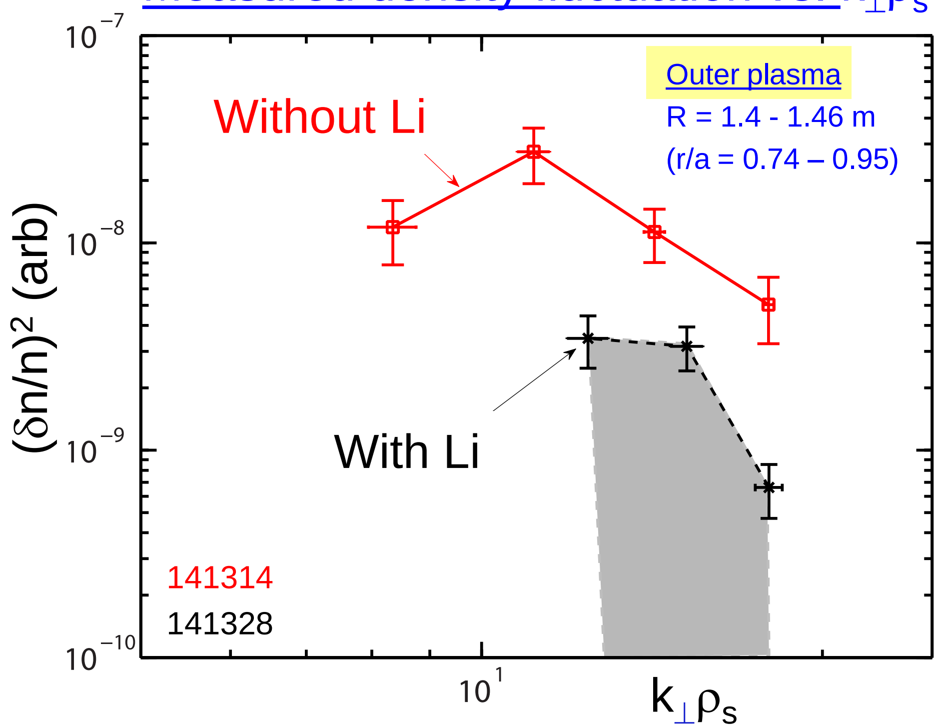
Plasma characteristics change nearly continuously with increasing lithium evaporation inside vessel



- Global parameters generally improve
- ELM frequency declines - to zero
  - ELMs stabilize
- Edge transport declines
  - As lithium evaporation increases, transport barrier widens, pedestal-top χ_e reduced

Maingi EX/11-2 **Canik EX/11-2**

Measured density fluctuation vs. $k_\perp \rho_s$



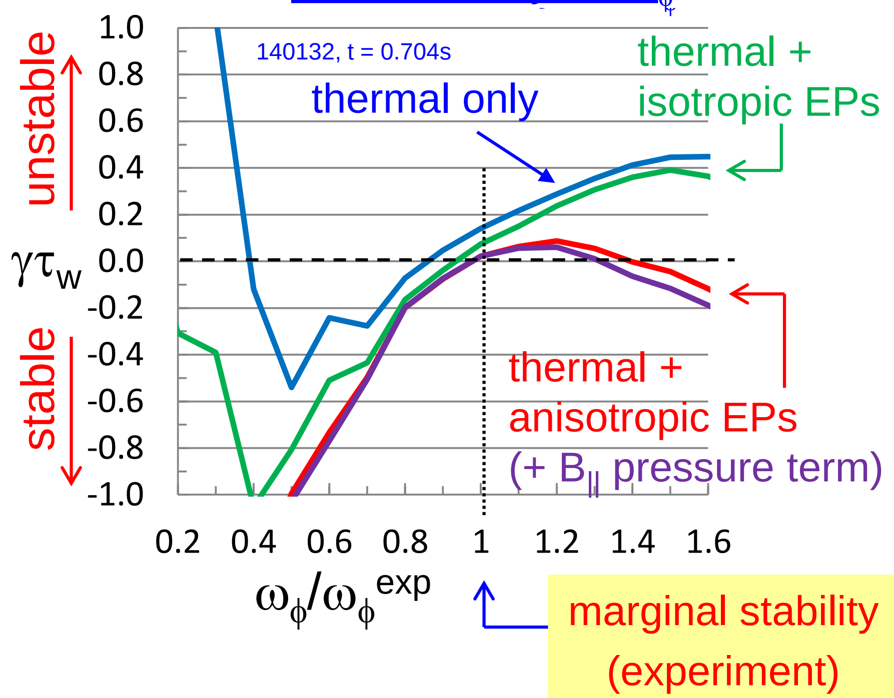
- Measured reduction in high-k turbulence consistent with reduced χ_e
- Impact of collisionality and ∇n on turbulence is under investigation
  - $B_t \tau_E \sim \nu_e^{*-0.8}$ observed

Ren EX/P7-02

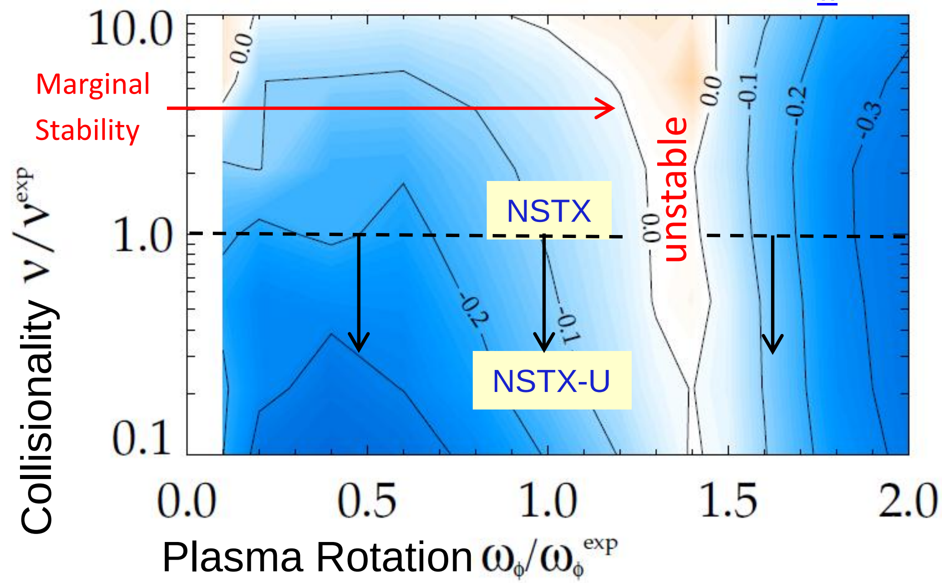
Kinetic RWM stability theory further tested against NSTX experiments, provides guidance for NSTX-U

MISK code

RWM stability vs. ω_ϕ



RWM growth rate contours ($\gamma\tau_w$)



- Improvements to physics model
  - Anisotropy effects
  - Testing terms thought small
    - Already good agreement between theory and experiment of marginal stability point improved

J. Berkery et al., PRL **106**, 075004 (2011)

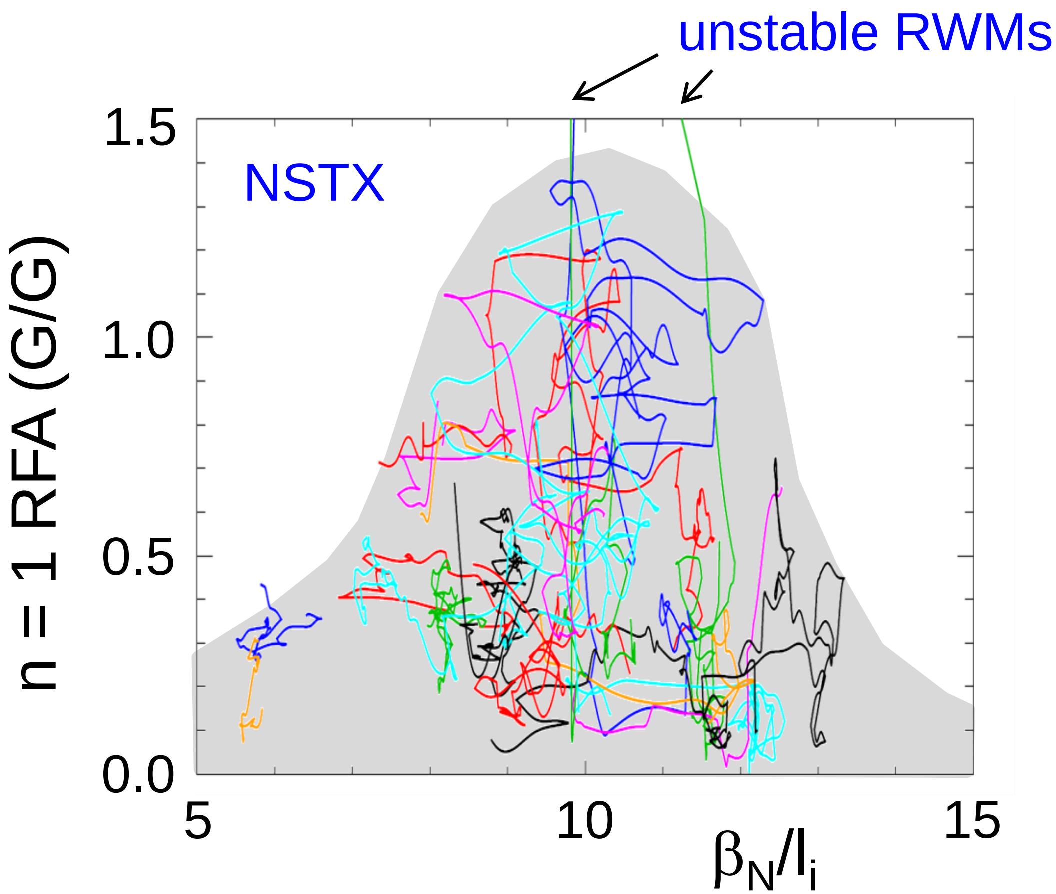
- Two competing effects at lower ν
  - Collisional dissipation reduced
  - Stabilizing resonant kinetic effects enhanced (contrasts early theory)
- Expectations at lower ν
  - More stabilization near ω_φ resonances; almost no effect off-resonance
    - Active RWM control important

Berkery EX/P8-07

Experiments using MHD spectroscopy show that highest β_N/l_i plasmas are not the least stable

$$RFA = \frac{B_{plasma}}{B_{applied}}$$

- Low frequency (40Hz) rotating n = 1 applied field used as seed field
- n = 1 resonant field amplification (RFA) of seed field used to measure global mode growth rate in stable plasmas
  - Higher amplitude = less stability

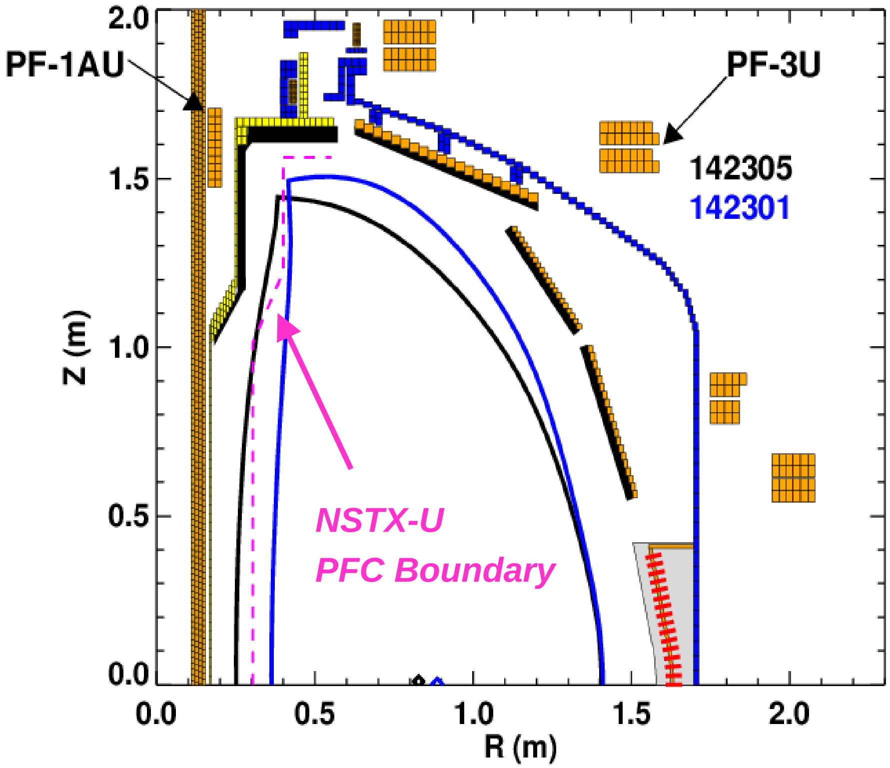


J.W. Berkery, S.A. Sabbagh

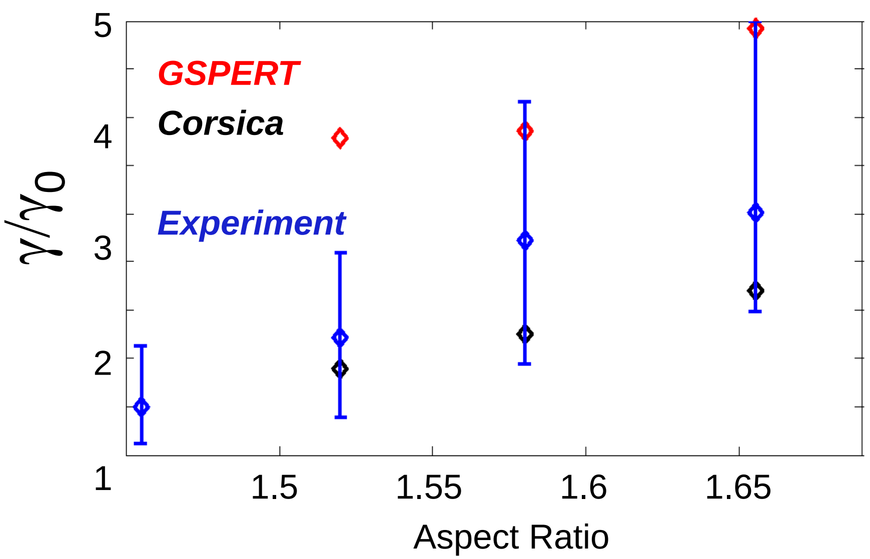
- Discharges with $\beta_N/l_i > 10$ have *greater* stability
  - Presently thought to be due to differing plasma rotation profile

Berkery EX/P8-07

Higher aspect ratio of NSTX-U tested in NSTX, vertical stability growth rate data obtained, compared to simulation



Vertical Stability Growth Rates vs. A



- NSTX Discharges have matched aspect ratio and elongation of NSTX-U without performance degradation

- Improvements to vertical control capability and understanding
  - Begun to compare measured growth rates to theoretical predictions (Corsica, GSPERT)
  - Improved plasma position observer
  - Modeled use of RWM coils for n=0 control

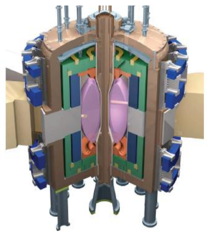
Kolemen EX/P4-28

Single summary slides follows

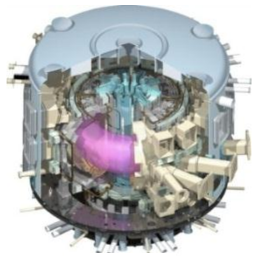
- NOTE: The single summary slide is not an adequate summary

OV/3-1: NSTX research targets needed predictive physics understanding crucial for fusion energy development

- Enable devices: ST-FNSF, ST-Pilot/DEMO, ITER
  - Leveraging unique ST plasmas provides new understanding for tokamaks, challenges theory



Fusion Nuclear Science Facility (FNSF)

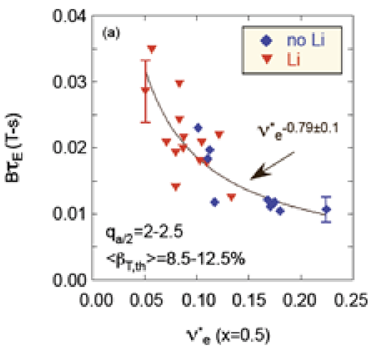


ITER

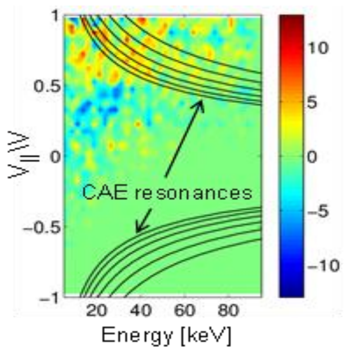
Highlights

- Transport, stability at reduced collisionality
  - τ_E scalings unified by collisionality; microtearing code matches XP χ_e, predicts lower χ_e at lower ν_e^*
  - Stabilizing kinetic RWM effects enhanced
- Pedestal
  - Width scaling stronger than usual $(\beta_p^{ped})^{0.5}$; measured δn_e correl. lengths agree w/non-linear gyrokinetics
- Pulse sustainment / disruption avoidance
  - Global stability increased + low disruptivity at high β_N
- Power/particle handling and first wall
  - Radiative snowflake divertor mitigates high heat flux both between & during ELMs, Li wall cond. effects
- Significant upgrade underway (NSTX-U)
  - Doubled B_T, I_p, NBI power, non-inductive sustainment

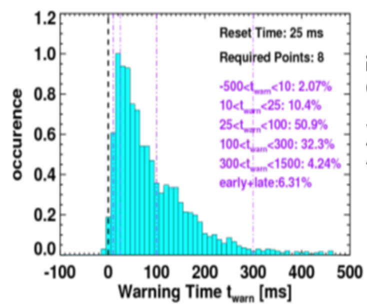
τ_e vs. collisionality



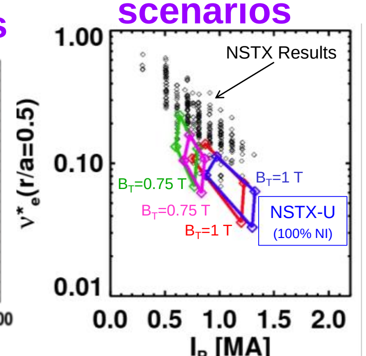
kink-induced fast ion redistribution



disruption warning analysis



non-inductive scenarios



Breakdown of Topics in Talk

Should have 18 slides total (17 plus 1 reference to other talks)

Advanced Scenarios and Control	2 slides
Boundary Physics / Pedestal	1 slides
Lithium Research	2 slides
Macroscopic Stability	4 slides
Solenoid-free Start-up / Ramp-up	1 slides
Transport and Turbulence	3 slides
Waves and Energetic Particles	1 slides
Title, intro, summary, reference	4 slides
Total	18 slides

NSTX Overview presentation and papers – preparation IAEA FEC 2012 (discussion on 8/13/12)

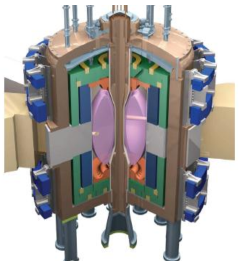
- Presentation length
  - Past talks: average of 13.5 slides in 17 minutes (0.8/min), suggests:
  - Overview: 17 slides in 21 minutes (+ 4 minutes for questions)
- Presentation status / needs (thanks for material sent so far!)
  - Working from NSTX PAC talk (approximately same length)
  - Will update with most recent analysis from NSTX Team
- Papers
  - 12 page proceedings paper
  - Longer Nuclear Fusion paper
- Paper Preparation
  - Most contributors sent slides, not text – please send text/references !
  - Some text available from EPS 2012 presentations, but not all topics are covered
  - Will send further requests for input / seek out new results

V1.0

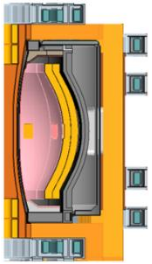
Alternate slides follow

NSTX research targets predictive physics understanding needed for fusion energy development facilities

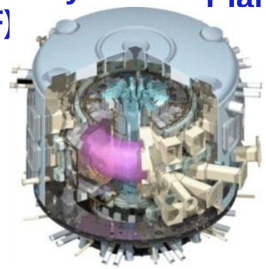
- Enable key ST applications
  - Move toward steady-state ST FNSF, pilot plant
  - Close key gaps to DEMO
- Extend understanding to tokamak / ITER
  - Leverage ST to test theory, develop predictive capability



Fusion Nuclear Science Facility (FNSF)



ST Pilot Plant



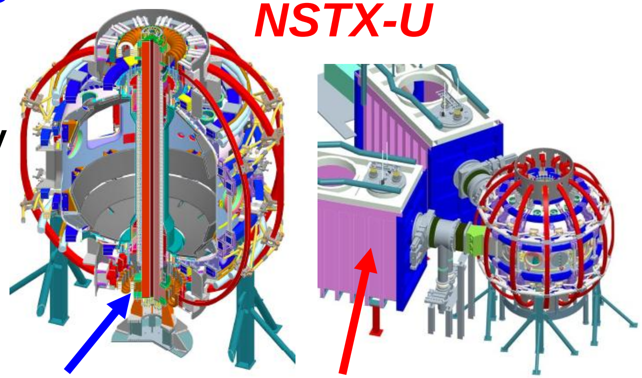
ITER

Outline

- Develop key physics understanding to be tested in unexplored, hotter ST plasmas
  - Study high beta plasma transport and stability at reduced collisionality, extended pulse
  - Prototype methods to mitigate very high heat/particle flux
  - Move toward fully non-inductive operation

3D effects are pervasive in this research

NSTX-U

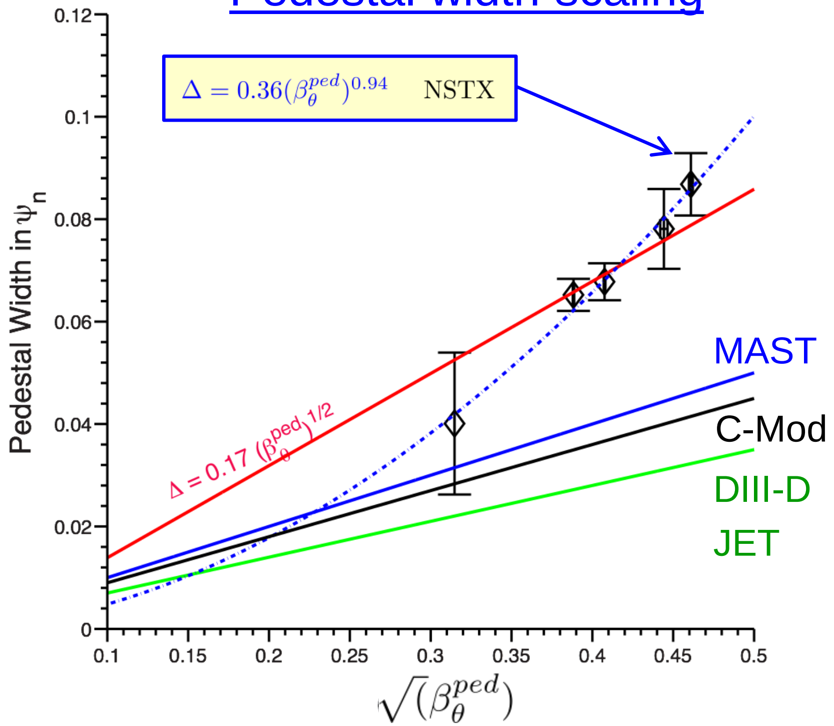


New center-stack   2nd Neutral Beam

$B_T \longrightarrow$ 1 T	$P_{NBI} \longrightarrow$ 12 MW
$I_p \longrightarrow$ 2 MA	pulse $\longrightarrow$ 5 s

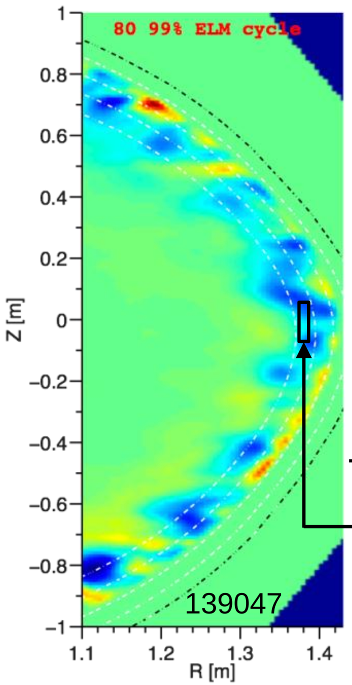
Pedestal scaling, structure, and dynamics studied theoretically and experimentally

Pedestal width scaling



Turbulence correlation lengths

(During inter-ELM period, at pedestal top)



	Theory (non-linear XGC1 code)	Experiment
radial	3 cm	2 – 4 cm (reflecto-metery)
poloidal	11 cm	10 – 14 cm (BES)

80% - 99% ELM cycle

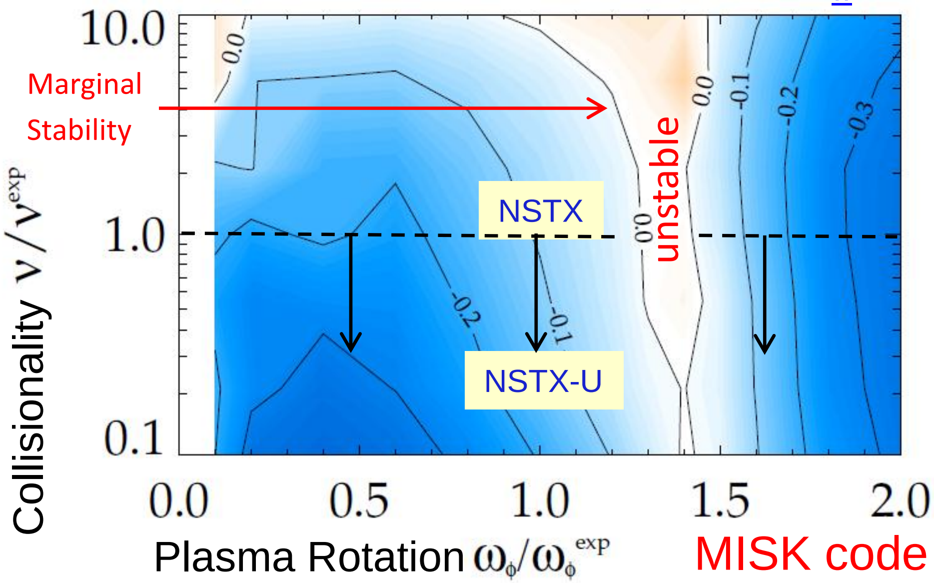
- Pedestal width scaling β_θ^α applies to multiple machines
- In NSTX, observed ped. width is larger
  - 1.7 x MAST, 2.4 x DIII-D
  - Data indicates stronger for NSTX: $\beta_\theta^{0.94}$ vs. $\beta_\theta^{0.5}$
- Measured correlation lengths at pedestal top are consistent with theory
  - BES and reflectometry
    - spatial structure exhibits ion-scale microturbulence ($k_\perp\rho_i \sim 0.2 - 0.7$)
    - Compatible with ITG and/or KBM

Diallo EX/P4-04

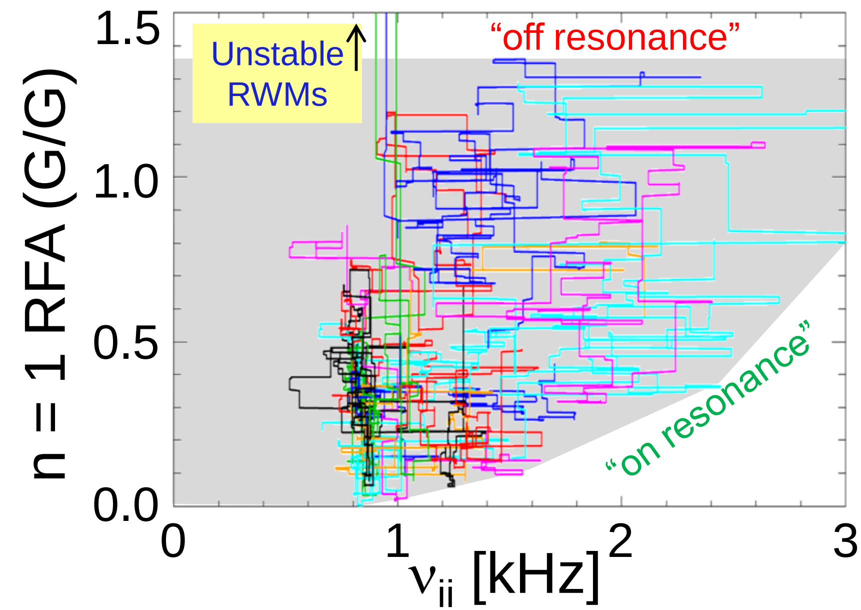
A. Diallo, C.S. Chang, S. Ku (PPPL), D. Smith (UW), S. Kubota (UCLA)

Experiments measuring global stability vs. ν further support kinetic RWM stability theory, provide guidance for NSTX-U

Theory: RWM growth rate contours ($\gamma\tau_w$)



Exp: Resonant Field Amplification (RFA) vs ν



(trajectories of 20 experimental plasmas)

- Two competing effects at lower ν
  - Collisional dissipation reduced
  - Stabilizing resonant kinetic effects enhanced (contrasts early theory)
- Expectations at lower ν
  - More stabilization near ω_φ resonances; almost no effect off-resonance

J. Berkery et al., PRL **106**, 075004 (2011)

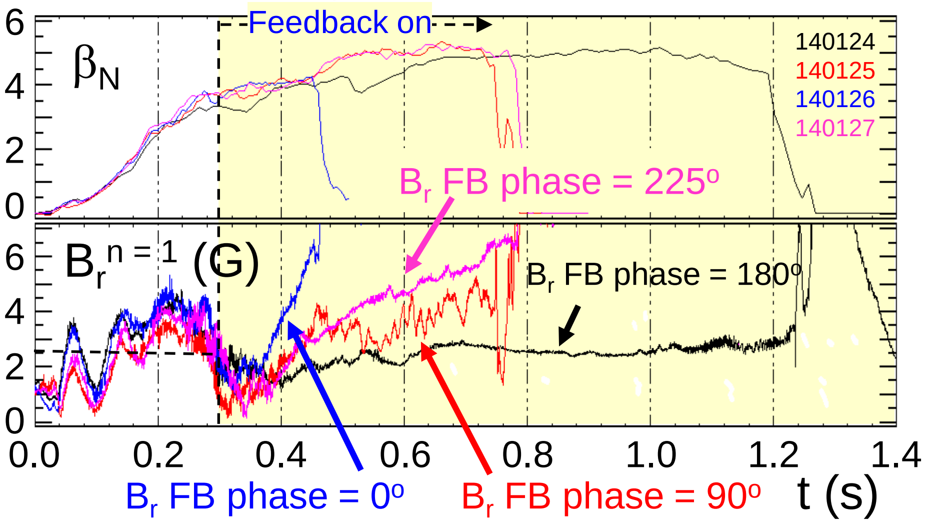
- Mode stability directly measured in experiment using MHD spectroscopy
  - Decreases with ν at lower RFA ("on resonance")
  - Independent of ν at higher RFA ("off resonance")

$$\text{RFA} = \frac{B_{plasma}}{B_{applied}}$$

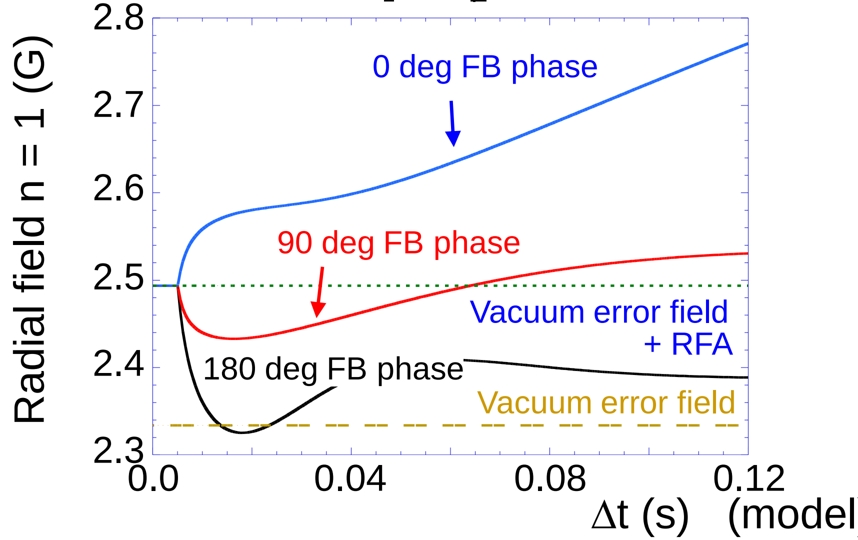
Berkery EX/P8-07

Improved stability control includes dual field component feedback and state space feedback with 3D structure

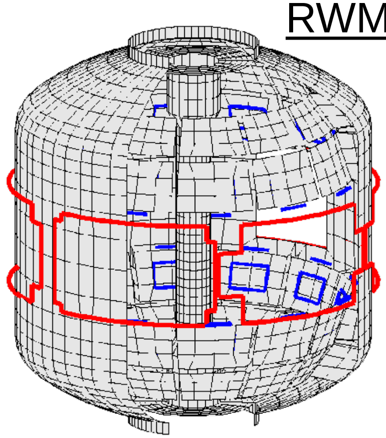
Active n = 1 B_p + B_R feedback (FB) control



Calculation of B_r + B_p control (VALEN)



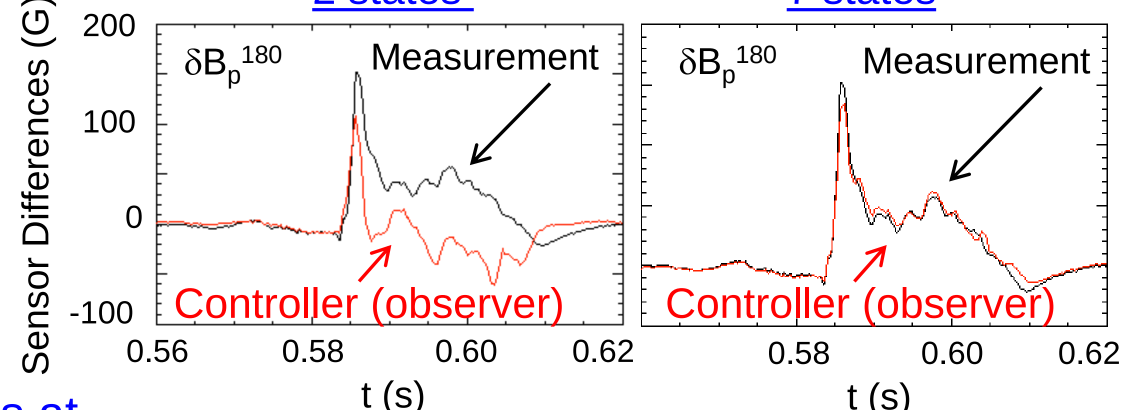
RWM State Space Controller



3D wall, ports, mode currents

2 states

7 states

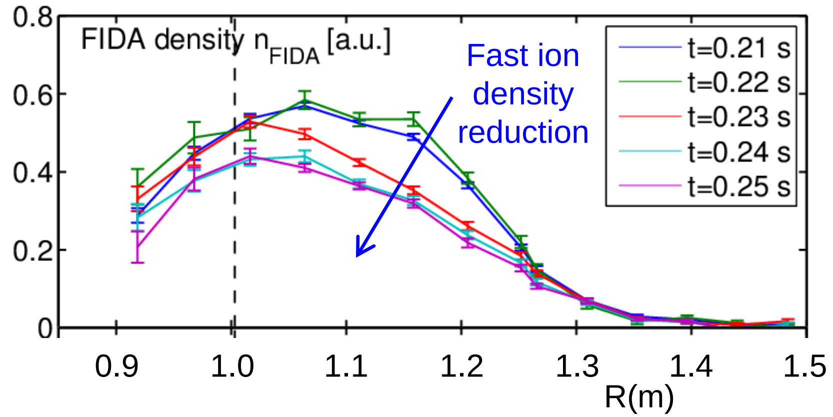


- Significantly reduced disruptions at high β_N in controlled experiments

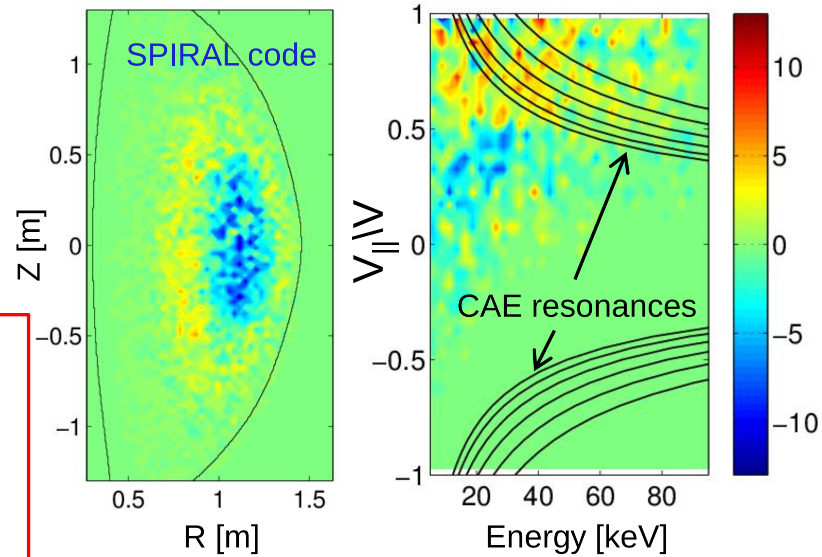
S.A. Sabbagh, O. Katsuro-Hopkins, J.M. Bialek

Fast ion redistribution associated with low frequency MHD measured by fast ion D_α (FIDA) diagnostic

- Caused by n = 1 global instabilities
  - Primarily n = 1, weaker n = 2 present
- Redistribution can affect stability of *AE, RWMs, other MHD
  - CAE activity observed *after* onset of low frequency MHD
- Full-orbit code (SPIRAL) shows redistribution in real and velocity space
  - Radial redistribution from core plasma
  - Particles shift towards $V_{||}/V = 1$



Change in distribution due to kink mode



- Measured CAE and GAEs (reflectometer) in H-mode core plasmas
  - mode #, frequency measured: modes peak in core, resonant with electron orbit frequencies

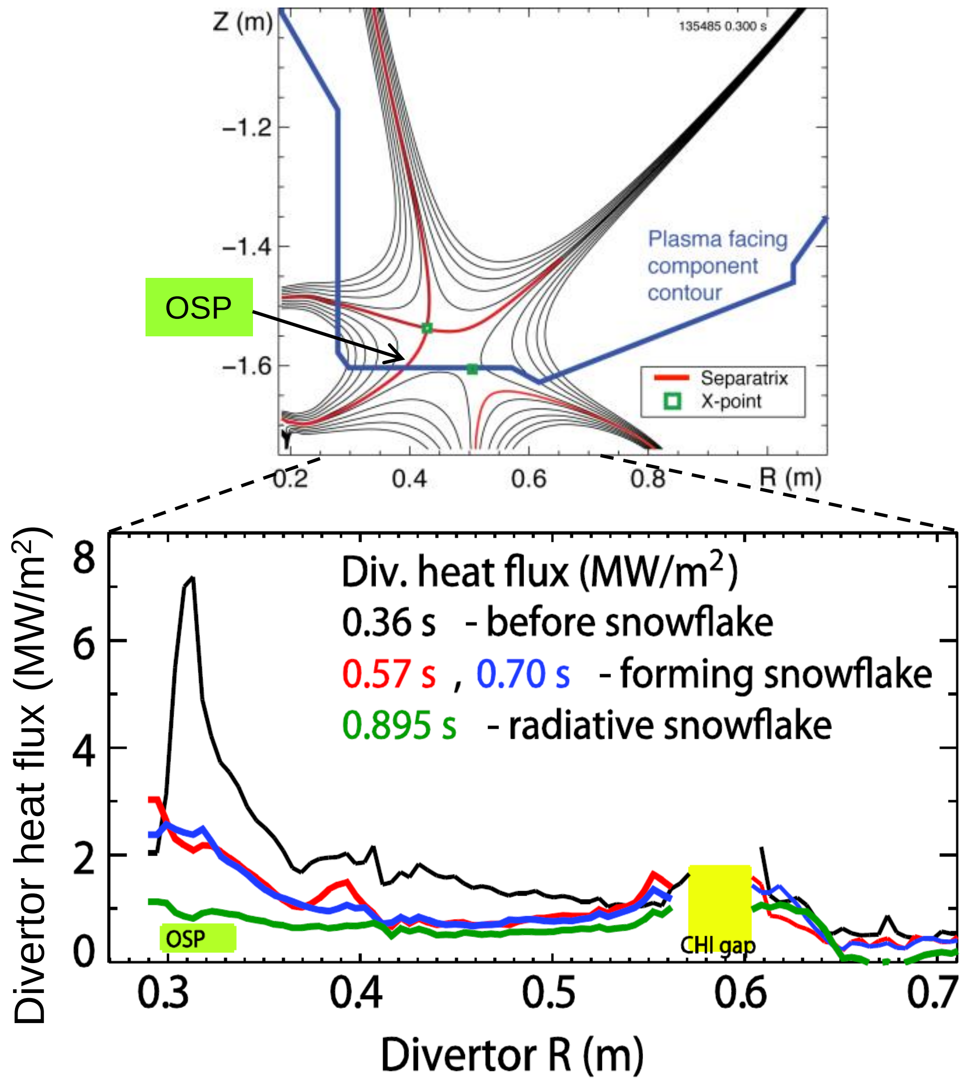
Crocker EX/P6-02

A. Bortolon

Snowflake divertor experiments provide basis for required divertor heat flux mitigation in NSTX-U

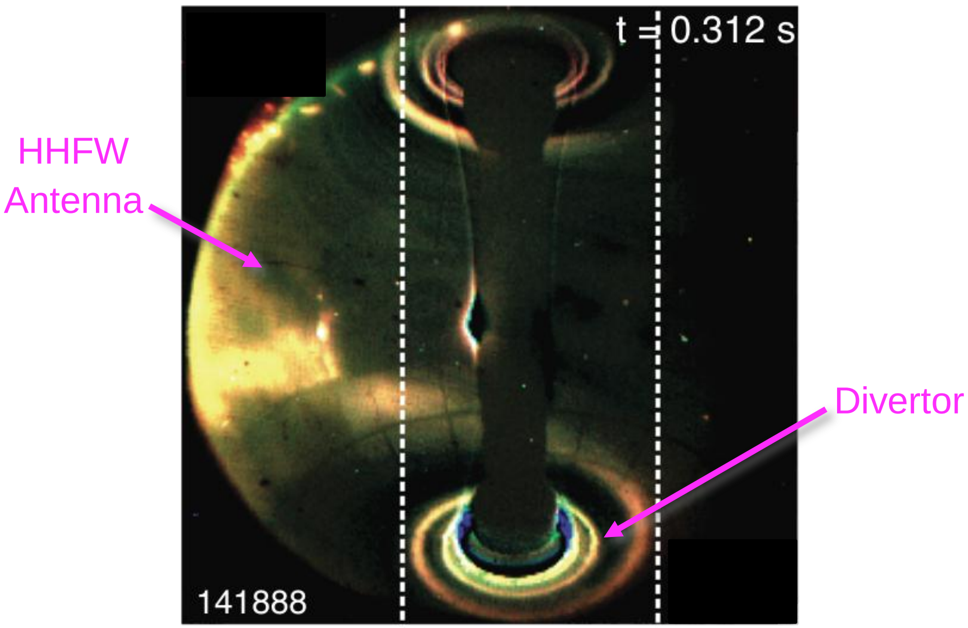
- Needed for NSTX-U, as divertor heat flux width strongly decreases as I_p increases
- Snowflake divertor experiments (P_{NBI} = 4 MW, P_{SOL} = 3 MW)
  - Good H-mode τ_E, β_N, sustained during snowflake operation
  - Divertor heat flux significantly reduced both during and between ELMs
    - during ELMs: 19 to < 1 MW/m^2
    - steady-state: 7 to < 1 MW/m^2
  - Achieved by a synergistic combination of detachment + radiative snowflake divertor

Snowflake divertor in NSTX

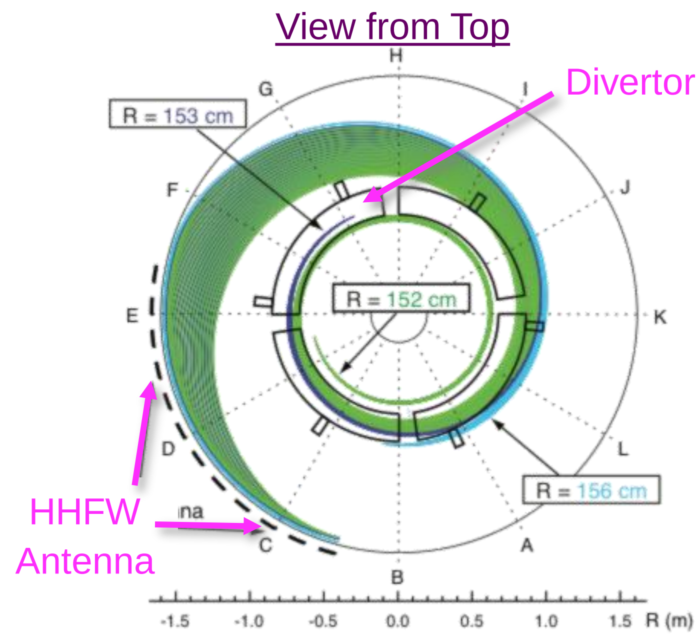


Soukhanovskii EX/P5-21

Significant fraction of the HHFW power may be lost in the SOL in front of antenna and flow to the divertor region



Visible camera image shows edge RF power flow follows magnetic field from antenna to divertor



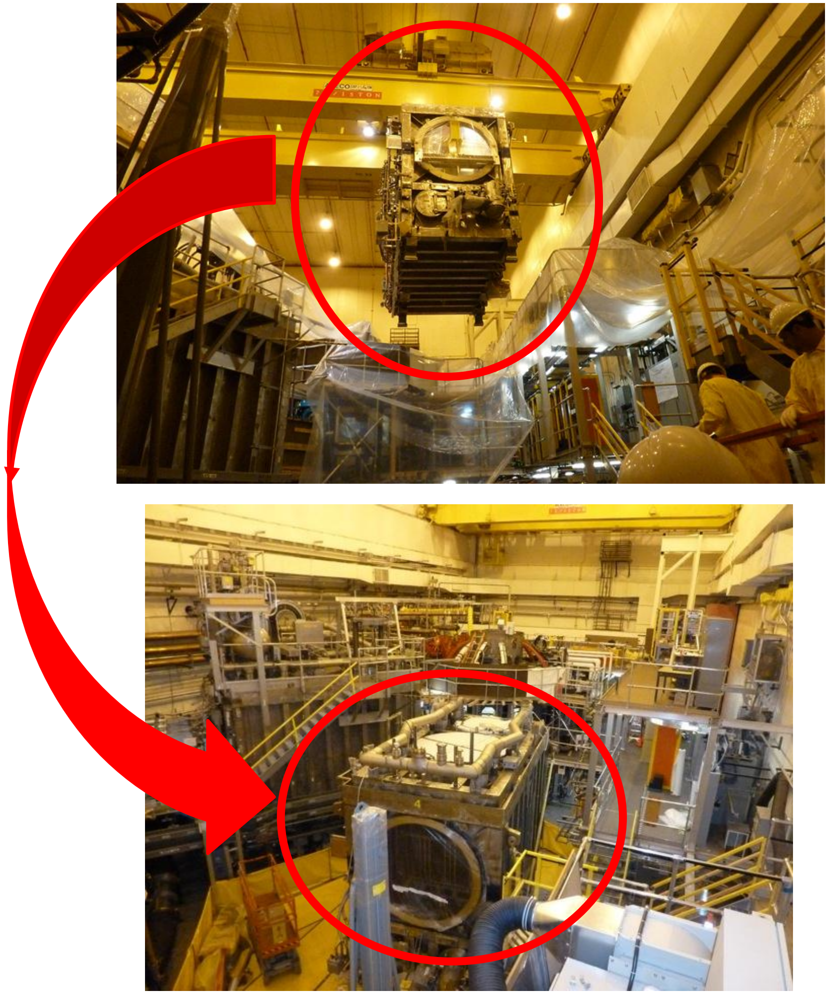
SPIRAL results show field lines (green) spiraling from SOL in front of HHFW antenna to divertor

- Field line mapping predicts RF power deposited in SOL, not at antenna face
  - 3D AORSA will assess surface wave excitation in NSTX-U
- Proposed DIII-D experiment to look for RF edge losses during 2012 run
- NSTX-U experiments and modeling to emphasize HHFW heating of high NBI power, long-pulse H-modes → assess effect of varying outer gap

R. J. Perkins, et al., PRL (2012)

Perkins EX/P5-40

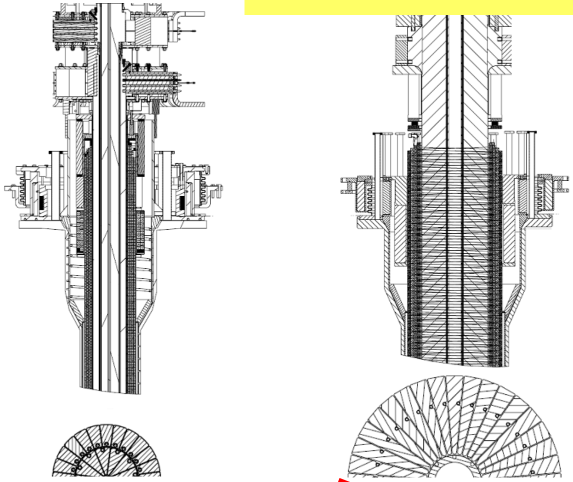
Rapid Progress is Being Made on NSTX Upgrade



- 2nd NBI box moved into place

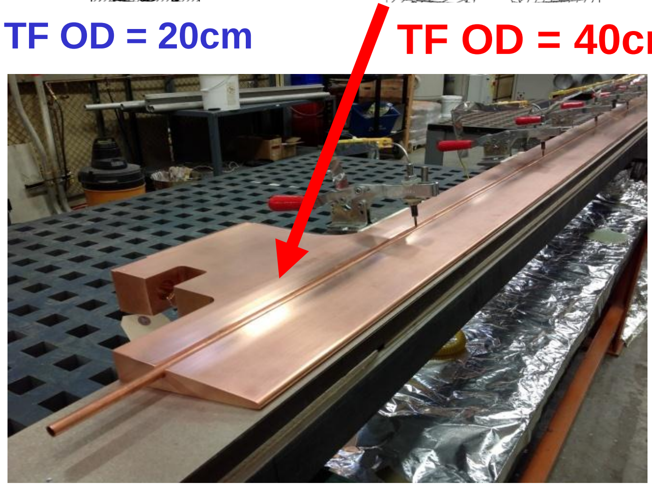
Old center stack

NEW Center Stack



TF OD = 20cm

TF OD = 40cm



- TF conductors being made

(first plasma anticipated June 2014)

Menard FTP/3-4