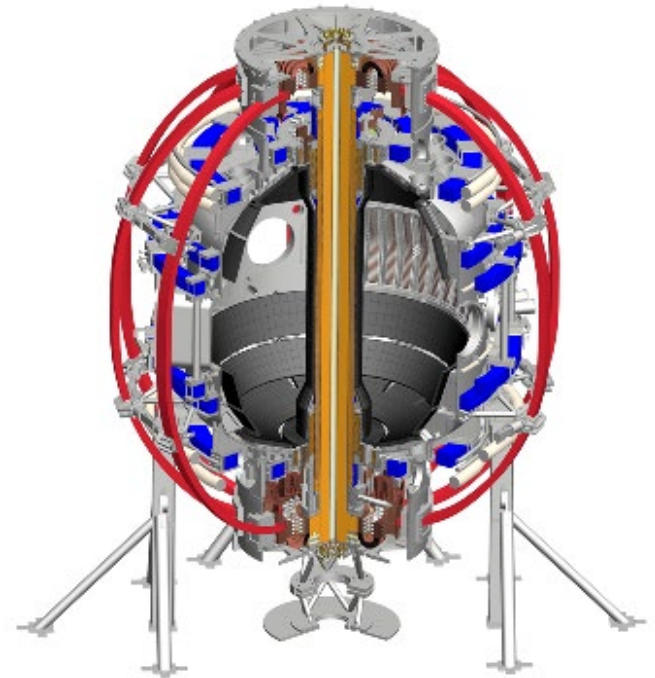


[OV/4-5Rb] Recent NSTX-U theory, modeling and analysis results

Walter Guttenfelder for the NSTX-U team

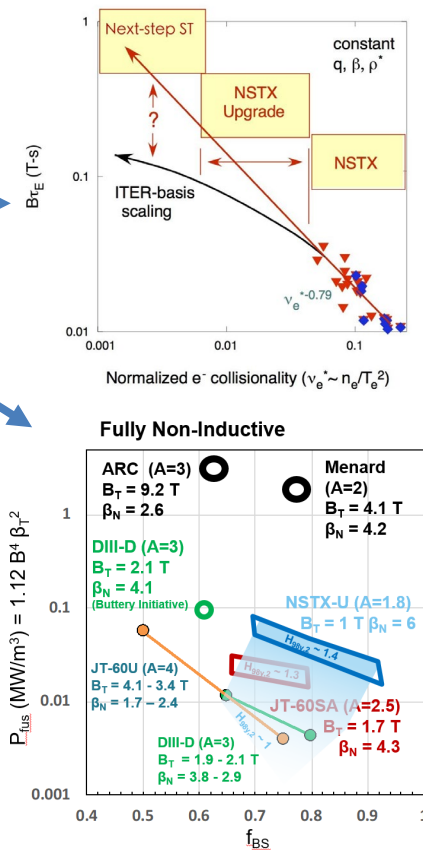
IAEA Fusion Energy Conference 2020
Virtual
May 10-15, 2021



NSTX-U Mission: Advance low-A physics basis for configuration optimization, support ITER & critical fusion development needs

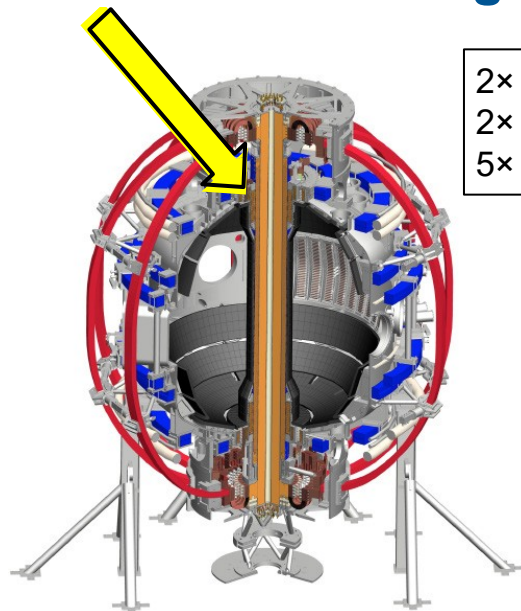
- Key objectives of NSTX-U research program:

- Extend confinement and stability physics basis at low-A and high beta to lower collisionality relevant to burning plasma regimes
- Develop operation at large bootstrap fraction and advance the physics basis required for non-inductive, high-performance and low-disruptivity operation of steady-state compact fusion devices
- Develop and evaluate conventional and innovative power and particle handling techniques to optimize plasma exhaust in high performance scenarios
- Support additional critical fusion science and technology development needs (e.g., ITER) utilizing the unique regimes accessible in NSTX-U



NSTX-U targeting major performance increase to explore new physics regimes

1. New Central Magnet



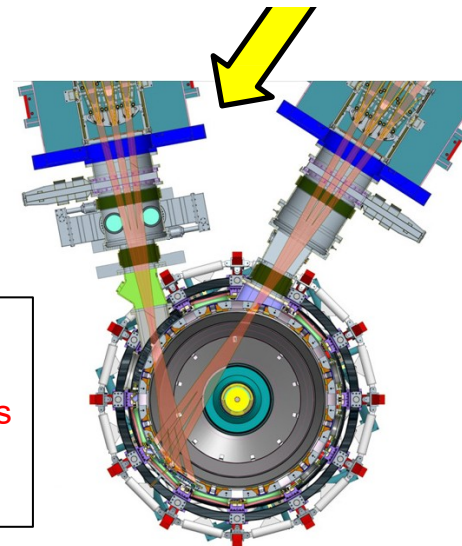
2× toroidal field (0.5 → 1T)
2× plasma current (1 → 2MA)
5× longer pulse (1 → 5s)

2× heating power (5 → 10MW for 5s)
• Tangential NBI → $2 \times \eta_{cd}$
• Up to 15 MW NBI + 6 MW RF for 1-2s
Up to 10× higher $nT\tau_E$ (~MJ plasmas)
4× divertor heat flux (→ ITER levels)

Unique regime (high β , low v_*)

Study new transport and stability physics

2. Tangential 2nd Neutral Beam



→ Sustain plasma without transformer

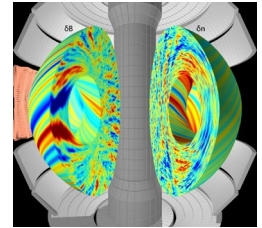
Not yet achieved at high- β_T , low v_*

Essential for any future steady-state ST/tokamak

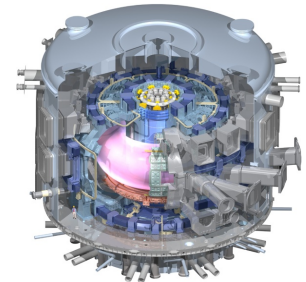
NSTX-U research addresses urgent issues for fusion science, ITER and next-step devices

- ST accesses unique regime of high β_T
 - Fundamental changes in nature of turbulence, MHD stability → Enhanced electromagnetic and super-Alfvénic effects
 - STs can more easily measure electron-scale turbulence → Important transport channel at all aspect ratio
 - Neutral beam fast-ions in present STs mimic DT α populations → Study burning plasma science
- **Expanded parameter space crucial for model validation**
- If physics results favorable, STs could provide more economical fusion development & energy systems
 - Potentially reduced magnet and device size and cost

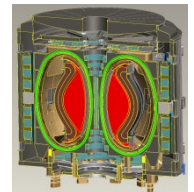
Electromagnetic turbulence



ITER



Pilot Plant



J.E. Menard, TECH/2-4 (FEC2021)

Significant progress made in analysis, theory and modeling in multiple areas addressing NSTX-U Mission

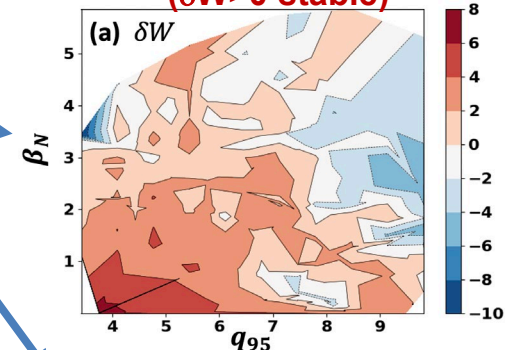
- Core MHD stability
- Energetic particle physics
- Transport and pedestal structure
- Boundary and divertor
- RF heating
- Scenarios and control

Core Stability: kink ($n=1$) + tearing ($2/1$) calculations identify stable (β_N , q_{95}) operating space

- Global kink and tearing modes often limited performance on NSTX, also cause disruptions
- Resistive DCON developed to identify scenarios stable to external kink ($n=1$) & NTM ($2/1$)
 - Benchmarked with MARS, PEST3 [Z. Wang, 2020]
 - For NSTX-U projection ($B_T=1.0$ T, $I_p=2.0$ MA, $P_{NBI}=12$ MW, $q_{\min} \rightarrow 1^+$), both stable for $\beta_N \approx 3$, $q_{95} = 7.5$ (regions in red are stable)
 - Can also be applied to optimize ramp-up
- 3D calculations (IPEC, M3D-C1) used to study sensitivity of plasma response & PFC strike-points to coil alignment
 - Tearing & locked-mode onset + PFC field line pitch used to guide engineering alignment tolerances [Ferraro, 2019]
 - Corresponding strike-point splitting & extended footprints contained to high-heat flux PFCs [Munaretto, 2019]

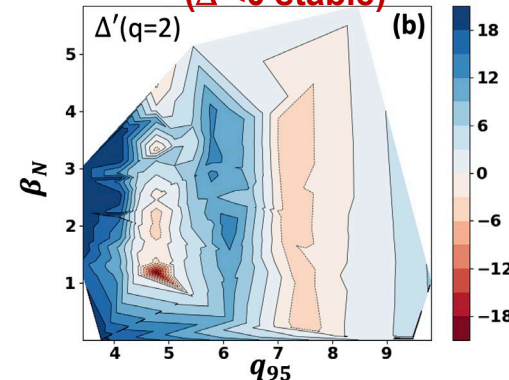
$n/m=1/1$ kink stability

($\delta W > 0$ stable)



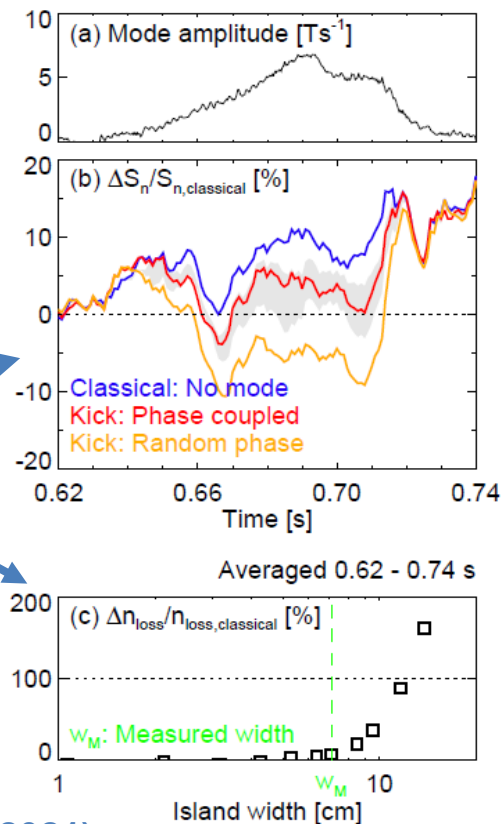
$n/m=2/1$ NTM stability

($\Delta' < 0$ stable)



EP: Coupled low-frequency modes modify fast ion transport synergistically

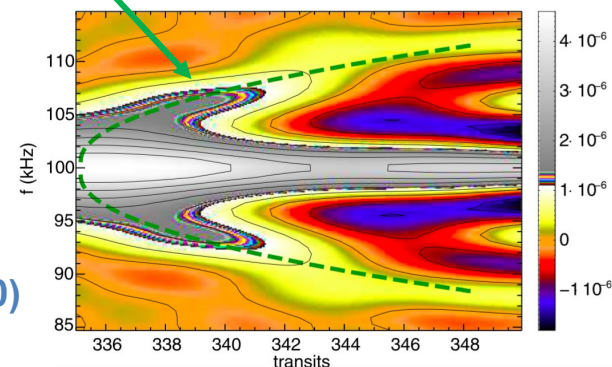
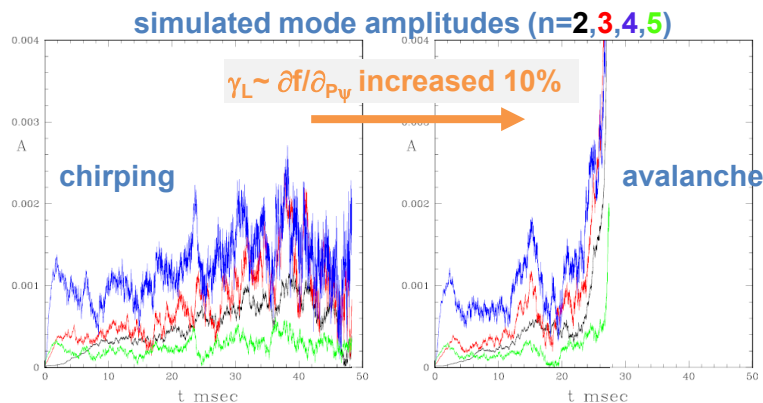
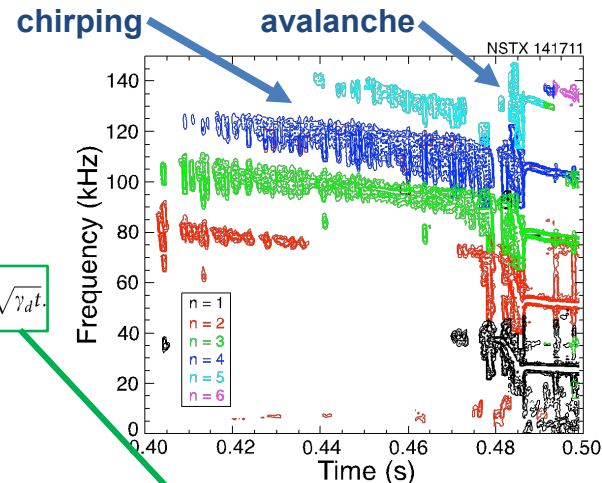
- Alfvénic and low-frequency (MHD) modes can interact nonlinearly to influence fast ion transport
- “Kick” model for energetic particle (EP) transport has been extended to include low-f perturbations
M. Podesta (2019), TH/P1-26 (FEC2021)
- Predicted neutron rate deficit (ΔS_n) correlates with amplitude of coupled kink ($n=1$) + tearing ($2/1$) modes
 - Important to model phase-coupled, as inferred in experiment; different than sum of randomly-phased modes
- Possible experimental evidence of NTM saturation level clamped by “stochastization” of EP phase space
- Kinetic module of M3D-C1 recently developed to simulate coupled MHD-EP interactions (C. Liu)



J. Yang (2021)

EP: Guiding center simulations (ORBIT) + delta-f formalism predict nonlinear chirping & avalanches due to AEs

- Chirping and avalanches are primary mechanisms of fast ion loss due to Alfvén eigenmodes (AEs)
 - Avalanches often observed in NSTX with super-Alfvénic beams ($V_{\text{beam}}/V_A > 1$) \rightarrow relevant to ITER α 's
- Frequency chirping predicted by simulations, consistent with analytic theory based on wave-particle nonlinearity $\delta f = \pm \frac{16\sqrt{2}}{\pi^2 3\sqrt{3}} \gamma_L \sqrt{\gamma_a t}$
- Avalanche onset requires multiple modes with sufficient resonance overlap, simulations predict threshold behavior



R.B. White (2019, 2020)
TH/P1-13 (FEC 2021)

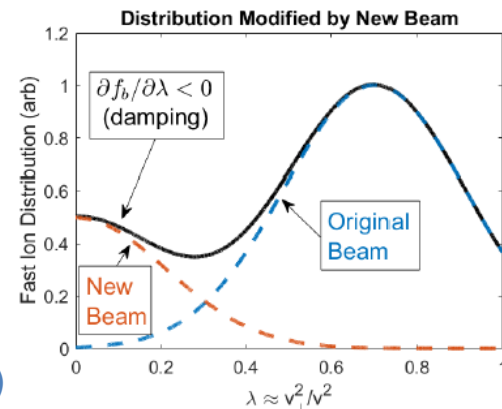
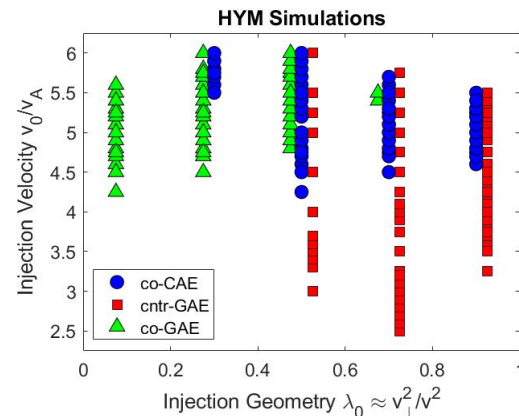
EP: Theory & simulation predict ways to stabilize high-frequency GAE/CAE, hypothesized to influence core T_e

- GAE/CAE observed at high power, correlated to flattening of T_e
 - Possibly due to stochasticized electron orbits (χ_e) and/or energy channeling to edge via CAE-KAW mode conversion
- Hybrid MHD-kinetic simulations (HYM) predict sensitivity of GAE/CAE stability with beam injection geometry and velocity
- Local analytic theory gives insight on how to stabilize

$$\gamma \propto \int h(\lambda, v) \left[\left(\frac{\ell \omega_{ci}}{\omega} - \lambda \right) \frac{\partial}{\partial \lambda} + \frac{v}{2} \frac{\partial}{\partial v} \right] f_b(\lambda, v) d^2 v$$

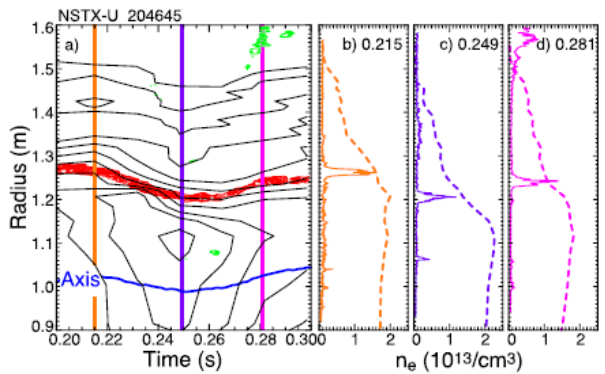
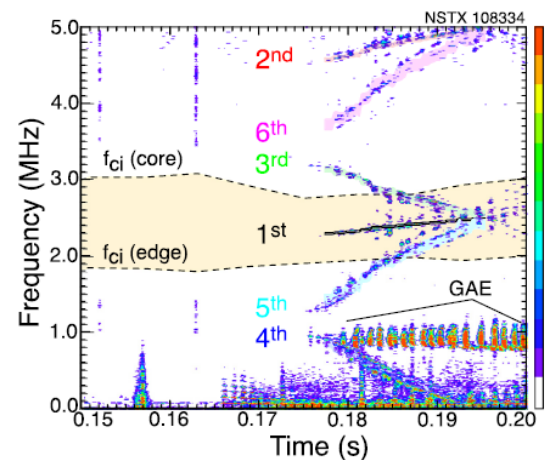
- Counter-propagating GAE requires perpendicular injection ($\lambda_0 \sim 0.5-0.7$ in NSTX) \rightarrow stabilized by more tangential NBI on NSTX-U ($\lambda_0 \sim 0$)
- Co-propagating GAE predicted with more tangential NBI at low field ($V_0/V_A > 4$) \rightarrow testable on NSTX-U to further validate theory
- Exp. and simulations for DIII-D suggest GAEs will be unstable in ITER, but much smaller growth rates & amplitudes than NSTX

J. Lestz (2020a,b, 2021); E. Belova, TH/P1-27 (FEC 2021)



EP: Detailed analysis of ion cyclotron emission (ICE) challenges present theories

- ICE being considered as an additional α -particle diagnostic in ITER, measurable via external Mirnov coils
- ICE observed in NSTX(-U) with ω_{ICE} from 1st to 7th Ω_{ci} harmonics, distinct variations identified:
 1. $\sim 100 \mu s$ bursts ($\gamma/\omega \sim 1\%$)
 2. quasi-stationary, strong 3-wave coupling to GAE
 3. longer, stronger bursts ($\gamma/\omega \sim 0.06\%$) with chirping
- ω_{ICE} scales with B (Ω_{ci}), not with density ($V_{Alfvén}$)
- ICE frequency, $\Omega_{ci}(R) = \omega_{ICE}/n$, often maps mid-radius (not core or edge), appears correlated with density gradient
- No correlation between neutron rate and ICE amplitude



E. Fredrickson (2019), EX/P7-6 (FEC2021)

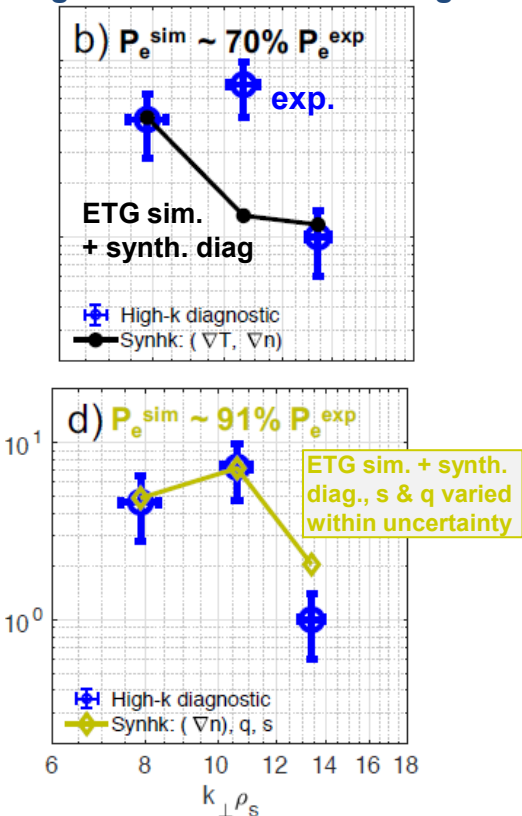
Transport: Comprehensive validation of electron-scale (ETG) gyrokinetic simulations using high-k microwave scattering

- Local high-k microwave scattering diagnostic + lower-field on NSTX enable detailed validation of electron-scale ETG turbulence
- Using novel synthetic diagnostic, nonlinear gyrokinetic simulations (GYRO) reproduce electron transport & high-k microwave scattering spectra for moderate- β NSTX H-mode
- Numerous parameter scans (∇n , ∇T , q , s) used to quantify sensitivity of predicted fluxes and synthetic high-k spectra

J. Ruiz Ruiz (2019, 2020a, 2020b)

- Global ion-scale simulations (GTS) consistent with negligible ion-scale transport ($\chi_i \approx \chi_{i,NC}$) in this discharge (Ren, 2020)
- Novel pseudolocal SXR tomography concept to measure high-k δT_e developed (X. Chen, 2021)

High-k microwave scattering

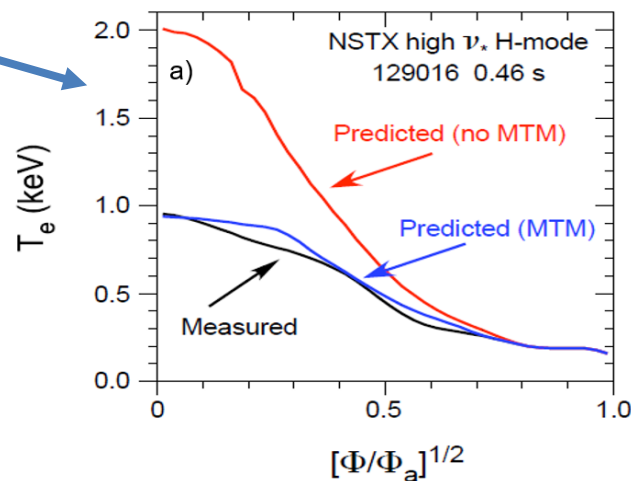
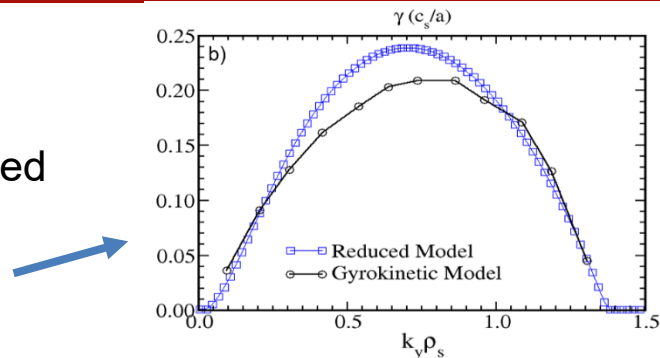


Transport: MTM reduced transport model qualified with gyrokinetics and validated via predictive modeling

- Significant microtearing mode (MTM) electron thermal transport predicted by gyrokinetics in high- β NSTX H-modes
- Hybrid fluid/kinetic MTM model developed to enable integrated predictive modeling
 - Recovers many linear gyrokinetic scalings ($k_\theta \rho_s$, β_e , a/L_{Te} and v_e)
 - $\delta B/B_0$ saturation model (from nonlinear dispersion) recovers some scalings from nonlinear GYRO simulations
- Predicted T_e profiles using MTM model + Multi-Model Model recover experiment for higher v_* discharges
 - Overpredicts transport at lower v_* as $\delta B/B_0$ saturation model scales too weakly with collisionality; other modes may also contribute

T. Rafiq (2021)

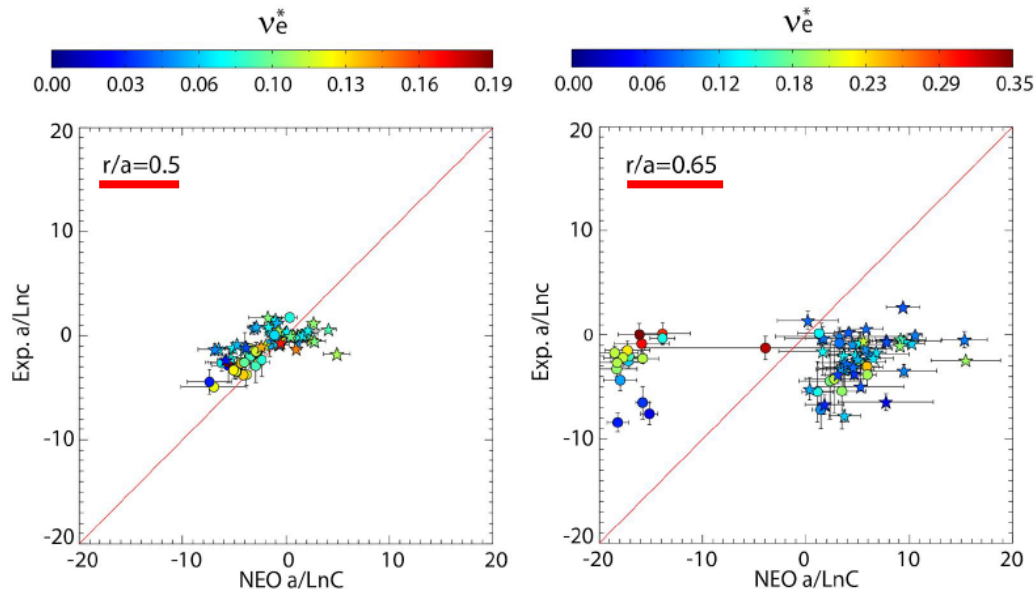
- Empirical χ_e model based on artificial neural networks also recently developed and being tested (Y.-S. Na)



Transport: Carbon impurity peaking near midradius consistent with neoclassical theory in NSTX database

- Expanded database analysis finds $a/L_{n,C} \approx a/L_{n,C,NEO} \sim (a/L_{n,D} - 0.5 a/L_{Ti})$ at mid-radius

Exp. $a/L_{n,C}$ compared to neoclassical $a/L_{n,C}$ @ $\Gamma_C=0$
computed by NEO (w/ poloidal asymmetry from rotation)



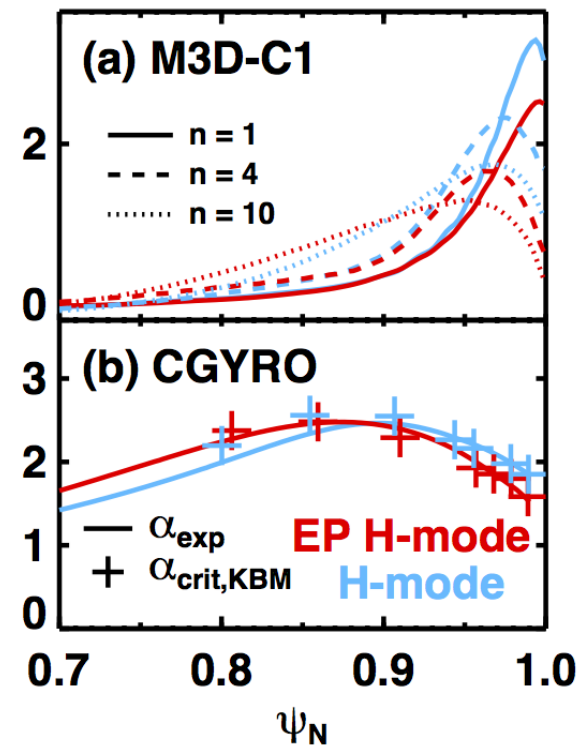
- Deviations further out ($r/a=0.65$) correlate most with wall conditioning
 - circles = boronization
 - stars = lithium wall conditioning
- Linear CGYRO predicts a mix of unstable modes, unable to account for the difference
 - MTM ($\Gamma_C \sim 0$) at high v_*
 - Ballooning modes at low v_* , but Γ_C same direction as NEO

N. Howard, PD/1-1 (FEC 2021)

Pedestal: Kink/peeling and KBM instabilities may enable Enhanced Pedestal H-modes (EPH)

- EPH is an attractive wide-pedestal, ELM-free scenario for optimized core-edge performance
 - $H_{98} > 1.3$ & $f_{BS} > 0.7$ at $f_{GW} > 0.7$
- Increased edge $\nabla T_i \approx \nabla T_{i,NC}$ typically accessed via transient reduction in v_{*i} (e.g. low edge density following large ELM)
- Hypothesis: particle transport from edge instabilities “locks-in” new edge profile state
 - Changes in BES fluctuations observed
 - Kink/peeling linearly unstable (resistive MHD: M3D-C1)
 - Profiles broadly at linear KBM threshold (gyrokinetics: CGYRO)
- SOLPS analysis in NSTX typically finds $(D_e/\chi_e) < 0.1 \rightarrow$ profile evolution likely dependent on other additional mechanisms predicted unstable (ETG, MTM, TEM)

Resistive MHD (M3D-C1) and gyrokinetic (CGYRO) KBM stability

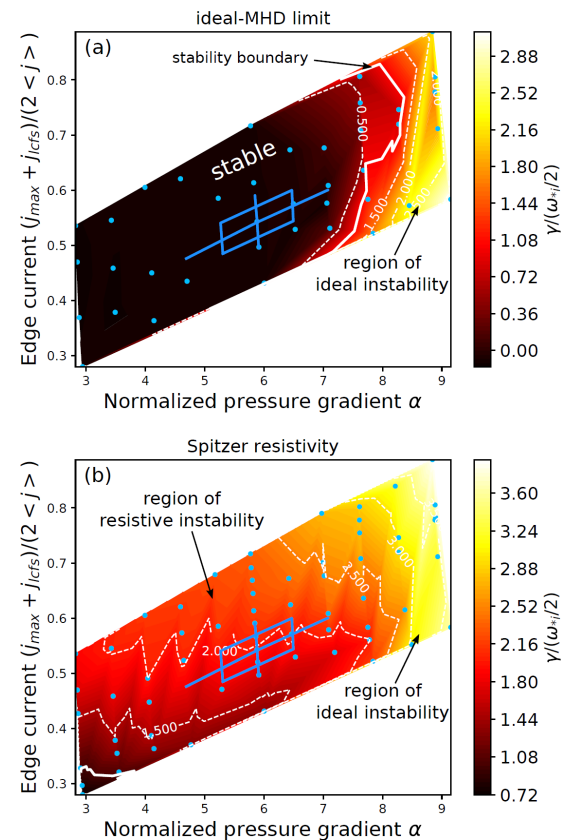


D. Battaglia (2020)

Pedestal: Resistive effects important for predicting MHD pedestal stability in NSTX H-modes

- Ideal MHD peeling-ballooning (P-B) growth rates in ELMy H-mode, $\gamma_{\text{NSTX,ELMy-H}} / (\omega_{*i}/2) \approx 0.1$, smaller than that for conventional tokamaks, $\gamma / (\omega_{*i}/2) = 1$
- Resistive MHD simulations (M3D-C1) predict larger P-B growth rates & change in stability boundary for NSTX ELMy H-modes
 - ELM stability boundary closer to $\gamma_{\text{resistive}} / (\omega_{*i}/2) = 2$
- Moving towards generalized pedestal structure model including non-ideal MHD + gyrokinetic KBM

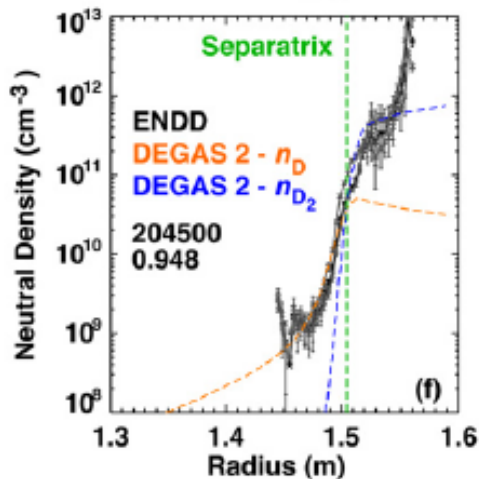
A. Kleiner (2021)



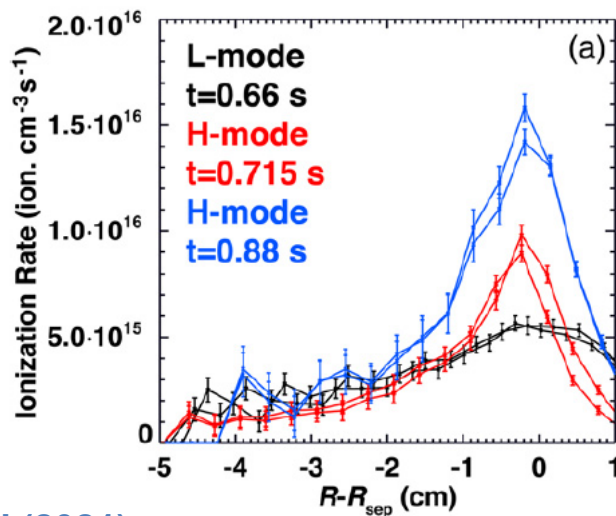
Pedestal: Neutral density & ionization measurements enable fueling and pedestal transport studies

- Neutral density and ionization rate inferred via inverting line-integrated 2D D_α (ENDD)
- Good agreement with DEGAS 2 over large database of NSTX / NSTX-U discharges
- n_D and ionization profiles narrow following $L \rightarrow H$, widths remain similar as n_e pedestal grows

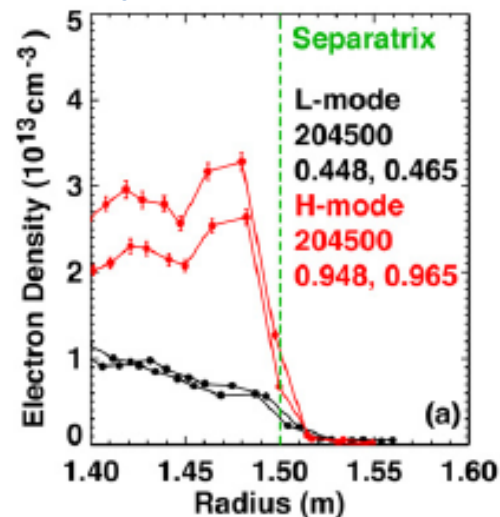
Neutral densities
exp. (ENDD), modeled (DEGAS 2)



Ionization rate (ENDD)



n_e pedestal formation

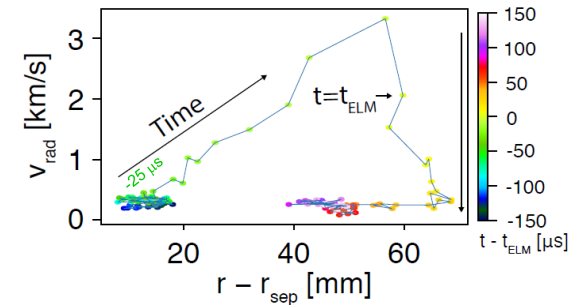


F. Scotti (2021)

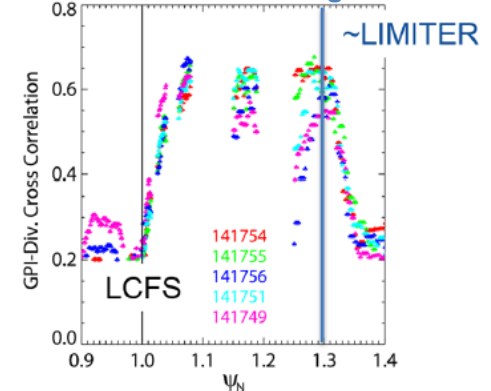
Boundary: Numerous turbulence observations challenge L/H, ELM and SOL theory

- 2D gas puff imaging (GPI) enables detailed edge turbulence studies
- **L/H**: No statistically significant change in average turbulence $\langle V_{\text{pol}} \rangle$ or gradient inside LCFS prior to L \rightarrow H (S. Zweben, 2021)
- **ELM**: Several SOL filaments coalesce into a single, circular filament, accelerates away from separatrix until ELM crash \rightarrow
 - Current filament model (Myra, 2007) consistent with coalescence, circular shape, and poloidal acceleration, does not explain $V_{\text{radial}} \sim (r-r_{\text{sep}})$
 - Possibility of reconnection contributing to current transport (Ebrahimi, 2017)
- **Inter-ELM**: wakes observed trailing SOL filaments (Zweben, 2019)
 - Possibly drift-Alfvén waves, not observed in earlier seeded blob turbulence simulations (Myra, 2013)
- **Upstream (GPI) / divertor target (Li I) correlation**: weak correlation measured near separatrix (Scotti, 2020) \rightarrow
 - Supports the role of X-point geometry and collisionality for disconnection of midplane instabilities from divertor target (Myra, 2006)

ELM filament radial velocity (V_{rad}) vs. distance from LCFS



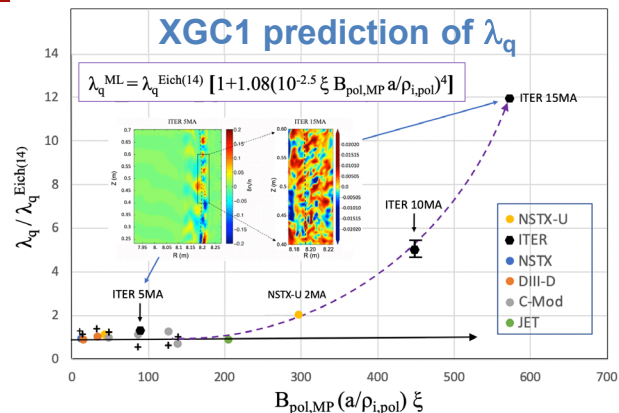
Correlation between upstream and divertor target



Boundary: Gyrokinetic simulations are approaching realism needed to simulate SOL dynamics

- XGC1 predictions of SOL λ_q for NSTX-U & ITER deviate from Eich scaling
- Due to emergence of TEM turbulence at large f_{trap} (low-A) and lower $v_{*e} \rightarrow$ opportunity to validate on NSTX-U

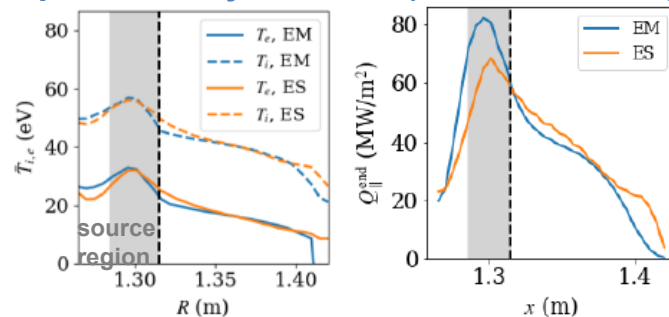
Chang (2021), TH/P4-04 (FEC2021)



- Electromagnetic effects incorporated into full-f gyrokinetic open-field line simulations for first time (GKEYLL)
- Used to model NSTX-like simple helical SOL
 - Does not yet include closed surfaces, shaping or X-point
- Predicts change in upstream gradients and target fluxes for scaled-up heating & fueling source rates ($\beta \uparrow$)

Mandell (2020); Hakim (2021), TH/3-4 (FEC 2021)

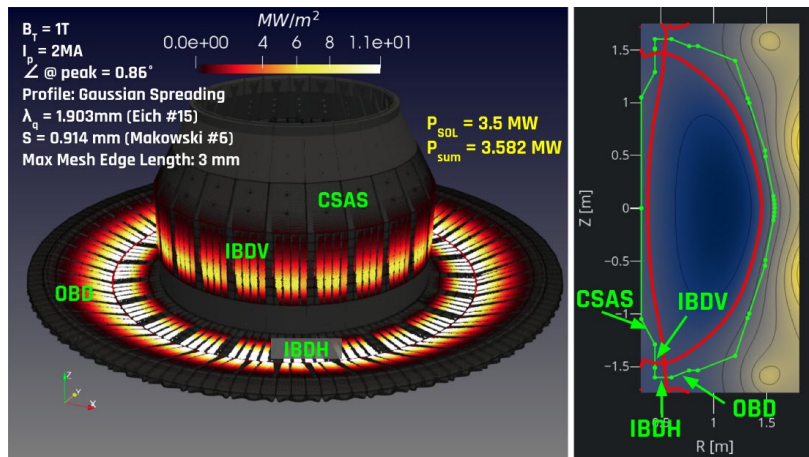
Temperature and target heat fluxes predicted by GKEYLL (scaled sources)



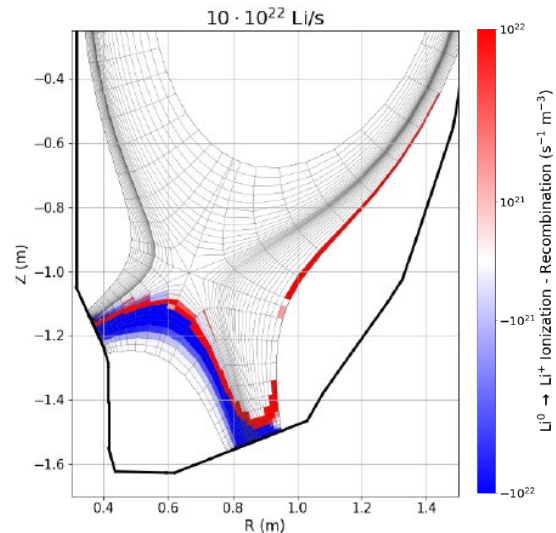
Boundary: New modeling developments to evaluate present PFC operational limits & future PFC concepts

- Heat flux Engineering Analysis Toolkit (HEAT) couples physics, CAD, ... to predict 3D heat flux & temp. on new castellated, fishscaled tiles
- Used to evaluate efficacy of strike-point sweeping to extend high power pulse lengths
- Lithium vapor box (LVB) predicted to reduce target q_{\perp} while maintaining stable detachment
- For NSTX-U LVB concept, SOLPS-ITER predicts upstream n_{Li}/n_e can be minimized (<2%) with sufficient D_2 puffing while maintaining detachment

Predicted 3D distribution of PFC heat fluxes (HEAT)



T. Looby (2021)



Volumetric Li ionization (with D_2 puff)

E. Emdee (2021)

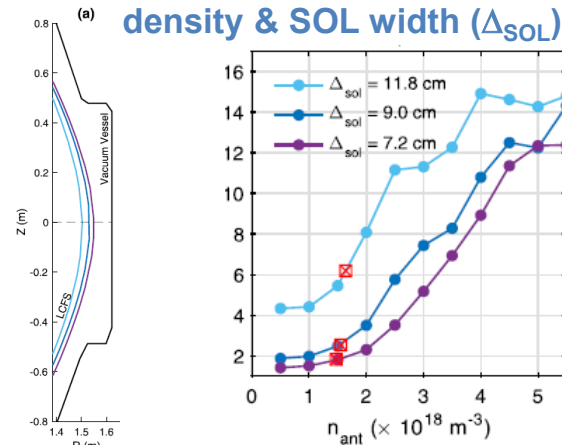
Additional concepts proposed (Ono, 2020; TECH/P7-11)

RF: 2D simulations used to investigate high harmonic fast wave (HHFW) coupling & SOL losses

- Significant HHFW power often lost to SOL in NSTX through cavity modes
- 2D full wave code (FW2D) updated to include realistic boundary \rightarrow predicted SOL loss minimized for lower density near antenna ($n_{\text{ant}} \sim$ fast wave cutoff)
 - Reduced at higher field (NSTX-U), smaller outer gap (Δ_{SOL})

E.-H. Kim (2019)

HHFW SOL power loss (%) vs. antenna density & SOL width (Δ_{SOL})

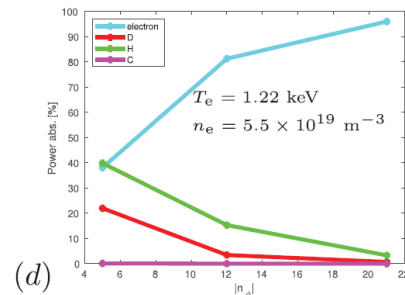
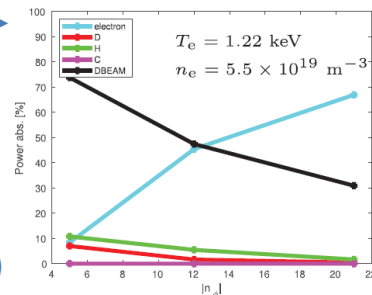


- Challenge to couple HHFW with NBI fast ions
- 2D full-wave simulations (AORSA, w/o SOL) predict competition between electron and fast ion absorption
 - Electron damping increases with phasing / toroidal wave number (n_{ϕ}), T_e/T_i and B_T

- Additional simulations show that a sufficient concentration of H+ minority species could enable new HHFW heating scenarios (without NBI)

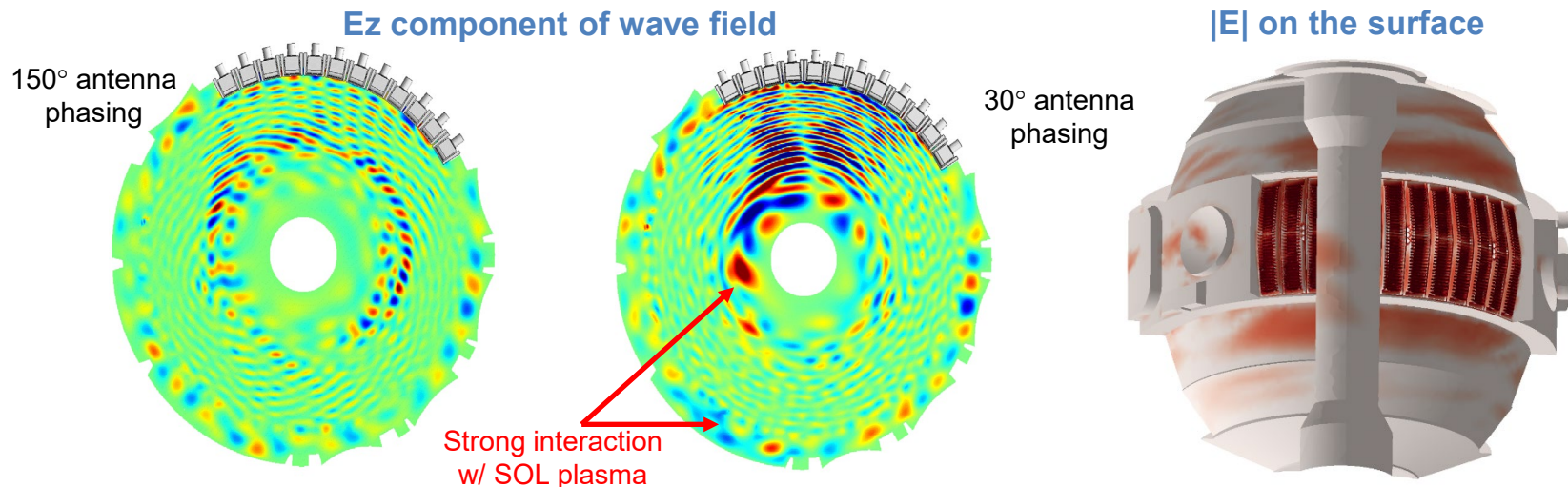
N. Bertelli (2019)

Predicted HHFW absorption w/ & w/o NBI fast ions



RF: Realistic 3D simulations (Petra-M) applied to study NSTX-U HHFW heating and SOL losses

- Petra-M developed for 3D simulations including SOL
 - 3D CAD for vessel & 12-strap antenna, EFIT for magnetic equilibrium
- Predicts increased SOL loss with lower antenna phasing (lower n_ϕ)
 - Stronger interaction with SOL plasma, far away from plasma
 - Stronger surface $|E|$, important for understanding impurity generation & RF sheath effects

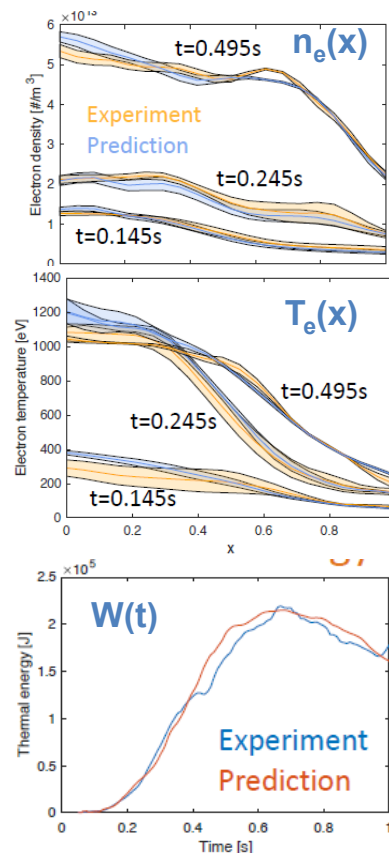


S. Shiraiwa, TH/7-2; N. Bertelli, TH/P2-16 (FEC2021)

Scenarios & Control: Many developments to establish, optimize and control high-performance discharges

- **Scenario optimization**: Optimizing steady-state scenario and actuator trajectories using ML acceleration integrated predictive modeling
 - Automated approach developed for optimizing scenarios & ramp-up trajectories (Wehner, 2019)
 - Accelerated (days \rightarrow seconds) through reduced models & machine learning \rightarrow for NBI (Ilhan, 2019; Boyer, 2019) and electron transport (Boyer, 2021)
- **Realtime (RT) control developments**:
 - Physics-based closed-loop RT control algorithm of snowflake divertor (Vail, 2019)
 - Safety factor control algorithm (Ilhan, 2019), tested using improved TRANSP closed-loop control modeling (Boyer, 2020)
 - Scalable framework for RT Thomson scattering (<17ms latency) using dedicated server & parallel analysis (Laggner, 2019)
- **Startup**: Semi-empirical model for designing inductive startup scenarios developed (Battaglia, 2019) \rightarrow used to help achieve MAST-U first plasma

M.D. Boyer, EX/P7-5 (FEC 2021)

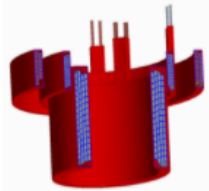


NSTX-U Recovery Project in construction & installation phase

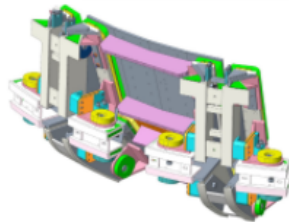
- Magnet fabrication is complete; strong progress in CS casing, PFCs, passive plates
- Due to COVID delays, early start date now August 2022

S. Gerhardt, TECH/P3-17

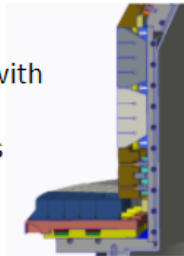
Six new inner-PF coils (PF-1a, -1b, -1c, upper and lower)



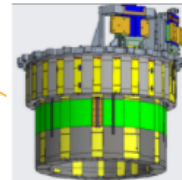
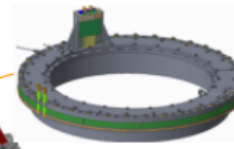
Repairs to passive plate bracketry to allow operation at full EM load



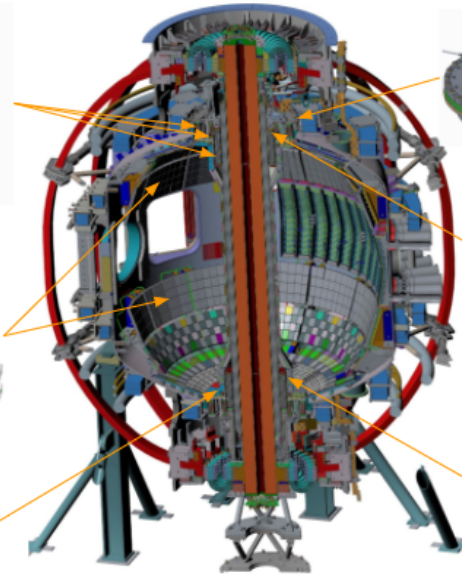
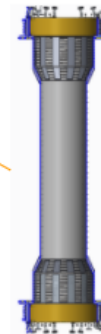
New graphite tiles with improved heat flux handling capabilities



PF-1a, -1b, -1c support structures and double O-ring seals



New Center Stack Casing With Improved Heating/Cooling Features



Summary: NSTX-U research addresses urgent issues for fusion science, ITER and next-step devices

- Advances in core transport validation and model development
- Advances in modeling fast ion transport and energetic particle stability
- Expanded understanding of transport and stability mechanisms setting pedestal structure
- Boundary studies addressing SOL turbulence and PFC modeling
- 2D and 3D RF modeling of HHFW coupling and SOL losses
- Advances in scenario optimization using machine-learning accelerated integrated predictive modeling and in real time control algorithms