

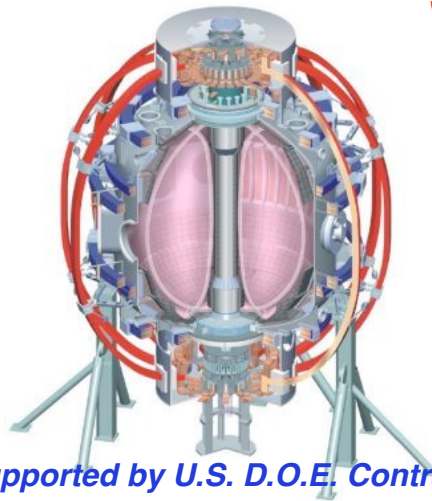
Coupling of Coaxial Helicity Injection plasma start-up to inductive ramp-up on the National Spherical Torus Experiment

D. Mueller, R. Raman, M. G. Bell, T.R. Jarboe, B.P. LeBlanc, B.A. Nelson, A.L. Roquemore, S.A. Sabbagh, and V.A. Soukhanovskii

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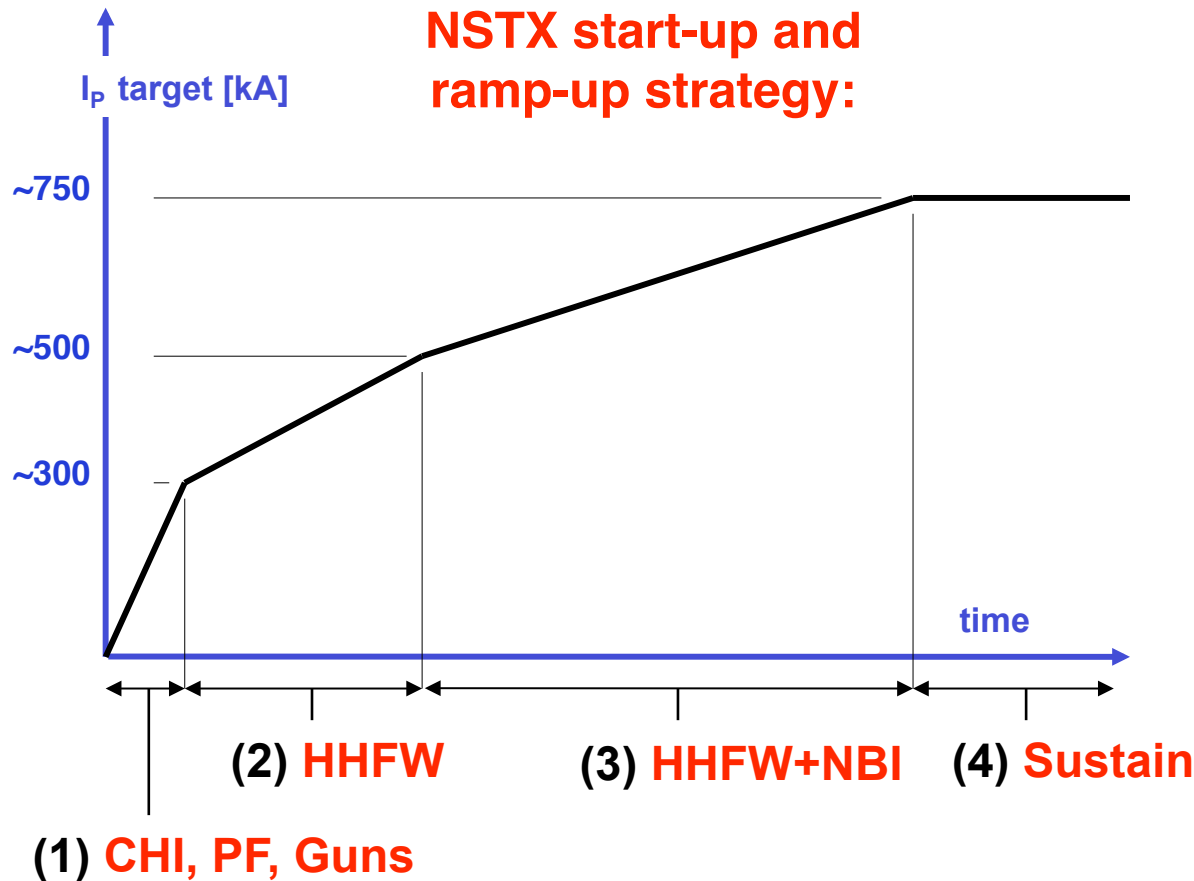
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Non-inductive start-up and ramp-up is a high priority for the future of the ST

NSTX start-up and ramp-up strategy:

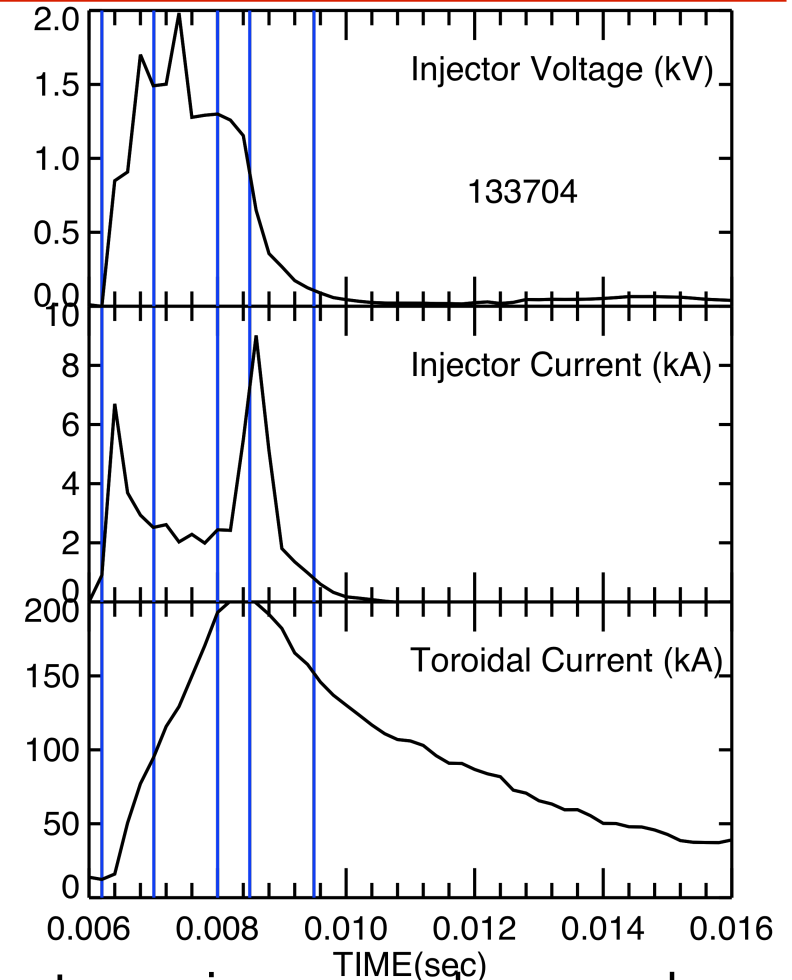
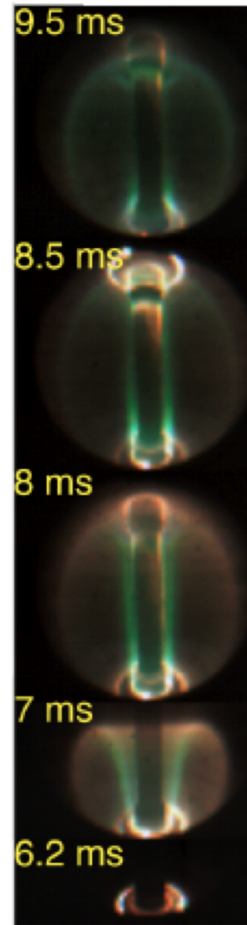
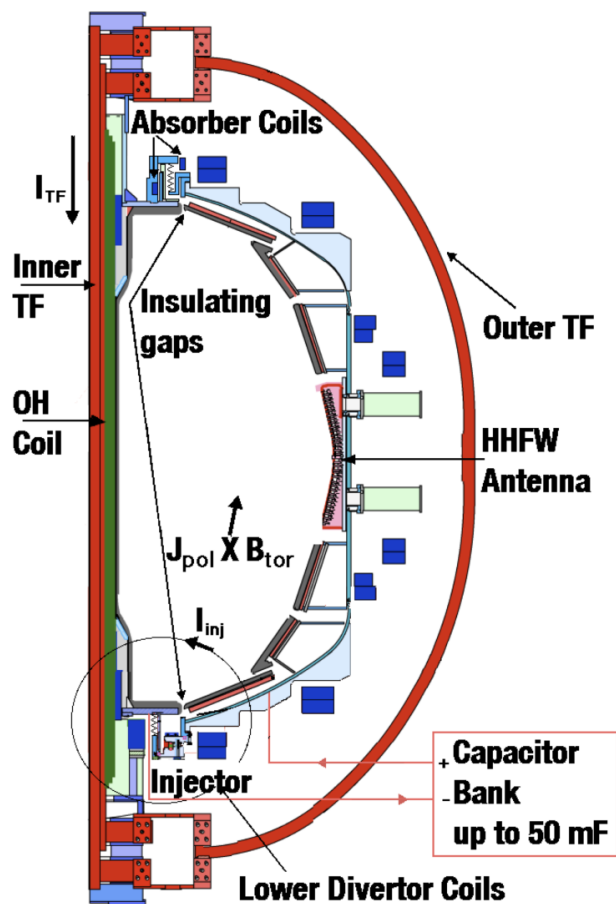


Start-up/ramp-up requirements:

- (1→2) I_p , T_e , RF coupling must be sufficiently high for HHFW to be absorbed
- (2) Sufficiently high P_{RF} , τ_E must be achieved for I_p overdrive using BS and HHFW current drive
- (2→3) Sufficiently high I_p needed to absorb NBI, high P_{HEAT} , τ_E , β_P needed for current overdrive
- (3→4) Ramp-up plasma must be consistent with sustained high- f_{NI} scenario

The NSTX non-solenoid plans are to progressively reduce the use of the central solenoid to investigate fully non-inductive discharges

Transient CHI: Axisymmetric reconnection leads to formation of closed flux surfaces



- Initial plasma on helical field lines in the injector region expands upward due to the $J_{pol} \times B_{tor}$ to fill the vessel
- Removing the injector voltage brings the injector current quickly to zero

Add inductive drive to CHI formed plasma

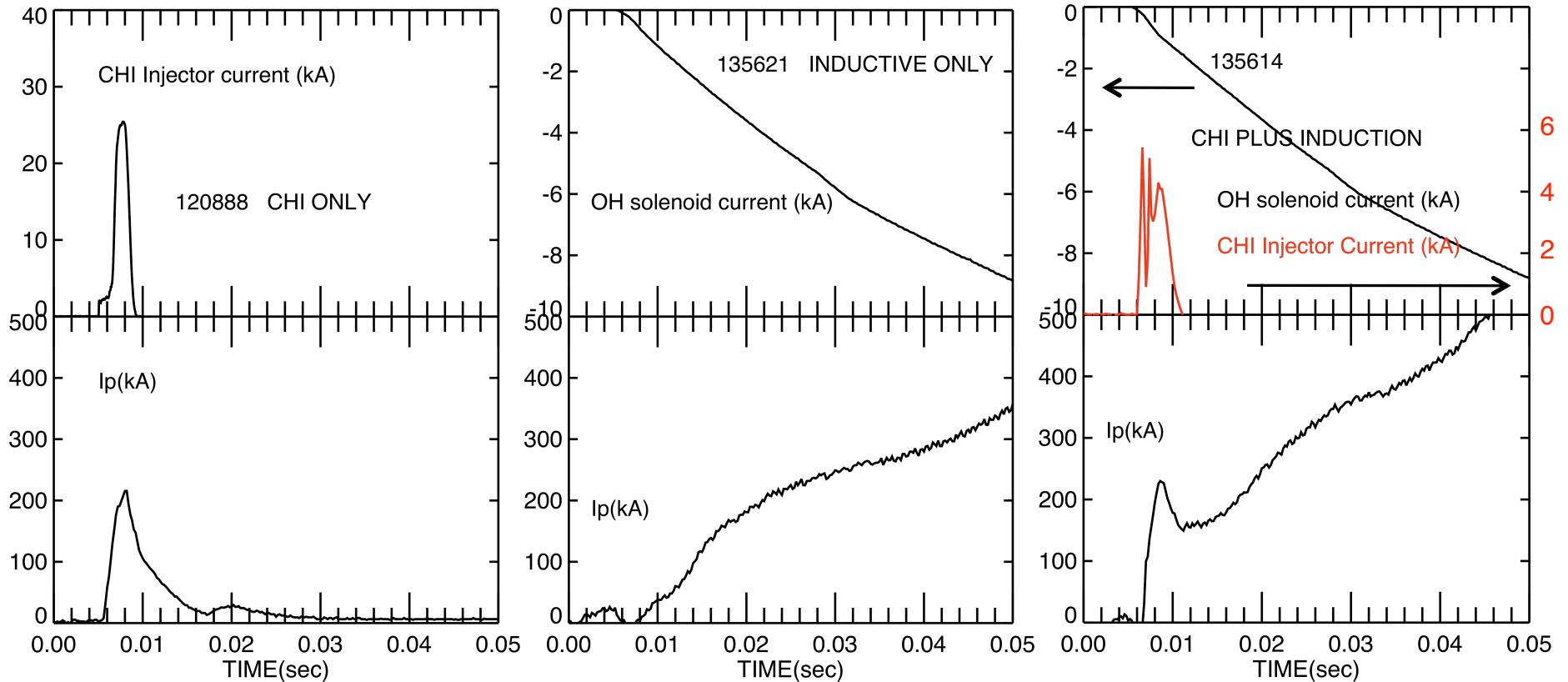
CHI START-UP



INDUCTIVE RAMP

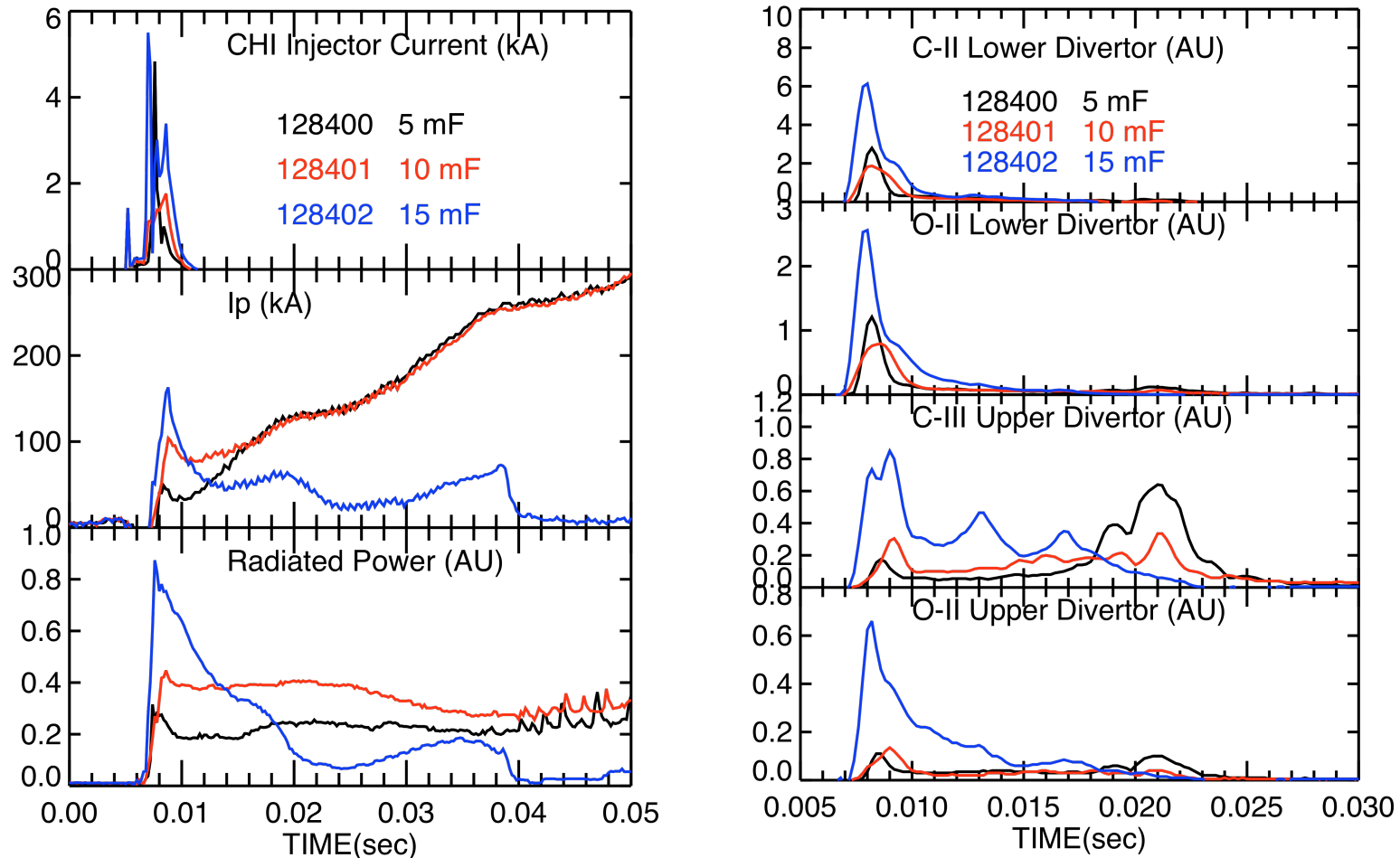


HIGHER CURRENT



- *The goal is to use CHI to establish a discharge that can be ramped up by other means*

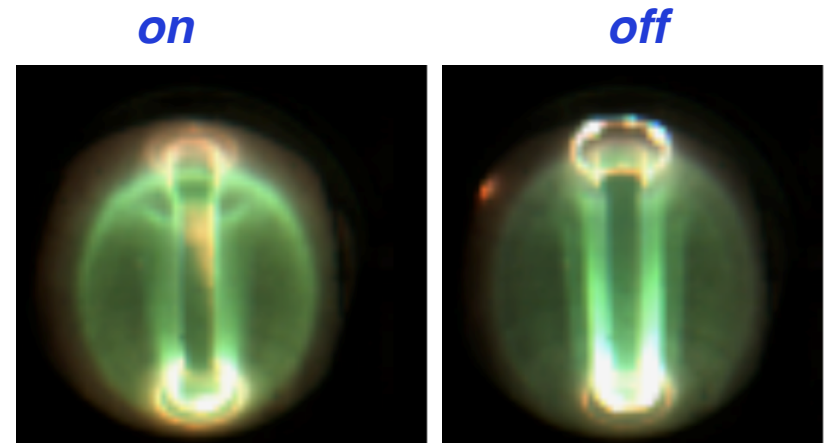
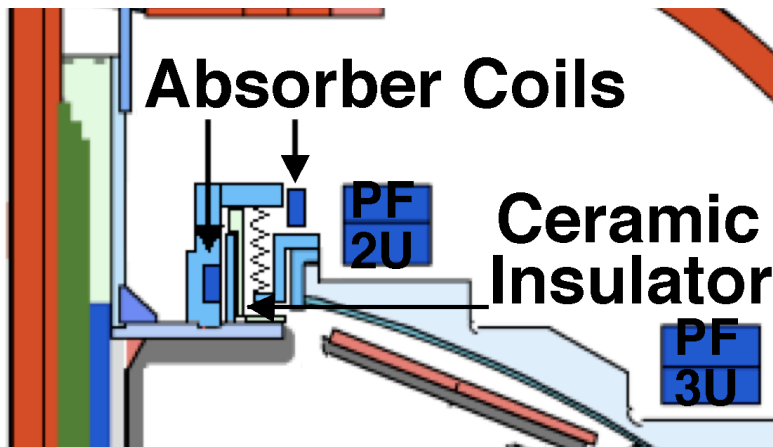
Low-Z impurities can limit the ability to ramp-up a CHI-initiated discharge with the central solenoid



- As the discharges grow upwards to fill the vessel absorber arcs occur.
- Applied $V_{loop} \sim 3.5$ V/turn
- Low-Z impurities increase with increasing injector current and capacitor energy

Use of the Absorber Coils can limit vertical growth of the plasma and prevent unwanted arcs in the absorber

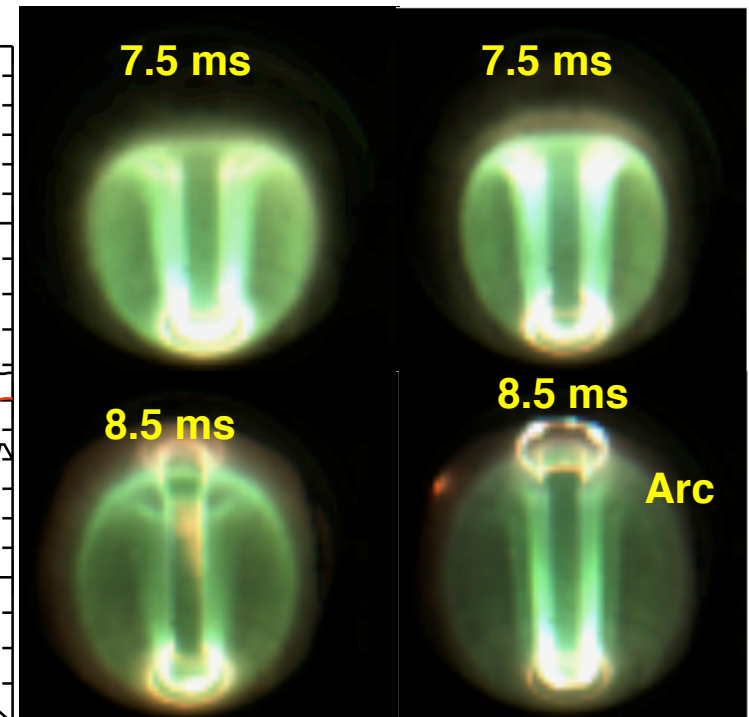
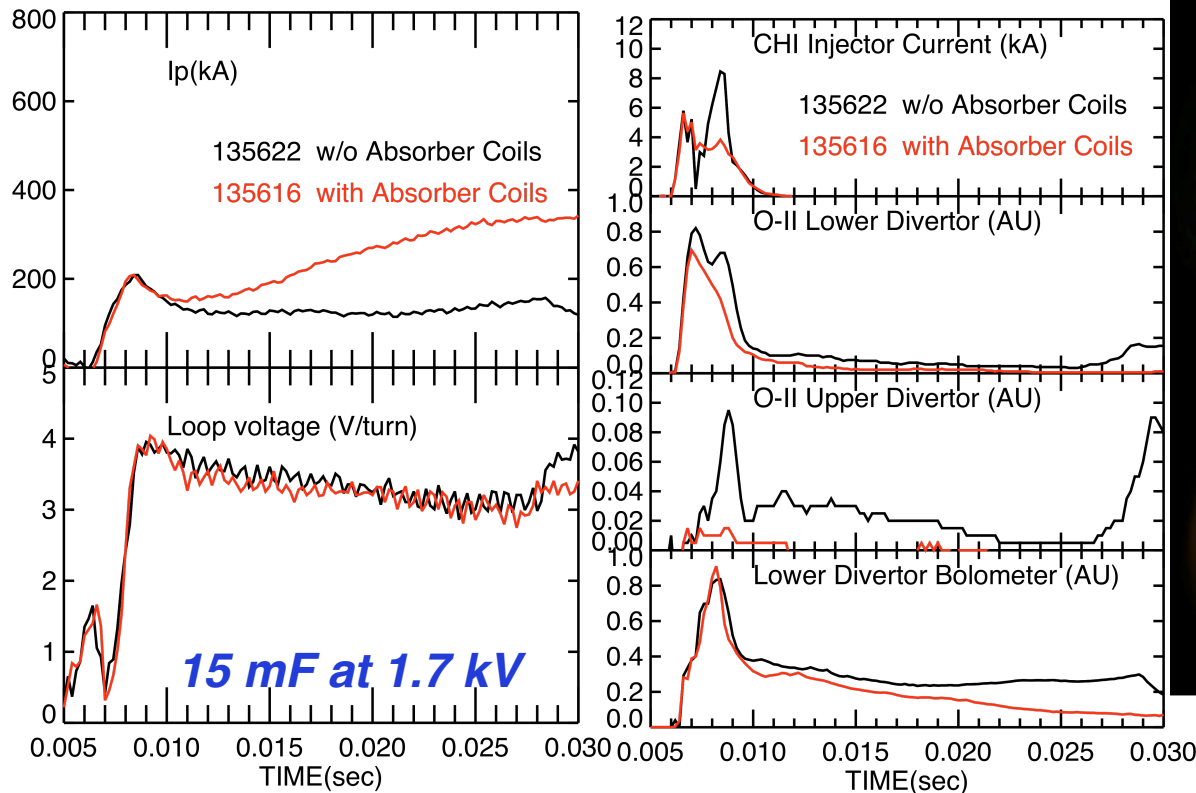
Absorber coils



- Providing a buffer flux prevents the plasma from growing into the absorber gap and eliminates arcs
- When arcs occur, low-Z impurities, primarily carbon and oxygen, are introduced into the discharge

Radial field from absorber coils prevent plasma from reaching absorber gap and reduces impurity influx

Red with Absorber Coils **Black without** **With Absorber Coils** **Without**

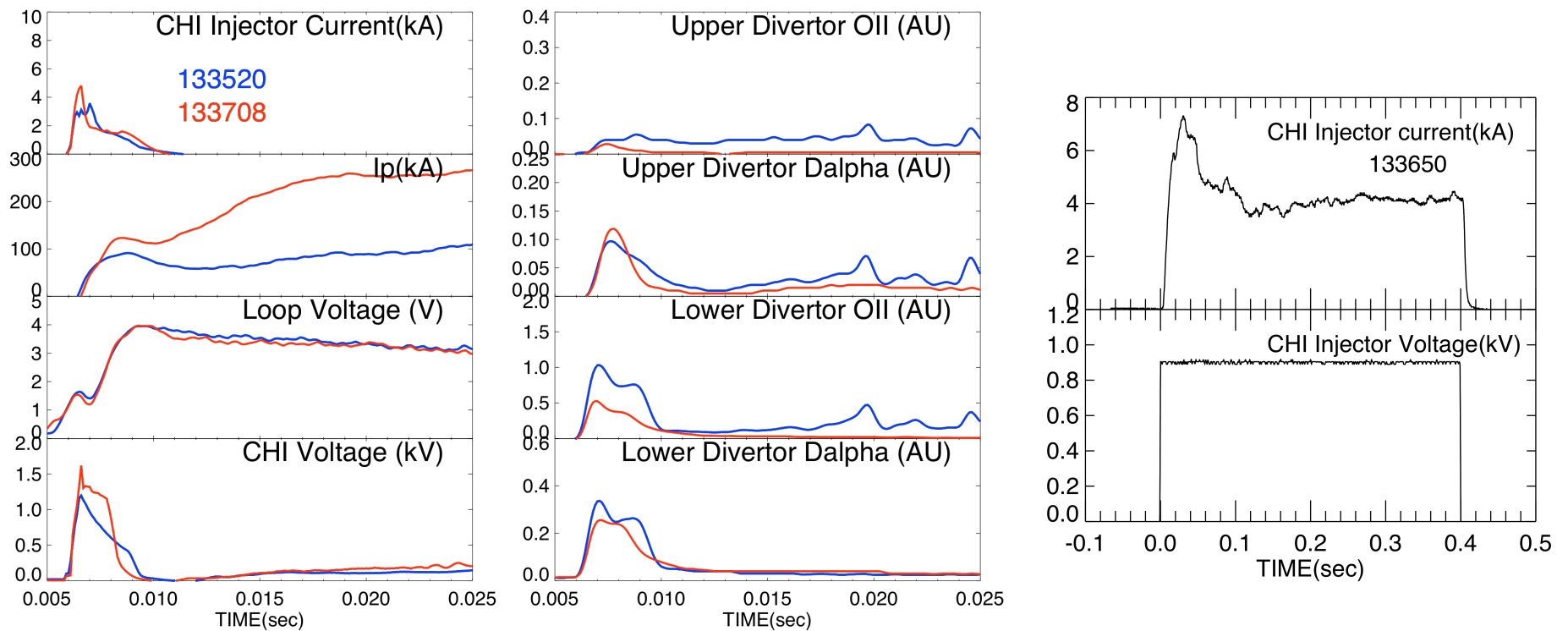


- Only the discharge without an absorber arc couples to inductive ramp-up
- Absorber arcs raise low-Z impurities, limit the ability to couple to ramp-up
- Use of the absorber coils reduces arcs, reduces impurity influx and improves coupling to inductive ramp-up

Discharge cleaning and lithium evaporation reduce low Z impurities and improve the coupling to inductive ramp-up

- *Blue before conditioning*
- *Red after conditioning and with lithium evaporation*

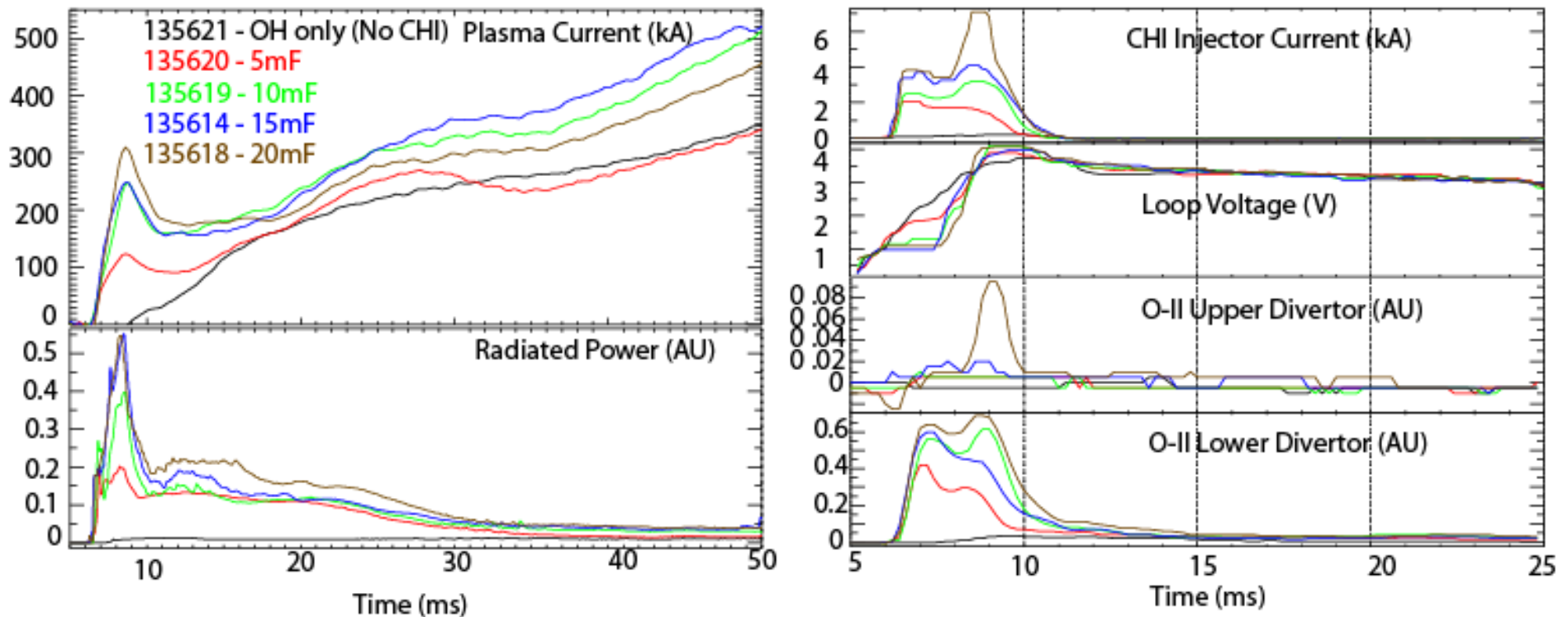
Conditioning Discharge
CHI rectifier supply – long pulse
High poloidal flux – limits expansion



- Conditioning of the lower divertor is also important

Final plasma current increases with CHI energy until absorber arc occurs

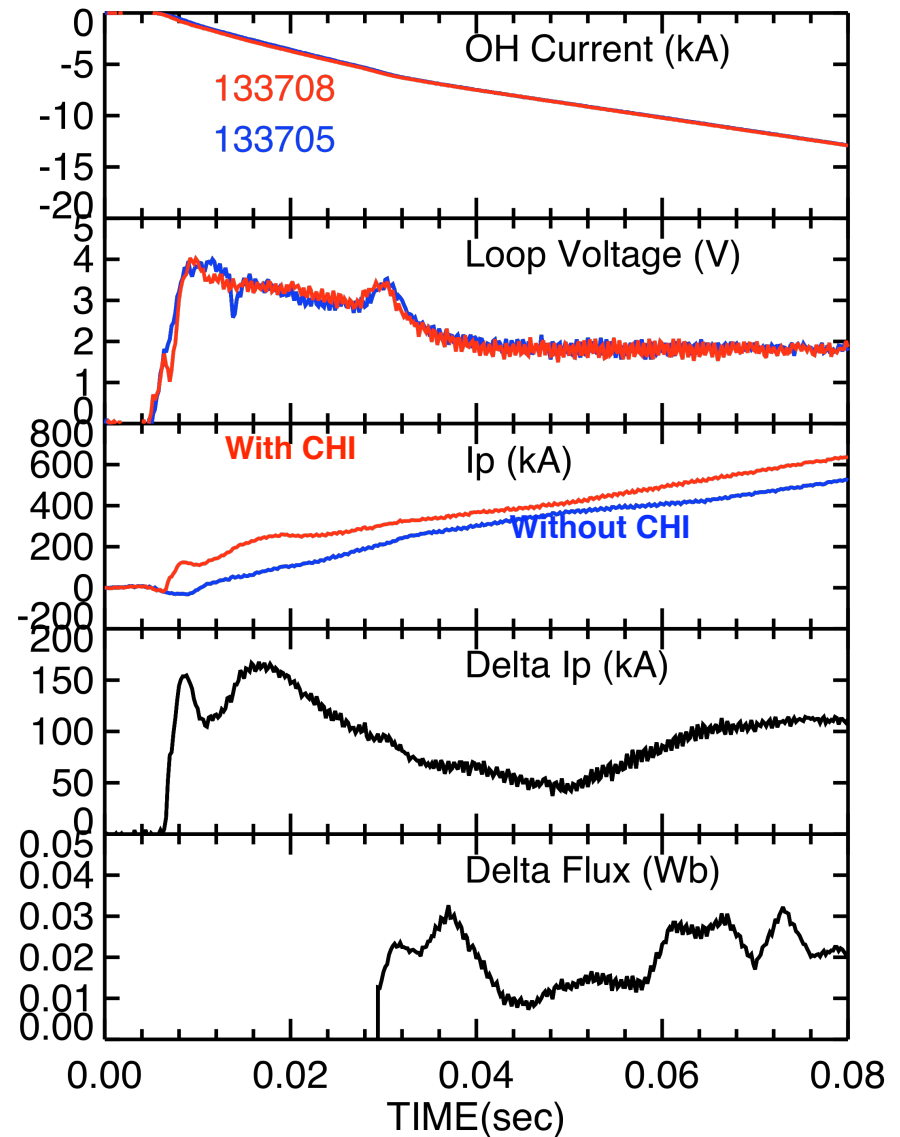
All discharges had identical solenoid (OH) current programming



- With CHI, $I_p \sim 200$ kA greater than ohmic only
- Successful coupling to induction using 20 mF (goal is arc-free with 50 mF)
 - Ignoring impurity effects, expect current to scale with capacitor energy

Comparison of well-controlled shots demonstrates higher I_p and greater internal poloidal flux with CHI initiation

- Compare discharges with identical solenoid current programming
 - Discharge in red with CHI (10 mF, 1.65 kV)
 - Discharge in blue is purely inductive
 - Both had low levels of $O_{||}$ emission
 - Density of CHI initiated discharge ~ 25% greater
- Final I_p is 110 kA greater in the CHI initiated discharge
- Internal inductance, plasma shape and radius from EFIT analysis are essentially identical
- Internal flux = $1/2 \mu_0 I_p I_i R_p$



NSTX is developing non-inductive start-up and ramp-up techniques for STs

- **Transient CHI a proven method to generate current on closed flux**
 - Start-up & inductive coupling of 200 kA non-inductive current demonstrated on NSTX **Goal: 500 kA start-up current**
 - Used absorber coils to reduce/eliminate absorber arcs
 - Used Li to reduce impurities during CHI
 - CHI initiated reached 700 kA in NBI heated H-mode plasmas
 - Will test CHI performance implications of metal electrodes (LLD plates); Can reducing the source of low Z impurities improve the start-up and coupling?
- **HHFW Heating and Current Drive for Ramp-up**
 - NSTX is investigating low current, low temperature targets for HHFW **Goal: replace inductive drive**
- **NBI Current Drive and Bootstrap Current routinely sustain about 65% of the plasma current non-inductively**
 - NSTX upgrade will increase that fraction **Goal: replace inductive drive**

Backup Slides

CHI Scaling

- From helicity and energy conservation, for a Taylor minimum energy state $\lambda_{inj} \geq \lambda_{tok}$

$$-\lambda_{inj} = \mu_0 I_{inj} / \psi_{inj}; \psi_{inj} = \text{poloidal injector flux}$$

$$-\lambda_{tok} = \mu_0 I_p / \psi_{tok}; \psi_{tok} = \text{toroidal flux in vessel}$$

- $I_p \leq I_{inj} (\psi_{tok} / \psi_{inj})$
- For similar B_T NSTX has 10 times ψ_{tok} of HIT-II

- Bubble burst condition:

$$I_{inj} = 2 \psi_{inj}^2 / (\mu_0^2 d^2 I_{TF}) \quad d = \text{flux footprint width}$$

-For HIT-II, $\psi_{inj} = 8 \text{ mWb}$, $d = 8 \text{ cm}$, for NSTX, $\psi_{inj} = 10 \text{ mWb}$, $d = 16 \text{ cm}$

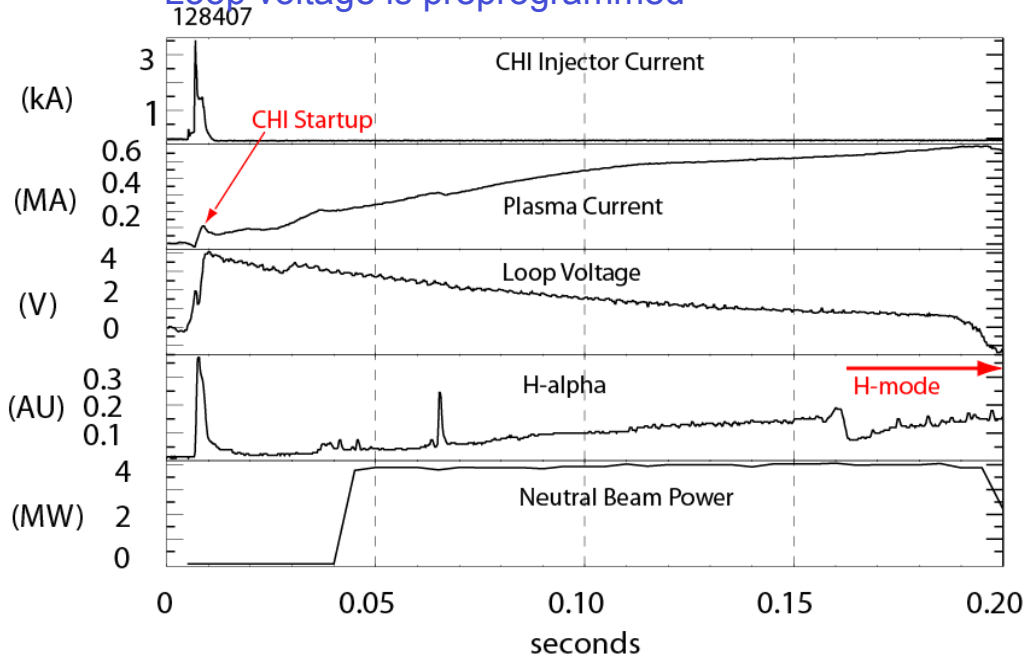
- $I_{inj} \geq 15 \text{ kA}$ for HIT-II, $I_{inj} \geq 2 \text{ kA}$ for NSTX

- Sufficient energy to produce CHI discharge $W_{cap} > W_{plasma}$; $\frac{1}{2} CV^2 > \frac{1}{2} L_p I_p^2$
 - L_p plasma inductance $\sim 0.5 \mu\text{H}$, and $C = 50 \text{ mF}$ limits I_p to $\sim 500 \text{ kA}$ for present NSTX system
- NSTX has achieved $I_p > 60 I_{inj}$; HIT-II has achieved $I_{inj} \sim 50 \text{ kA}$
 - $\Rightarrow I_p$ over 2.5 MA is possible for CTF if $I_{inj} \sim 50 \text{ kA}$

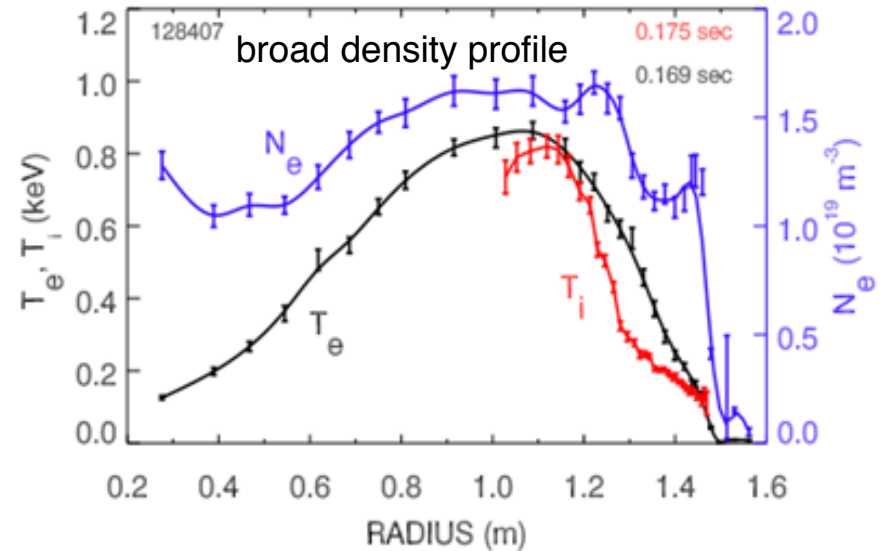
CHI started discharge couples to induction and transitions to an H-mode

demonstrates compatibility with high-performance plasma operation

- Discharge is under full plasma equilibrium position control
 - Loop voltage is preprogrammed



Te & Ne from Thomson
Ti from CHERS



Central Te reaches 800eV
Central Ti > 700eV

- Projected plasma current for $CTF > 2.5 \text{ MA}$ $[I_p = I_{inj}(\psi_{Tor}/\psi_{Pol})]^*$
 - Based on 50 kA injected current (Injector current densities achieved on HIT-II)
 - Current multiplication of 50 (achieved in NSTX)
 - In HIT-II nearly all CHI produced closed flux current is retained in the subsequent inductive ramp

CHERS: R. Bell, Thomson: B. LeBlanc

*T.R. Jarboe, Fusion Technology, 15 (1989) 7