#### **Novel divertors and NHTX simulations\***

#### T.D. Rognlien in collaboration with Dmitri Ryutov, Ron Cohen, and Maxim Umansky LLNL

Divertor discussion PPPL Nov. 20, 2007

\*This work was performed under the auspices of the U.S. Department of Energy by Lawrence Livermore National Laboratory in part under Contract W-7405-Eng-48 and in part under Contract DE-AC52-07NA27344.

#### Outline



- Heat load problem
- Some NHTX-specific results for orientation
- Divertor options
  - Plate tilting
  - Radiation
  - Flux-surface expansion
    - Snowflake divertor (1 VG; full discussion from Ryutov)
    - X-divertor
  - Lithium (NSTX results and prospects; kinetic edge models)
- Discussion

#### Future tokamaks have a major heat load problem







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### Nominal NHTX open divertor is symmetric double null



- Use UEDGE to model heat flux with "standard" D = 0.2 m<sup>2</sup>/s and  $\chi_{e,i}$  = 1 m<sup>2</sup>/s
- High recyling:  $R_p = 0.99$ ,  $R_w = 1.0$
- Scan core-edge density and power input



## Heat flux to outer divertor is very large in expected operating range (~50 MW)



- Electrons carry most of energy
- Results similar to Canick, but need to be more closely compared
- 1% neon fixed-concentration shows only small heat-flux reduction



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## Consider case 200058 with strike-point on outward-facing plate



- Here  $n_{core} = 1.5 \times 10^{20}$  and  $P_{core} = 30$  MW again
- Heat flux broadens and is reduced by ~x3
- Divertor density lower, T<sub>e</sub> is high; far from detachement



## Plate rotation "toward" strike-point can induce detachment - the ITER solution



- Neutrals are roughly directed normal to plate
- Pushing neutrals toward separatrix increases plasma density, recycling
- Reduction of heat flux larger than cos(theta)



## Radiation efficiency requires high density; transport effects can increase radiation



- Even lithium can give substantial radiation in a "detached-like" regime
- BUT high recycling Li plate doesn't give high enough density



Li multi-species with transport; wall is Li, but plate high recycling



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## Snowflake divertor produces a (near) 2nd order field null for flux expansion (Ryutov, PoP '07)



- Simple two-wire X-point model generalized by adding another divertor wire
- Stability of configuration obtained by operating off perfect snowflake - see Ryutov



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#### X-divertor utilizes multiple divertor coils to produce a nearby flux expansion (Kotchenreuther, PoP '07)



- Technology may be more complicated
- Offers possibility of stabilizing MARFE formation away from X-point and core plasma
- 1/2 radiation still localized near plates





- Kinetic modifications needed for edge-plasma transport
- Low-recycling (lithium) divertor (NSTX example)
  - higher edge temperatures
  - plate heat flux about the same
- Core contamination by lithium appears acceptable
- Developing kinetic transport codes (e.g., TEMPEST and XGC) with be very useful here

# Parallel plasma transport is modified for a hot, low-density edge plasma



- Mean-free-path  $\lambda \propto T^2$  /  $n_e$  and  $L_{||}$  is B-field-line length
- When λ ~ L<sub>||</sub>, the velocityspace distribution is non-Maxwellian
- Plasma heat and momentum transport coefficients change
- Sheath boundary conditions are modified & normalized potential eφ/T<sub>e</sub> can decrease



![](_page_12_Picture_1.jpeg)

• Flux-limiting of transport coefficients have the form

$$q_c = -\kappa_e \frac{\partial T_e}{\partial s_{\parallel}} \to -\kappa_e [1 + (q_c/q_f)^2]^{-1/2} \frac{\partial T_e}{\partial s_{\parallel}},$$

where  $\kappa_e$  is the classical heat conductivity,  $s_{\parallel}$  is the distance along B, and  $q_f = c_e n T_e (2T_e/m_e)^{1/2}$  with  $c_e \approx 0.15$ .

• Thermal force term has kinetic correction of the form

$$0.71n\nabla_{\parallel}T_e \rightarrow \frac{0.71n\nabla_{\parallel}T_e}{(1+\lambda_{mfp}/L_{\parallel})}$$

![](_page_13_Picture_1.jpeg)

• Electron kinetic energy lost at the collisional sheath is

$$2T_e\Gamma_e = 2T_e \frac{n\bar{v}_e}{4} \exp(e\phi_s/T_e)$$

• Monte Carlo simulations show that long mean-free-path regime can be treated by the extended formula

$$2T_e\Gamma_e \to 2T_e \frac{n\bar{v}_e}{4} \frac{\exp(e\phi_s/T_e)}{(1+\tau_p/\tau_c)}$$

where  $\tau_p$  is long mean-free path Pastukhov confinement time, and  $\tau_c$  is the confinement time for the collisional (full velocity loss-cone) sheath-limited regime. Here the correction factor is

$$1/(1+\tau_p/\tau_c) \approx 1/[1+\alpha_c(\lambda_{mfp}/L_{\parallel})(e\phi_s/T_e)]$$

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## Effect of low-recycling is illustrated for an NSTX case

![](_page_14_Picture_1.jpeg)

- Use 2D UEDGE fluid transport with kinetic corrections
- Begin with a base-case with high recycling (shot 109034, Porter)
  - P<sub>core</sub> = 2 MW
  - $D = 0.5 \text{ m}^2/\text{s}, c = 1.5 \text{ m}^2/\text{s}$
  - R = 1.0
  - Wall gas albedo = 0.95
  - Carbon impurity
- Solutions for R = 1.0, 0.9, 0.5, and 0.2 on outer divertor only

![](_page_14_Figure_10.jpeg)

## Low recycling increases midplane temperatures by factor of ~2

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![](_page_15_Picture_1.jpeg)

![](_page_15_Figure_2.jpeg)

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#### Peak divertor heat flux largely unchanged by R

![](_page_16_Picture_1.jpeg)

- Direct energy loss from escaping particles scales as nT T<sup>1/2</sup> = nT<sup>3/2</sup> at the divertor
- For high collisionality, nT ~ constant along B-field
  - implies heat flux ~  $T^{1/2}$
- However, for low recycling, nT|<sub>midplane</sub> >> nT|<sub>divertor</sub>

![](_page_16_Figure_6.jpeg)

#### A substantial convective particle loss about the outer midplane can decrease effect of R

![](_page_17_Picture_1.jpeg)

- Add a radial convective velocity increasing from 10 m/s --> 100 m/s from core --> wall
- Increased wall flux gives increased pumping (albedo=0.95) model dependent
- Effectiveness of divertor pumping is decreased

#### **Electron temperature**

![](_page_17_Figure_6.jpeg)

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# Contamination of core from lithium divertor being modeled by coupling UEDGE & WBC MC code

![](_page_18_Picture_1.jpeg)

- Heat and particle flux to module computed by UEDGE
- Temperature rise of Li surface from heat transfer (Ulrickson)
- Sputtering of Li from U. III. composite model (Allain et al.)
- WBC calculates lithium source near the divertor plate (Brooks)
- UEDGE uses this Li source to calculate lithium density throughout the edge region

#### NSTX divertor region + Li module

![](_page_18_Figure_8.jpeg)

# Lithium flows throughout the SOL, but core boundary concentration appears low

![](_page_19_Picture_1.jpeg)

- Lithium concentration peaks in outer SOL and private-flux regions
- Primary forces keeping Li in divertor are E<sub>p</sub> & hydrogen drag
- Lower recycling good because
  - Lower sputtering hydrogen flux
  - Monotonic downward E<sub>p</sub>;
    R=0.2 much better than R=0.9
  - Higher sputtering rate is bad

![](_page_19_Figure_8.jpeg)

#### Lithium concentration, n\_li/n\_e

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#### Summary

![](_page_20_Picture_1.jpeg)

- Simulating low-recycling plasmas uses known modifications to fluid transport models
- Substantial increase in edge temperature is calculated
- Plasma convection shows some reduction of core-edge T increase
- Lithium contamination studies for NSTX begun
  - Impurity screening at low R aided by  $E_p \&$  downward hydrogen flow
  - Role of convection on Li must be included