



University

### TRANSP use for Neutron and Fast lons interpretation at MAST and JET



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### Content

Use of TRANSP at **MAST** for forward modelling of:

- neutron camera measurements: routine
- FIDA diagnostic (with FIDASIM): routine
- Charged Fusion Product Detector (W. Boeglin, R. Perez from FIU and D. Darrow from PPPL): in progress

Use of TRANSP at **JET** for forward modelling of:

- TOFOR & MPRu neutron spectrometer data

Physics topics for which TRANSP is used:

- fast ion physics: confinement and redistribution
- sawtooth
- current drive (see D. Keeling presentation)
- neutron spectrometry analysis
- fuel ion ratio in DD and DT plasmas

### MAST

Mega Ampere Spherical Tokamak:

- Aspect ratio	$R/a \approx 1.3~(R \approx 0.85~{\rm m},~a \approx 0.65~{\rm m})$
- high elongation	$1.5 \le \kappa \le 2.5$
<ul> <li>high triangularity</li> </ul>	$\delta \le 0.5$
- plasma current	$I_p \leq 1.5 \text{ MA}$
- electron temp.	$T_{_e} \leq 2~{\rm keV}$
- plasma density	$n_{_e} \approx 0.1$ – $1.0 \times 10^{20} \text{ m}^{-3}$
- toroidal field	$B_{_T} \approx 0.35$ – 0.6 T (at $R \approx 0.75$ m)
- NBIs	$P_{_{N\!B\!I}} \le 5$ MW, $\le$ 75 keV D ions
	$v_{\scriptscriptstyle NBI}^{}\approx 2.5\;v_{\scriptscriptstyle A}^{}$

Fast Ion Diagnostics (FIDs):

- Vertical and tangential Fast Ion  $\rm D_{a}$  (FIDA) system
- Fission chamber, D<sub>a</sub> edge monitors
- Prototypes:
  - Neutron Collimated flux monitor (NC)
  - Charged Particles Detector array (CPD)



### Fast ions in MAST

Typically 90 % of DD reactions are:

$$D_{fi} + D_{th}$$
  $\longrightarrow$  <sup>3</sup>He + n  
 $T + p$ 



### Vertical Fast Ion D- $\alpha$ diagnostic



### Tangential Fast Ion D- $\alpha$ diagnostic



### **Charged Particles Detector**

4

Number of channels:

 $\rightarrow$  <sup>3</sup>He + n



#### Neutron collimated flux monitor



8

29



### Forward Modelling



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### **TRANSP** input preparation

The MAST Chain Control Centre -  $mc^3$  - is an interactive data preparation tool developed by Rob Akers at CCFE to allow the user to generate a self consistent set of MAST data for TRANSP.

Extremely power and flexible:

- data from multiple diagnostics integrated (TS, CXRS, MSE, FC, Neutrals, magnetics, Zeff, ...)
- and mapped onto EFIT flux surfaces
- EFIT flux surfaces detailed analysis
- MSE constrained
- multiple smoothing/ interpolation methods
- time slicing
- FI pressure from TRANSP
- iterative equilibrium reconstr. with diagnostics constrains
- automatic generation of UFILES
- interactive generation of TR.DAT file





### TRANSP and MC<sup>3</sup> for FI pressure

TEQ solver in TRANSP (from CORSICA) cannot account for:

pressure anisotropy
 toroidal flow

NUBEAM used to determine the FI velocity distribution function and pressure:

 $f_{\rm FI}(R, Z, E, p)$ 





# TRANSP modelling of NC measurements

Field of view of NC not pencil-like: non-flux averaged neutron emissivity required.





## TRANSP modelling of NC measurements: H/D ratio

Plasma discharge	Hydrogen puff (ms)	$n'_{\rm H}/n'_{\rm D}$ Eq. (1)	$n'_{ m H}/n'_{ m D}$ TRANSP
30457	0	0	0
30460	40	0.08	0.09
30464	65	0.30	0.29
30458	75	0.55	0.55







## TRANSP modelling of NC measurements: AFID



Time-dependent anomalous fast ion diffusion necessary to reproduce FC and NC Measurements within experimental errors.

Choice of appropriate diffusion coefficient usually enough to get good agreement but for those scenario with very strong MHD activity



### TRANSP modelling of NC measurements: fishbones



Effect on fast ion distribution function:

- Loss Model 1: energy and v\_/v Loss Model 2: energy and T\_{\_{DEPTH}}



### TRANSP modelling of NC measurements: fishbones





## TRANSP use for modelling of FIDA measurements

NUBEAM models the FI from beam injection to thermalization including CX loss process but does not provide the re-neutralized FI trajectories nor distribution among atomic levels.

FIDASIM with FI distributions from TRANSP/NUBEAM used to simulate FIDA measurements on MAST





# TRANSP/NUBEAM modelling of NC & FIDA measurements: an integrated approach

Example of the same TRANSP run output used to simulate:

- NC via LINE2
- FIDA via FIDASIM

In presence of TAEs using the anomalous fast ion diffusion coefficient.





# TRANSP/NUBEAM modelling of NC & FIDA measurements: an integrated approach to FBs





# TRANSP/NUBEAM modelling of NC & FIDA measurements: an integrated approach to sawtooth

Kadomtsev mixing model with full mixing (actually mixing parameter not making a huge impact).

TRANSP indicates that the crash causes a large radial redistribution of fast ions from the core to regions of the plasma outside the mixing radius.

Passing fast ion population is reduced at each crash while the trapped population is increased: pitch angle scattering







## Neutron spectrometer at JET: MPRu

### The upgraded magnetic proton recoil (MPRu) spectrometer for DT mainly

- Neutron energies deduced from the positions of elastically scattered protons on an array of scintillators.
- G. Ericsson et al, Review of Scientific Instruments, 72, 759 (2001)
- E. Andersson Sundén et al, <u>Nuclear Instruments and Methods in Physics Research A</u>, 610 , 682 (2009)









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~19 m

### Neutron spectrometer at JET: TOFOR





#### The time-of-flight spectrometer TOFOR

- Neutron energies deduced from time-of-flight
- Optimized for DD operiation

M Gatu Johnson *et al*, Nuclear Instruments and Methods in Physics F



~1.5 m



# TRANSP for neutron data analysis at JET

- Fast ion distributions from NUBEAM are used for neutron spectrometry analysis
- The expected neutron spectrum from beam-target and beam-beam reactions can be calculated and compared with measured data (C. Hellesen et al, Plasma Phys. Control. Fusion 52 (2010) 085013)
- RF capabilities in TRANSP would make this framework even more useful



TOFOR data from JET discharge 68138 heated with 116 keV NBI, and a fit of the calculated spectral components. Energy representation of the fitted components



# TRANSP for neutron data analysis at JET

- The calculated spectral shapes and the corresponding reactivities can be used to estimate the fuel ion ratio in DT plasmas
  - $n_t$  and  $n_d$  are related the components intensities (obtained from the spectral analysis of the data) and the corresponding reactivities (calculated from the NUBEAM distributions)
- n<sub>t</sub>/n<sub>d</sub> in DT plasmas from the JET DTE1 campaign has been estimated using data from the Magnetic Proton Recoil spectrometer (C. Hellesen, J. Eriksson *et al*, Nucl. Fusion 55 (2015) 023005)



Beam-target T-on D (BT-TD) and thermonuclear DT (TH-DT) neutron emission components fitted to MPR data from JET discharge 42840



Energy representation of the fitted components



# TRANSP for neutron data analysis at JET

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*Time evolution of fitted beam-target T-on D (BT-TD) and thermonuclear DT (TH-DT) neutron emission components* 



Estimated values of  $n_t/n_d$  in the plasma core, compared with Penning trap (KT5P) measurements of  $n_t/n_d$  at the edge



### Wishlist

#### TRANSP/NUBEAM code

- Faster NUBEAM MC module: 50.000 particles take a long time! Needed for smoother FI velocity distribution functions
- SXR synthetic diagnostic
- Integration with orbit following code
- RF heating modelling
- "kick" model for fast ion transport developed by PPPL

#### TRANSP/NUBEAM user support

- netCDF viewer (http://meteora.ucsd.edu/~pierce/ncview\_home\_page.html)
- examples
- training sessions



#### Fast Ion D- $\alpha$ diagnostic





### Proton Detector (2013)





### Neutron measurements (2010)

Large neutron shield and collimation assembly.

Liquid scintillators  $(2 \times 5 \times 1.5 \text{ cm})$  coupled to PMTs

Full digital acquisition 250  $\mathrm{MS/s}$  and post-processing with PSD.

