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# Status and Plans for the PEGASUS Toroidal Experiment

R. J. Fonck for the Pegasus Team

presented to the

NSTX Five Year Plan Ideas Forum

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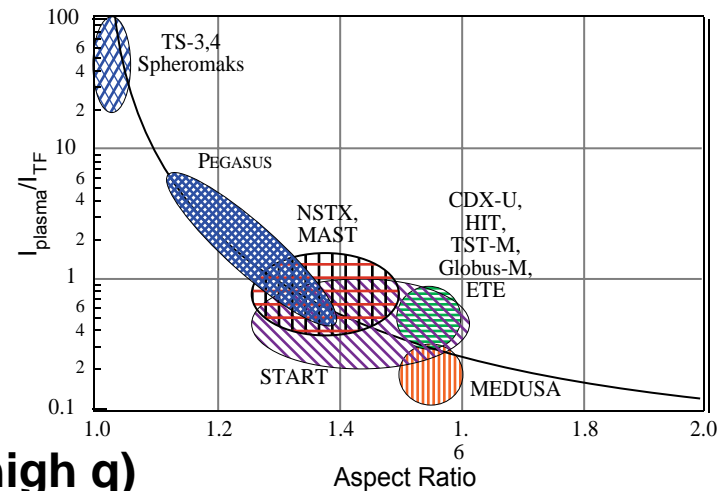


# Role of Pegasus in the Fusion Community

*An extremely low-aspect ratio facility exploring quasi-spherical high-pressure plasmas with the goal of minimizing the central column while maintaining good confinement and stability.*

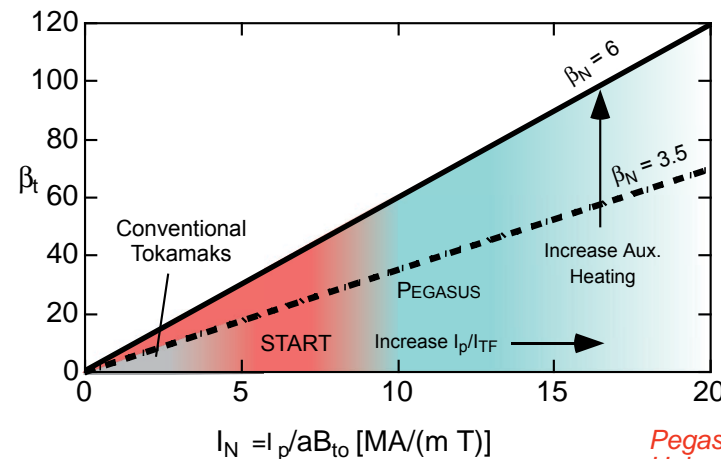
- **Physics of  $A \approx 1$  plasmas as an Alternate Concept (low  $q$ )**

- Extreme toroidicity ( $A \rightarrow 1$ )
- Very high TF utilization ( $I_p/I_{TF}$ )  $> 3$
- Stability at very low TF ( $\beta \approx 1$ )
- Relaxation stability at tokamak/spheromak boundary
- RF heating and CD schemes (HHFW, EBW)
- Trade-offs: CD, recirculating power, and  $A \approx 1$ , low-TF operation



- **Contribute to development of the ST (high  $q$ )**

- Stability limits for  $A \rightarrow 1$  (vs.  $I_p/I_{TF}$ ,  $q_\psi$ ,  $N_e$ ,  $\beta_t$ ,  $\beta_{pol}$ ,  $\kappa$ ,  $A$ , etc.)
- $\beta$  limit dependencies
- Access high  $\beta_t$  at extreme  $I_N$  w/o conducting shell
- Confinement  $A < 1.3$
- New startup schemes (e.g., plasma gun, EBW)





# Pegasus is a mid-sized university spherical torus

- Achieved Parameters:
  - $R = 0.2 - 0.45 \text{ m}$
  - $B_t \leq 0.07 \text{ T}$
  - $\kappa = 1.5 - 3.7$
  - $\langle n_e \rangle = 1 - 5 \times 10^{19} \text{ m}^{-3}$
  - $A = 1.15 - 1.3$
  - $I_p \leq 0.15 \text{ MA}$
  - $\Delta t_{\text{pulse}} = 10 - 30 \text{ ms}$
  - $\beta_t \leq 20\%$
- High-strength solenoid magnet is enabling technology





# PEGASUS Experiment Group

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- **Pegasus Personnel - Experiment Team:**

Staff:

G. Garstka	R. Fonck
B. Lewicki	P. Nonn (+ MST)
B. Ford	P. Probert (+ HSX)
G. Winz	

Graduate Students:

C. Ostrander  
N. Eiditis  
A. Sontag  
K. Tritz  
E. Unterberg

Undergraduates:

S. Diem	B. Wilson	C. Putre
M. Reinke	J. Boerner	R. Radel
R. Slowoski	A. Norseck	T. Wagner
R. Curtiss		

- **Associated Theory (CPTC)**

J. Callen	C. Sovinec	C. Hegna
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# Vacuum vessel interior hardware

**EBW antenna**

**New centerstack armor**

**Outboard limiter**

**HHFW antenna**



**Extensive magnetic diagnostics**

**Segmented divertor plates**



# Pegasus diagnostic set

## • Presently Operating Diagnostics

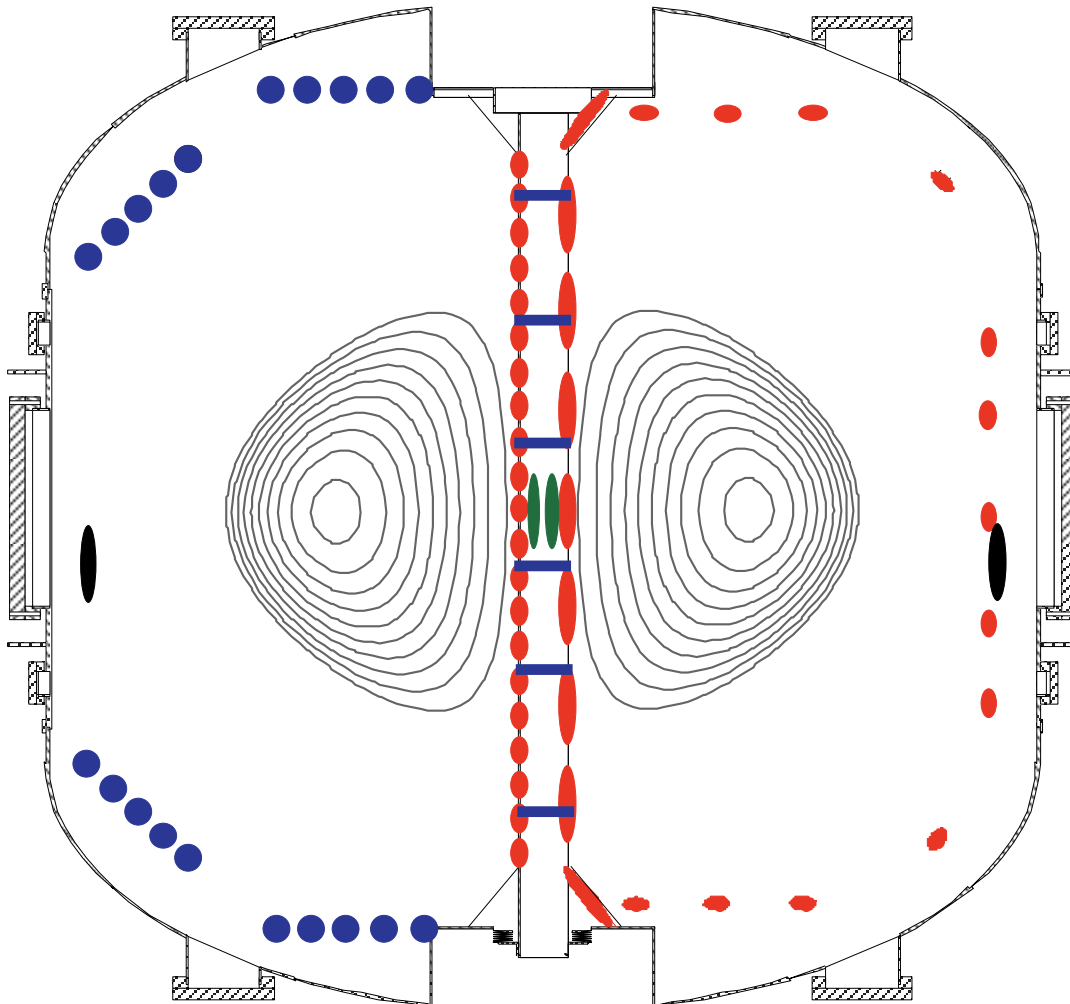
<u>Diagnostic</u>	<u>Capability</u>	<u>Measures</u>
Core Flux Loops	(6)	$V_L, \Psi_{pol}$
Wall Flux loops	(6)	Vessel currents
Int. Flux loops	(20)	$\Psi_{pol}$
Rogowski Coils	(2)	$I_p$
Diamagnetic Loop	(2)	$\Phi_{tor} / \beta_p$
$B_p$ , Mirnov Coils	(56)	$B_r, B_z /$ MHD activity
VUV (SPRED)	central chord	Impurity monitor
Filterscopes	central chord	Oxygen, Carbon, VB, $D_\alpha$
Interferometer	single chord	$N_e I$
High Res. Camera	1000 fps	Plasma shape/position
2-D SXR Camera		Internal Shape/ $j(R)$
Poloidal SXR Diode Array	(19)	MHD Activity

## • Near-Future Diagnostics

<u>Diagnostic</u>	<u>Capability</u>	<u>Measures</u>	<u>Status</u>
Tangential CCD PHA	single chord	$T_e(t)$	In Development
Tangential Bolometer Array	~20 chords	$P_{rad}$	Testing
Ross Filters	4 chords	$T_{e0}(t)$	Testing
2-Color X-ray	4 chords	$T_e$	Testing
Tangential VB Array	~20 chords	$Z_{eff}(R,t), N_e(R,t)$	Testing
DNB		$N_e(R,t), T_e(R,t), j(R)$	Proposed
EBW Radiometer		$T_e(t)$	Proposed



# Magnetics diagnostics installed in 2001



- Flux Loops (26)
- Poloidal Mirnov Coils (22 + 21)
- LFS Toroidal Mirnov Coils (6)
- HFS Toroidal Mirnov Coils (7)

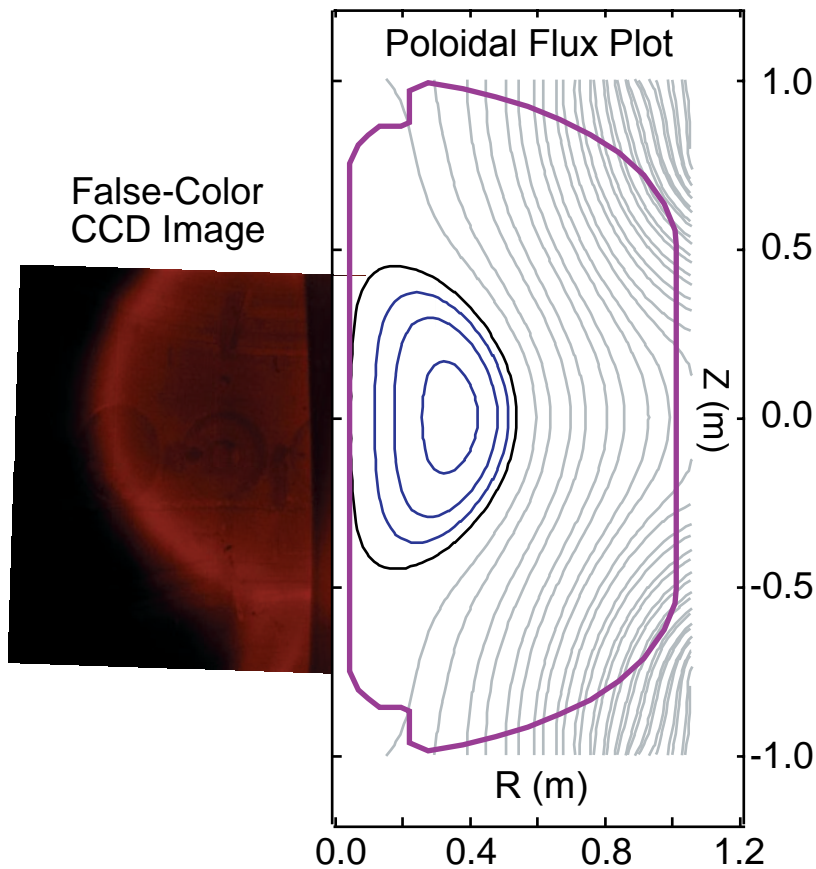
Not shown:

- Plasma Rogowski Coils (2)
- Diamagnetic Loops (2)
- Diamagnetic Compensation Loop
- External Flux Loops (6)
- Internal  $B_{tan}$  Coils (15)



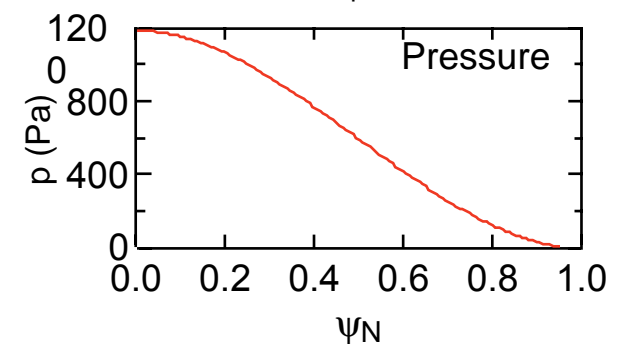
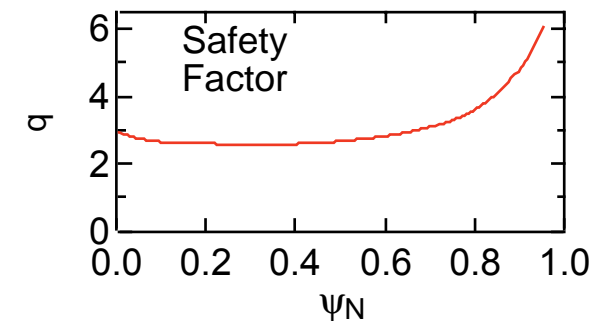
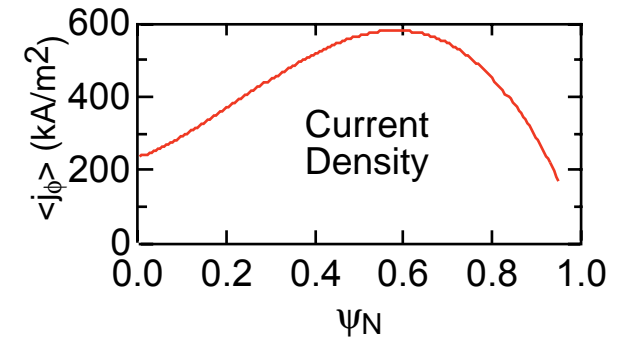
# Equilibrium reconstruction is a primary analysis tool

- A new equilibrium code has been developed for Pegasus
  - Robust, cross-platform, easy incorporation of new diagnostics
- Uses a two-step iterative method to determine equilibrium
  - Gauss-Seidel multigrid relaxation to solve Grad-Shafranov equation
  - Levenberg-Marquardt method to minimize  $\chi^2$
- Benchmarked against existing codes
- Sample reconstruction:



### Fit Results

$I_p$	151.4 kA
$R_0$	0.305 m
$a$	0.249 m
$A$	1.22
$\kappa$	1.8
$RB_t$	0.003 T-m
$\beta_t$	16%
$I_i$	0.35
$q_0$	2.8
$q_{95}$	6.2

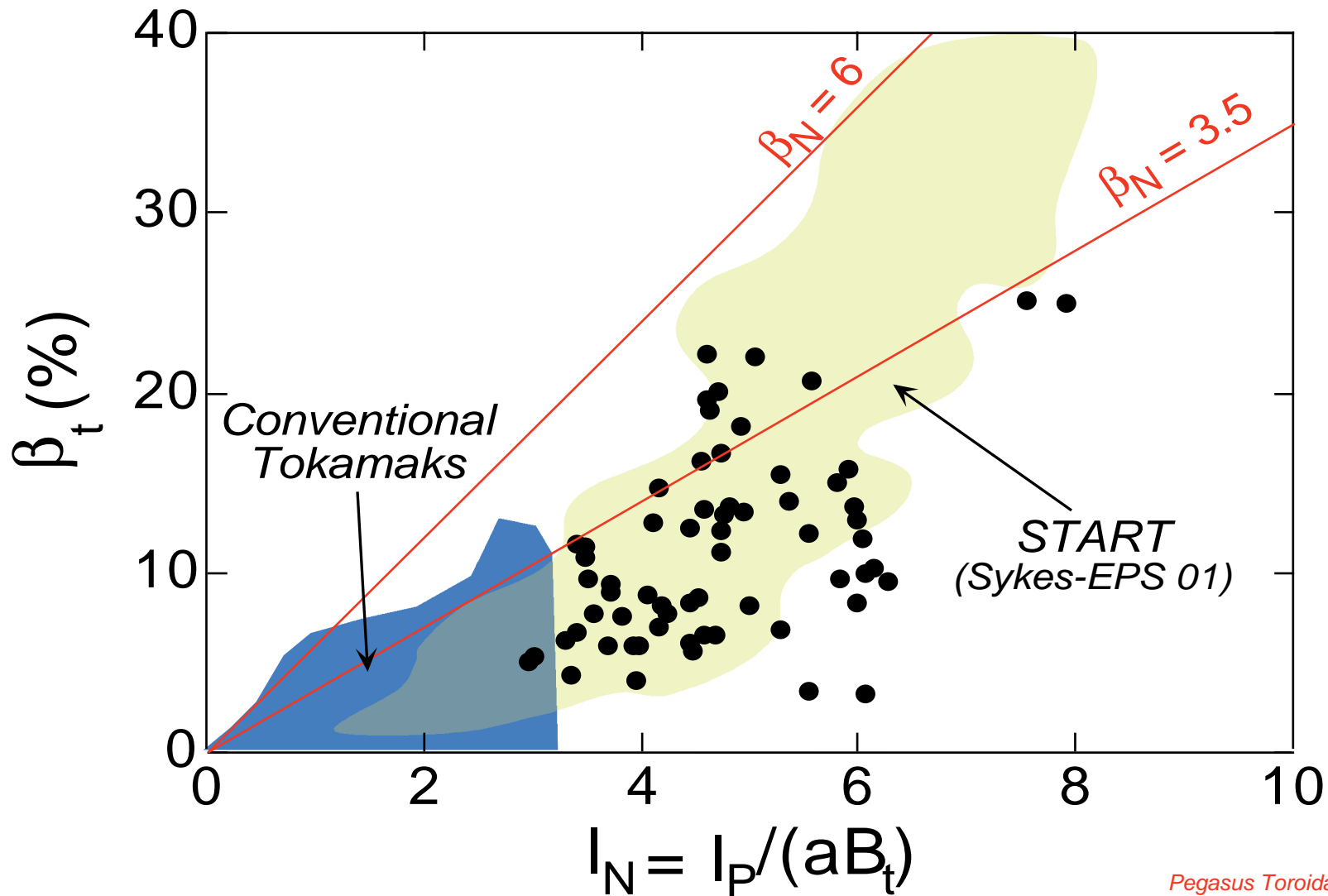






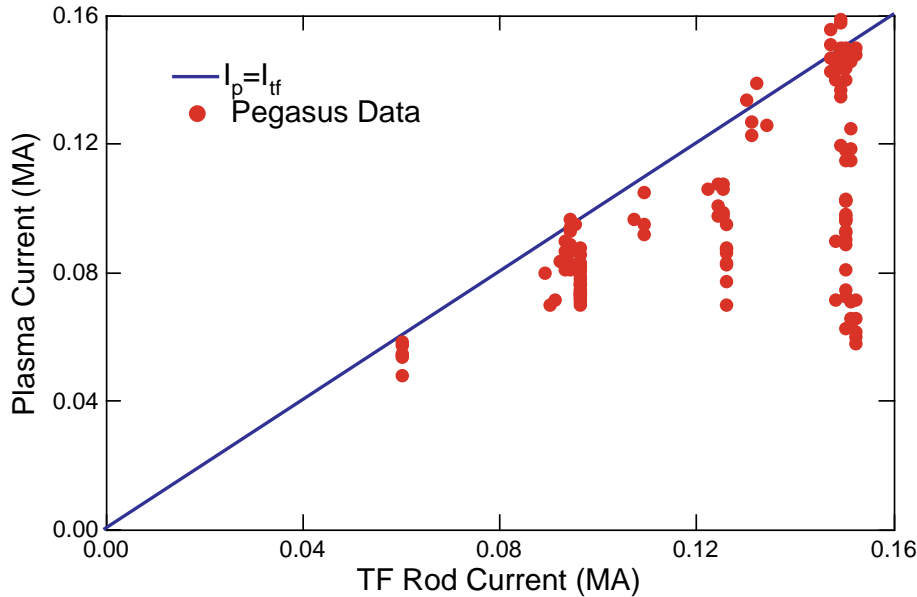
# PEGASUS is Accessing High- $\beta_t$ ST Regime

- High  $\beta_t$  achieved at high density and low toroidal field
  - All results achieved with ohmic heating only
  - Highest normalized currents achieved at low field



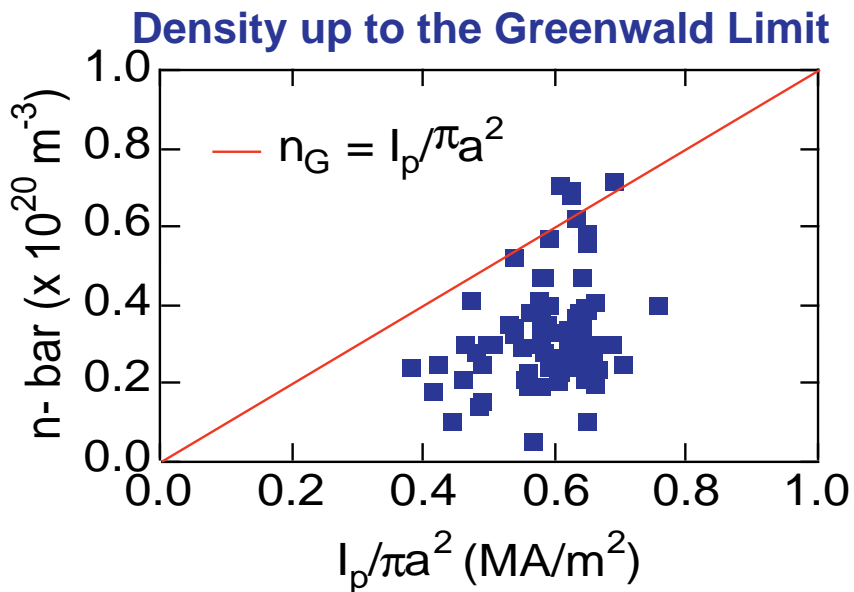


# Low-TF, High Density, Low- $I_i$ Operation

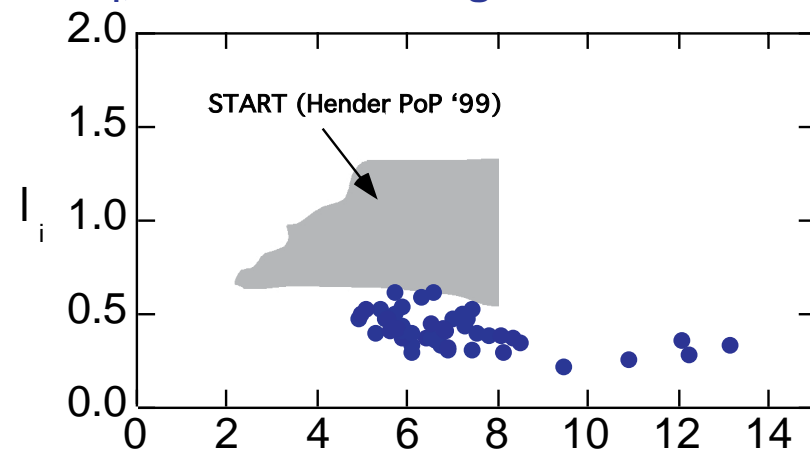


## TF utilization: “soft” limit at $I_p \approx I_{TF}$

- Increased V-s consumption required for breakdown at lower TF
- Large-scale resistive MHD during current ramp (exacerbated by low  $I_i$ ?)

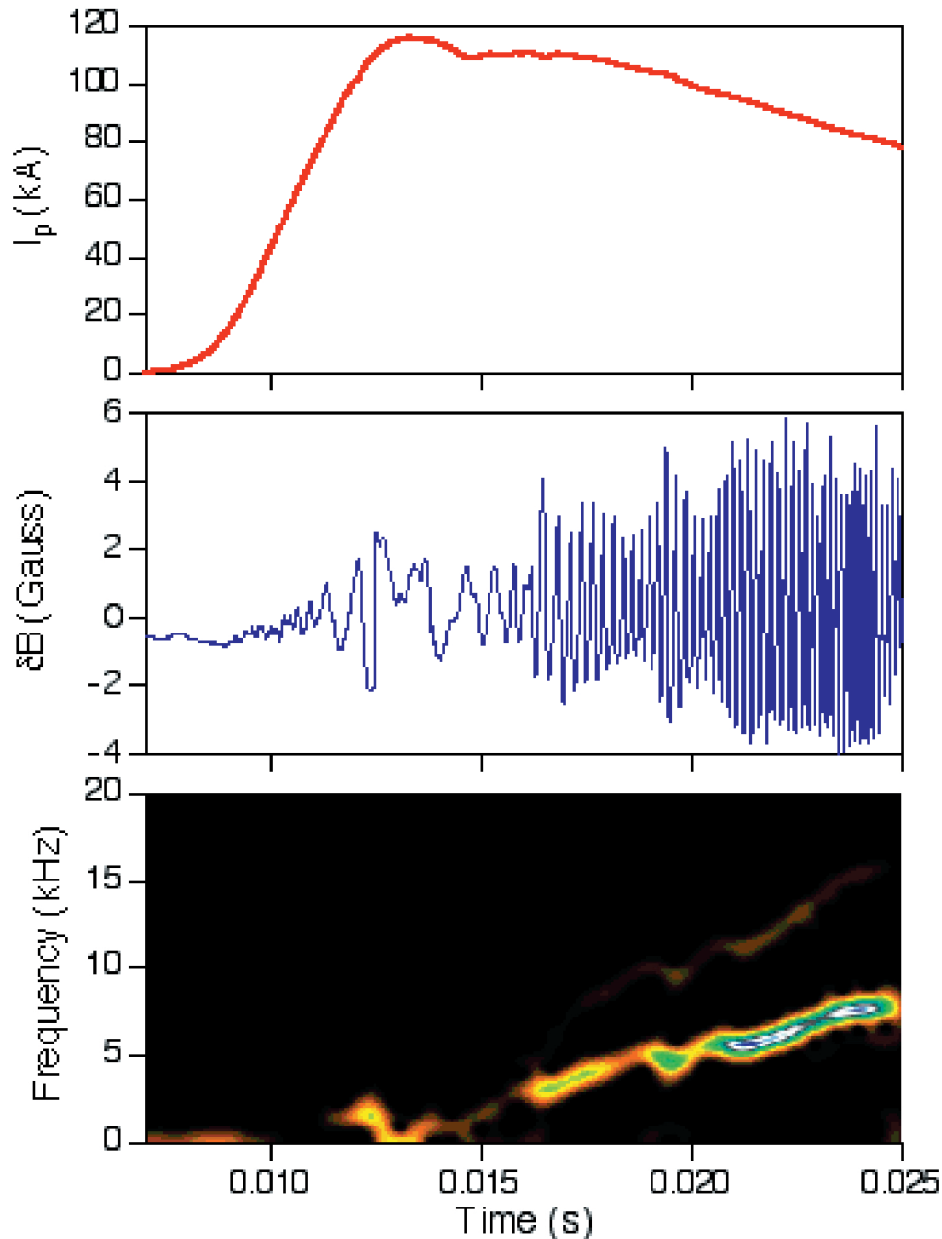


## Low $I_i$ ; increases during current relaxation





## A dominant feature has been a rotating $m/n=2/1$ mode

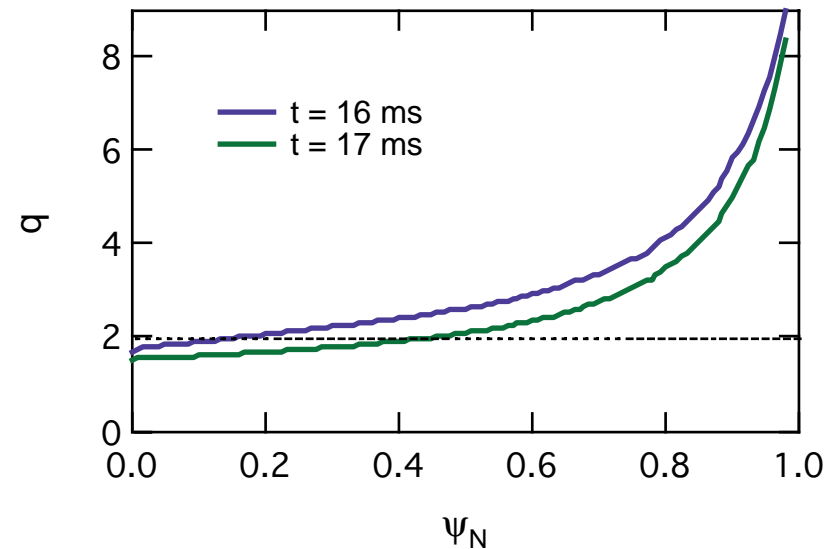
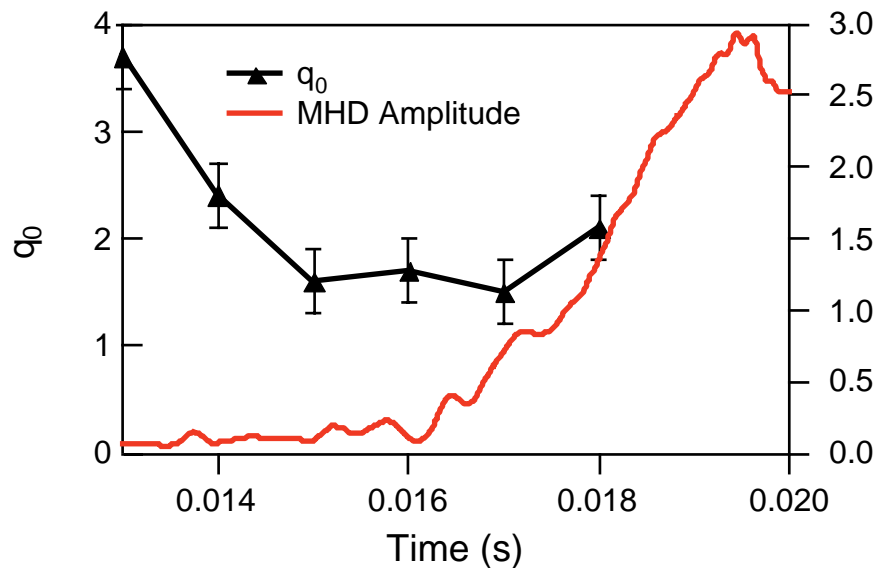
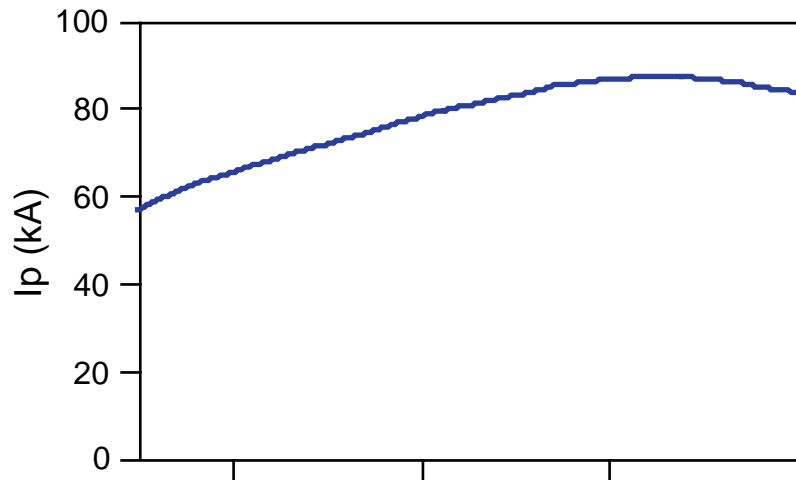


- Mode present in all significant discharges
- Rotates in electron diamagnetic direction
  - Mode is likely magnetic island
- Frequency is typically 2-10 kHz
  - No evidence of mode locking
- Little shear stabilization of island growth
  - Central shear is nearly zero



## 2/1 mode not observed until $q_{\min} < 2$

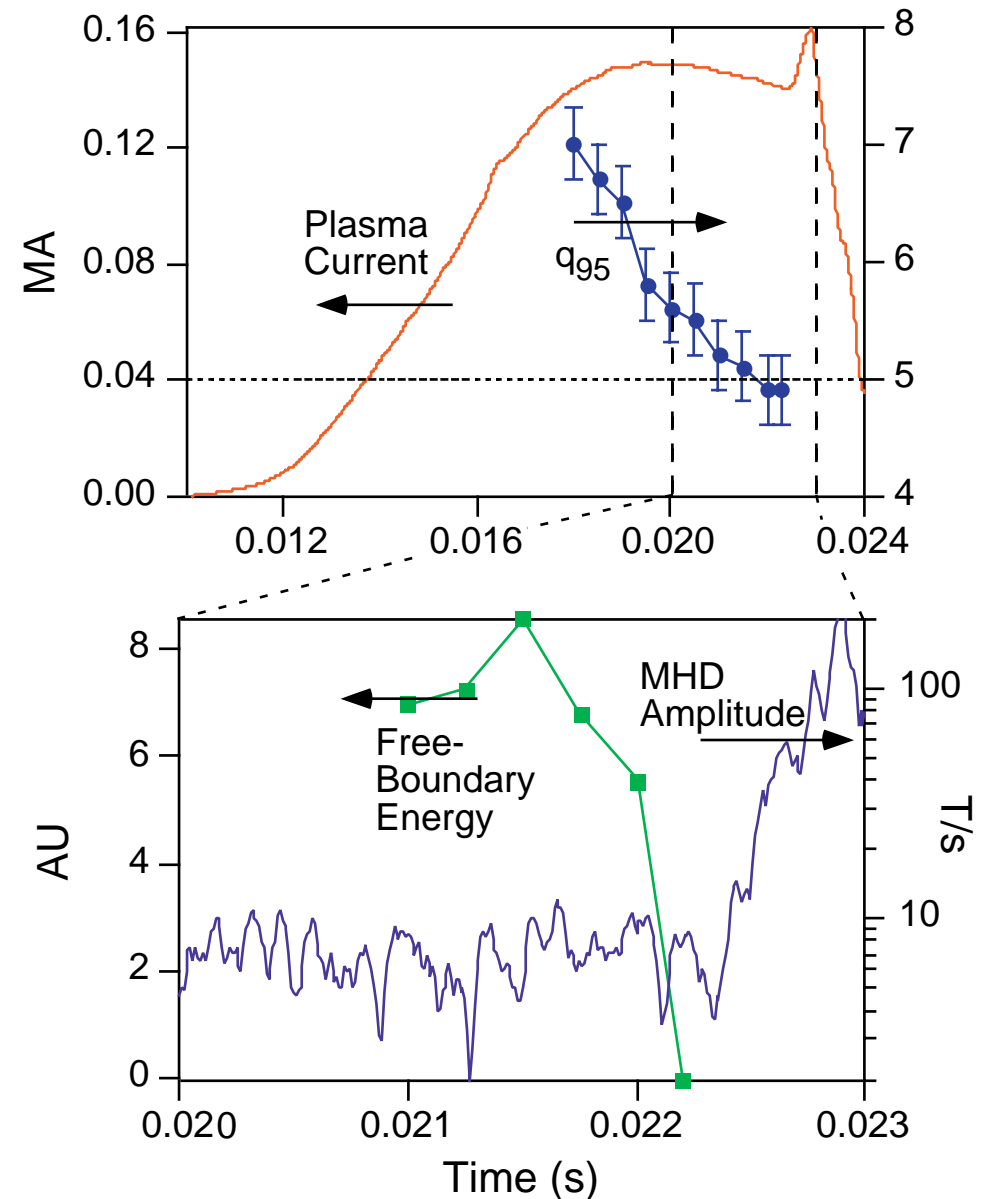
- Consistent with observed helicity of the mode
- A large region of low shear exists about the  $q=2$  surface
- Central  $q$  inferred from Equilibrium fit
- In general, strong MHD activity appears to relate to  $I_p$  limits





# External kink modes observed in highest-current shots

- Higher-current discharges (150 kA class) often terminate in abrupt disruptions
  - Lower-current shots have IREs followed by gradual plasma termination
- $n=1$  fluctuations are observed on Mirnov coils immediately prior to disruption
  - Dominant frequency is roughly 10 kHz
  - Mode observed a few 100  $\mu$ s before IRE
- Observed disruptions are associated with edge kink limits
  - Oscillations not observed until  $q_{95} \approx 5$
- Calculated free-boundary energy (DCON) approaches zero as oscillations begin
  - Negative value indicates instability to external kink
- Consistent with theoretical understanding of ideal kink stability at near-unity  $A$ 
  - As  $A \rightarrow 1$ , unstable  $q_a$  increases
  - Roles of finite  $\beta$ , low  $l_i$  under study

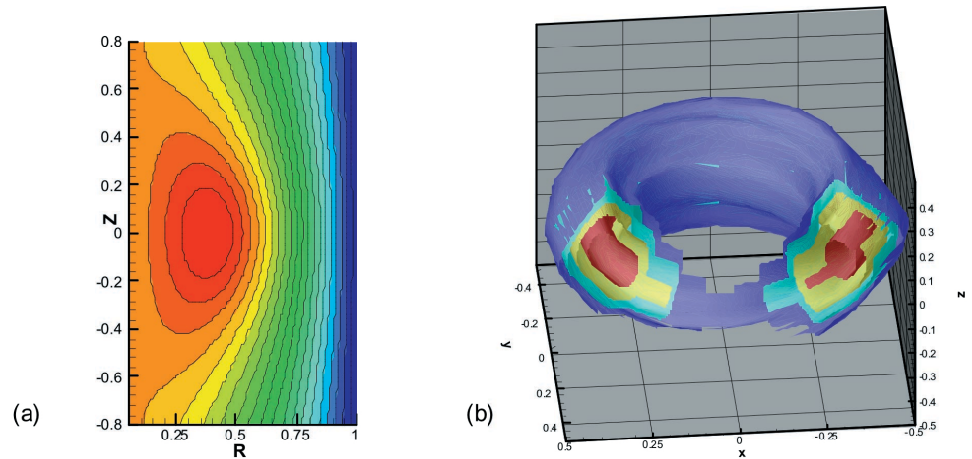




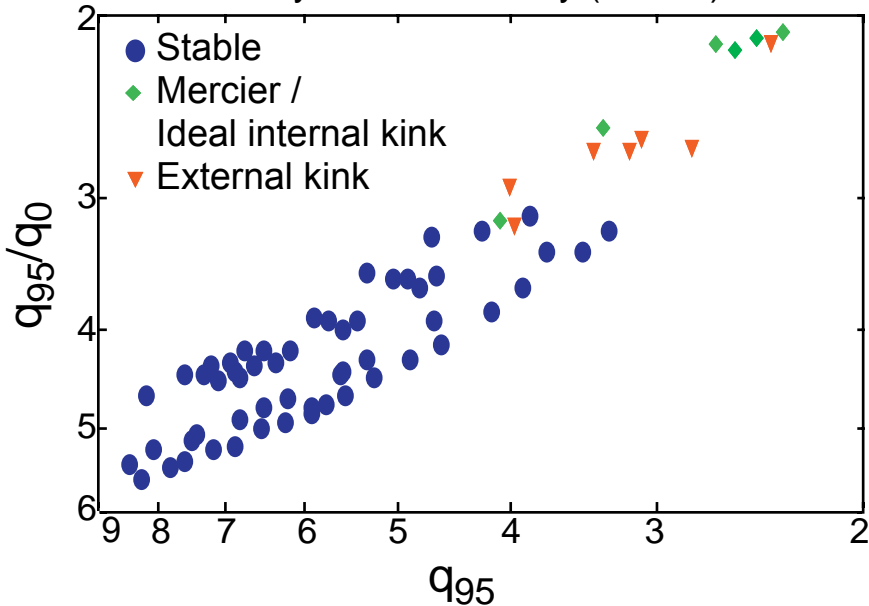
# A variety of computational tools are being implemented

- Equilibrium code
- Wall currents code
- Energy confinement code
- DCON: ideal stability analysis
- TSC: predictive discharge modelling
- GATO: instability growth rates
- NIMROD: resistive MHD analysis

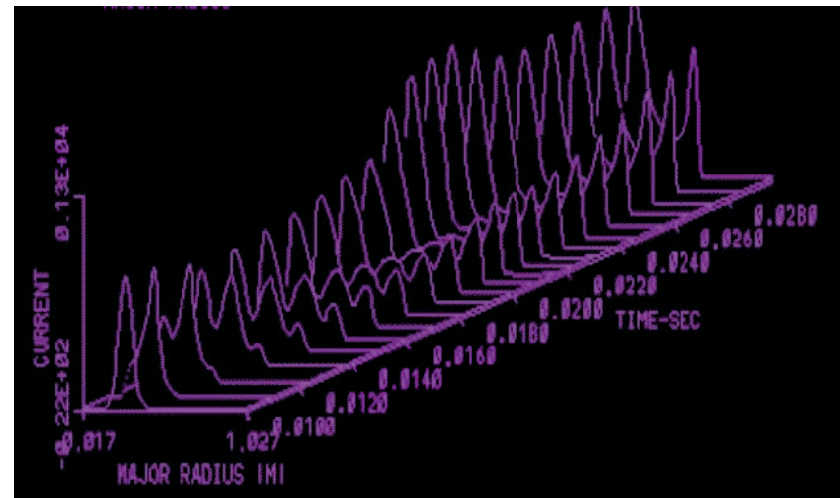
Preliminary Model of Resistive Stability (NIMROD)



Study of Kink Stability (DCON)



Evolution of  $J(r)$  during TF rampdown (TSC)





# Planned upgrades will allow further study of low- $q$ high- $\beta$ plasmas

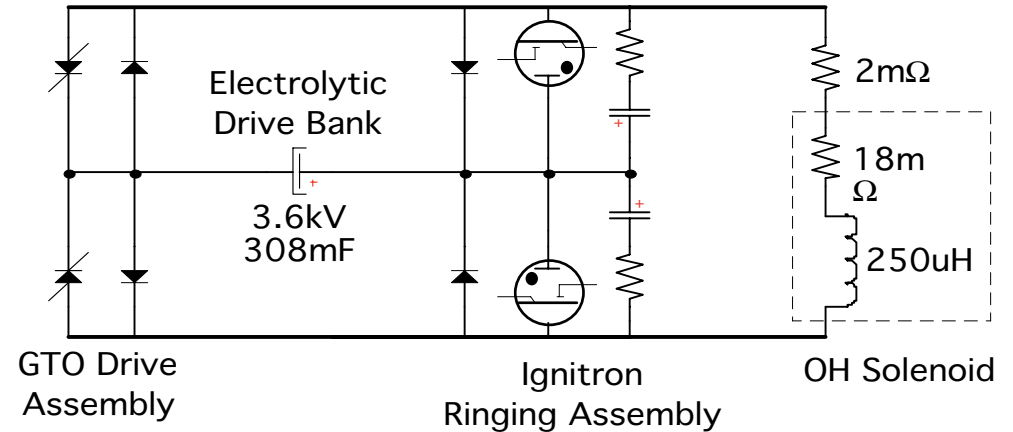
- **Goals require increased control of plasma conditions**
  - Density control and shot reproducibility = between-shot gettering
  - Improved equilibrium field control
- **Suppression of large internal MHD modes**
  - Increasing  $I_p$  ramp time = increased programmable V-sec from ohmic solenoid
  - Attain higher  $T_e(0)$  during formation = increased  $B_T$ , improved position control
  - HHFW heating = increased RF power operation, improved position control
  - Maintain  $q(0) > 2$  during plasma formation = increased  $B_T$
- **Control onset of suspected external kink modes**
  - Maintain  $I_p$  ramp time = increased programmable V-sec from ohmic solenoid
  - Maintain high  $q_{95}$  during formation = increased  $B_T$  w/rampdown
  - Controlled gas puff for edge cooling = continuous gettering
  - Separatrix operation = energize divertor coils
- **Access to very high  $\beta_T$  regime**
  - Increase  $T_e(0)$  during formation = increased  $B_T$  w/fast-rampdown
  - Increase  $I_p$  and  $N_e$  = increased V-sec
  - High-power HHFW heating = increased RF power operation



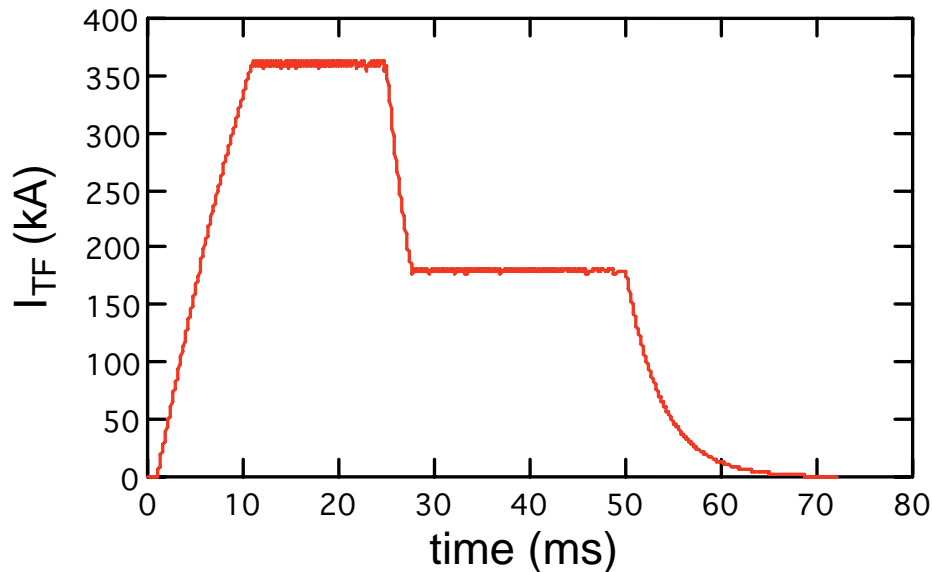
# New power supplies will provide dramatically improved control over plasma evolution

- Bipolar switching supplies replacing resonant L-C systems to provide flexible waveform control
  - *TF*: much higher  $I_{rod}$  and fast rampdown capability w/ new center rod
  - *OH*: near full use of available flux and programmable  $V_{loop}$  control
  - *EF*: programmable position control

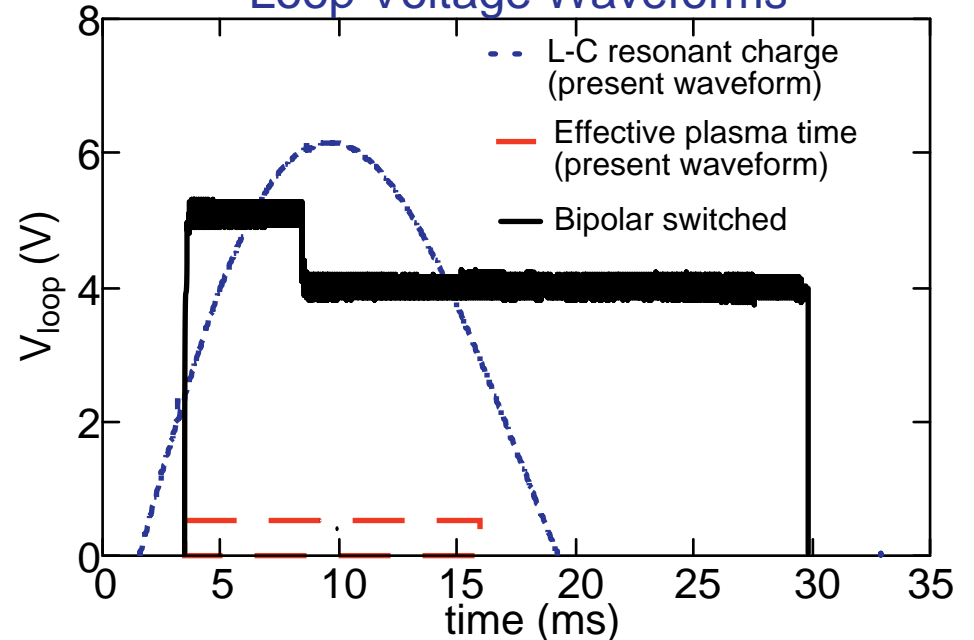
## Four Quadrant Ohmic Switch



## TF Waveform with Fast Rampdown



## Present and New Loop Voltage Waveforms







# Status & Project Plans for next 3-years

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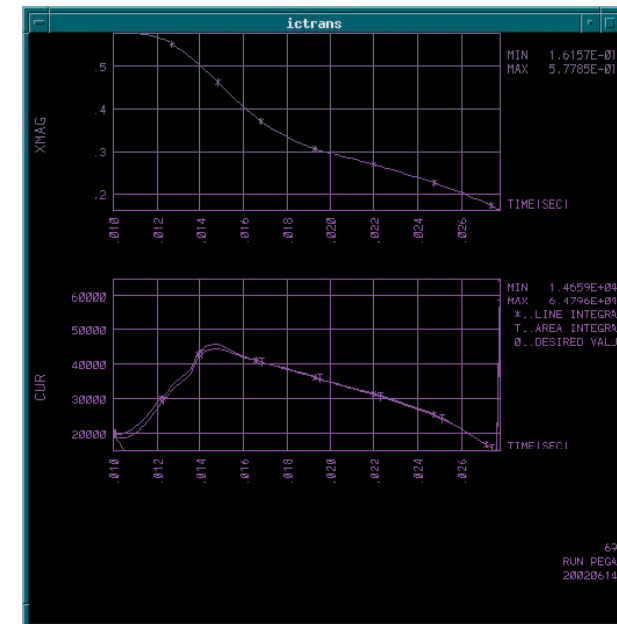
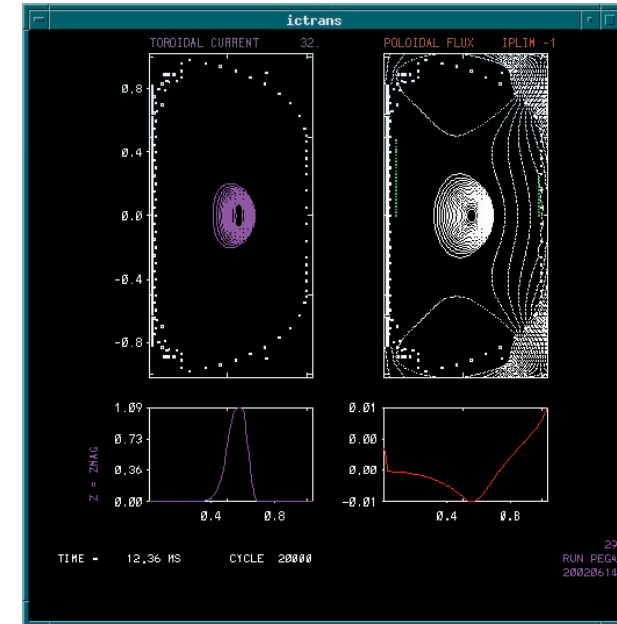
- **Renewal proposal submitted to continue for another 3 years**
  - *Year 1: Tool development and MHD suppression*
    - Programmable EF control; Increased V-sec;  $B_{TF}(t)$
    - Lab reconfiguration
  - *Year 2:  $I_p$  limits; access high  $\beta_t$ , low  $B_{TF}$  regime*
    - Internal MHD suppression thru  $q(0,t)$  and resistivity manipulation
    - Vary external mode onset thru edge cooling and shear control, plus geometry and  $l_i$  evolution
  - *Year 3: Stability boundaries documentation*
    - Increase stored energy through increased  $\Delta t_{pulse}$ ,  $I_p$ ,  $T_e$ ,  $N_e$ , and  $P_{aux}$
    - Documentation and parameter scans in tokamak-spheromak transition region
- **Have started ~11 month down period for renovation and upgrades**
  - *Lab reconfiguration or possible move to new high-bay area*
  - *Front-loading proposed major upgrades to present down period*
    - Upgraded OH power system  $\Rightarrow$  2-3 times effective V-sec
    - New EF power system  $\Rightarrow$  active radial position and shape control
    - Low-inductance TF system  $\Rightarrow$  time-variable TF



# Relation to NSTX and ST PoP Program

- **Present program is mainly complementary**
  - Explore extreme in  $A$ ; tokamak-spheromak transition region
  - Expand ST database for stability limits, etc.
  - Appropriate role for CE component of ST PoP program
- **Interested in developing future interactions with NSTX program...**
  - *EBW tests*
    - 500 kW, 2.45 GHz system feasible
    - Hoping to explore PPPL-ORNL-UW collaboration
  - *Diagnostic developments*
    - Tangential X-ray imaging
    - Low-field  $j(r)$  measurements
  - *Move to tests of startup and non-inductive ST techniques*
    - e.g., ECH, EBW, HHFW, external coil induction-compression, etc.
  - *Fueling with spheromak injection into diamagnetic plasma (UC-Davis)*
  - *Here to explore any and all ideas...*

*TSC: induction-compression startup with external coils*





# Summary: Progress in Development and Understanding Plasmas at Very Low A & B<sub>t</sub>

- **Facility and analysis developments ⇒ increased capability**
- **Plasma equilibria show low-A characteristics**
  - $\beta_t \sim 25\%$
  - $I_p/I_{TF} \sim 1.2$
  - $2/1, 3/2, \text{ double tearing modes, IREs, external kink}$
  - $\beta_N \sim 5$
  - $I_N \sim 8$
  - $n_e \sim n_{GW}$
  - $A \approx 1.16$
- **Access to low-B<sub>t</sub>, low-A operation: configuration and physics**
  - V-sec capability can limit access to interesting physics
  - Large internal modes (2/1, 3/2) degrade plasma evolution
    - *Susceptible due to large, low shear region and low T<sub>e</sub>?*
  - Evidence of access to external kinks at low  $l_i$
- **Proposed direction: access & document low-q, high  $\beta_t$  @ A → 1**
  - Characterize tokamak-spheromak overlap regime
  - Improved plasma control to manipulate MHD onset
    - *Increased  $I_p$ ; position/shape control; B<sub>TF</sub>(t)*
  - Separatrix operation for edge q control and possible H-mode