

# Development of Advanced Spherical Torus Operating Scenarios in NSTX

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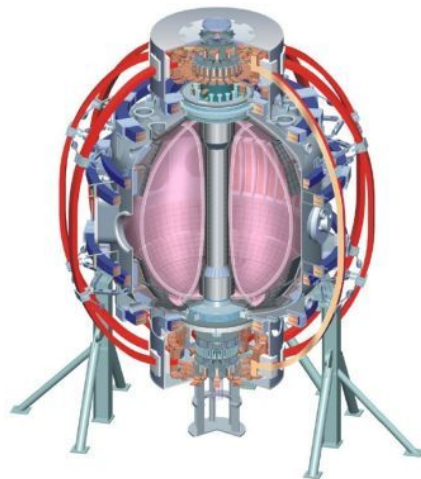
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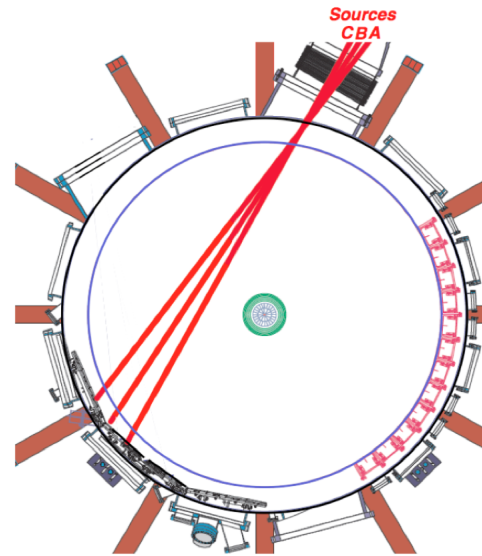
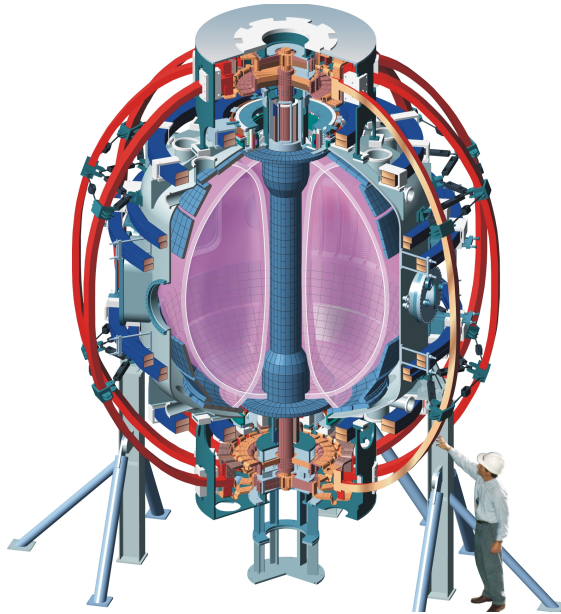
*Culham Sci Ctr*  
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*U Quebec*

# The mission of NSTX is to demonstrate high $\beta$ scenarios consistent with steady state operation

- Overview of steady state operation requirements
- Global performance studies
- Confinement scaling studies
- Current profile analysis
- Global Stability and non-axisymmetric control
- New plasma control capabilities
- Summary

# NSTX and Steady State Operation Research

# NSTX is a Medium Sized Spherical Torus With Significant Capabilities for High- $\beta$ Scenario Research



## 3-D Field Coils Important For Scenario Development

Pre-programmed  $n=3$  correction  
 Main VF coil is not a perfect circle  
 $n=1$  feedback system  
 Internal  $B_R$  and  $B_P$  sensors  
 Slow response: error field correction  
 Fast response: RWM control  
 Now testing state-space RWM controller.

Aspect ratio A	1.27 – 1.7
Toroidal Field $B_{T0}$	0.35 – 0.55 T
Plasma Current $I_p$	$\leq 1.4$ MA
NBI (<100kV)	7 MW

**Lithium conditioning of PFCs via a dual evaporator system.**

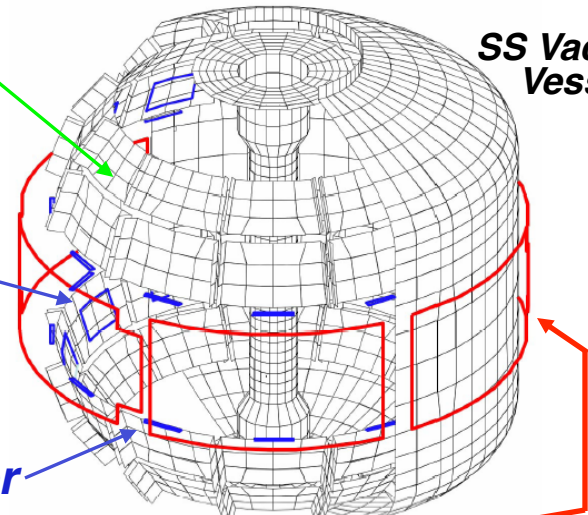
Copper passive conductor plates

SS Vacuum Vessel

$B_R$  Sensor

$B_P$  Sensor

6 ex-vessel midplane control coils



# Plasma shaping and $\beta_N$ are determining factors for performance of steady-state burning plasmas

- Reactor designs aim at the highest shaping that technology can support
- High shaping (S) requirement for AT-like scenarios comes from self-sustainment at high  $\beta_t$

- Define sustained  $\beta$  figure of merit:

$$\beta_{sus} \equiv f_{bs} \beta_t \sim S \beta_N^2$$

- (high  $\beta_t$  is required to compete with TF losses)

- **Indicates need to optimize both  $\beta_N$  and  $S$**
- **For NSTX simulations indicate possible 100% non-inductive operation with  $\beta_N \sim 7$  only with strong shaping**

**Shape factor definitions:**

$$\varepsilon = a/R_0$$

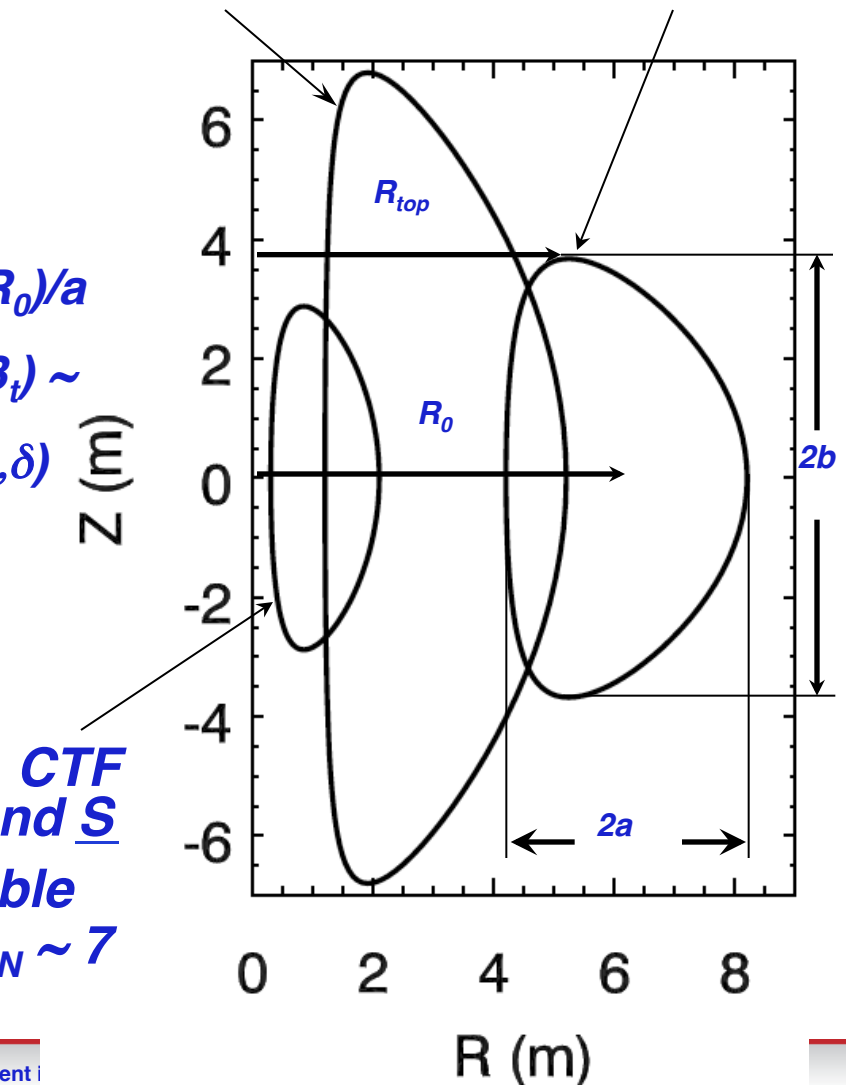
$$\kappa = b/a$$

$$\delta_{top} = (R_{top} - R_0)/a$$

$$S = q_{95}(I_p/aB_t) \sim (1+\kappa^2)f(\varepsilon, \delta)$$

**ARIES-ST**

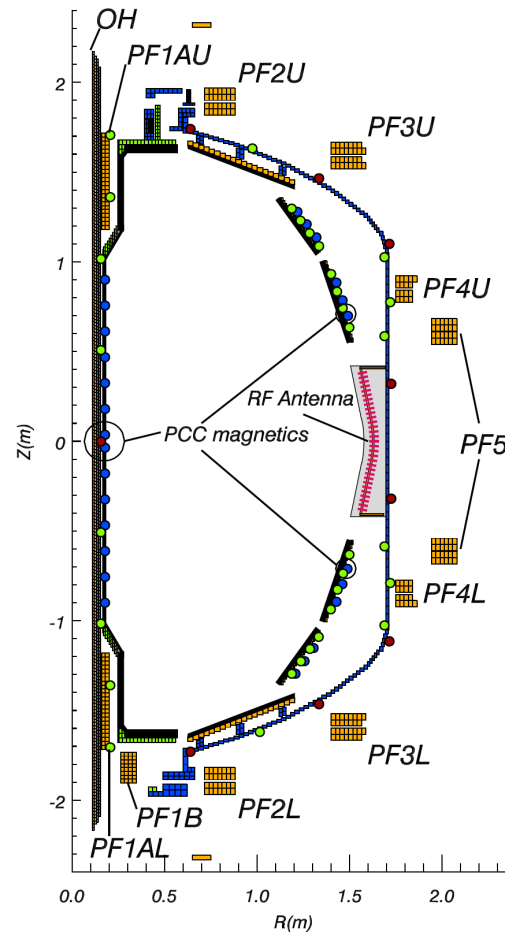
**ITER**



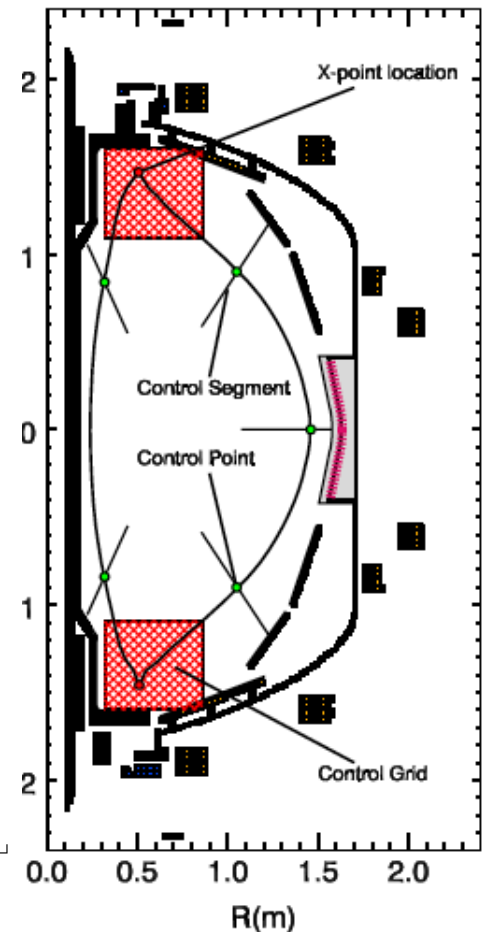
# rtEFIT+isoflux controls shape

- Measure magnetic fluxes, fields, coil and vessel currents
- Solve GS equation
  - Assume a polynomial for the current and pressure profiles
- User defines plasma boundary
- Control segments chosen to intersect boundary
- Control point is intersection of segment and boundary
- Control difference between x-point and control point flux

*Location of measurements used in real time reconstruction*



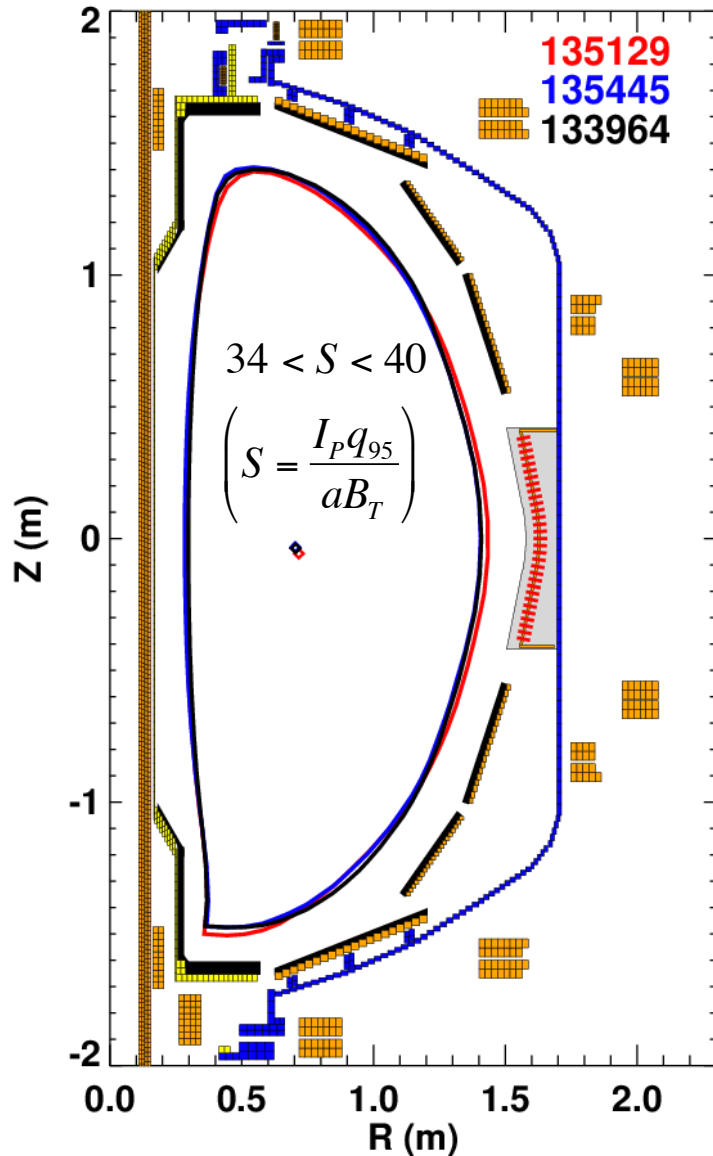
*Geometry of control grids and segments*



# Global Performance Studies



# High-Elongation Configurations Developed to Challenge Limits in $\beta_T$ , Non-inductive Current Fraction and Sustainment



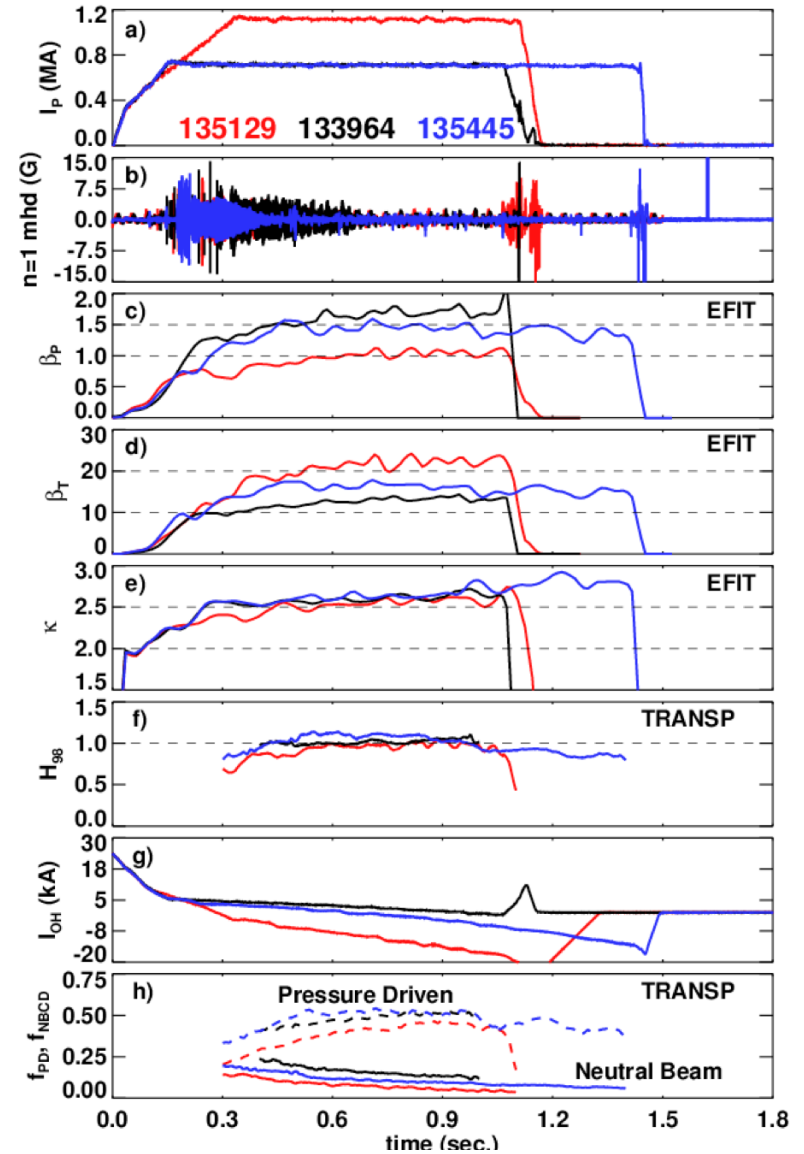
$$q^* = \frac{\varepsilon(1 + \kappa^2)\pi a B_{T0}}{\mu_0 I_p}$$

**High- $\beta_T$**   
 $q^*=2.8$   
 $B_T=0.44 \text{ T}$   
 $I_p=1100 \text{ kA}$

**Long Pulse**  
 $q^*=3.9$   
 $B_T=0.38 \text{ T}$   
 $I_p=700 \text{ kA}$

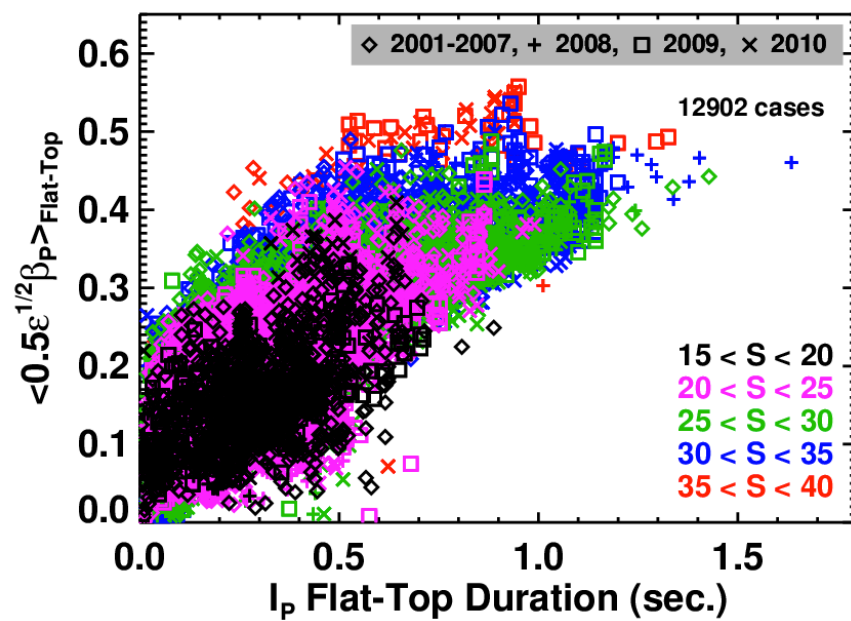
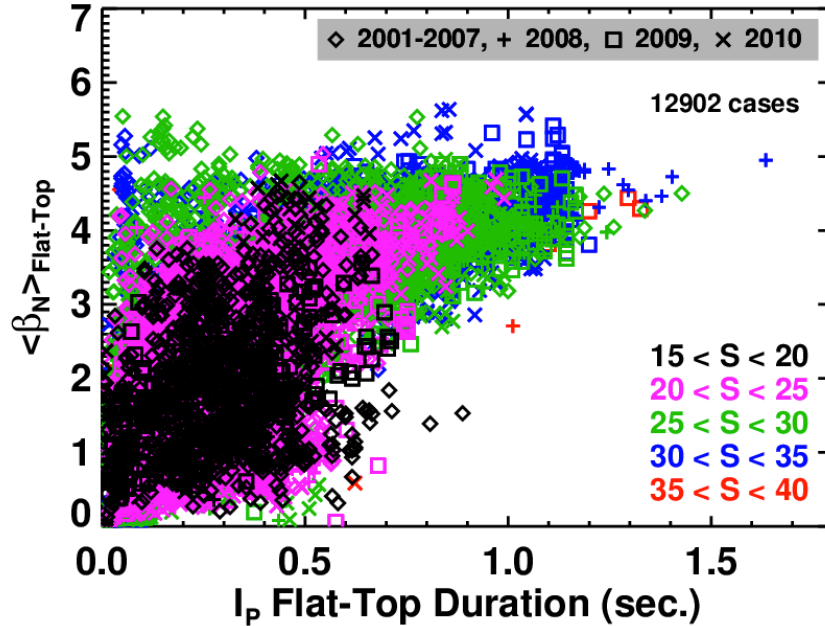
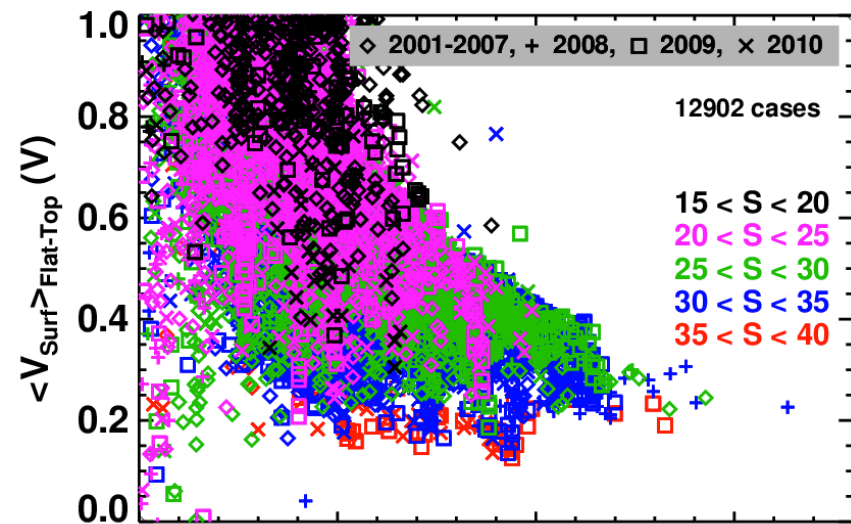
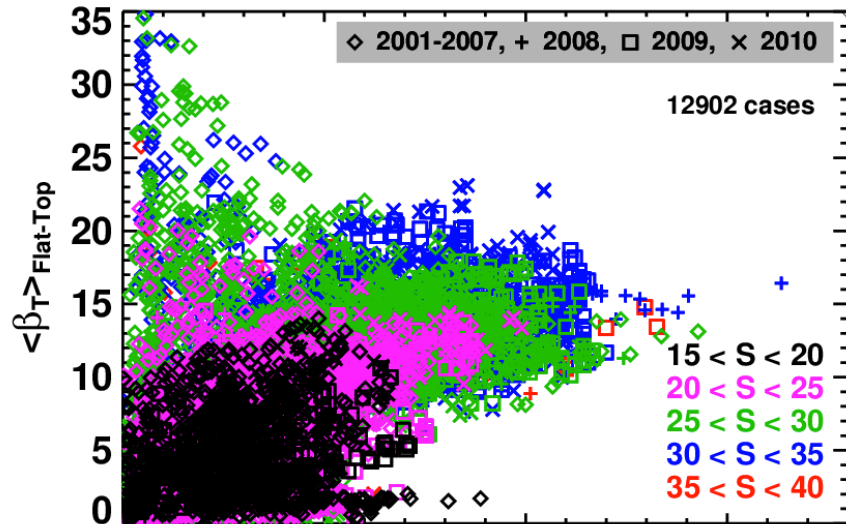
**High- $\beta_p$**   
 $q^*=4.7$   
 $B_T=0.48 \text{ T}$   
 $I_p=700 \text{ kA}$

**All**  
 $H_{98} \geq 1$   
 $\kappa = 2.6-2.7$

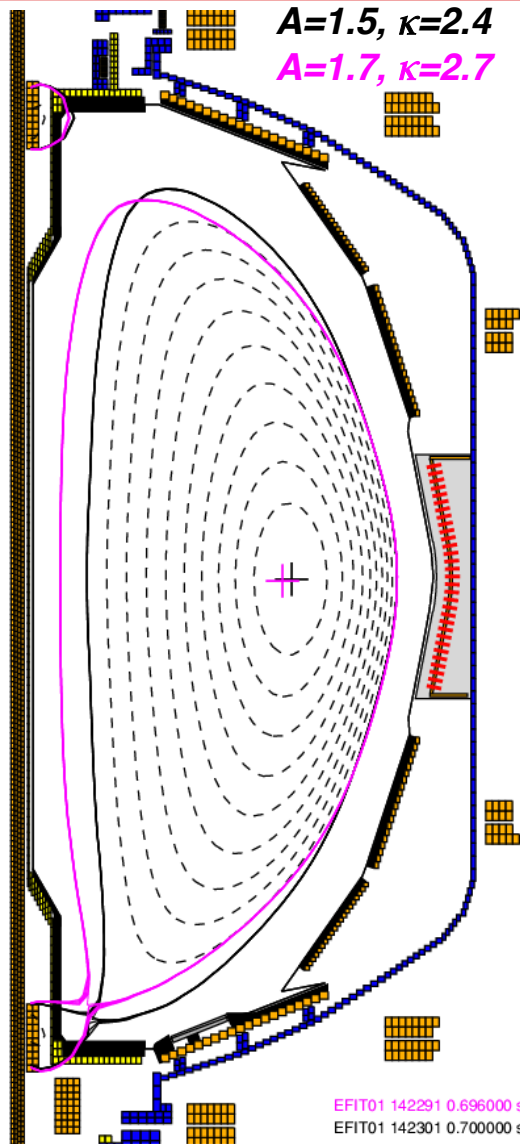




# Strong Shaping has Helped NSTX Make Continued Progress on a Range of Optimization Targets

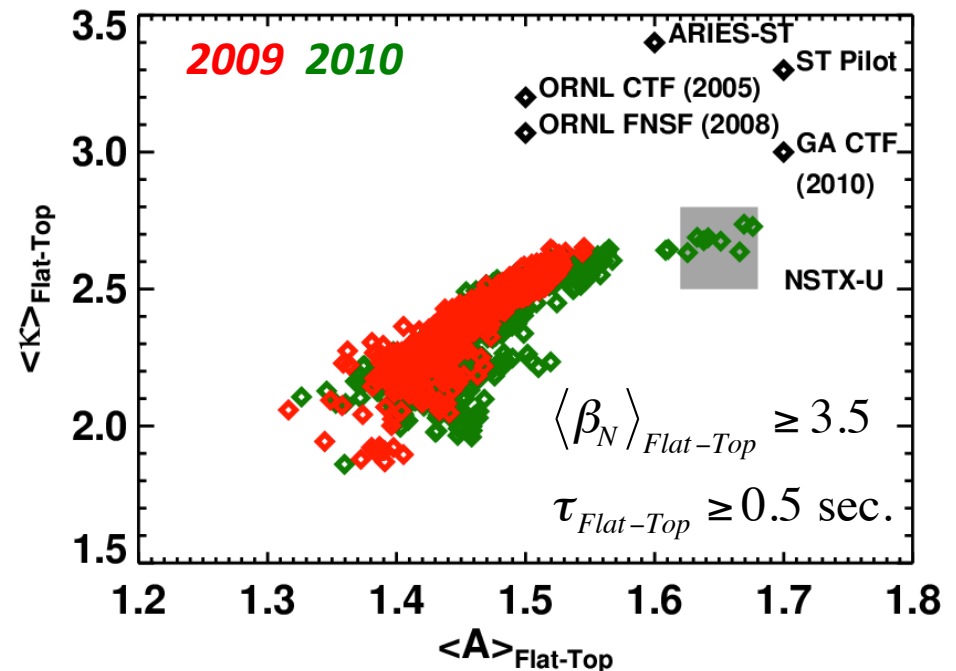


# Next-Step STs Motivate Study of Larger Aspect Ratio Plasmas



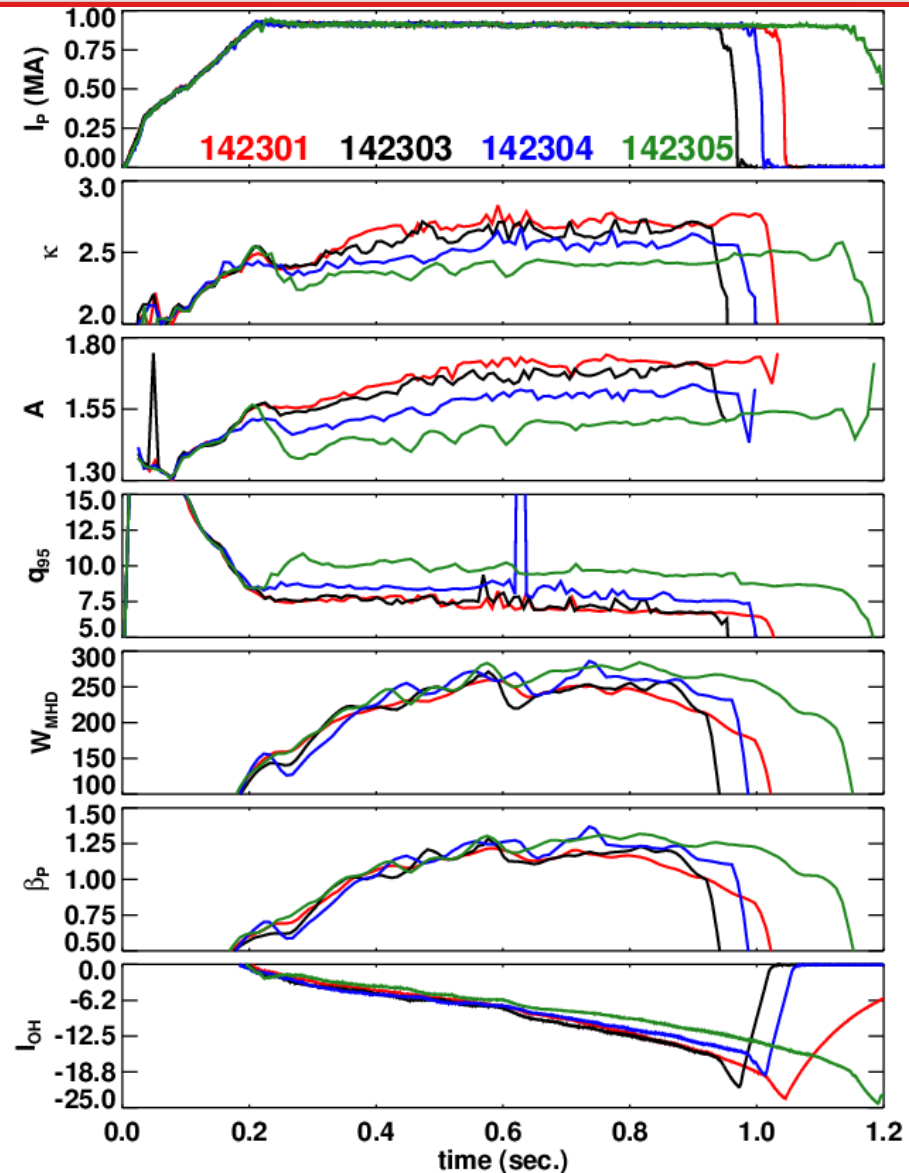
- Recent ST studies looking at higher aspect ratio.
  - NSTX-Upgrade
  - NHTX
  - GA versions of ST-FNST
  - PPPL ST Pilot Plant
- Likely deleterious to both  $n=0$  and  $n=1$  stability.
- Achieved in NSTX by two steps:
  - Increase the inner divertor coils current to max. level
  - Increase the elongation.

Typical high-performance plasma boundary.  
 Higher aspect-ratio boundary.



## Initial Studies of Increasing the Aspect Ratio Show No Strong Degradation

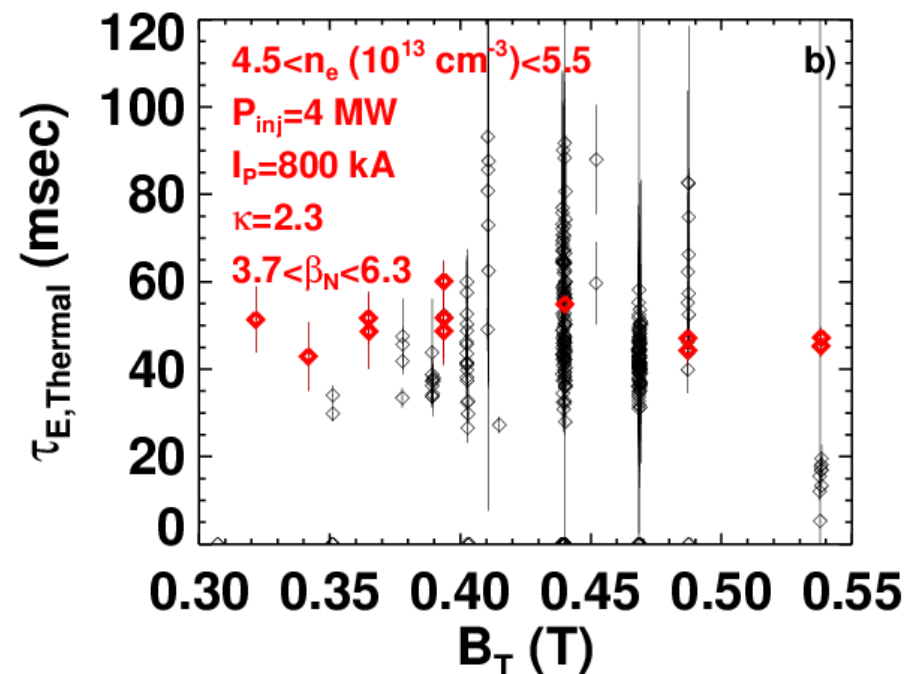
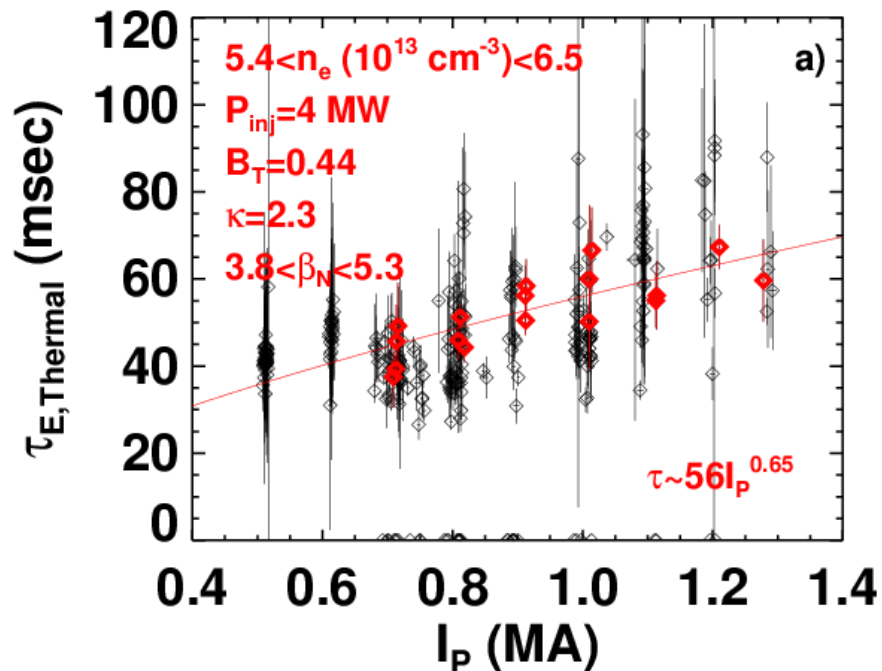
- Increased elongation does not compensate increased aspect ratio in edge  $q$ .
- May be some degradation of confinement
- Demonstrates the existence of a viable NSTX-U operational scenario



# Confinement scaling

# Dedicated Scans Show Confinement Trends in Lithiumized High-Performance Plasmas

- Dedicated scans as part of the 2010 JRT on SOL physics.
  - Red below, black is full database.
- $I_p$  scaling intermediate between ITER-98 and previous NSTX.
- $B_T$  scaling is very weak.
- Difference due to Lithium, collisionality?



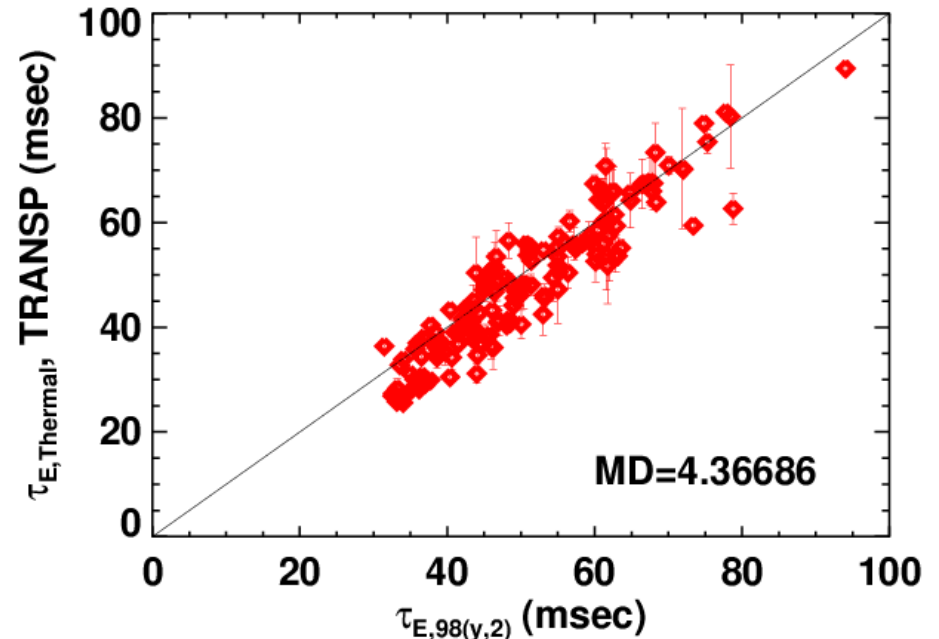
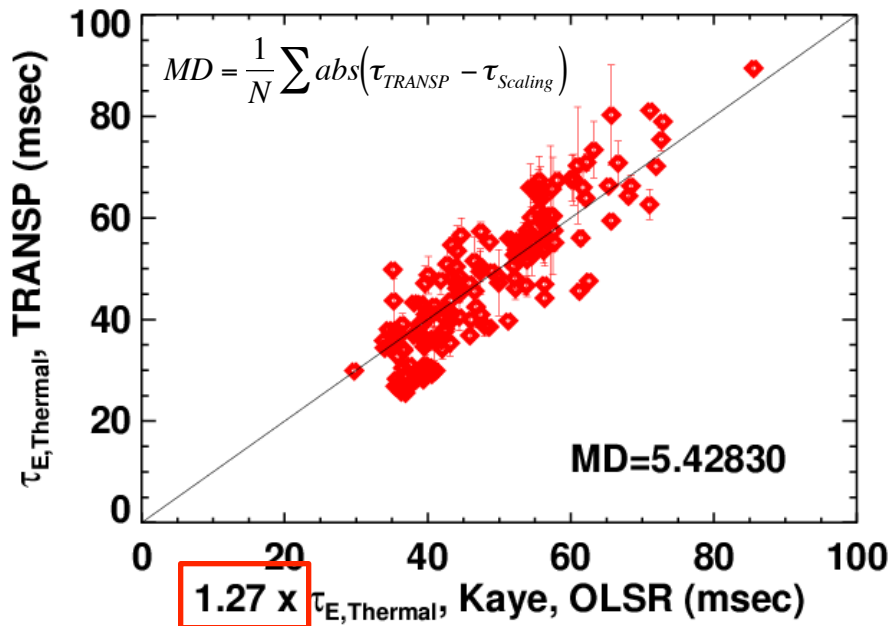
# Lithiumized Discharges Shows Confinement Scaling Similar to Higher Aspect Ratio

Consider > 75 msec averaging windows, at least one current diffusion time into the  $I_p$  flat-top, at high- $\kappa$  and  $\delta$ , in lithium conditioned discharges

Criterion excludes many high-confinement discharges

$$\tau_{E,th,Kaye,OLSR} \propto I_P^{0.57} B_T^{1.0} n_e^{0.44} P_{abs}^{-0.73}$$

$$\tau_{E,th,ITER-98} \propto I_P^{0.93} B_T^{0.15} n_e^{0.41} P_{abs}^{-0.69} \kappa^{0.8}$$



- Confinement exceeds previous low-A scaling by ~30%.
  - Lithium conditioning, strong shaping, higher  $\beta_N$  and longer-pulse duration.
- Working to revise ST-scalings for  $\tau_E$  in this class of discharge.



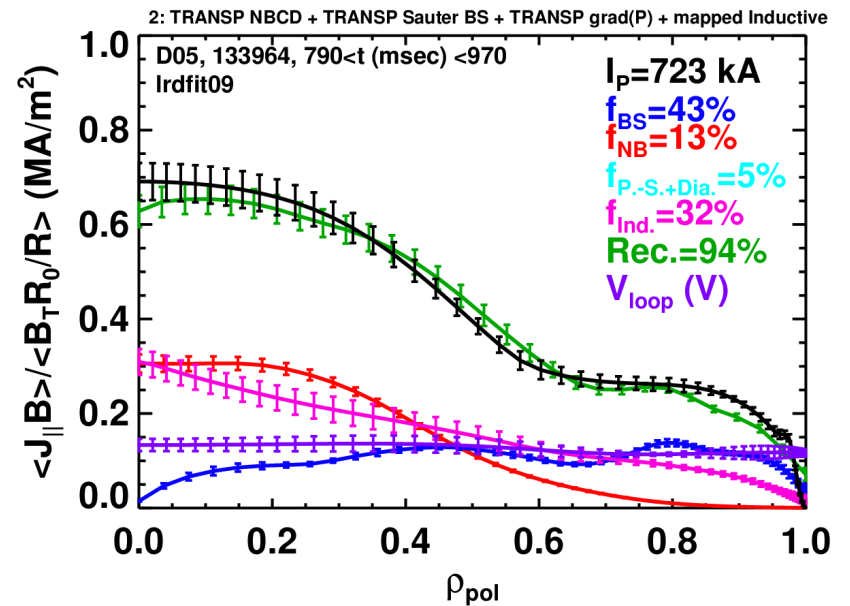
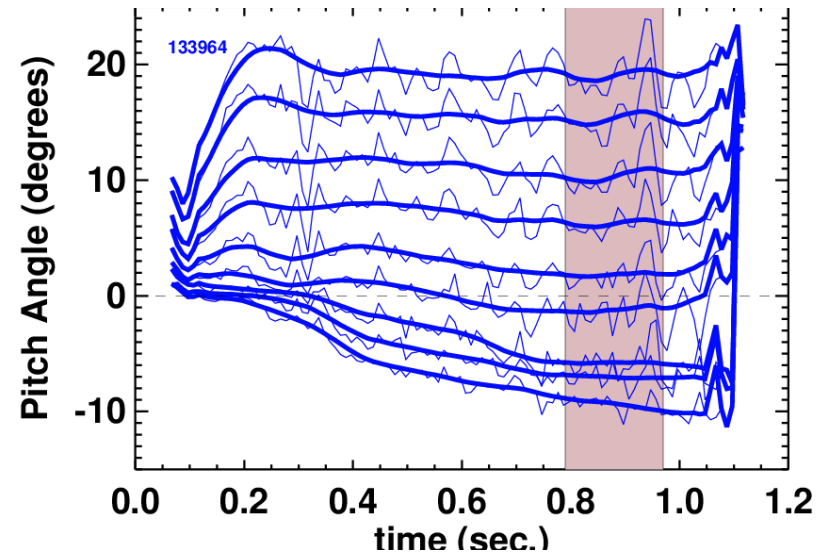
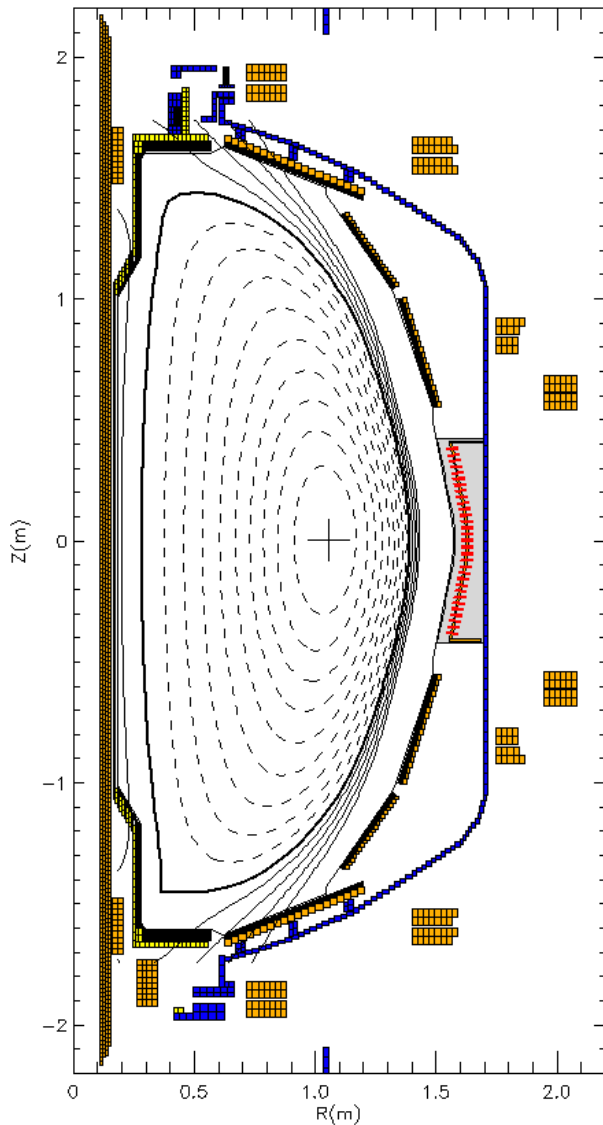
# Current Profiles and the Non-Inductive Fraction

# J( $\rho$ ) Profile Record of Low $V_{loop}$ Shot Can Be Understood Without Anomalous Fast Ion Diffusion

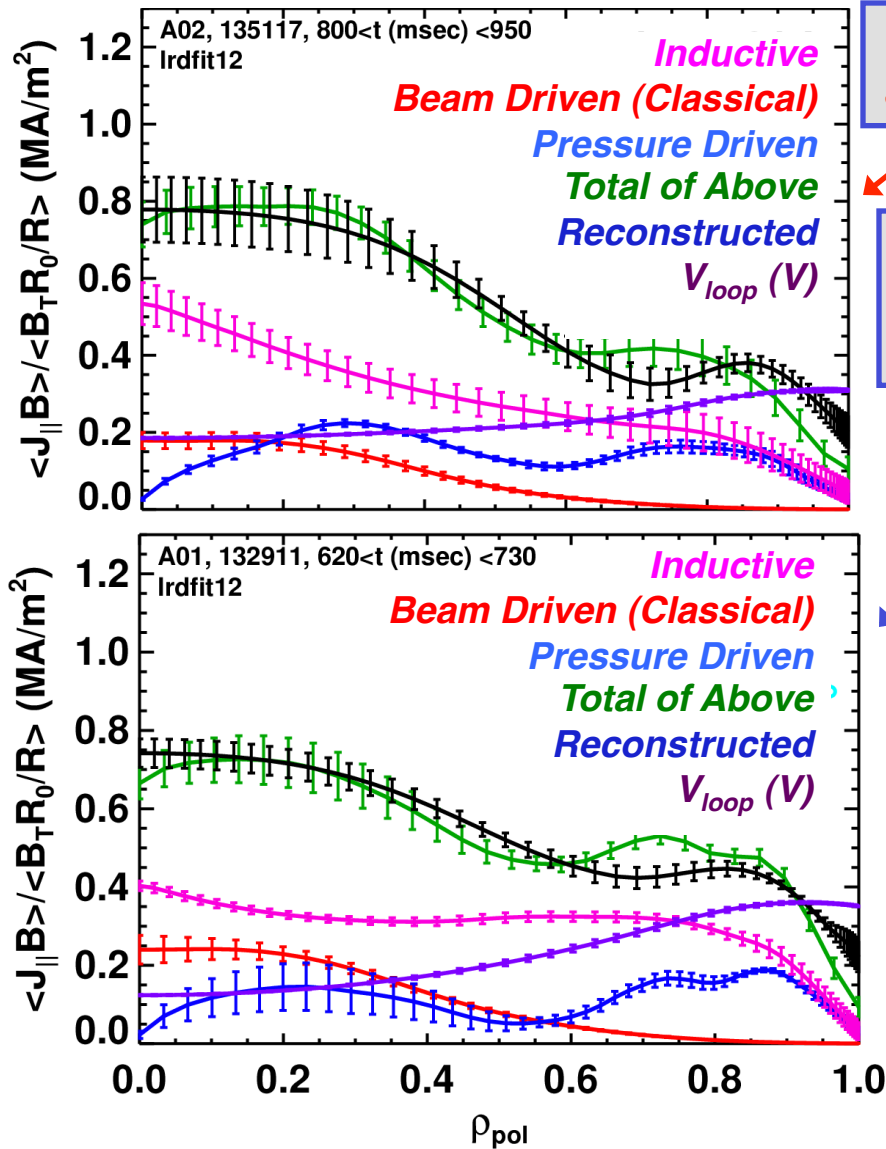
$\kappa=2.6$   
 $I_p=700$  kA  
 $B_T=0.48$  T

Near Fully Evolved Pitch Angles

Good Agreement Between Reconstructed Current Profile and the Constituent Sum



# Current Profile Reconstructions Have Been Done For a Wide Range of Plasmas

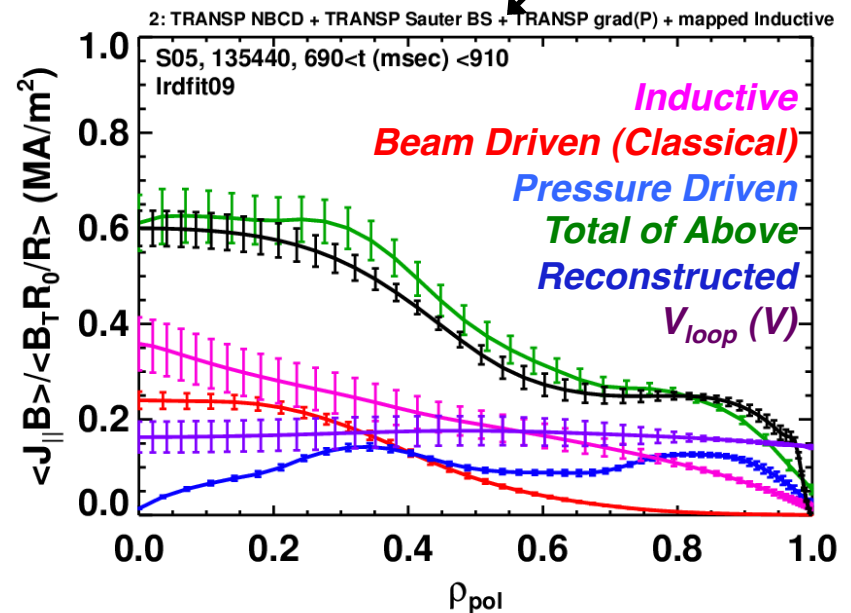


1100 kA, Optimized For Sustained High  $\beta_T$

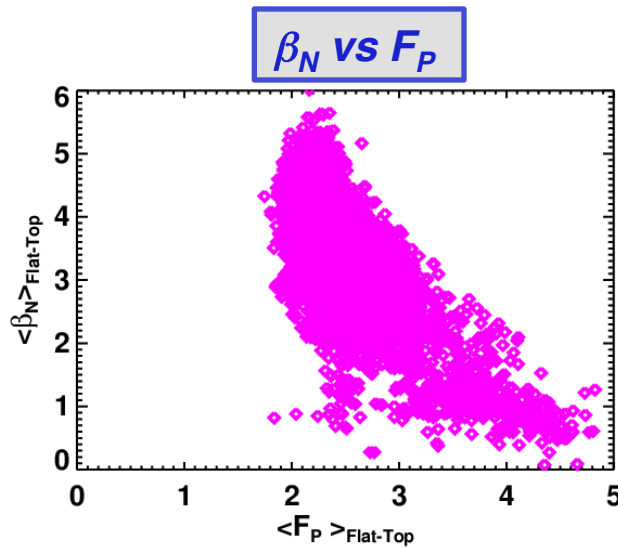
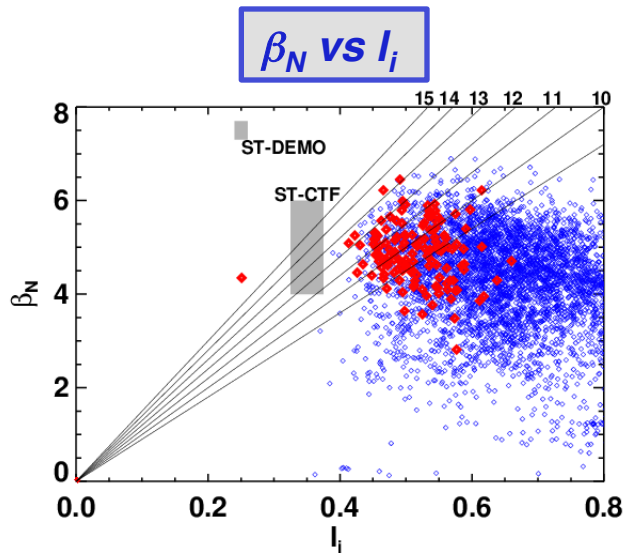
1300 kA, Optimized For Sustained For Large Stored Energy

All analysis during MHD free periods, with no anomalous fast ion diffusion.

700 kA, Optimized For Long Pulse



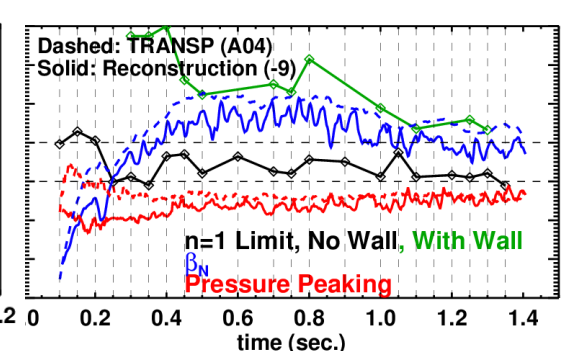
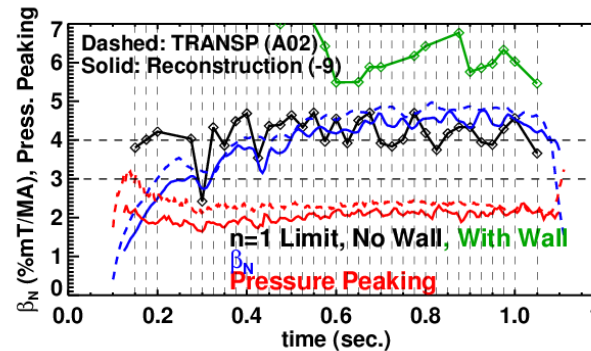
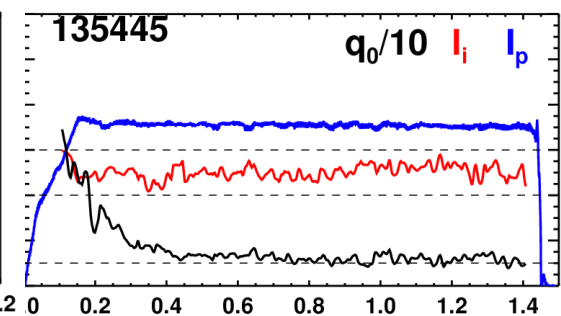
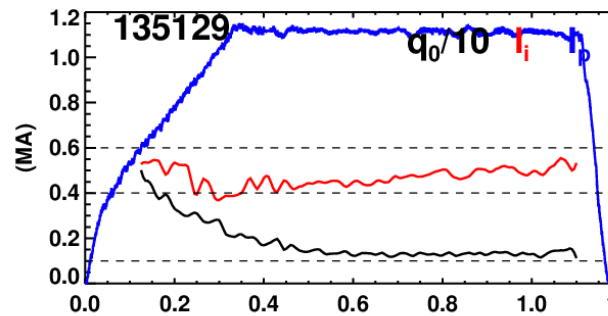
# No-Wall $\beta_N$ Limit Can Vary Widely Depending on Profiles; Best Shots Near With-Wall Limit



- MSE constrained equilibria using EFIT code.
- Use CHEASE to scale the pressure profile.
- DCON to evaluate n=1 no- & with-wall limits.
- Repeat calculation for many times during discharge.

**135129**  
High- $\beta_T$  target  
 $I_p=1100$  kA,  $B_T=0.45$  T

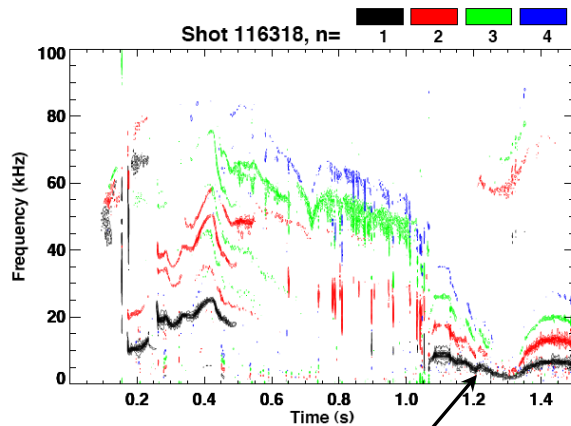
**135129**  
Long Pulse Target  
 $I_p=700$  kA,  $B_T=0.38$  T



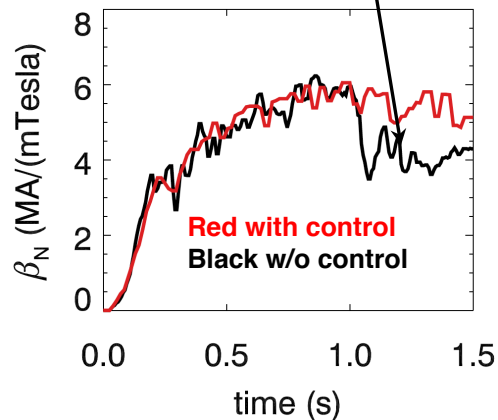
# Global Stability and non-axisymmetric control

# n=1 RFA/RWM control combined with n=3 error correction increases $\beta$ and extends pulse

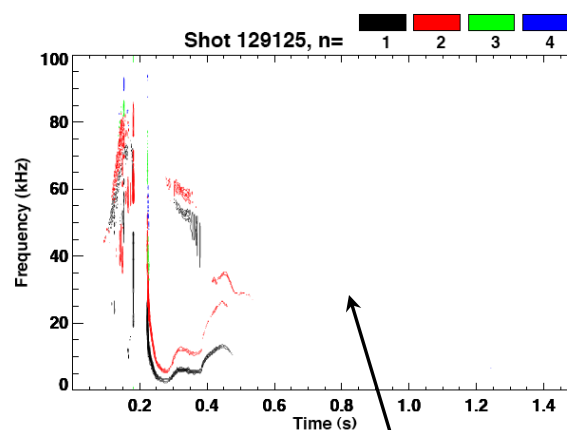
**MHD spectrogram w/o n=1 feedback and n=3 correction**



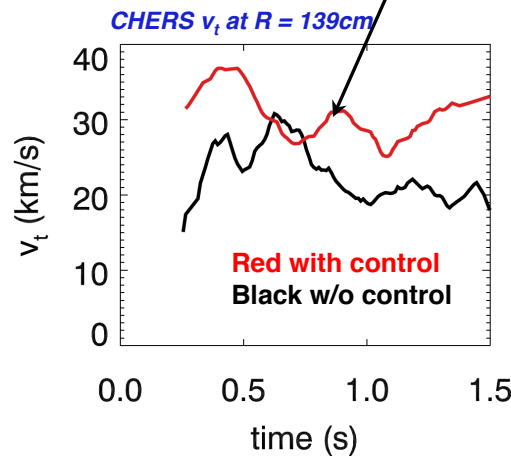
n=1 mode drops  $\beta$



**MHD spectrogram with n=1 feedback and n=3 correction**

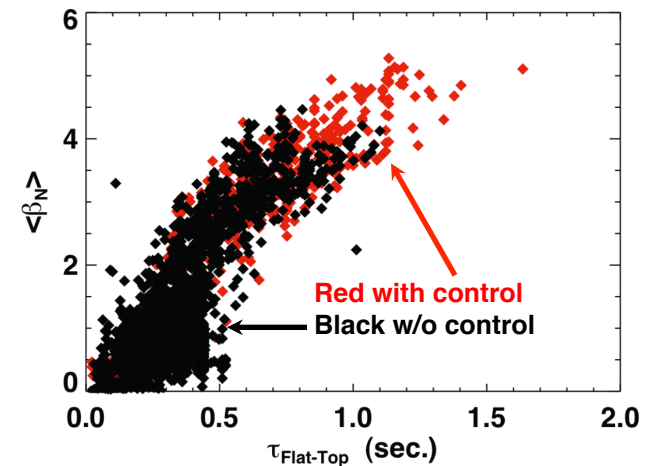


No MHD,  $\beta$  and rotation maintained



- Non-axisymmetric feedback algorithm has been developed using unique feedback training scheme
  - Prevents onset of MHD modes
  - Plasma rotation is maintained throughout discharge
- Control statistically raises  $\beta$  and increase pulse length

**Pulse averaged  $\beta_N$  vs. current flat-top**





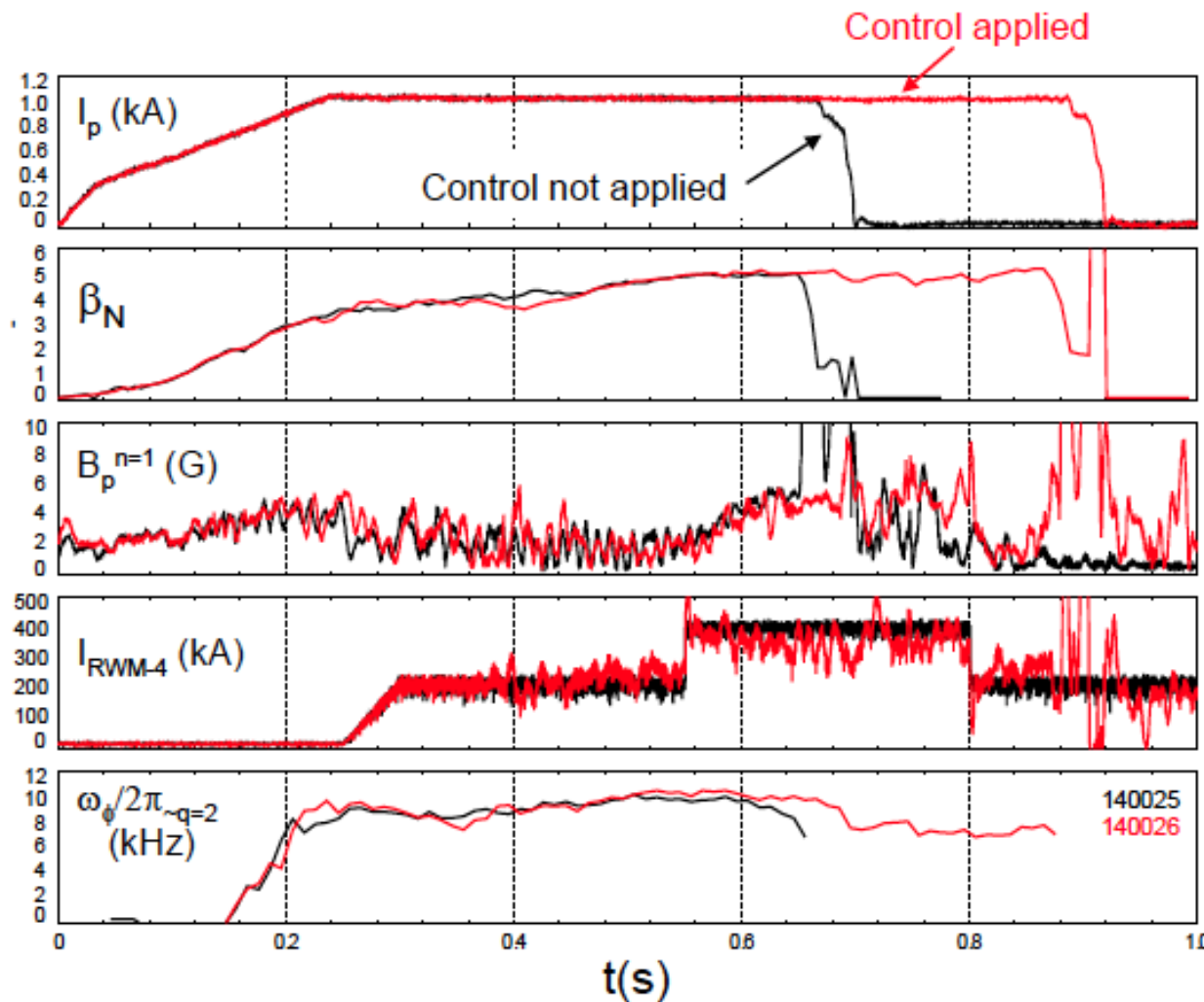
# RWM state space controller sustains otherwise disrupted plasma caused by DC n = 1 applied field

## □ n = 1 DC applied field

- Simple method to generate resonant field amplification
- Can lead to mode onset, disruption

## □ RWM state space controller sustains discharge

- With control, plasma survives n = 1 pulse
- n = 1 DC field reduced
- Transients controlled and do not lead to disruption
- NOTE: initial run – gains NOT optimized



# New control system capabilities

## Substantial investment in NSTX control capability is continuing

- Improved control capability is the engine that drives advanced scenario development
- $\beta_N$  control developed and used successfully to improve reproducibility of experimental performance
- Simultaneous strike point control and X-point control developed in support of LLD and NSTX-U
- Real-time rotation diagnostic has been developed – will now be tested first on NSTX-U
- Snowflake divertor control being developed for upgrade
- Working towards current profile control using radial array of beam sources early in the NSTX-U project
- Evaluating control system upgrade for the outage period

## NSTX has developed the physics basis for NSTX-U and has answered many questions about the viability of an ST-FNSF

- Highly shaped, high  $\beta_N$ , *high*  $f_{NI}$  plasmas with good confinement have been reproducibly controlled
  - Demonstrated NSTX-U shape
  - Measured confinement scaling
  - Understood current drive sources
  - Measured and controlled  $\beta$ -limiting modes
- Investments in advanced control capability have been key to this success
- NSTX-U will benefit directly from this effort and will continue the advancement of the ST physics basis