

Recent PPPL/DIII-D Activities and Plans

by

B.A. Grierson

Princeton Plasma Physics Laboratory

Presented at the NSTX-U/Magnetic Fusion Science Weekly
Physics Meeting

B318, PPPL

August 20, 2018



PPPL is the Largest Collaborating Institution on DIII-D



**Raffi
Nazikian**



**Brian
Grierson**



**Alessandro
Bortolon**



**Nikolas
Logan**



**Shaun
Haskey**

- Seven Ph.D. Physicists*
- One graduate student*



**Arash
Ashourvan**



**Qiming
Hu**



**Matthias
Knolker**

* Permanent off-site

PPPL Leads and Manages Major Engineering and Operations Elements of the DIII-D Program



Off-Site Staff Maintain Close Connection to the Lab Through the Physics Topical Areas

3D Fields & Stability

- ELM control with RMPs
- Entrain MHD modes and RT stability
- NTV and plasma rotation

Integrated Modeling & Scenarios

- Thermal transport, heating and current drive
- EP transport
- Scenario design and model validation
- Extrapolation to ITER

Core-Edge Integration

- Impurity delivery and control
- Pellet pacing
- Pedestal structure and transport
- Heat flux mitigation

Off-Site Staff Maintain Close Connection to the Lab Through the Physics Topical Areas

3D Fields & Stability

- J-K Park, N. Ferraro, M.D. Boyer, M. Okabayashi, K. Erickson, W. Tang, Z.R. Wang
- M3D-C1
- IPEC
- GPEC
- DCON

Integrated Modeling & Scenarios

- F. Poli, S. Kaye, W. Guttenfelder, W.X. Wang, M. Podesta, N. Gorelenkov, G. Kramer
- TRANSP
- RBQ
- GTS

Core-Edge Integration

- R. Maingi, A. Diallo, R. Lunsford, R. Hager, W. Guttenfelder, C.S. Chang
- Impurity technology
- XGC, CGYRO

Select Recent Physics Results*

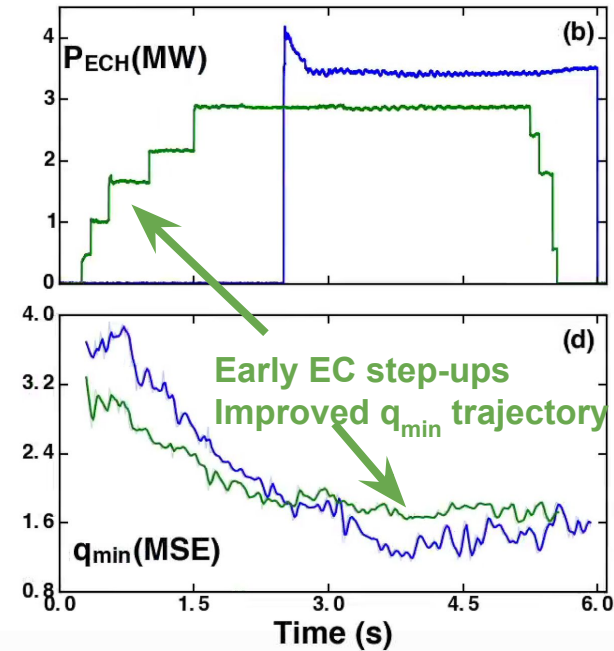
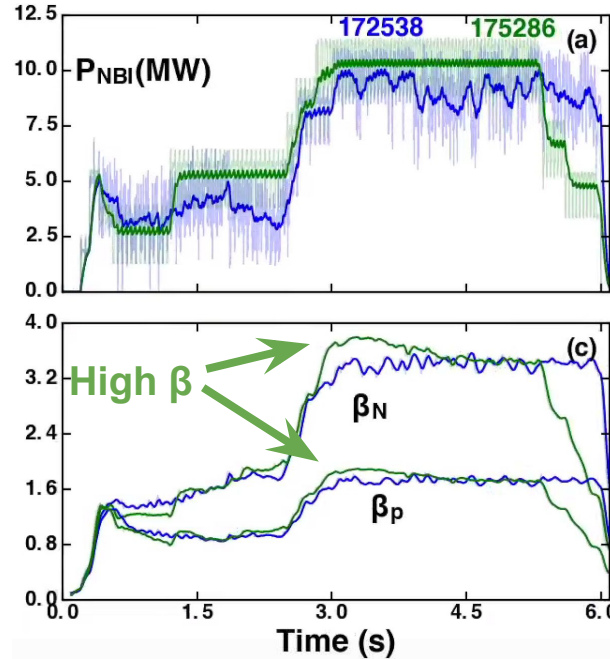
* Too many to cover in short time, apologies



"Predict First" EC and NBI Trajectory Achieved High Beta Access with Smooth, Elevated and Sustained q_{min}

- Challenge to achieve operating point in high- β steady-state AT

- Operational space limited by onset of TMs and degraded confinement
- Empirical "recipe" developed in control room
- #1 Goal: Test predictive TRANSP simulations



"Predict First" EC and NBI Trajectory Achieved High Beta Access with Smooth, Elevated, Sustained q_{min}

Shot 175286

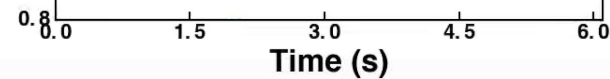
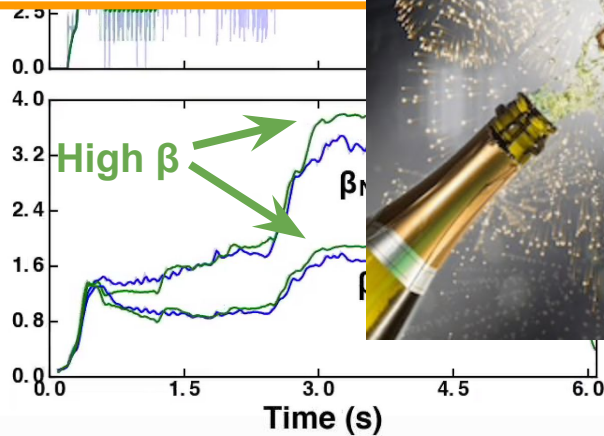
SESSION_LEADER: victorb

Preshot:

Early EC power and changing the NBI to achieve higher q_{min} . Shots designed by F. Poli

requested ip: 0.85 MA, btor: 1.65 T, pnbi: 0 MW,
Step in Miniproposal 2018-22-01 experimental plan

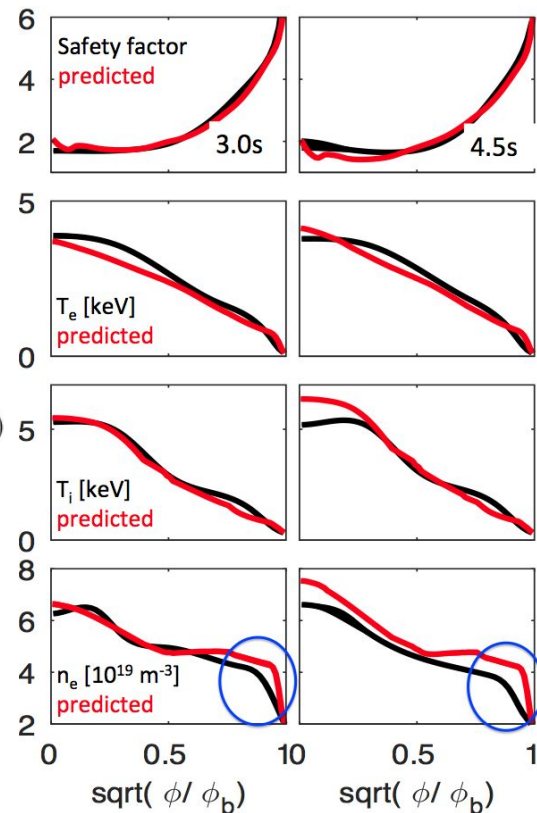
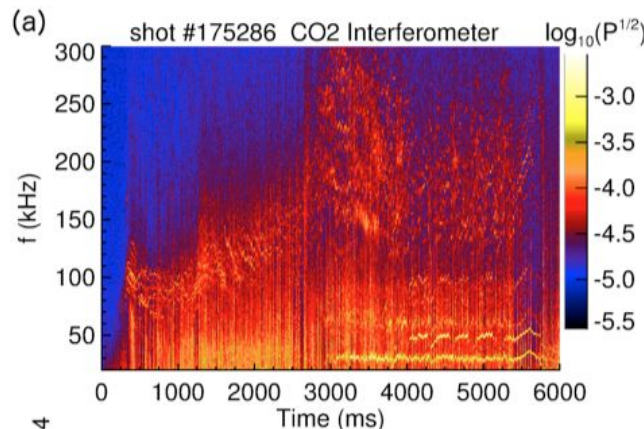
- Empirical "recipe" developed in control room
- #1 Goal: Test predictive TRANSP simulations



Validation of Predictions Against Achieved High qmin Scenario Identifies Key Areas Where Model Improvements are Required

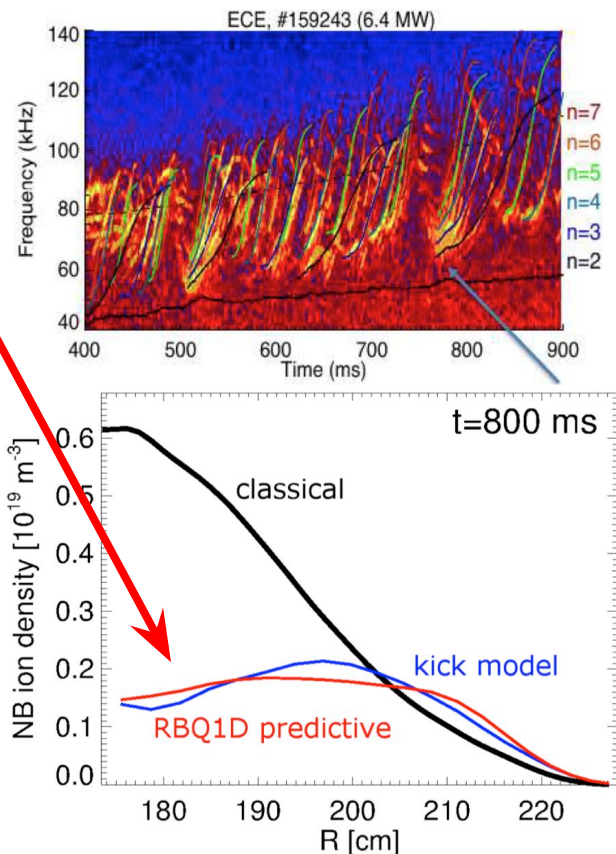
- Investigation beyond global agreement with predictions reveals
 - Edge density pedestal needs improved particle transport model
 - Neutron discrepancy requires AE transport

- Integrate reduced AE transport models
- Improve pedestal density predictions



Reduced Models of AE-Induced Transport are Showing Promising Performance for Use in Scenario Design

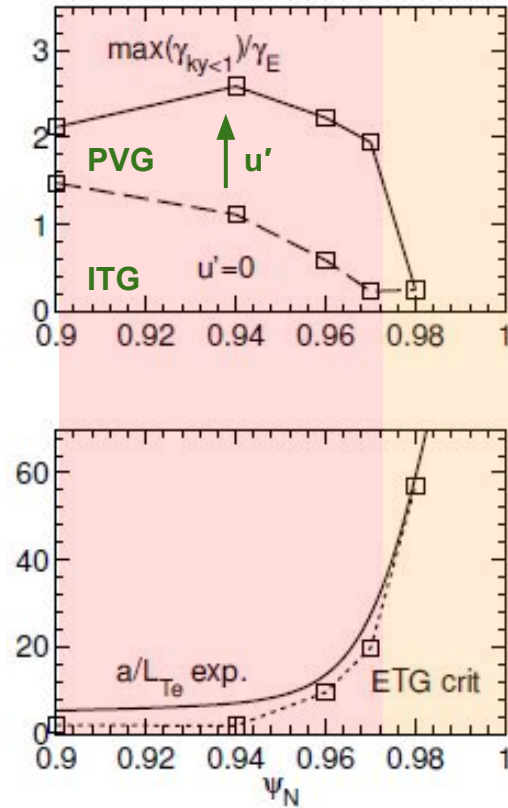
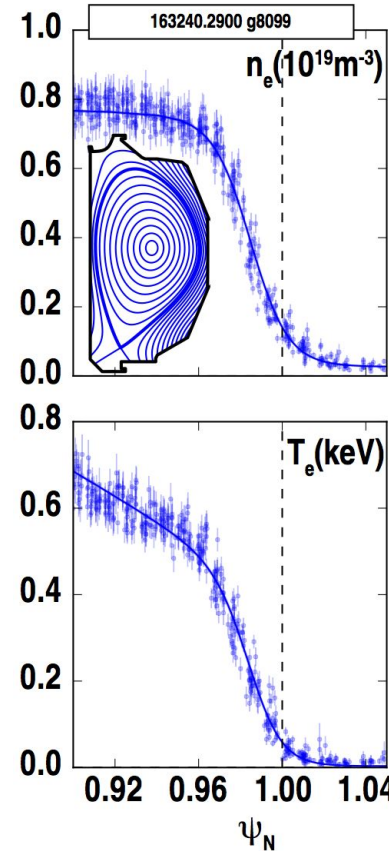
- RSAEs cause transport by perturbing the fast-particle distribution function
- RBQ-1D provides diffusion in canonical angular momentum space and captures *profile shape*
- Mostly passing particles near injection energy of 60-70 keV
 - Depletes NBCD
- Perturbed distribution function available for validation with FIDA



N.N. Gorolenkov AAPPs-DPP Invited (2017)

CGYRO Simulations Indicate Electrostatic Instabilities Dominant From Top of Pedestal into Steep Gradient

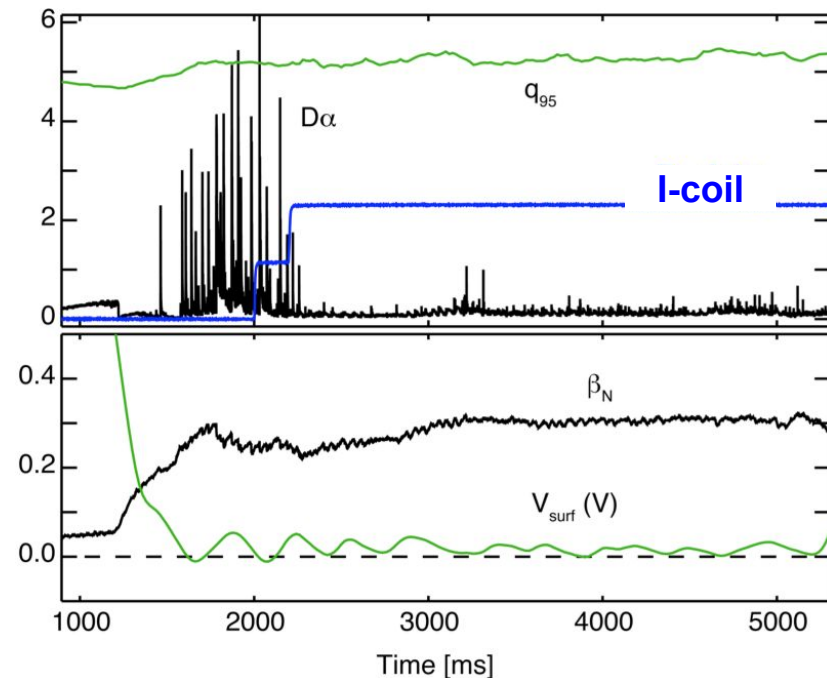
- Goal for FY19 JRT is quantifying *particle transport*
- Linear simulations show transition from ITG/PVG \rightarrow ETG modes radially
 - Rotation shear drives PVG
 - $\gamma(\text{ITG/PVG}) > \gamma(\text{ExB})$ inside 0.96
- Pedestal a/L_{T_e} tracks critical gradient for ETG ($\eta_e \text{ crit} \sim 1.2$)
- Nonlinear simulations in progress to quantify energy and particle fluxes



ITER Q=5 Steady State Target with ELM Suppression Extended to q95=5.2; Enters Oscillating Two-State Pedestal

- Pedestal “pulsations” driven by cyclic variations in the resonant field strength at top of pedestal
- Bifurcations between ELM suppression and Grassy-ELM regime moderates divertor heat flux
- Dramatic change in pedestal structure
 - Density flattens, zero of ExB shear widens
 - Pedestal grows wider, higher
 - Drives need to understand edge rotation and radial electric field E_r

R. Nazikian NF (2018), Press Release

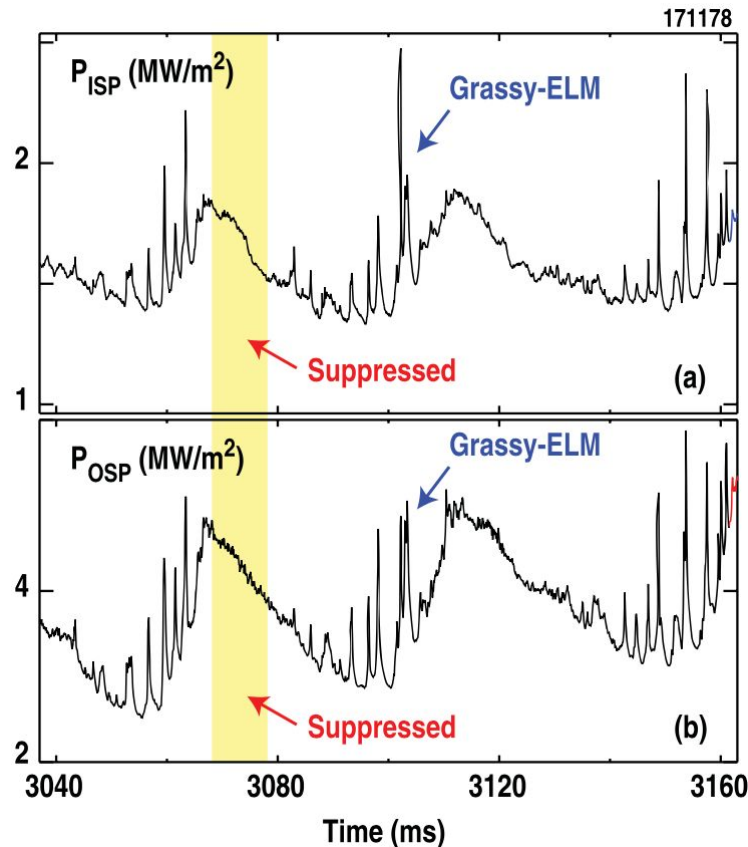


Ohmic transformer turned off

ITER Q=5 Steady State Target with ELM Suppression Extended to q95=5.2; Enters Oscillating Two-State Pedestal

- Pedestal “pulsations” driven by cyclic variations in the resonant field strength at top of pedestal
- Bifurcations between ELM suppression and Grassy-ELM regime moderates divertor heat flux
- Dramatic change in pedestal structure
 - Density flattens, zero of ExB shear widens
 - Pedestal grows wider, higher
 - Drives need to understand edge rotation and radial electric field E_r

R. Nazikian NF (2018), Press Release



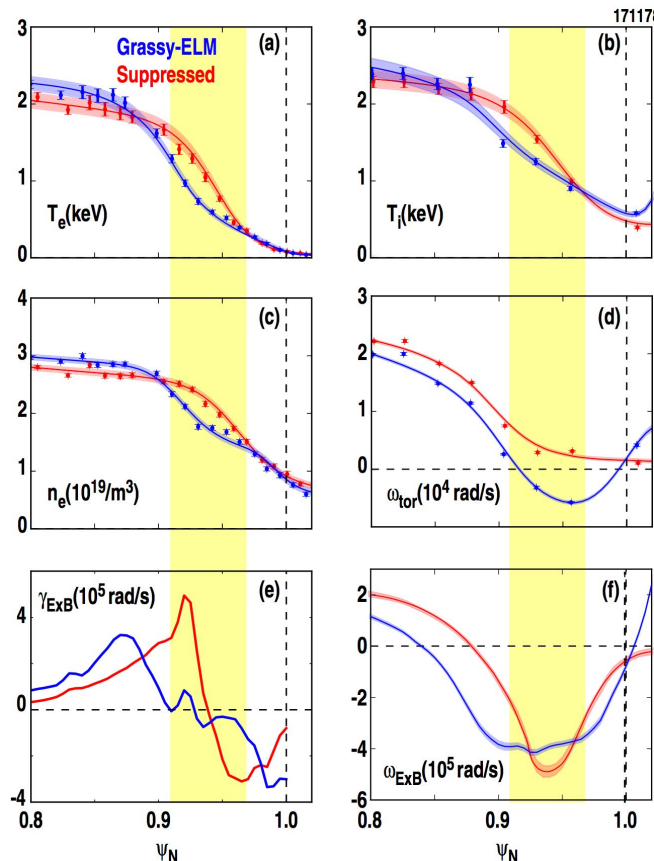
ITER Q=5 Steady State Target with ELM Suppression Extended to q95=5.2; Enters Oscillating Two-State Pedestal

- Pedestal “pulsations” driven by cyclic variations in the resonant field strength at top of pedestal
- Bifurcations between ELM suppression and Grassy-ELM regime moderates divertor heat flux
- Dramatic change in pedestal structure

- Density flattens, zero of ExB shear widens
- Pedestal grows wider, higher
- Drives need to understand edge rotation and radial electric field E_r

R. Nazikian NF (2018), Press Release

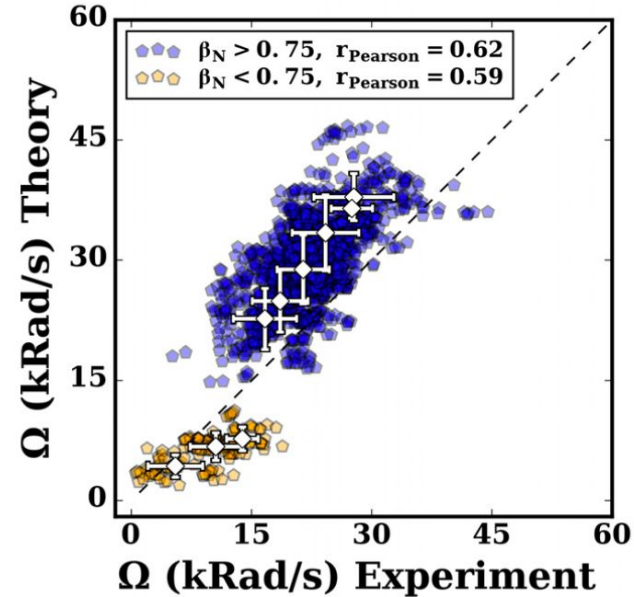
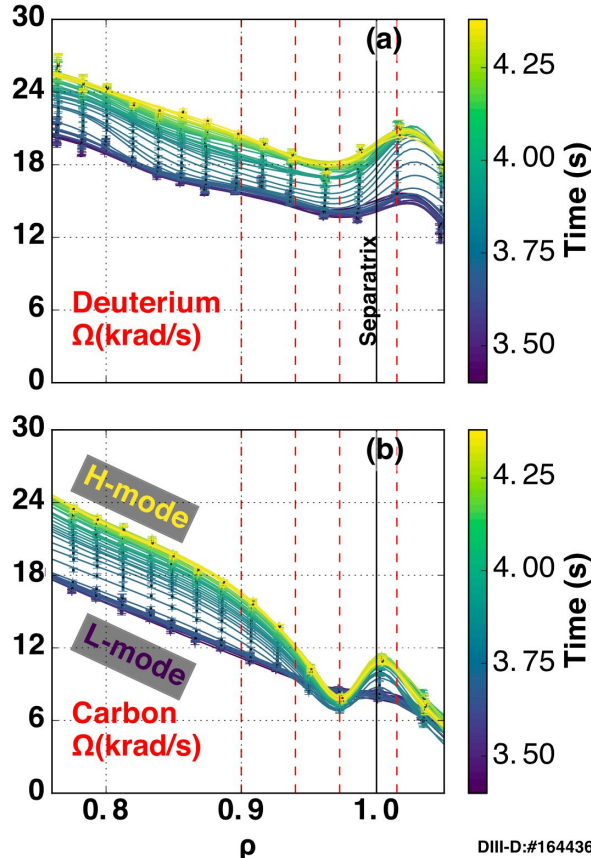
A. Ashourvan (NSTX-U MFS July 19 2018)



New Deuterium Measurements Have Improved our Understanding of Main-ion Properties of the Pedestal

- First ever capability
- Reveals dramatic differences with impurity rotation
- Used to validate intrinsic rotation model [TSD]
- Revealing lower main-ion temperature in pedestal
 - Implications for model validation

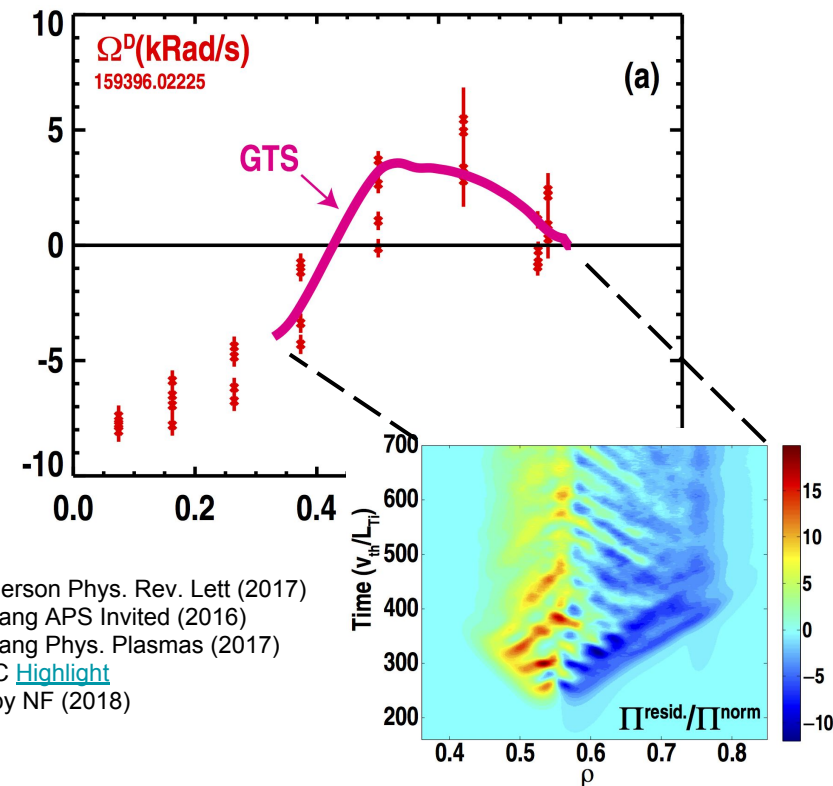
B.A. Grierson, RSI (2016)
 S.R. Haskey, RSI (2016)
 S.R. Haskey, RSI (2018)
 Ashourvan APS Invited (2017)
 Ashourvan Phys. Plasmas (2018)
 S.R. Haskey HTPD Invited (2018)
 S.R. Haskey PPCF (2018)



DIII-D:#164436

Global Nonlinear Gyrokinetic Simulations Capture the Core Main-ion Intrinsic Rotation Profile Structure

- Well known “rotation reversal” in ohmic L-mode plasmas a challenge to simulate
 - $\Pi_{\text{resid}} \equiv 0$ in local limit
- GTS global simulations show *intensity gradient* and *ZF ExB shearing* dominate k_{\parallel} symmetry breaking
 - Momentum pinch weak
 - Balance of $\Pi_{\text{resid}} \equiv 0$ and χ_{ϕ}
 - Recently confirmed on ASDEX*



B.A. Grierson Phys. Rev. Lett (2017)
W.X. Wang APS Invited (2016)
W.X. Wang Phys. Plasmas (2017)
DOE SC [Highlight](#)
*Hornsby NF (2018)

PPPL Staff at DIII-D Have Led Community-Based Analysis Capability Supporting Domestic and International Cooperation

- Through GA's OMFIT PPPL and collaborating institutions have expanding the usage of TRANSP*
- Now >65 registered TRANSP users at DIII-D
- Streamlined and standardizing profile analysis with advanced fitting methods
- OMFIT + TRANSP integration continues to grow incorporating the world's tokamaks
- ASDEX-Upgrade, C-Mod, COMPAS, DIII-D, JET, MAST, NSTX

N.C. Logan Fus. Sci. & Tech (2018)
 B.A. Grierson Fus. Sci. & Tech (2018)
 F. Poli NSTX-U MFS July 30 2018



FUSION SCIENCE AND TECHNOLOGY
 © American Nuclear Society
 DOI: <https://doi.org/10.1080/15361056.2017.1386985>

Check for updates

Orchestrating TRANSP Simulations for Interpretative and Predictive Tokamak Modeling with OMFIT

B. A. Grierson,
 J. Buchanan
 *Princeton, NJ
 †General Atoms
 ‡CCFE, Culham

Received August
 Accepted for Publication

FUSION SCIENCE AND TECHNOLOGY
 © American Nuclear Society
 DOI: <https://doi.org/10.1080/15361056.2017.1386943>

Check for updates

OMFIT Tokamak Profile Data Fitting and Physics Analysis

N. C. Logan,^{a,*} B. A. Grierson,^a S. R. Haskey,^a S. P. Smith,^b O. Meneghini,^b and D. Eldon^b
 *Princeton Plasma Physics Laboratory, Princeton, New Jersey 08540
 †General Atoms, San Diego, California 92121

Received July 26, 2017
 Accepted for Publication

OMFIT TOKAMAK PROFILE DATA FITTING AND ANALYSIS - LOGAN ET AL. 7

Abstract — for interfacing the many diverse spatial and temporal scales of tokamak profile data fitting and analysis, OMFIT provides a user-friendly interface for the many diverse spatial and temporal scales of tokamak profile data fitting and analysis. OMFIT provides a user-friendly interface for the many diverse spatial and temporal scales of tokamak profile data fitting and analysis. OMFIT provides a user-friendly interface for the many diverse spatial and temporal scales of tokamak profile data fitting and analysis.

Keywords — Some of the world's tokamak experiments (JET, JET-1, ASDEX-Upgrade, C-Mod, COMPAS, DIII-D, JET, MAST, NSTX) quantify fusion energy, particle confinement, and the resulting sensitivities to the core-most data may cause significant variations in the peakness of the fit even after τ_a when the edge dynamics have settled down. If the user believed the plasma to be in a stable equilibrium after this point, this phenomenon could be avoided by expanding the convolution window, smoothing a 2-D fit further, or even using a fit with a fixed m -axis value.

More rigorous metrics of the fit are also available through the OMFITprofiles interface. The χ^2 and reduced- χ^2 are stored as a function of time and are

immediately available for comparison with previous fit methods/settings. Once a particular fit method is chosen based on the physics and overall performance, these goodness-of-fit metrics focus refinement efforts by immediately identifying problematic time slices. A typical fitting workflow is shown in Fig. 8. After fitting 1-D splines, the reduced- χ^2 is used to identify poor fits and the interactive plots are used to investigate the cause. Here, the reduced- χ^2 plot identified 1520 and 1720 ms as poor fits. Comparing the profiles to a good fit at 1520 ms, we see the spline fit at 1520 ms was allowed to condense its knots to create an unphysical scale length at the top of the pedestal while the fit at 1720 ms contains a faulty channel (this is the only time for which this channel took data). Both issues are easy to fix by setting the minimum knot separation and deselecting the faulty channel in the OMFITprofiles GUI.

V. PHYSICS CONSEQUENCES AND CONSTRAINTS

For physics applications, the true quality of a fit takes into account not only mathematical measures such as the

I. INTRODUCTION

The world tokamak experiments (JET, JET-1, ASDEX-Upgrade, C-Mod, COMPAS, DIII-D, JET, MAST, NSTX) quantify fusion energy, particle confinement, and the resulting sensitivities to the core-most data may cause significant variations in the peakness of the fit even after τ_a when the edge dynamics have settled down. If the user believed the plasma to be in a stable equilibrium after this point, this phenomenon could be avoided by expanding the convolution window, smoothing a 2-D fit further, or even using a fit with a fixed m -axis value.

More rigorous metrics of the fit are also available through the OMFITprofiles interface. The χ^2 and reduced- χ^2 are stored as a function of time and are

I. MOTIVATION

The expanding capabilities of the fusion diagnostic inputs to intricate physics modeling (Energy (DOE) core simulations of fusion prioritize streamlining tokamak data and simulation and validation of whole-grated modeling tasks require fundamental physics and time.

WMD efforts such as move beyond individuals

Figure 7 shows two panels of 2-D visualizations. Panel (a) shows a contour plot of the fit for a specific time slice, with a color scale on the right ranging from 0.0 to 4.0. Panel (b) shows a similar plot but with a horizontal spline fit overlaid, demonstrating how the fit can be smoothed over time.

Figure 8 consists of two subplots. Subplot (a) is a line graph showing the time-dependent reduced χ^2 for various fit parameters, with peaks at 1520 ms and 1720 ms. Subplot (b) shows interactively overlaid profiles for different time slices, with a color-coded legend indicating the fit quality and the specific parameters used for each fit.

PPPL Staff in DIII-D Program Communicate High Impact Scientific Results Through Publications and Press Releases

DOE Web Highlight

Fusion Energy Sciences (FES)

04.27.17

Plasma Turbulence Generates Flow in Fusion Reactors

Heating the core of fusion reactors causes them to develop sheared rotation that can improve plasma performance.

The Science

Improved stability and confinement in fusion reactors comes through generation of sheared flow. High-energy particle beams commonly drive the flow. Researchers ran new simulations that are showing how the plasma can self-organize to generate its own sheared flow even when

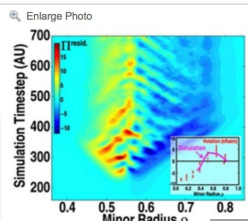


Image courtesy of W.X. Wang

Simulation of plasma turbulence generate positive (red) and negative (blue) residua that drives rotation shear. Comparison b measured and simulated rotation profile

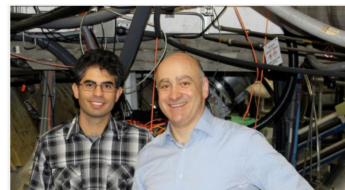
NEWS

PPPL and General Atomics scientists make breakthrough in understanding how to control intense heat bursts in fusion experiments

By Raphael Rosen

March 13, 2015

Twitter Google+ Share Print



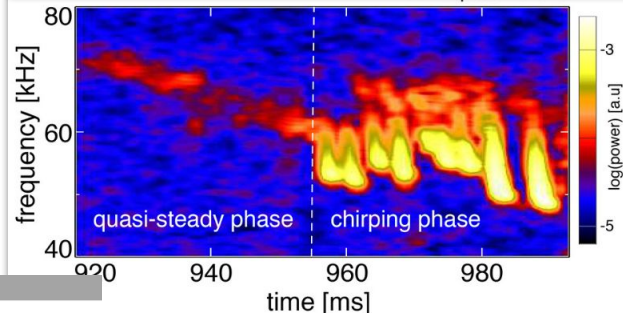
Carlos Paz-Soldan, left, and Raffi Nazikian at the DIII-D tokamak. (Photo by Lisa Petrillo/General Atomics)

AIP SciLight

DECEMBER 2017 <https://doi.org/10.1063/1.5018736>

A close look at fast ion microturbulence in tokamaks builds predictive capability for fusion

J. H. Majors



AIP Physics of Plasmas

HOME BROWSE INFO FOR AUTHORS COLLECTIONS

Featured

Multi-scale transport in the DIII-D ITER baseline scenario with direct electron heating and projection to ITER

B. A. Grierson, C. M. Staebler, W. M. S.

Featured Article



Physicists Brian Grierson of PPPL, left, and Gary Staebler of General Atomics

(Photo by Shaun Haskey)

Press Releases

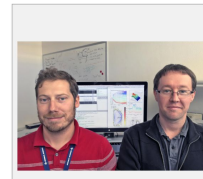
PPPL and General Atomics team up to make TRANSP code widely available

DOE/PRINCETON PLASMA PHYSICS LABORATORY

Facebook Twitter YouTube Instagram SHARE

PRINT E-MAIL

Plasma transport analysis, the study of how plasma particles, heat and momentum drift across magnetic field lines, is a necessary first step for understanding how well fusion reactors are performing. Teams of scientists from the U.S. Department of Energy's (DOE) Princeton Plasma Physics Laboratory (PPPL) and General Atomics (GA) have joined forces to bring PPPL's premier transport code, TRANSP, to beginning users and experts alike.

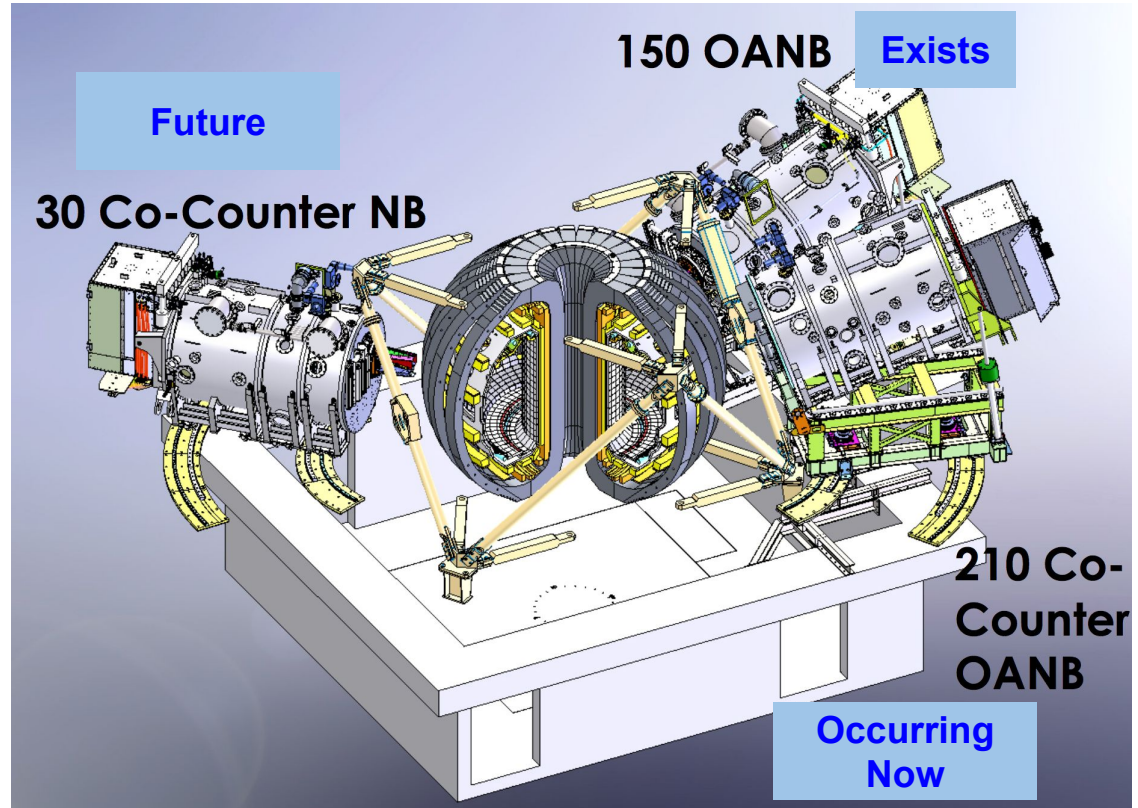


Recent Engineering and Operations Activities



During the Current Long Torus Opening (LTO) DIII-D is Making NB 210 Co/Ctr Steerable and Off-Axis

- Primary goal for CCOANB is achieving $\beta_N \sim 5$ with broad pressure and current profile
- Target completion March 22nd
- Alex Nagy: Deputy Proj. Manager



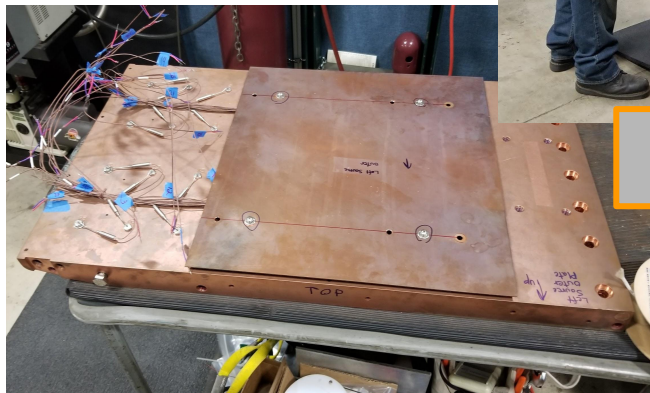
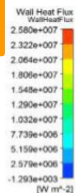
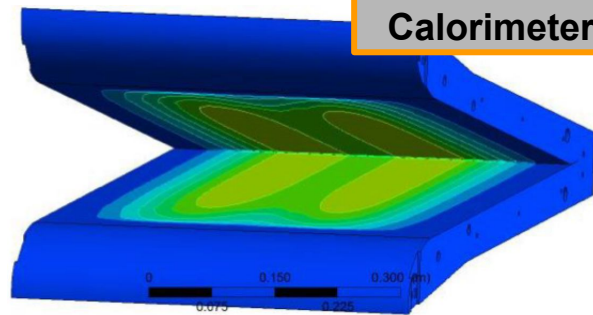
PPPL Supplying Calorimeters and Pole Shields for Existing 150 OANB and New 210 CCOANB



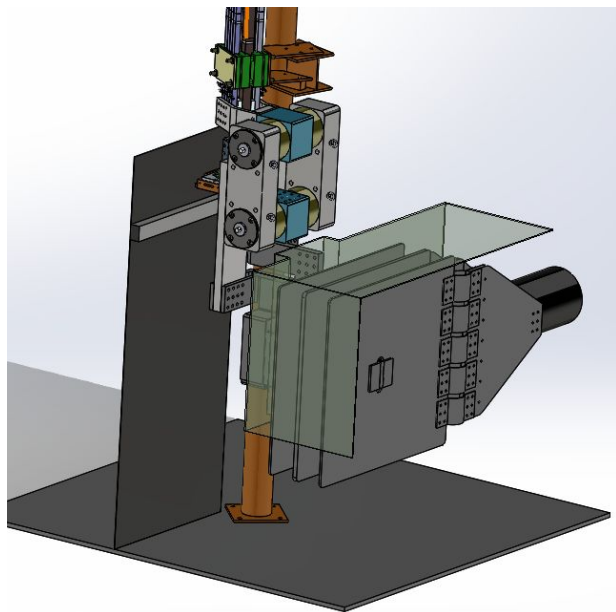
**NB150
Calorimeters**



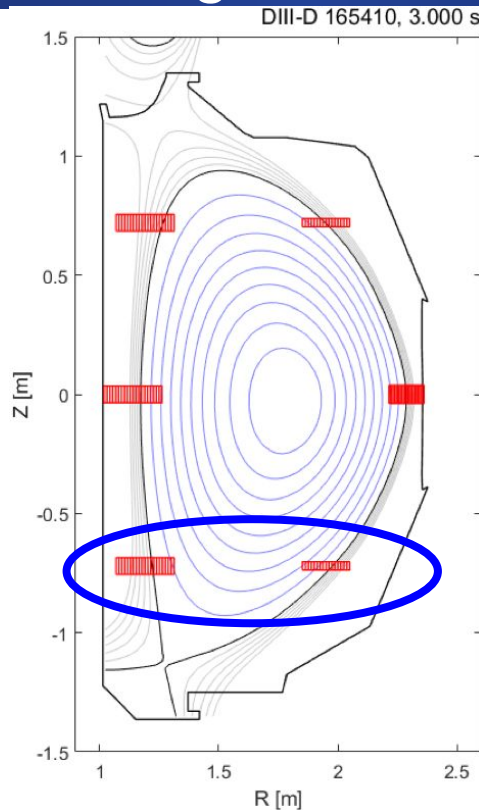
**NB210
Pole Shields**



New Toroidal Field Reversing Switch and Ly-alpha Diagnostic Enhance Divertor and Core/Edge Studies



TF Switch enables day-to-day (shot-to-shot?) reversing of toroidal field for divertor drifts and 3D field studies



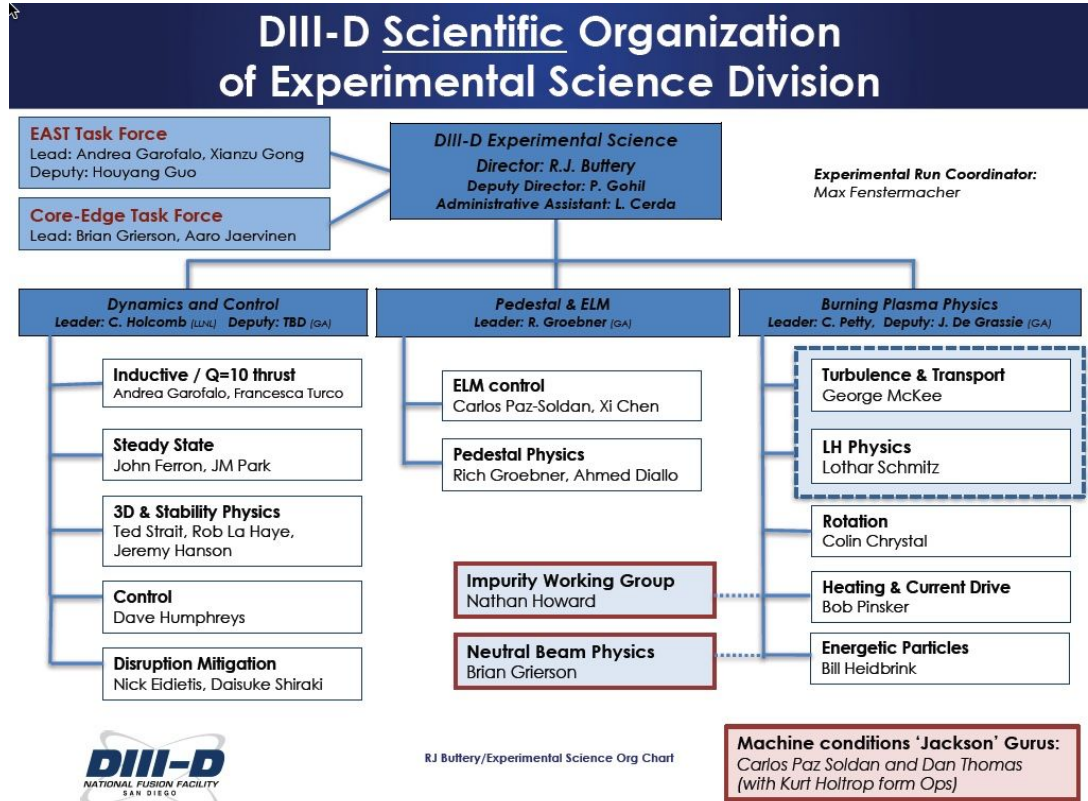
First pair of HFS/LFS Ly-alpha arrays installed during LTO for neutral density and radiation
A. Bortolon, A. Rosenthal (MIT)

Near Term Activities and Long Term Goals



Get (Stay) Involved Through DIII-D's Research Opportunities Forum and Topical Research Groups

- **Core-Edge Integration**
 - Brian, Alessandro
- **3D Fields and Stability**
 - Nik, Raffi
- **Dynamics and Control**
 - Brian, Francesca, Arash
- **Pedestal and ELM**
 - Raffi, Ahmed, Brian
- **Burning Plasma**
 - Brian, Mario, Shaun, Arash
- **Research Forum late Dec '18 or Jan '19**



Come See Us at Major Meetings and Discuss PPPL Plans for DIII-D Over the Next Five Years

- **IAEA FEC**
 - **Mario Podesta(oral)**, Brian Grierson, Ahmed Diallo, Dan Boyer, Florian Laggner, Michio Okabayashi, Robert Hagar
- **APS-DPP (Invited)**
 - Nik Logan, Zhirui Wang, Egemen Kolemen, Brian Grierson
- **PPPL companion to the DIII-D Five Year Proposal**
 - <https://diii-d.gat.com/diii-d/Home>
- **Reach out, visit and stay involved**
It's an exciting program!

