Team discussion of NSTX support of ITER/ITPA high priority research and joint expts.

September 29th, 2008

Presented by Charles Skinner for the BP TSG.

Menard Email Sept 4th:

•"For each ITPA topical group, the corresponding NSTX ITPA members and TSG leaders are asked to work together to determine which areas NSTX can make strong contributions to - focusing on the top 2-3.

• You are also asked to discuss and determine the theory & modelling needed to support NSTX contributions, to include/invite the ITPA 'modellers' below, members of the PPPL theory group, and other theorists.

• Any additional diagnostic needs should also be noted.

2. Skinner - SOL and Divertor

Proposed topics from 23 Sept. BP TSG meeting:

- SOL width and turbulence.
- Divertor/wall heat flux during transient events.
- Dust.
 - (PFC contributions limited by lack of W, Be)

1. SOL widths and turbulence:

- IO request (High Priority Research Tasks 2008-2009 ("DRAFT for discussion at ITPA CC 2008"): An improved assessment of <u>SOL width and local power</u> deposition during disruptions / VDEs, including limiter plasmas."
- For FY10 Joule Milestone to "improve understanding of the heat transport in the tokamak scrape-off layer (SOL) plasma", and for SOL physics experiments or theory aimed at ITER:
- Present Joint Experiment DSOL-15 "Inter-machine comparison of blob characteristics" (with some NSTX involvement). Database of blob characteristics in progress for ITPA CC meeting in Dec. 08. Potential extension to investigations of SOL width under discussion.

ASSETS:

- SOL turbulence / blob diagnostics with fast cameras (Magueda/Zweben).
- Fast IR camera λ_q for SOL heat flux width in divertor (Maingi/Gray) Langmuir probes, $\lambda_{Ne} \lambda_{Te}$ both midplane (Boedo) and new LLD probe array (Kallman, Kaita).
- BES, (high-K?)
- Turbulence codes & theory e.g. Lodestar, (D. Ippolito), BOUT (Umansky)...

POTENTIAL NSTX CONTRIBUTIONS (Zweben/Magueda/Stotler/Maingi/Myra/Ryutov):

1. Measurement of blob velocities, sizes, shapes, packing fraction, PDF distribution,

 I_{sat} n_e, T_{e} characteristics.

- Comparison of NSTX blob data with Lodestar SOLT simulations and analytic blob models. 2.
- Measurement of the connection of blobs between midplane and divertor plate. 3.
- Connect measured SOL widths $\lambda_{\textrm{q}}\,\lambda_{\textrm{Ne}}\,\lambda_{\textrm{Te}}$ to turbulence. 4.
- Study of mechanisms for active SOL width modification. 5.

Additional Theory and Modeling Needs ? Additional Diagnostic Needs ?

- Desire a PPPL theorist or postdoc to work on SOL turbulence theory and simulation (in addition to people from outside PPPL).
- Need more TS channels in the SOL.

2. Divertor/Wall Heat Flux during transient events:

- Proposed by SOL/Div ITPA (High Priority Research Tasks 2008-2009 (DRAFT for discussion at ITPA CC 2008):
- "Understand the effect of ELMs/disruptions on divertor and first wall structures." IO request :
- "An improved assessment of SOL width and local power deposition during disruptions / VDEs, including limiter plasmas."

ASSETS:

New fast camera (1.675 kHz full frame, 10 kHz with reduced FoV) will be installed in FY 09 (new ORNL postdoc: T. Gray)

POTENTIAL NSTX CONTRIBUTIONS (Maingi/Gray/Gerhardt):

- SOL power loadings during disruptions, VDEs 1.
- SOL power loadings during ELMs 2.
- SOL power loadings during application of 3-D fields 3
- ٠
- Additional Theory and Modeling Needs? Modeling of effect of 3-D fields on heat flux profile Additional Diagnostic Needs?
- ٠
 - Need wall heat load diagnostics during disruptions

3. Dust

High Priority Research Tasks 2008-2009 (Diagnostics ITPA DRAFT for ITPA CC 2008):

• "Development of measurement requirements for measurements of dust, and assessment of techniques for measurement of dust and erosion"

IO request :

• "Need to test most promising diagnostic approach, a dust microbalance, in a tokamak."

ASSETS:

- Extensive experience with electrostatic dust detector
- Work on novel dust conveyor begun in PPPL/SEAS collaboration
 - will be reported at IAEA.

POTENTIAL NSTX CONTRIBUTIONS:

- 1. First measurements of surface dust in a tokamak.
- 2. Demonstration of remote dust removal in tokamak initial collaborations with SEAS. Discussions begun with Calle at NASA.
- Additional Theory and Modeling Needs ?
 - Dust modeling (e.g. Pigarov) in progress this focus is experimental.
- Additional Diagnostic Needs ?
 - Existing fast cameras useful to corroborate dust measurements.

3b. Potential dust collaboration with NASA

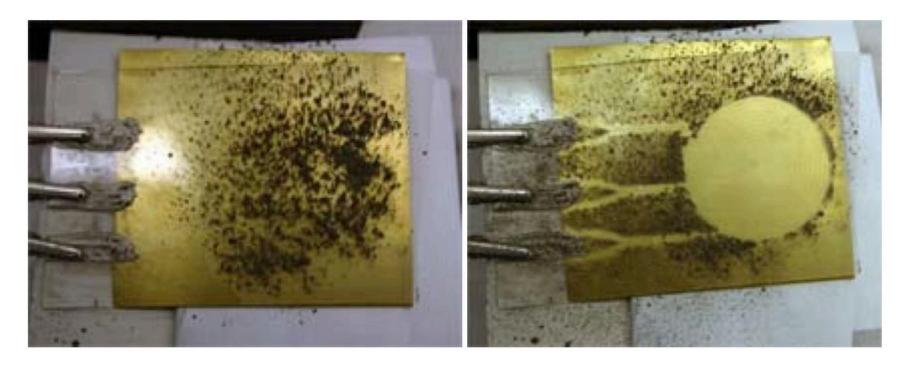


Fig. 4: Before and after photographs of a Dust Shield prototype on a metal plate to provide proof of concept for a sample handling system.

From *"Dust Particle Removal by Electrostatic and Dielectrophoretic Forces with Applications to NASA Exploration Missions"* C.I. Calle et al ESA Proc. Annual Meeting on Electrostatics 2008.

Joint experiments:

ITPA DSOL-15: Inter-machine comparison of blob characteristics

- Database of blob characteristics in progress for ITPA CC meeting in Dec. 08.
- Extension to investigations of SOL width under discussion.

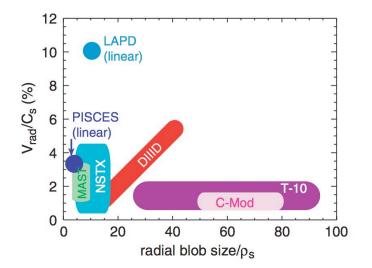


Figure 3. Turbulence filament cross-section size and radial velocity in the SOL normalized to the local ion Larmor radius at the electron temperature, ρ_s , and the local ion sound speed, c_s , for a number of devices.

Lipschultz Nucl. Fus. 47 (2007)1189

ITPA DSOL-17: Cross-machine comparisons of pulse-by-pulse deposition,

- Data shared between NSTX, JET, Asdex-U. Joint reports and individual publications (e.g. on NSTX dynamic retention and Li, JET disruptions, Asdex deposition)....
- Modelling needed to get science out of inter-machine comparisons. WallPSI (Pigarov) a potential possibility.

Backup slides from 23 Sep. mtg.

Boundary Physics Topical Science Group meeting

September 23rd, 2008

Charles Skinner

- "Group discussion of NSTX contributions to the DSOL and PEP ITPA groups, and ITER high-priority research. (DSOL = Divertor and scrape-off layer, PEP = Pedestal and Edge Physics)."
 - Introduction brief history of ITPA
 - List of topics proposed by Div/SOL ITPA and requested by IO.
 - Discussion of potential areas NSTX can contribute to:
 - 1. MAPP (Charles)
 - 2. Diagnostics (Charles)
 - 3. RF erosion (Joel / Charles)
 - 4. ELMs, SOL width (Rajesh)
 - 5. SOL width (Stewart)
 - 6. Blobs (DSOL-15) Ricky
 - 7. Modeling (Daren)
 - 8. All.....

What is the ITPA?

- Genesis is ITER Expert Groups during ITER EDA.
- After US left ITER it was renamed "International Tokamak Physics Activity" to permit US participation without Congressional opposition.
- With ITER construction it has now moved back to the auspices of the ITER IO and is more directly coupled to ITER R&D needs.
- From preamble to new ITPA Charter:
 - "The International Tokamak Physics Activity (ITPA) provides a framework for internationally coordinated fusion research activities. The ITPA continues the tokamak physics R&D activities that have been conducted on an international level for many years resulting in the achievement of a broad physics basis essential for the ITER design and useful for all fusion programs and for progress toward fusion energy generally."
 - "The ITPA will provide support to ITER in the fulfillment of its mission by helping to create a common international research programme organized around scientific issues and will facilitate the participation of the ITER Members in the ITER scientific programme."

What does Div/SOL ITPA do?

- Meets every 6 months. Met last week in Nagasaki September 15-18, 2008. Chaired by Lipschultz and Tsitrone.
- Sessions are organized on high priority topics. For Nagasaki mtg last week these are:
 - Fuel Retention:
 - Ongoing work for ITER predictions talks & discussion
 - H level in today's tokamaks talks & discussion
 - Fuel retention in carbon talks & discussion
 - Fuel retention in metals talks & discussion
 - Campaign integrated dust production and corresponding erosion talks & discussion
 - Dust mechanism, effects and dynamics talks & discussion
 - Wall transient thermal loads talks & discussion
 - Material migration talks & discussion
 - Discussion of new IEA/ITPA proposals
- Joint multi-machine experiments proposed and reported at Div/SOL mtg.

Current NSTX involvement in Div/SOL area

ITPA-IEA_JointX_NSTX-07-input-at-mtg.xls, Dec 4 2006

	ID No	Topical Group	Proposal Title	Keypersons 1	Devices 2	2005 Ext	Туре	NSTX Lead / Actions	NSTX input at meeting
15	DSOL-3	Divertor & SOL	Scaling of radial transport	Bruce Lipschultz, Tony Leonard, S. Lisgo, A. Kallenbach	C-mod, ,MAST, DIII-D,JET, AUG, JT-60U	YES	E	High priority for NHTX; Maingi to discuss with Lipschultz to determine appropriate NSTX contribution, and present XP at Forum	No
16	DSOL-8	Divertor & SOL		Naoko Ashikawa (LHD), Jiansheng Hu (HT-7 and EAST), Volker Philipps (TEXTOR), Volker Rohde (ASDEX- Upgrade)	LHD, HT-7, EAST, TEXTOR, AUG	<u>NEW</u>	E	No	No
17	DSOL-13	Divertor & SOL	Deuterium codeposition with carbon in gaps of plasma facing components	K. Krieger, A. Litnovsky, K. Krieger, C. Wong,Ch. Brousset,B. Lipschultz, G. Counsell	ASDEX Upgrade, TEXTOR, DIII-D, Tore-Supra, C- MOD, MAST	YES	E	No	No
18	DSOL-15	Divertor & SOL		J. Terry (C-Mod), G. Antar (PISCES), N. Asakura (JT-60U), J. Boedo/D. Rudakov (DIII-D), J. Egedal (VTF), W. Fundamenski (JET), O. Grulke (IPP-Griefswald), Albrecht Herrmann (AUG), C. Hidalgo (JET and TJ-II), R. Maqueda/S. Zweben (NSTX), ? (TEXTOR)	TEXTOR, VTF, NSTX, TJ-II, JET, TCV, HT-7, Tore- Supra, AUG, JT-	YES	Ρ	Zweben / broad worldwide participation; Maqueda will present XP at Forum	Yes
19	DSOL-17	Divertor & SOL	Cross-machine comparisons of pulse- by-pulse deposition	C. Skinner, V. Rohde, A Kreter	NSTX, AUG, JET		E	Skinner / congrats for making it after 2-year effort; will present XP at Forum (piggyback)	Yes, Add A Krater from JET to list
20	DSOL-19	Divertor & SOL		A. Loarte, W. Fundamenski, R. Pitts, M. Fenstermacher, M. Groth, A. Herrmann, V. Rohde, D. Whyte, T. Nakano and G. Counsell.	AUG, JET, DIII-D, C-mod, JT-60U, MAST, ?NSTX?			Vlad and Kevin Tritz will discuss with Loarte to determine appropriate NSTX contribution, and possibly present XP at Forum	Considering

What is goal of this meeting?

Menard Email Sept 4th:

- "For each ITPA topical group, the corresponding NSTX ITPA members and TSG leaders are asked to work together to determine which areas NSTX can make strong contributions to - focusing on the top 2-3.
- You are also asked to discuss and determine the theory & modelling needed to support NSTX contributions, to include/invite the ITPA 'modellers' below, members of the PPPL theory group, and other theorists.
- Any additional diagnostic needs should also be noted.
- Once this information is prepared, we will hold an inclusive meeting to discuss the results from all the groups in mid to late September.....
- Draft meeting agenda (TBD):
- 1. Rich will give a 15-20min overview presentation on ITER needs
- 2. Stan can comment on ITPA coordinating committee process/needs (or other)
- 3. NSTX ITPA members present summaries of proposed NSTX contributions (top 2-3) no more than 3 VGs each with ample time for discussion."

Candidate areas ?

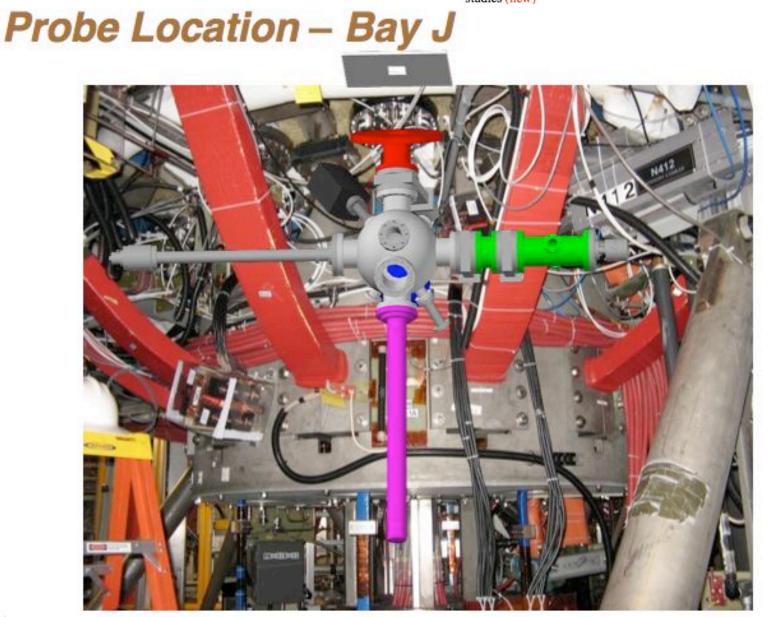
Clear focus on non carbon materials makes JET-ILW most relevant.

High Priority Research Tasks 2008-2009 (DRAFT for discussion at ITPA CC 2008)

	DRAFT for discussion at ITPA CC 2008)
Proposed by SOLDIV TG	Requested by IO
 Improve understanding of Tritium retention and development of efficient T removal methods. Compile high-Z experience regarding hydrogenic retention in tokamaks and laboratory studies (new) Initiate studies on neutron damage and how that leads to T retention (new) High-Z operational experience (compatibility with core) Exposure of tungsten to He fluence and effects on surface properties (new) Characterize the level and processes involved in RF enhancement of erosion (new) Z. Joel result 	 A range of PWI issues will need to be resolved to build confidence that reliable operation can be sustained: establishment of requirements for carbon/ carbidic compound removal at divertor changeout (eg need to identify distribution of redeposited material) T-retention in W/ Be and their compounds, including neutron irradiation effects tungsten/ beryllium material damage and dust production rates (steady-state, transients) performance of Be-coated tungsten PFCs development of modelling capability for beryllium and tungsten PWI simulation When are results required? early quantitative information on key safety-related questions (T-retention, dust production) would be important - ie 2-3 years 1. MAPP possibilities should aim for a complete picture of W/ Be PWI issues on 5 year timescale
 Understand the effect of ELMs/disruptions on divertor and first wall structures Exploration of the effect on the SOL and power loadings of ELM mitigation (ongoing) 3. Rajesh Study runaway effects in disruptions and how to nullity them (new) 	 a validated modelling capability for runaway generation and loss to allow an improved assessment of first wall energy deposition An improved assessment of SOL width and local power deposition during disruptions/ VDEs, including limiter plasmas improved data and analysis within the next 12 months would be valuable 3. Rajesh, 4. Stewart, 5. Ricky
 Improve measurements & understanding of plasma transport to targets and walls to better predict heat loads and effects on the core Code-code comparisons including impurities - specifically carbon (underway) Identify discrepancies between codes and experiment for SOL and divertor (new) 	
 High-Z operational experience (compatibility with core) Exposure of tungsten to He fluence and effects on surface properties (new) Characterize the level and processes involved in RF enhancement of erosion (new) 	

MAPP = Materials Analysis Particle Probe New collaboration with JP Allain - Purdue. Validation of safe levels of dust in the vacuum vessel required for machine operation

Compile high-Z experience regarding hydrogenic retention in tokamaks and laboratory studies (new)

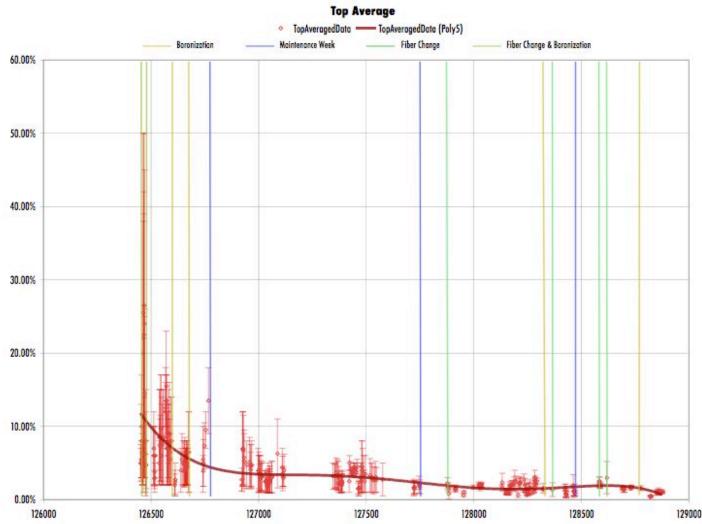


Proposed by Diagnostics TG	Requested by IO	
Development of methods of measuring the energy and density distribution		
of confined and escaping α 's. Assessment of the calibration strategy for the neutron diagnostics and the		
calibration source strength needed.		
Determination of life-time of plasma facing mirrors used in optical systems.	 Plasma facing mirrors and optical elements in divertor: these components will likely have a finite lifetime, which will impact on reliability of optical diagnostics 	Laser
	 several developments are needed, including models of erosion and deposition process 	mirror
	 mitigation of erosion and deposition by design development of shutters and baffles in-situ calibration techniques 	cleaning
	 deliverable: recommendation on most rugged first mirror arrangement for ITER, based on experiment and modelling 	proposed.
Development of measurement requirements for measurements of dust, and	 timescale: several years Dust: Timescale: 2-3 years (by end-2011) 	
assessment of techniques for measurement of dust and erosion.	 Validation of safe levels of dust in the vacuum vessel required for machine operation 	Continue to
	 Need to test most promising diagnostic approach, a dust 	
Roquemore / Pigarov?	 microbalance, in a tokamak Deliverable: develop, install, exploit and report on prototype 	develop dust
Skinner?	 Hot Dust: Timescale: 1.5 years (end-2009) Be dust on PFCs with T > 600°C must be limited to <6kg for 	detector /
	 safety Need to evaluate feasibility of significant quantities of dust being 	remover
	able to survive in regions of high heat flux using modelling and survey data from existing tokamaks	
	 Deliverable: provide an estimate of the quantity of hot dust in ITER, including quantitative evaluation of uncertainties 	
	 Divertor Erosion: timescale: 2-3 years (by end-2011) divertor erosion is expected to be the major source of impurities 	
	 and dust, as well as limiting the divertor lifetime need to test most promising diagnostic approach to remote 	
	divertor erosion measurements, based on laser ranging techniques	
	deliverable: develop, install, exploit and report on prototype remote divertor erosion measurement diagnostic using laser	
	 ranging Hydrogen background: 	
	 In non-active phase, outgassing and DNB will affect base level of hydrogen, influencing fuel retention studies 	Report
	 need to assess evolution of hydrogen levels in existing 	· · · · · · · · · · · · · · · · · · ·
	 devices and develop an appropriate model deliverable: estimates of hydrogen background in initial 	NSTX
	phase of ITER operation and assessment of required	11/D motio
	accuracy of gas balance measurements to permit analysis of fuel retention in non-active phase	H/D ratio
	o timescale: 2-3 years	
	Retained tritium:	
	 require validated techniques for assessing level of retained tritium in vacuum vessel 	
	 need to select and prototype candidate diagnostic techniques, 	
	including extrapolation from local to global measurements;	
	 also need to determine accuracy in estimate of T-burnup deliverable: validated technique for estimation of retained 	
	tritium in vacuum vessel	
	o timescale: 2-3 years	15/

Diagnostics request by IO:

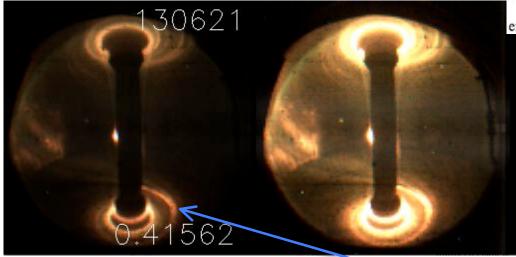
Preliminary NSTX 2008 data (John Robinson)

- Hydrogen background:
 - In non-active phase, outgassing and DNB will affect base level of hydrogen, influencing fuel retention studies
 - need to assess evolution of hydrogen levels in existing devices and develop an appropriate model
 - o deliverable: estimates of hydrogen background in initial

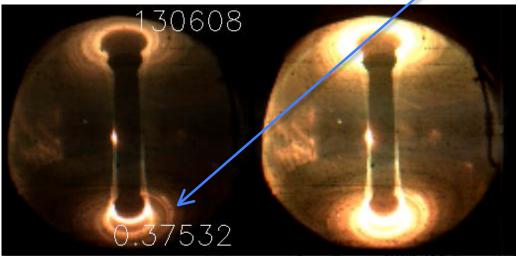


RF Effect Seen Outside Divertor Strike-point J. Hosea

Phase = - 90° just prior to arc before elm



Phase = -150° just prior to arc before elm



 $P_{RF} \sim 1.8 \text{ MW}, I_{P} = 1 \text{ MA}$

Characterize the level and processes involved in RF enhancement of erosion (new)

Searching for edge RF power loss processes

- RF interaction is localized toroidally
 - Appears to be linked with antenna along field lines
- Intensity may be dependent on phase
- Dies away after RF is removed
 - decay in 15 20 msec

Next campaign

- Need to measure heating with infrared camera and thermocouples to deduce RF power lost
- Need RF power and phase scans to see if power lost correlates with observed heating efficiency 17/6

Physics Questions on the SOL Width

S.J. Zweben, R.J. Maqueda, D.P. Stotler, et al

For FY10 Joule Milestone to "improve understanding of the heat transport in the tokamak scrape-off layer (SOL) plasma", and for SOL physics experiments or theory aimed at ITER:

- 1) What are the possible mechanisms which could affect the divertor heat flux SOL width ?
 - SOL turbulence (both ES and EM)
 - MHD (e.g. ELMs) and magnetic error fields
 - convective cells (e.g. with RF SOL fields),
 - neoclassical transport and parallel flows

SOL Physics Questions...cont...

2) What models will be useful to explain the experimental results on the heat flux SOL width ?

- BOUT, SOLT, XGC computational models
- blob models, i.e. nonlinear analytic models
- quasilinear / marginal stability estimates
- 3) What are the potential methods to control the this SOL heat flux width ?
 - modified magnetic divertors (e.g. ergodic, super-X)
 - convective cell generation (electrodes or RF)
 - radial electric field generation (e.g. as in TdeV)
 - radiative / detached divertors (via atomic physics).

SOL Physics Questions...cont...

- 4) More specifically, we need to understand from theory:
 - what is the relationship between the SOL width for density and temperature (both ion and electrons) ?
 - what is the relationship between the SOL widths at the midplane to those at the divertor plate ?
 - how do these widths scale with the dimensionless parameters of the SOL (e.g. C, β , ρ , M, etc) ?
 - does the SOL width also depend on the plasma inside the separatrix, e.g. through blob formation ?

Things that are probably NOT useful

- "Empirical scaling" of SOL widths from existing machines to ITER, since there is no physics basis for them, and they will be highly unreliable and misleading
- Applying existing theory and/or simulation to predict the ITER SOL, unless these have been validated using data on SOL turbulence and transport from existing machines (not yet done in any case !)
- Applying validated theory or models to situations in which they are NOT valid, e.g. collisional SOL models from present experiments to a collisionless ITER SOL

ITPA DSOL-15: Inter-machine comparison of blob characteristics R. J. Maqueda, S. J. Zweben

Understanding will only be gained through comparison with theoretical/numerical modeling

Numerical/theoretical models:

- **BOUT (LLNL)**: code development has been carried out for very many years, yet results for NSTX are <u>very</u> limited ...if any.
- **SOLT (Lodestar)**: new effort, considerable recent progress. Incorporates "synthetic GPI" using atomic emission tables from D. Stotler.
- Blob model (Lodestar): model for radial blob propagation in SOL. Good initial match with NSTX experimental data. Nevertheless, "comparison space" is limited -> "blob regime" physics still unchallenged by experimental data.
- **Other models** (B. Scott, etc): "good physics" but limited scope yield limited direct applicability to experimental data.

Model – experiment comparison based on "statistical" characterization and not "event" characterization

Possible blob measurements and characterization

- Blob velocities: blob radial and poloidal components. Are different blob regimes seen?
- Blob sizes: both radial and poloidal size.
- **Blob shape:** how to compare statistically?
- Packing fraction.
- **PDF distribution**: skewness and kurtosis "holes"?
- **Poloidal extension of blobs**: correlation with measurements in divertor region.
- I_{sat} characteristics (reciprocating probe), flows within blobs?
- **n**_e, **T**_e characteristics: diagnostic set?

Possible blob measurements and characterization (cont.)

- **Connection to edge turbulence** ($\tilde{n}, \tilde{\phi}, S(k,w), etc$) -> origin of blobs.
- Connection to SOL flows: zonal flows?
- Connection to: MHD modes, interchange/ballooning?
- Connection to "near edge" turbulence: BES.

NSTX can provide valuable "challenging" data points for benchmark of modeling codes

Experiments with full range of NSTX's parameter space will be of additional value for this effort

Improve understanding of plasma transport: SOL & Divertor Discrepancies

- Referring to PSI 2008 presentations?
- E.g., talk by Chankin & Coster:
 - Connected simulations of SOL flow to problems in matching divertor T_e & n_e profiles,
 - And to discrepancies in E_r profiles.
 - Related by Pfirsch-Schluter formula.
 - Main conclusion: codes may be getting parallel electron transport wrong.

Possible NSTX contributions to ITPA in boundary physics

R. Maingi Boundary Physics Topical Group Meeting Sept 23, 2008





NSTX can contribute steady and ELM heat flux profile analysis for DSOL group

Understand the effect of ELMs, disruptions on divertor and first wall structures

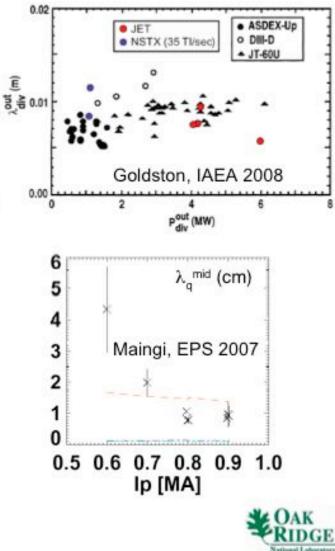
- Exploration of the effect on the SOL and power loadings of ELM mitigation (ongoing)
 - ITER IO wording: An improved assessment of SOL width and local power deposition during disruptions/ VDEs, including limiter plasmas (RM note: this doesn't mention ELMs)
 - NSTX can contribute SOL power loadings during disruptions, ELMs, during 3-d field application with new fast camera
- Study runaway effects in disruptions and how to nullify them (new)
- New question (RM): how is heat flux carried across separatrix, i.e. split between electron and ion channels, steady vs. transient, LFS vs. HFS?



ST steady heat flux patterns show similarities and differences with higher R/a devices

- NSTX λ_q^{mid} comparable to higher R/a devices when computed with Loarte's formulation
 - Caution: other devices have little/no gas puff during H-mode phase, whereas NSTX is using ~ 35 torr-I/s (HFS)
 - Gas puff rate itself not important parameters - maybe SOL v* better?
- NSTX λ_q^{mid} large compared with high R/a collisional scalings from Counsell, Connor
 - > Large ion gyro-radius -> min. SOL λ_q^{mid} ?
 - > How does turbulence impact λ_q^{mid} ?
- New fast camera (1.675 kHz full frame, 10 kHz with reduced FoV) will be installed in FY 09 (new ORNL postdoc: T. Gray)

Mostly piggyback - lots of analysis



Applicable NSTX Modeling / Theory

- Pigarov / UCSD,
 - WallPSI simulation of retention experiments?
 - UEDGE modeling of LLD & lithium behavior.
 - Dust modeling?
- BOUT?

٠

- Is it applicable? C-Mod is a better target.
- Who would run it?
- XGC-0
 - Now being applied to NSTX.
 - Is not yet comprehensive (i.e., not a UEDGE replacement).
- DEGAS 2
 - Being coupled into XGC-0,
 - Will address "role of neutral penetration length on pedestal density width",
 - And "incorporate comprehensive neoclassical and turbulence transport models into pedestal simulation codes".
 - And will be preparing for LLD.

Big Picture Comments

- ITPA tasks loaded with V&V related concepts,
 - Discussion of validation tools & nomenclature from a fusion perspective: P. W. Terry et al., *Phys. Plasmas* 15, 062503 (2008).
- Use the ECC to coordinate theory / modeling related to these ITPA tasks?

– As well as the FY10 Joule milestone.

- What's the role of the BPO here?
 - Rognlien envisions an ECC / BPO meeting to focus on FY10 milestone.

Discussion: Need to pick 2-3 areas for Jon

Selection criteria - as discussed with Stan K. :

- 1 Outstanding science
- 2 Can collaborate with other institutions
- 3 Requested by IO
- DSOL-15: Inter-machine comparison of blob characteristics? (modelling needed to get science out of comparisons)
- DSOL-17: Cross-machine comparisons of pulse-by-pulse deposition, (modelling needed to get science out of comparisons, carbon not relevant for ITER-DT)
- Materials probes: MAPP (NSTX)/ DiMES- MiMES (DIII-D) / S³ (C-mod) / TEXTOR....
- Dust ingress: MAPP / fast cameras (NSTX); DiMES fast cameras (DIII-D)
- Dust detection: NSTX / Tore Supra
- RF enhanced erosion. NSTX / C-mod...
 - (ready for prime time ?)
- SOL width
- Blobs
- Others.....

Theory and Modeling needed in support ?

Additional diagnostics needed ?