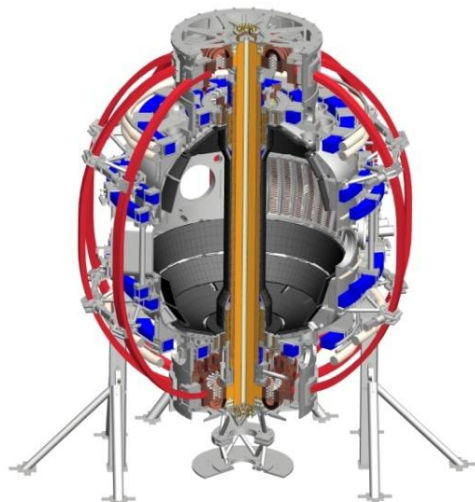


NSTX-U Research Highlights and Plans

Jonathan Menard, PPPL
For the NSTX Research Team

Coll of Wm & Mary
 Columbia U
 CompX
 General Atomics
 FIU
 INL
 Johns Hopkins U
 LANL
 LLNL
 Lodestar
 MIT
 Lehigh U
 Nova Photonics
 ORNL
 PPPL
 Princeton U
 Purdue U
 SNL
 Think Tank, Inc.
 UC Davis
 UC Irvine
 UCLA
 UCSD
 U Colorado
 U Illinois
 U Maryland
 U Rochester
 U Tennessee
 U Tulsa
 U Washington
 U Wisconsin
 X Science LLC

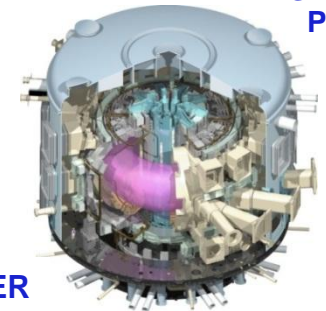
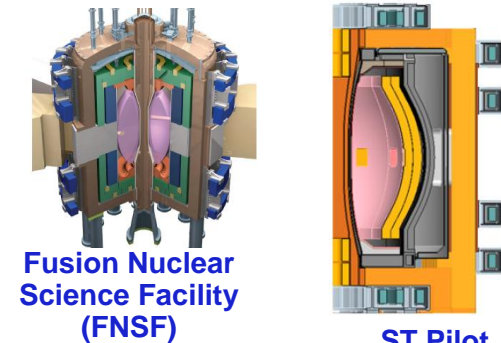
IAEA FEC Meeting
October 2012



Culham Sci Ctr
 York U
 Chubu U
 Fukui U
 Hiroshima U
 Hyogo U
 Kyoto U
 Kyushu U
 Kyushu Tokai U
 NIFS
 Niigata U
 U Tokyo
 JAEA
 Inst for Nucl Res, Kiev
 Ioffe Inst
 TRINITY
 Chonbuk Natl U
 NFRI
 KAIST
 POSTECH
 Seoul Natl U
 ASIPP
 CIEMAT
 FOM Inst DIFFER
 ENEA, Frascati
 CEA, Cadarache
 IPP, Jülich
 IPP, Garching
 ASCR, Czech Rep

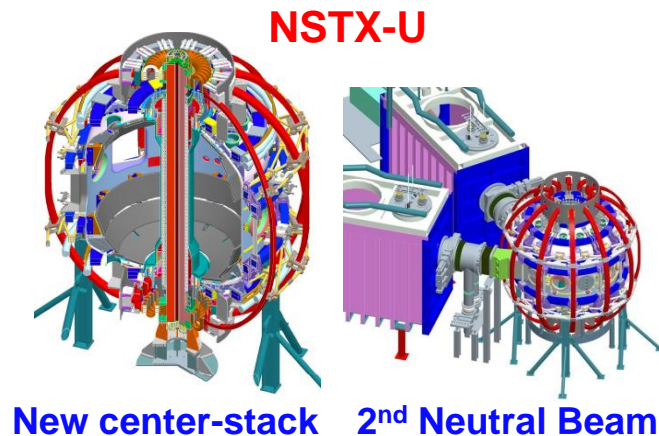
NSTX research targets predictive physics understanding needed for fusion energy development facilities

- Enable key ST applications
 - Move toward steady-state ST FNSF, pilot plant
 - Close key gaps to DEMO
- Extend understanding to tokamak / ITER
 - Leverage ST to develop predictive capability



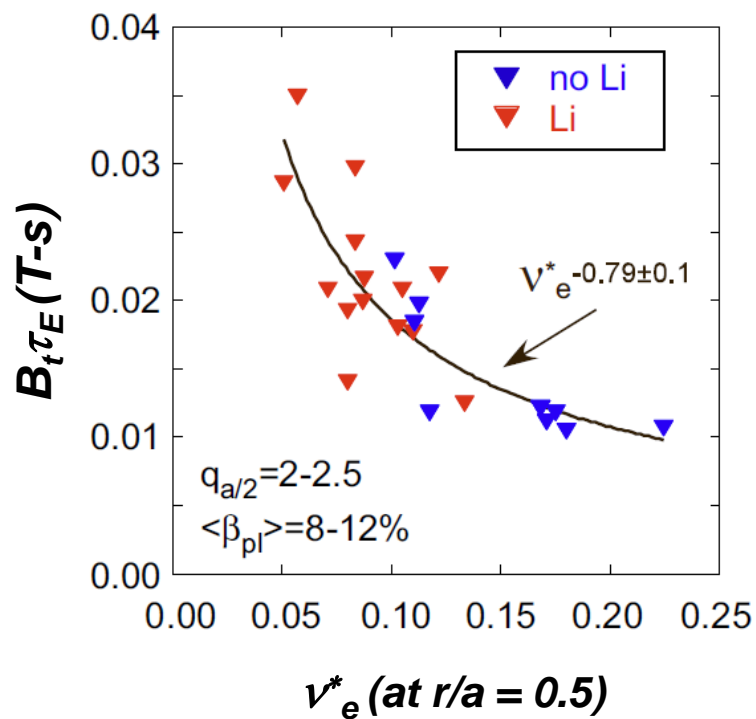
Present Research

- Develop key physics understanding to be tested in unexplored, hotter ST plasmas
 - Study high beta plasma transport and stability at **reduced collisionality, extended pulse**
 - Prototype methods to mitigate **very high heat/particle flux**
 - Move toward **fully non-inductive operation**



Nonlinear microtearing simulations for NSTX consistent with measured electron heat transport dependence on collisionality

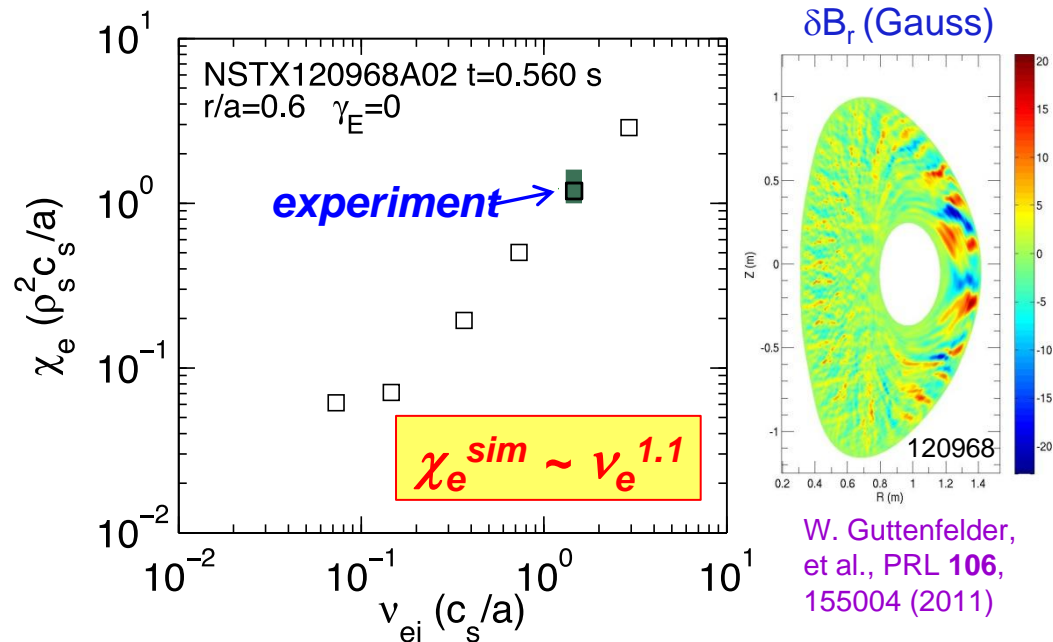
Experiment



- Increase in τ_E as v_e^* decreases
- Trend continues when lithium is used

Kaye EX/7-1

Theory

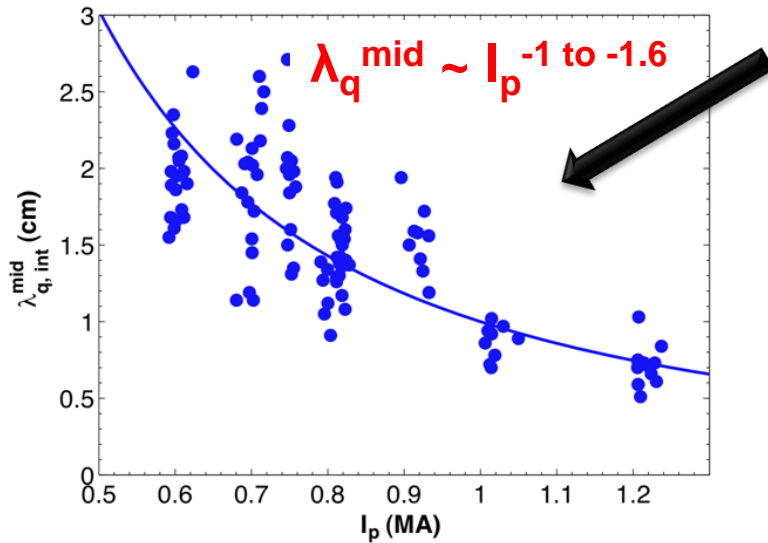


- Predicted χ_e and scaling $\sim v_e^{1.1}$ consistent with experiment ($\Omega \tau_E \sim B_t \tau_E \sim v_e^{*-0.8}$)
- Transport dominated by magnetic “flutter”
 - $\delta B_r / B \sim 0.1\%$ - possibly detectable by planned UCLA polarimetry system

Guttenfelder TH/6-1

- NSTX-U computed to extend studies down to $< 1/4$ of present v_e^*

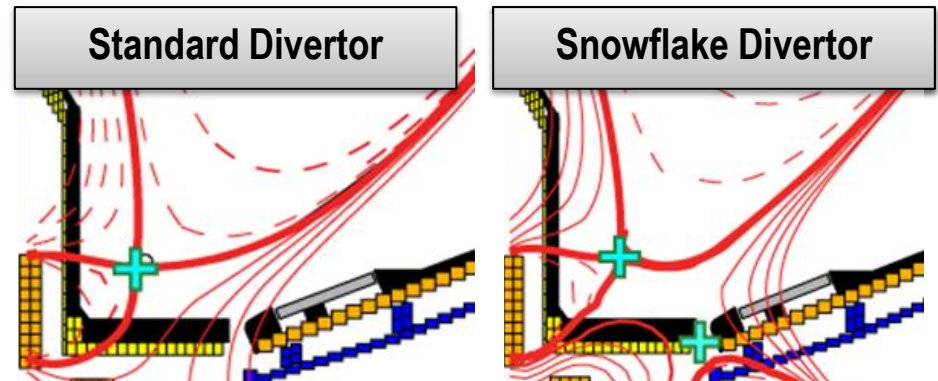
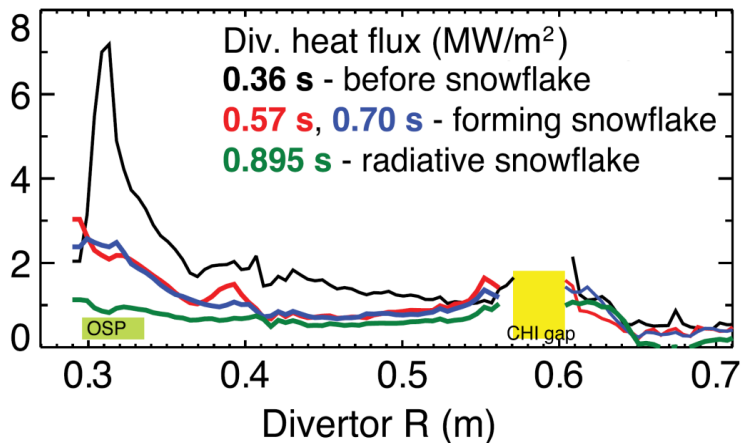
NSTX/NSTX-U investigating snowflake divertor and detachment physics for large heat-flux reduction for FNSF, ITER, and Demo



- Divertor heat flux width decreases with increased plasma current I_p

→ 30-45MW/m² in NSTX-U with conventional LSN divertor at full current and power

NSTX data



- **Snowflake** → high flux expansion = 40-60
lowers incident q_{\perp} , promotes detachment

Soukhanovskii – EX/P5-21

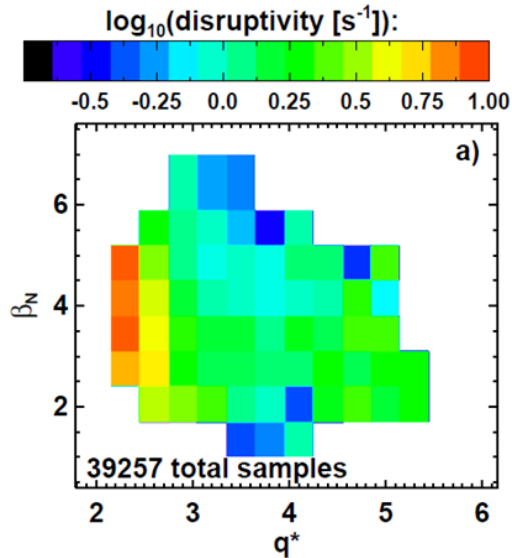
NSTX-U: U/D balanced snowflake has < 10MW/m² at $I_p = 2\text{MA}$, $P_{\text{AUX}}=10\text{-}15\text{MW}$

Disruption detection & warning analysis of NSTX being developed for disruption avoidance in NSTX-U, potential application to ITER

Disruptivity

- All discharges since 2005 with 1/3 ms sampling time

- Recorded equilibrium and kinetic parameters, disruption statistics



- Physics results

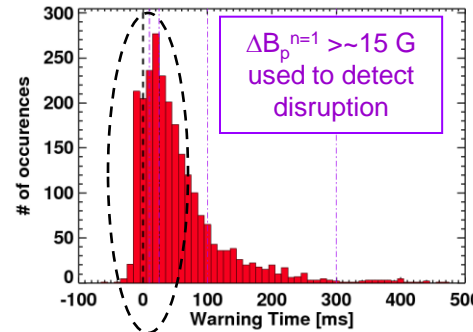
- Minimal disruptivity at relatively high $\beta_N \sim 6$; $\beta_N / \beta_N^{\text{no-wall}(n=1)} \sim 1.3-1.5$
 - Consistent with specific disruption control experiments
- Strong disruptivity increase for $q^* < 2.5$
- Strong disruptivity increases for lowest rotation

Gerhardt EX/9-3

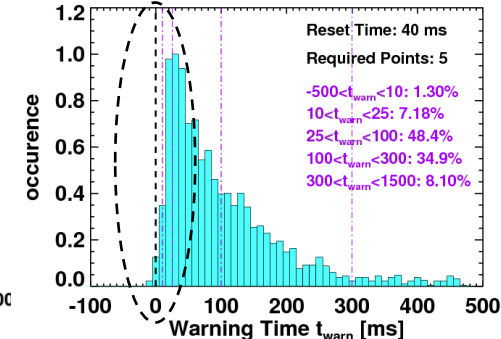
Warning Algorithms

- Disruption warning algorithm shows high probability of success
 - Based on combinations of threshold based tests; no machine learning

Statistics for a single threshold test



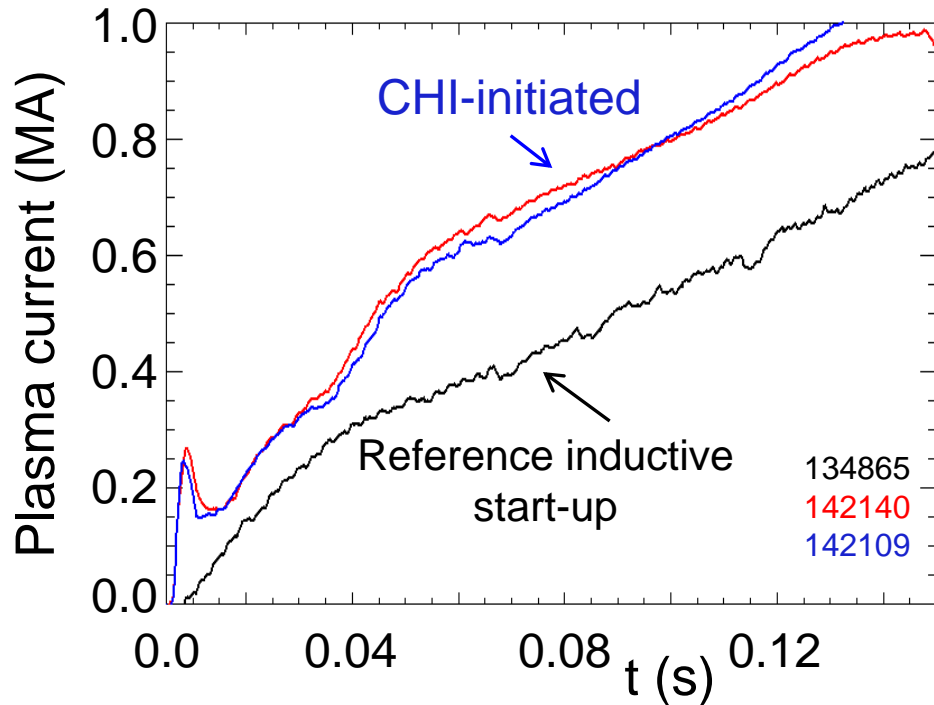
Statistics of full warning algorithm



- Results & Physics implications

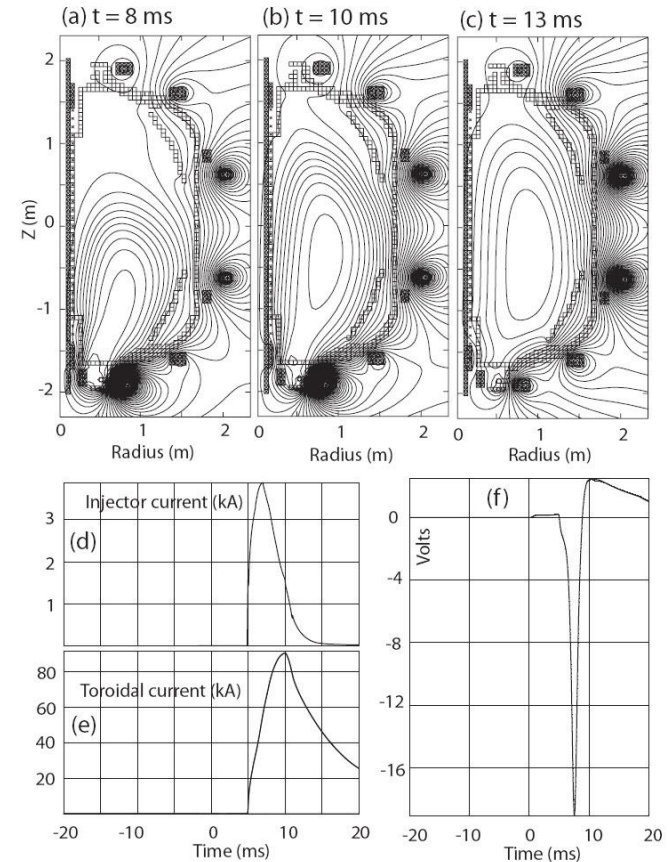
- ~98% disruptions flagged with at least 10ms warning, ~6% false positives
- Most false positives are due to “near disruptive” events
 - Early MHD slows ω_ϕ
 - recoverable Z motion

L-mode discharge ramping to 1MA requires 35% less inductive flux when coaxial helicity injection (CHI) is used



- ❑ Reference inductive discharge
 - ❑ Uses 396 mWb to get to 1MA
- ❑ CHI initiated discharge
 - ❑ Uses 258 mWb to get to 1MA (35% less inductive flux)
- ❑ Doubling of CHI closed flux current projected for NSTX-U: 200→400kA

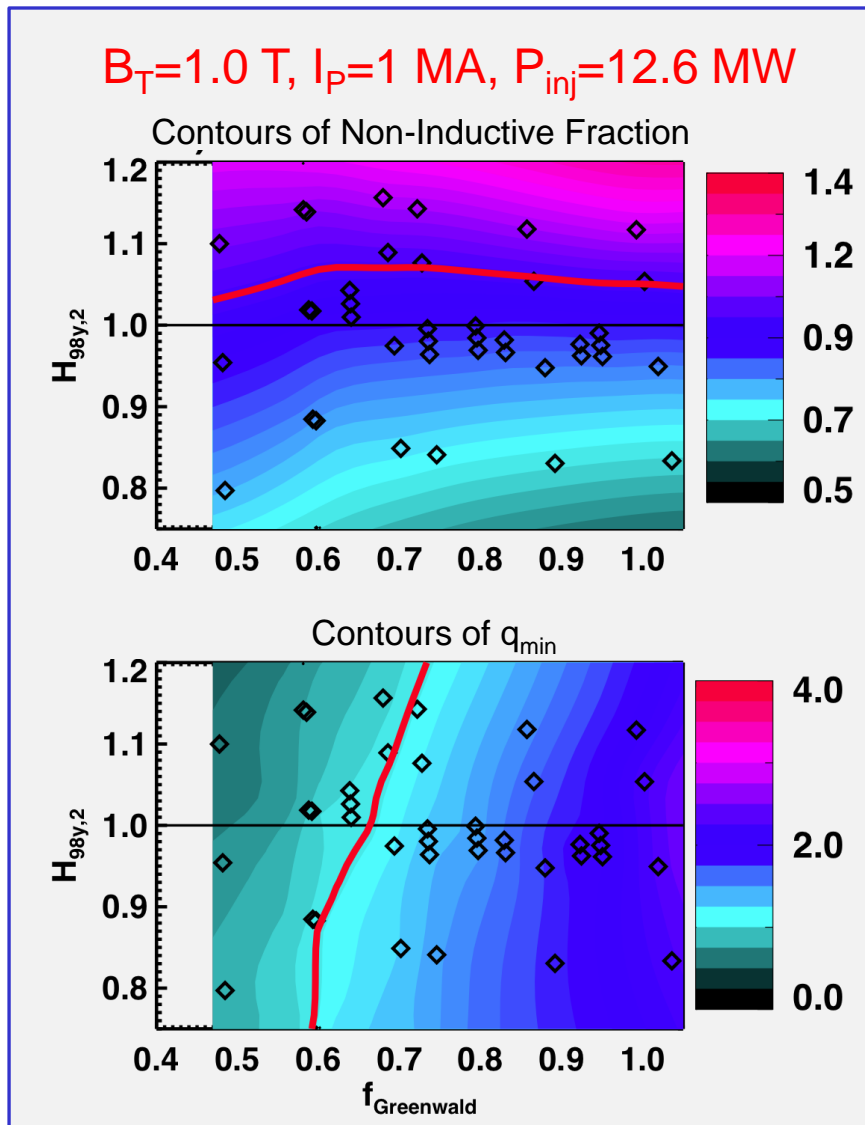
TSC simulation of NSTX CHI startup



- ❑ Decaying poloidal flux induces positive loop voltage, causes flux closure

Raman EX/P2-10

100% non-inductive NSTX-U operating points projected for range of toroidal fields, densities, and confinement levels



Projected Non-Inductive Current Levels for $\kappa \sim 2.85$, $A \sim 1.75$, $f_{GW} = 0.7$

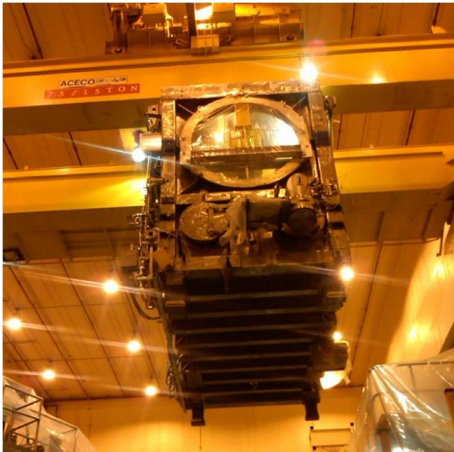
B_T [T]	P_{inj} [MW]	I_p [MA]
0.75	6.8	0.6-0.8
0.75	8.4	0.7-0.85
1.0	10.2	0.8-1.2
1.0	12.6	0.9-1.3
1.0	15.6	1.0-1.5

- From GTS (ITG) and GTC-Neo (neoclassical):
 - $\chi_{i,ITG}/\chi_{i,Neo} \sim 10^{-2}$
 - Assumption of neoclassical ion thermal transport should be valid

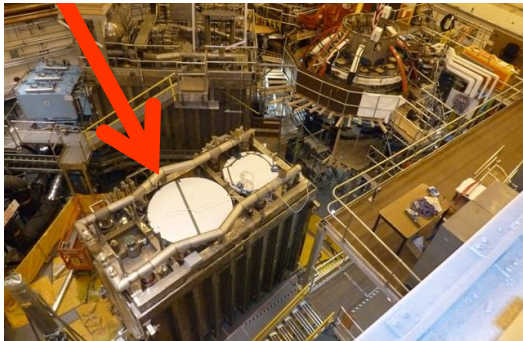
Rapid progress being made on NSTX Upgrade Project

First plasma anticipated mid-2014

Beam box craned over NSTX



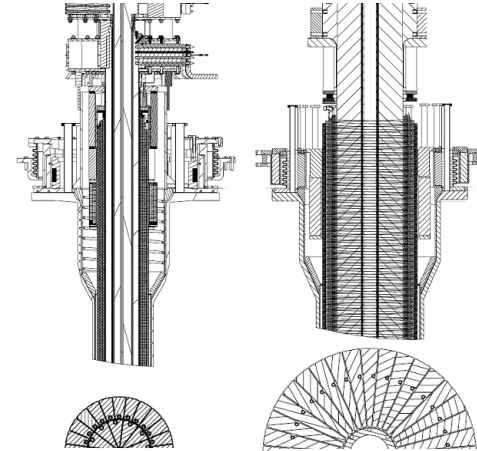
Box + cryo-panels next to NSTX



- 2nd NBI box moved into place
 - 1 month ahead of schedule

Old center stack

NEW Center Stack



TF OD = 20cm

TF OD = 40cm



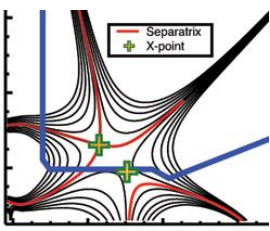
- Center-stack upgrade TF conductors being fabricated

Formulating FY2014-18 5 year plan to access new ST regimes with Upgrade + additional staged & prioritized upgrades

2009	2010	2011	2012	2013	2014	2015	2016	2017	2018
------	------	------	------	------	------	------	------	------	------

1 MA Plasma	Upgrade Outage	1.5 → 2 MA Plasma
-------------	----------------	-------------------

- CHI Control Coils
- LLD
- Moly-tile
- HHFW Upgrade



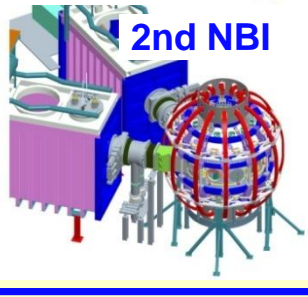
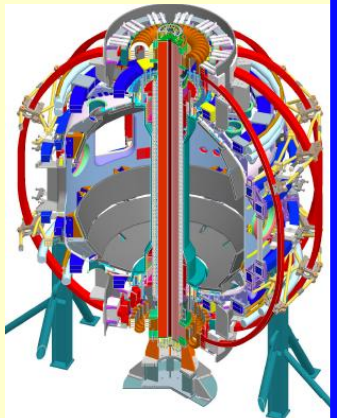
“Snowflake”



Lithium

- New Center-Stack
- 2nd NBI

New Center-stack

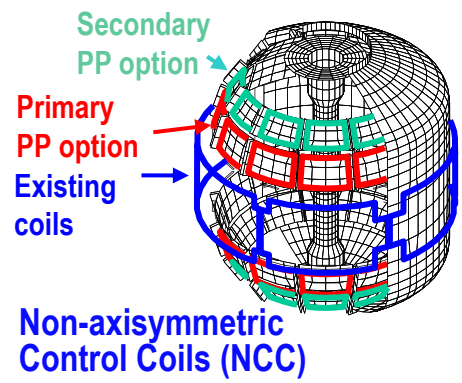


2nd NBI

- 0.5 MA CHI
- 1 MA CHI / Plasma Gun
- ECH/EBW 1MW → 2 MW

- 0.5 MA Plasma Gun
- Long-pulse Divertor

- NCC Upgrade



Non-axisymmetric Control Coils (NCC)

NSTX Upgrade research goals in support of FNSF and ITER

- Low collisionality plasma regimes
- 100% non-inductive operation
- Long-pulse, high power divertor
- Advanced high-β scenarios



NOTE: Upgrade operation would be delayed ~1 year to mid-2015 w/o incremental, other follow-on upgrades are further delayed