Divertor heat flux reduction and detachment studies in NSTX

Status Report by Vlad Soukhanovskii (LLNL)

- Components of divertor heat flux reduction and detachment studies in NSTX:
 - Studies in low δ , κ LSN plasmas (highest heat flux to date 10 MW/m²) with D₂ puffing
 - Studies in high δ,κ LSN plasmas with high flux expansion, with D_2 puffing
 - Studies with impurity puffing (CD₄, Ne)
 - DN studies
 - Lithium
- This work: steady-state divertor heat load in H-mode plasmas with D₂ inj.
- IAEA presentation an extended (detailed) version of PSI 17 paper
- Data analysis in good shape
- Modeling is in progress:
 - Two point model with losses plan to finish by IAEA paper deadline
 - UEDGE edge transport modeling for a generic LSN case plan to include in the NF version of IAEA paper



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IAEA FEC 2006 poster P4-28: Divertor heat flux reduction and detachment in NSTX

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- Introduction
- Inner divertor leg, MARFEs
- Experiment steady-state heat flux reduction at OSP in δ =0.4, κ =2 *H-mode* plasmas with D₂ puffing
 - Partially detached divertor x4 peak heat flux reduction
 - High-recycling radiative divertor same
 - Divertor data: heat flux profiles, recycling and carbon profiles, volume recombination, estimates of e-i recombination sink, Langmuir probe data I_{sat}, T_e, n_e, spectroscopic data from hign-n Balmer series lines T_e, n_e, divertor P_{rad}
 - Core data during gas injection (confinement, carbon concentration, rotation)
- Discussion
 - Operational window in NSTX
 - Why is it difficult to detach OSP in NSTX? (Ion momentum and rad. power loss from 2PM as a result of NSTX open divertor and short connection length)
 - Prospects for heat flux reduction in NSTX and future ST-based devices



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