

TRANSP for Integrated Tokamak modeling

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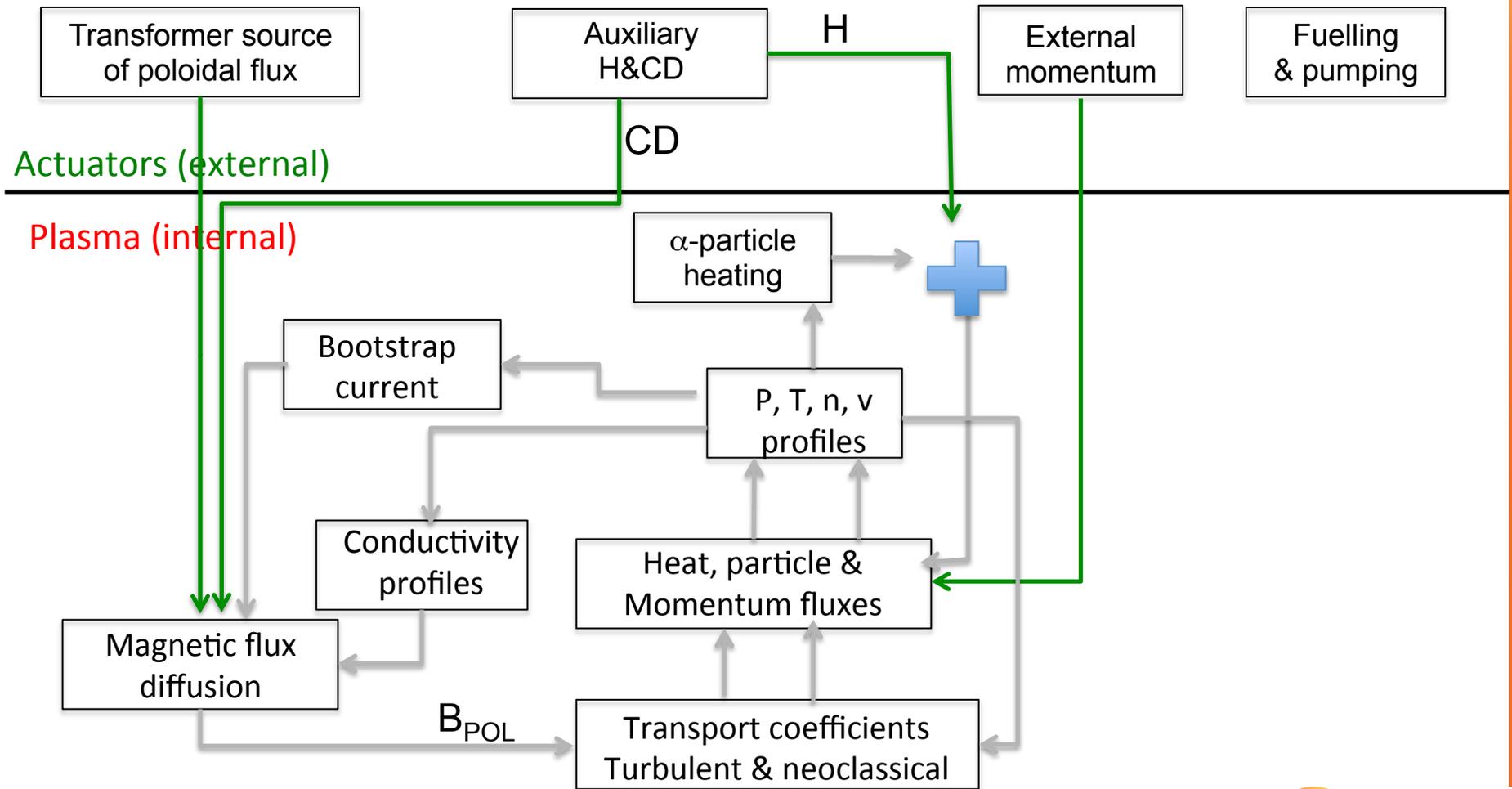
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Outline

- Integrated modeling, the golden goal
 - what should be included in a tokamak simulator?
- What do I use TRANSP for?
 - predictions, scenario modeling
 - analysis and interpretation
- What could I do better?
 - what upgrades would I like to see and how to facilitate this?

(do not ask what TRANSP can do for you, but what you can do for TRANSP)

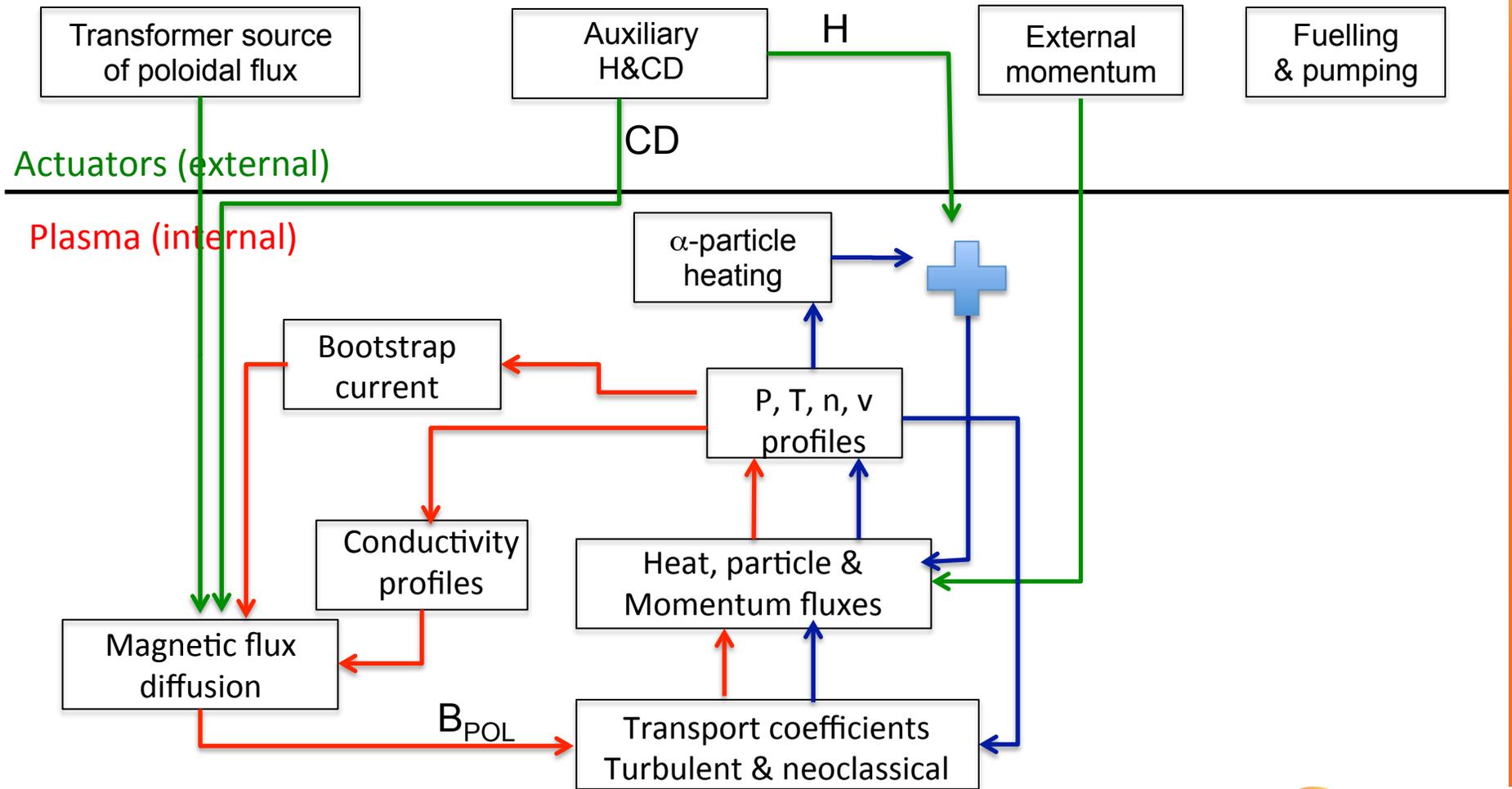
Integrated modeling is a complex task: evolve self-consistently plasma and actuators



adapted from Politzer NF 2005 and X. Litaudon, ITER summer school 2014



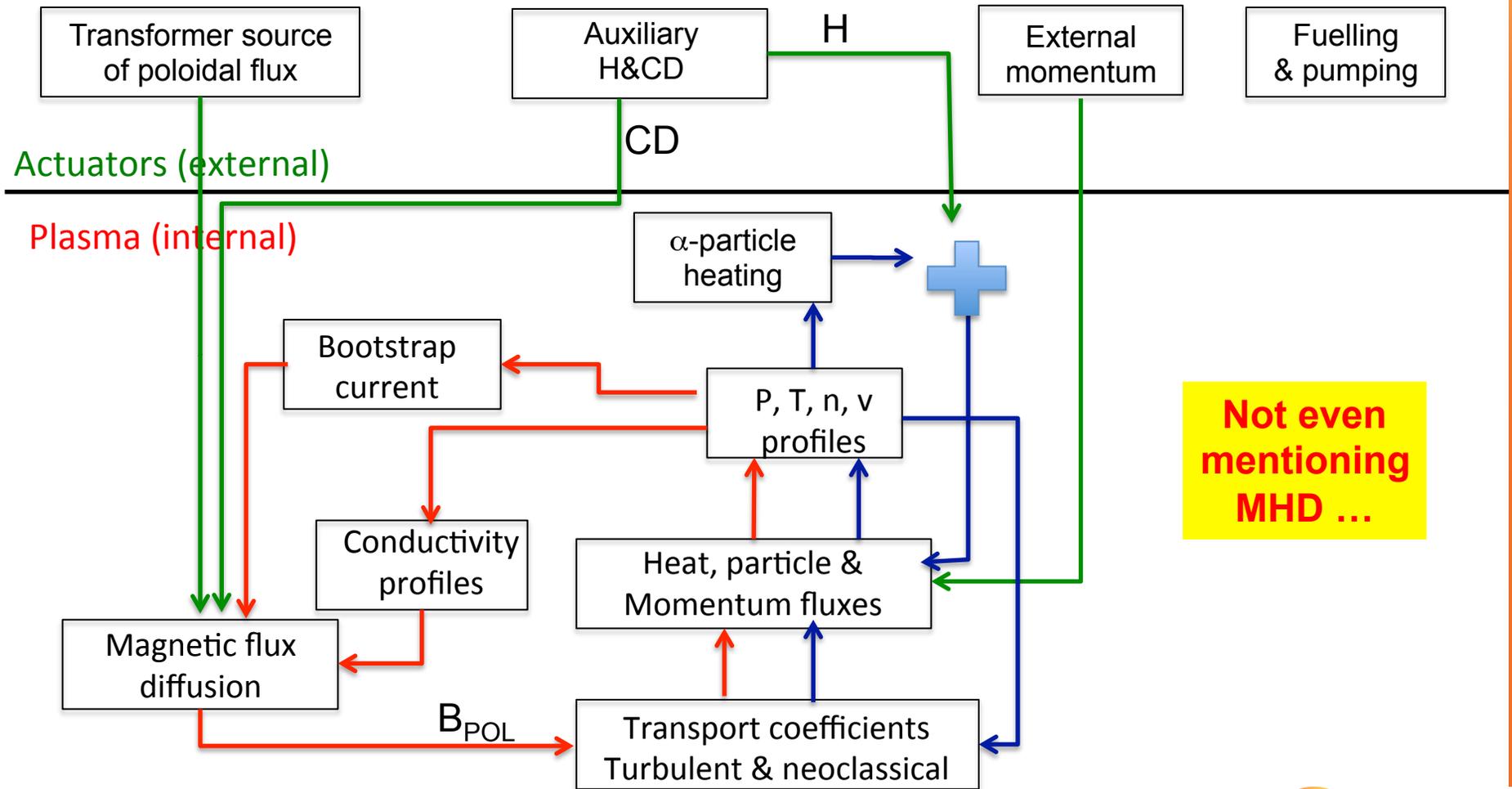
Integrated modeling is a complex task: combine fast (transport) & **slow** (current diffusion) time scales



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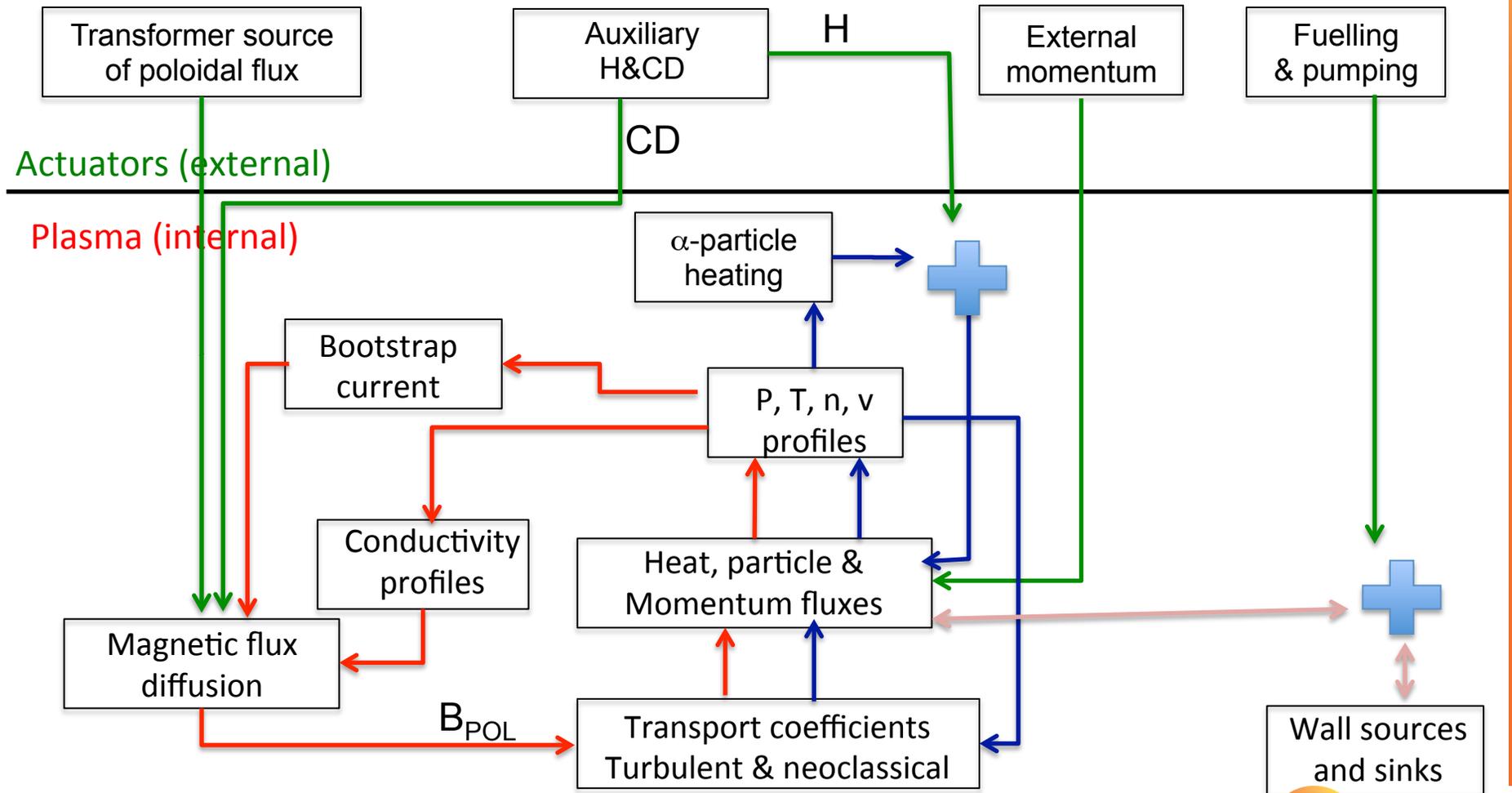
Integrated modeling is a complex task: combine fast (transport) & **slow** (current diffusion) time scales



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Integrated modeling is a complex task: Integrate the core with the edge and the boundary



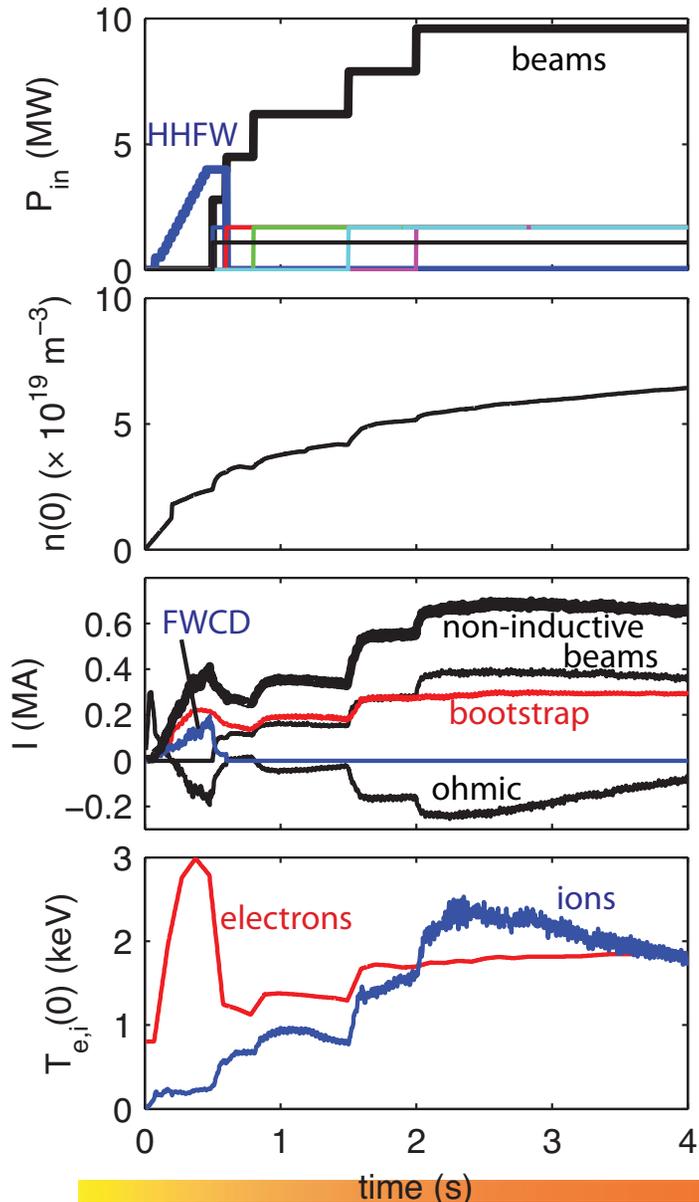
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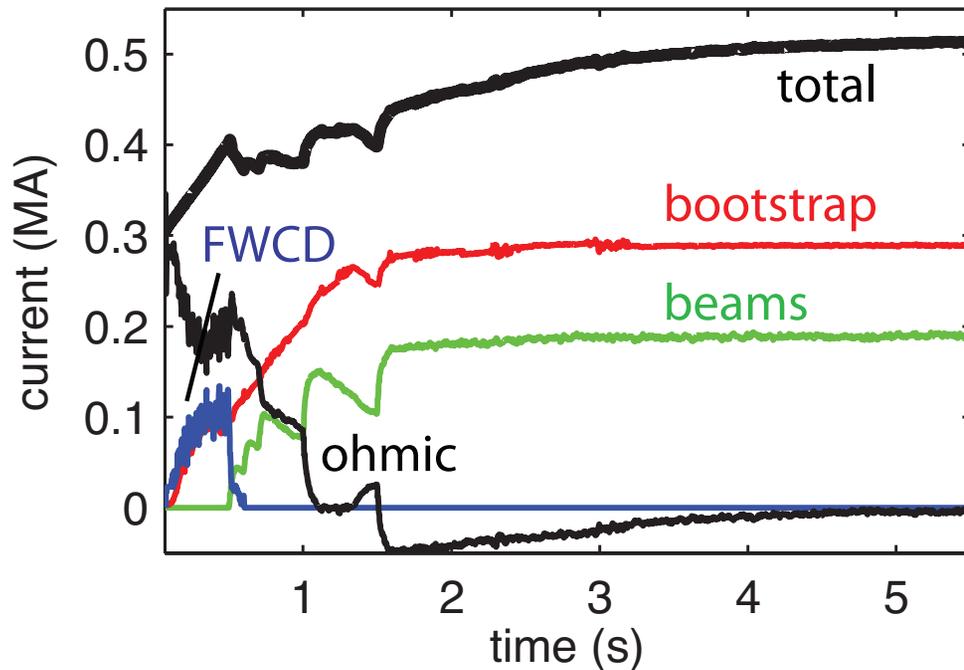
- Integrated modeling, the golden goal
 - what should be included in a tokamak simulator?
- What do I use TRANSP for? Three examples
 - predictions, scenario modeling (NSTX-U, ITER)
 - analysis and interpretation (C-Mod)
- What could I do better?
 - what upgrades would I like to see and how to facilitate this?
(do not ask what TRANSP can do for you, but what you can do for TRANSP)

Free-boundary TRANSP is playing a critical role in predictive simulations of non-inductive current ramp-up on NSTX-U



- free-boundary TRANSP with
 - HCD source module
 - bootstrap current and
 - thermal transport
 - combined with control algorithms is a great tool for scenario development
- Evolve self-consistently equilibrium and HCD
 - used here analytic density, BUT
 - predictive density evolution possible WITH
 - density feedback possible
 - shape control possible
 - beta control possible

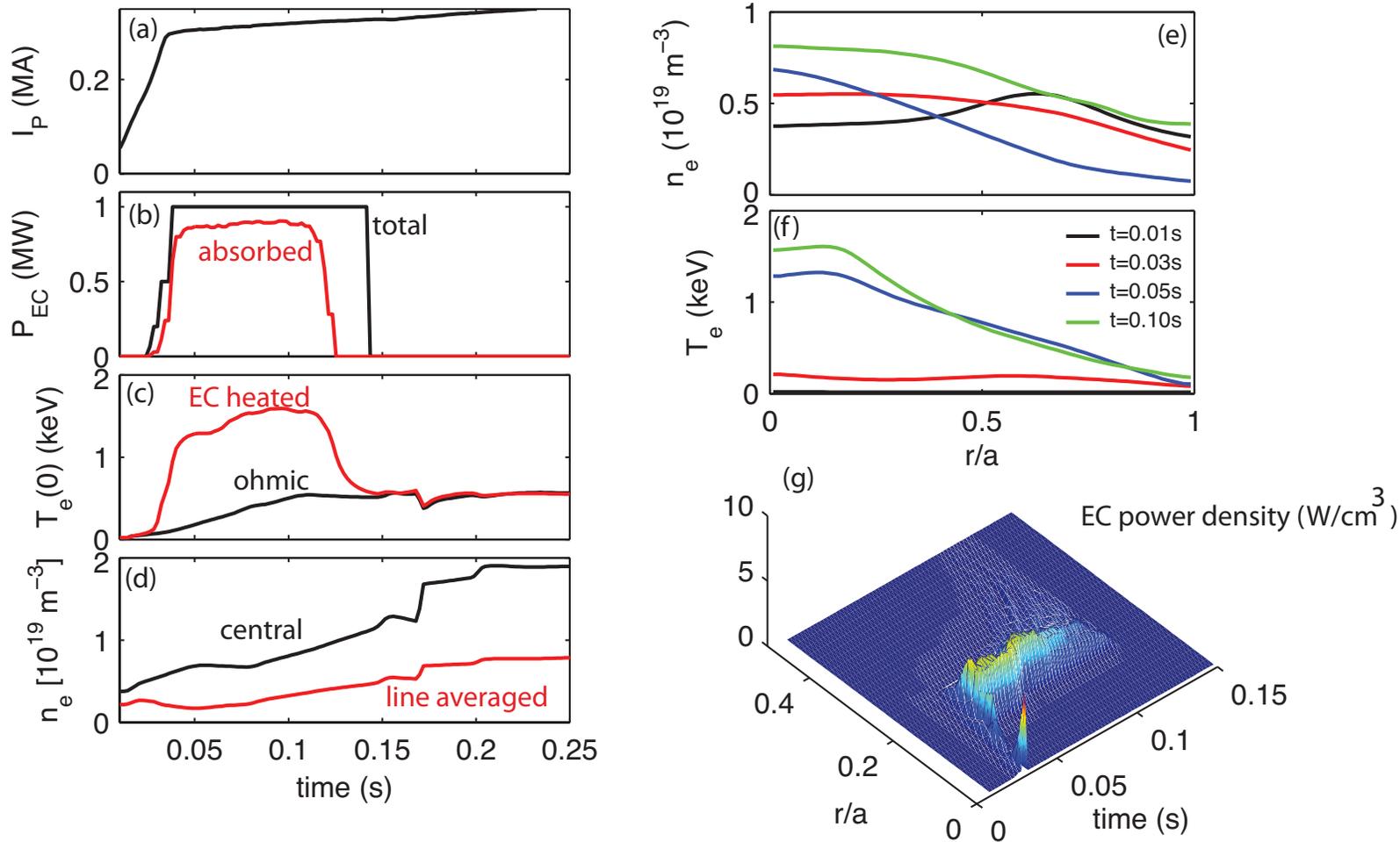
... and in the preparation of experiments towards the achievement of non-inductive ramp-up on NSTX-U



Simulation of a discharge on NSTX-U, with $B_T=0.75$ T, $n_{e,lin}=0.75n_G$ 4 MW of HHFW and 10 MW of NBI

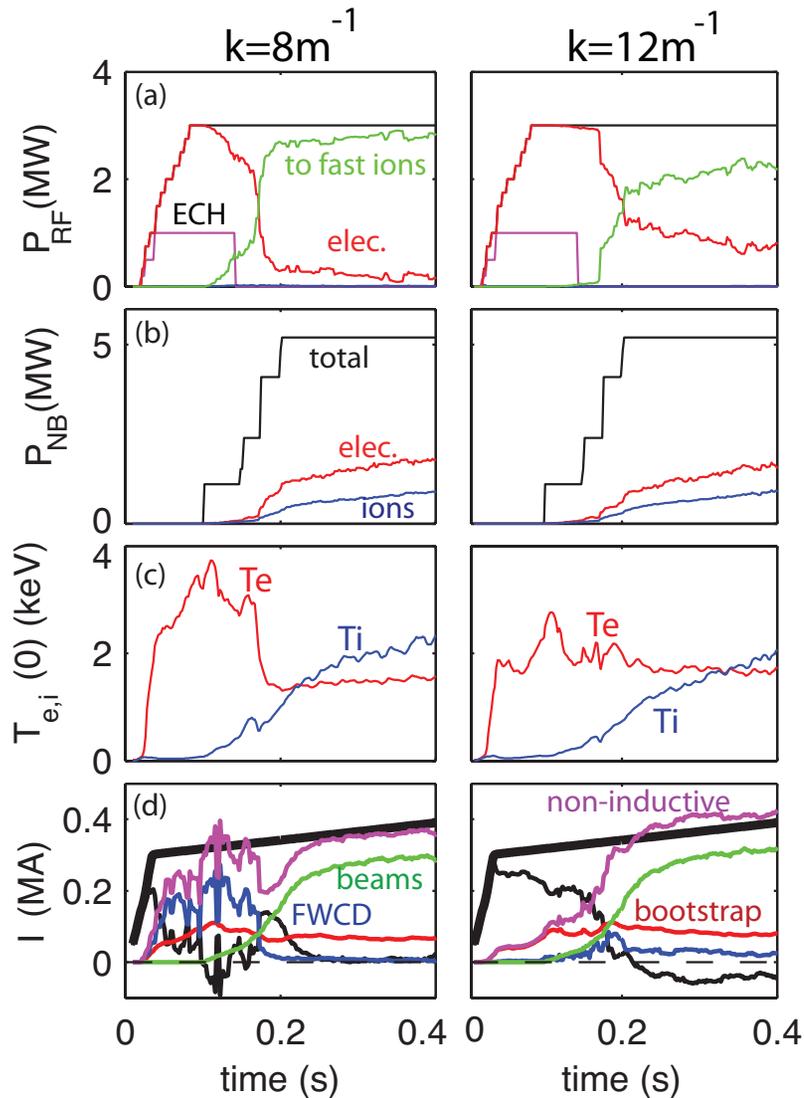
Combined with systematic validation activity, we are using free-boundary TRANSP to design experiments ahead of time and go “prepared” to the control room.

Self-consistent evolution of equilibrium and kinetic profiles are being used for assessing feasibility of EC upgrade



Simulation of 1 MW of Electron Cyclotron heating on NSTX-U start-up plasma

Assessing synergy and interactions between IC and NBI is important for experiments and for modeling



Correct inclusion of non-Maxwellian distribution function is a priority for fusion research

to correctly study

- interaction between RF and NBI

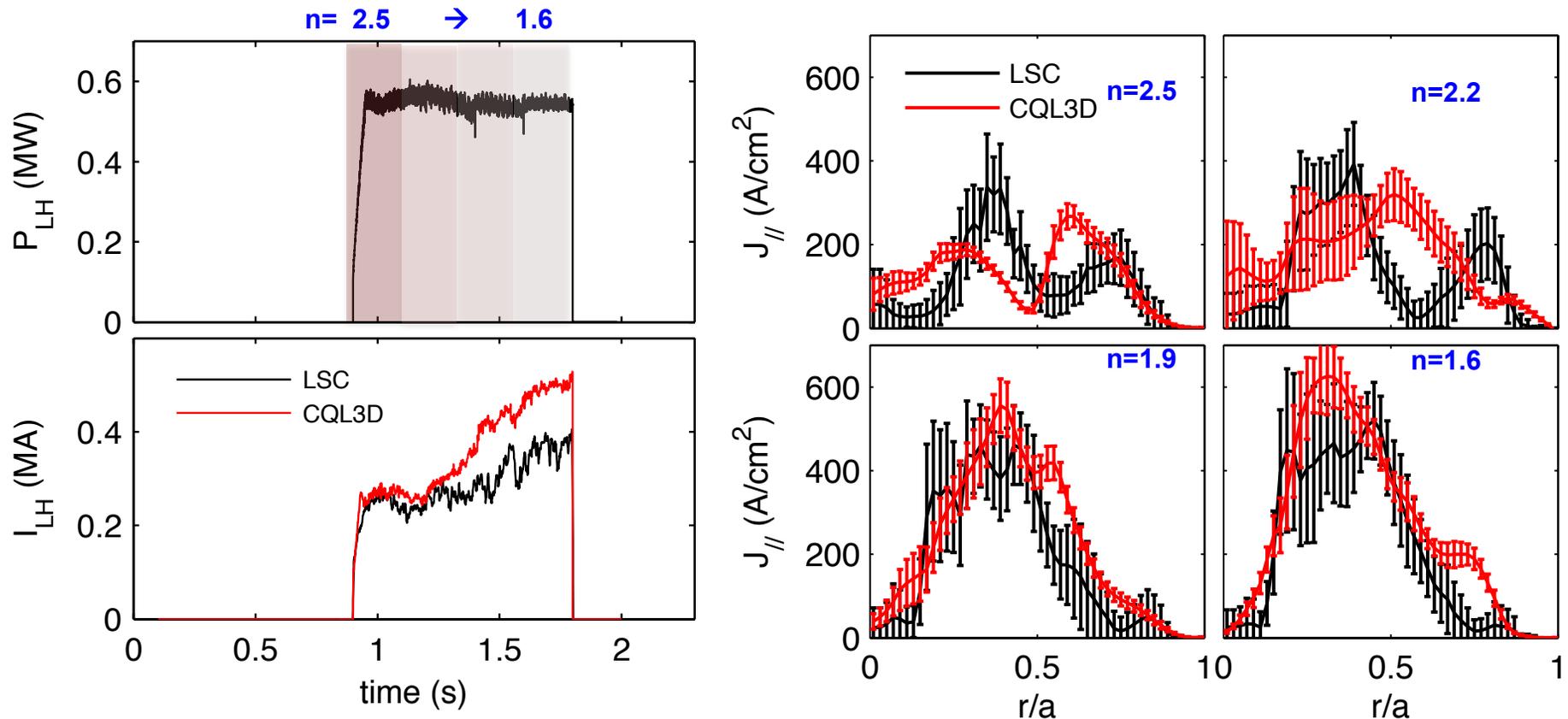
and for ITER and fusion devices

- correct treatment of minority heating

- treatment of alpha particles

Some upgrade needed here ...

Implementation of 3D Fokker-Planck CQL3D represents a sensible improvement in the study of LHCD physics



decrease $n_{||}$ in discharge => expect inner LH wave damping
 => trend visible in GENRAY+CQL3D, not evident in LSC at large $n_{||}$
 => difference of about 20% in total LH current (acceptable, depending on purposes)
 Sensitivity studies ongoing in TRANSP on C-Mod LHCD discharges



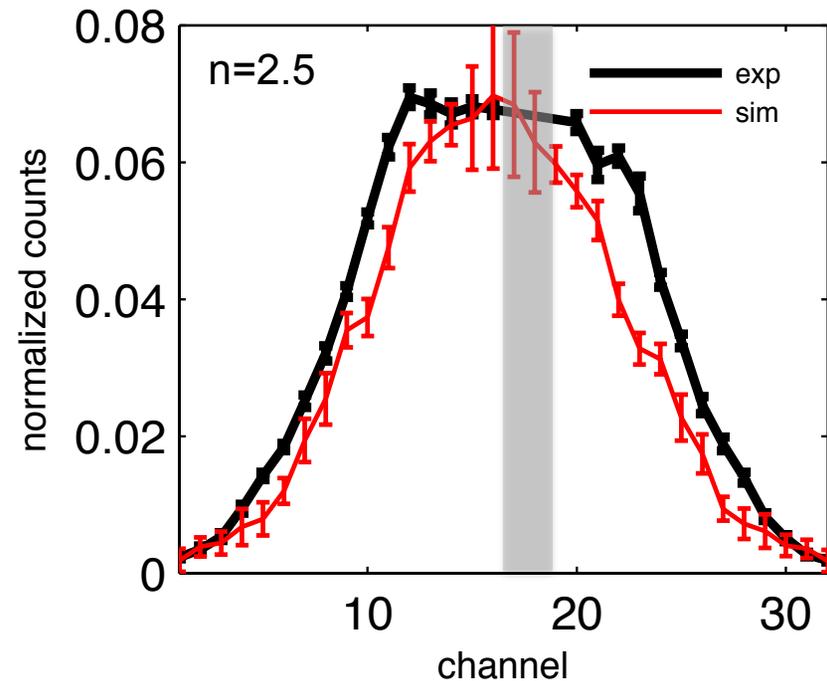
Using time-dependent calculations, LHCD model can be validated in experimental-like conditions

Time-dependent analysis accounts for:

- evolution of poloidal field diffusion
- including the coil current evolution
- use of self-consistent profile of Vloop
- can read equilibrium from EFIT-MSE for cross-check accuracy of diagnostics

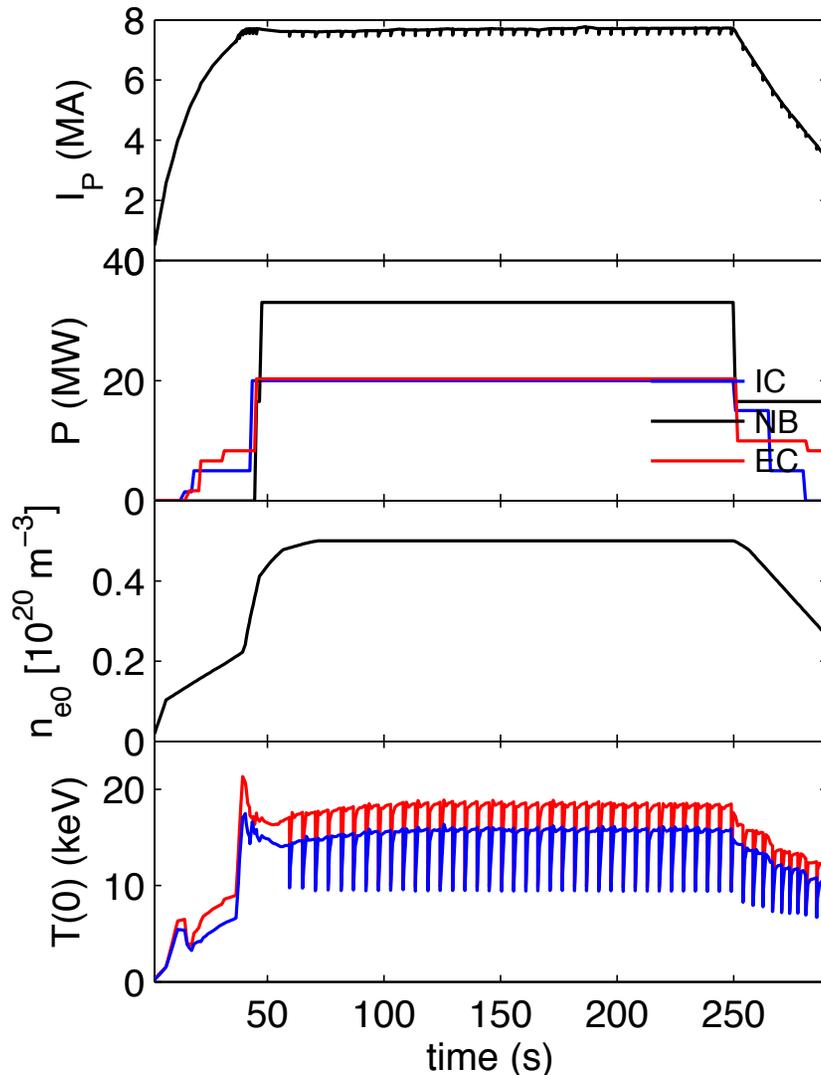
Exp. HXR integrated over 80 ms

Synthetic HXR integrated over 80 ms



TRANSP very good tool for model validation and for diagnostics cross-consistency

Prediction of ITER operational scenarios needs high fidelity HCD source models, thermal transport, fast ion transport ...



Simulate plasma evolution from startup to termination

Combine heating and current drive source
Assist L-H transition

Entry to burn

Ramp-down, H-L transition, termination
density evolution (prediction)
feedback control (shape, density, beta)

Evolve temperature and equilibrium self-consistently, including MHD
(here sawteeth with Porcelli trigger).

Outline

- Integrated modeling, the golden goal
 - what do we need to model a tokamak ?
- What do I use TRANSP for?
 - predictions, scenario modeling
 - analysis and interpretation
- What could I do better if we had ...

Upgrades wishing list ...

Short term (1-2 years) **CORE PHYSICS**

- RF synergies and interactions with fast ions (beams, alphas) [high priority ITER, NSTX-U]
- reduced pedestal model and consistent core-pedestal coupling [in progress]
- core impurity transport [in the pipeline]
- reduced MHD (NTMs, fast ion stabilization in STs) [in progress, ITA]
- fast ion transport physics

Medium term (>5 years) **EDGE PHYSICS**

- reduced models for SOL, PMI, ELMs

Long term (> 10 years) start now **MODULARITY, SPEED (more ITER oriented)**

- fast algorithms/reduced models/more efficient modules
 - Ex: TORBEAM 5s/1CPU – replaces GENRAY 25s/48CPU
 - Ex. EPED1 lookup table and parametrization
- Ability to run in ‘control’ mode for inter-shot predictive runs