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### Electromagnetic Particle Injector (EPI) for Fast Time Response Disruption Mitigation in Tokamaks

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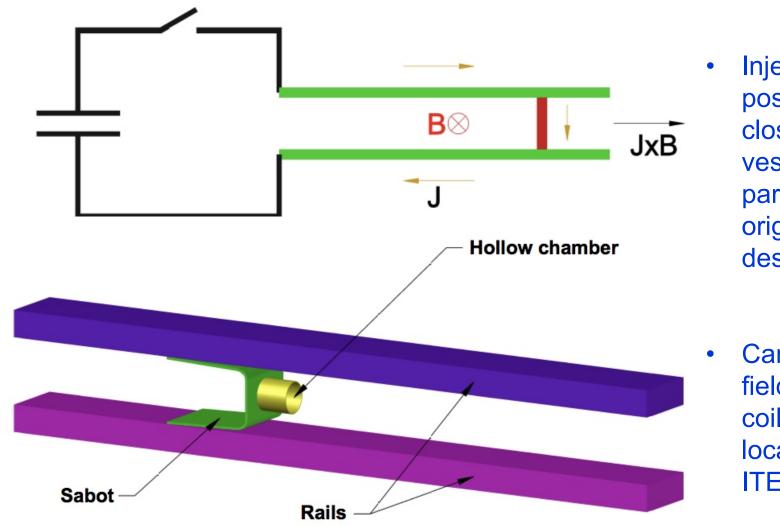
#### R. Raman<sup>1</sup>, R. Lunsford<sup>2</sup>, et al., <sup>1</sup>University of Washington <sup>2</sup>Princeton Plasma Physics Laboratory

### **APS-2019 Invited Talk Outline**

29 April 2019 Magnetic Fusion Science Meeting B318, PPPL

Culham Sci Ctr U St. Andrews York U Chubu U Fukui U Hiroshima U Hyogo U Kyoto U Kyushu U Kvushu Tokai U NIFS Niigata U **U** Tokyo JAEA Hebrew U loffe Inst **RRC Kurchatov Inst** TRINITI **KBSI** KAIST POSTECH ASIPP ENEA, Frascati CEA, Cadarache **IPP**, Jülich **IPP**, Garching ASCR. Czech Rep U Quebec

# EPI accelerates a metallic sabot to high-velocity in a simple rail gun configuration



- Injector can be positioned very close to the vessel, if EPI is part of the original reactor design
- Can use highfield SC boost coils (>8T) if located outside ITER port plug

## Controlling Major Disruptions on ITER and Tokamak-Based Reactors is an <u>Essential Requirement</u>

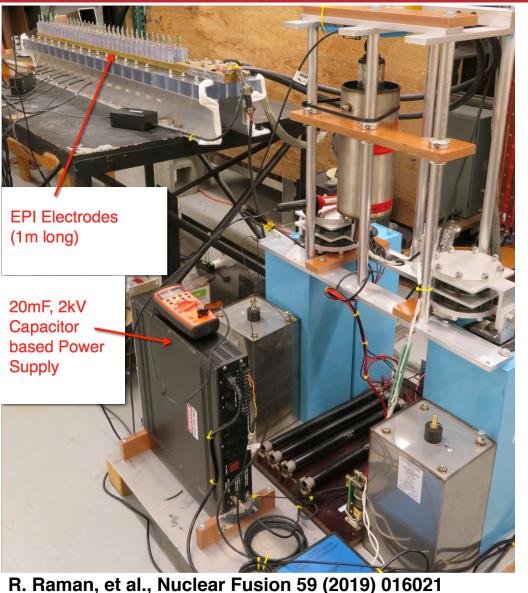
- The present baseline system for ITER is based on SPI, but this may change, and other improved technologies may be implemented
  - > The Electromagnetic Particle Injector (EPI) is a possible candidate
- Disruption mitigation on ITER would be much more challenging
  - High edge Te will make radiative penetration to q=2 quite difficult
  - On DIII-D some SPI fragments penetrate to q=2 surface, but simple, single fragment injection, calculations indicate that these would be ablated at the edge in ITER plasmas.
  - ➢ Will SPI on ITER be more like MGI on DIII-D?
  - Would MHD mixing be needed to transport the radiative payload to the core (MGI) or would inside-out quench using new technologies be superior and provide greater safety margin on ITER?
  - How well can we control the amount of required radiative material as the plasma energy content changes (to control the current quench time duration)
  - How well would runaway currents be controlled on ITER
- Reliable and safe operation of ITER is absolutely necessary for the future of any tokamak-based reactors, and for magnetic fusion energy in general

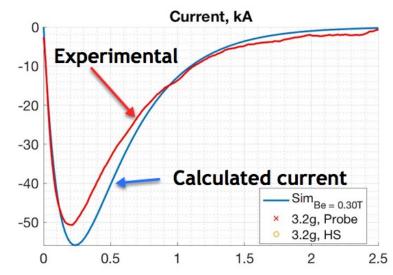
## Major uncertainties exist in the present baseline DMS for ITER

- EPI has the potential for protecting the ITER infrastructure (fast response time & high velocity)
- Off-line setup (with EPI-2) has demonstrated key aspects of concept, including 300 m/s velocities with 1.5ms response time consistent with calculations. (@ ~50% energy of EPI-1 for 150m/s)
  - Payload separation from sabot and sabot capture demonstrated at 160m/s and the concept can be extended to >2km/s
- Goals for APS Talk
  - Briefly summarize key results on radiative payload penetration from present experiments (will SPI on ITER be more like MGI on DIII-D)
  - Describe the EPI concept with results from EPI-1 and EPI-2
    - On EPI-2, extend operation to full 2T (possibly higher) boost coil fields
    - Possible test inside a vacuum chamber
  - Include a M3D-C1 simulation on typical EPI fragment injection into a DIII-D/KSTAR scale plasma

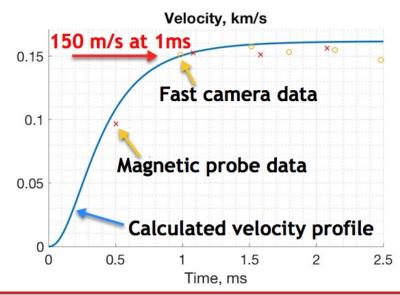
#### **Back-up slides**

## Measured EPI-1 system parameters with 0.3T B-field augmentation in agreement with calculations

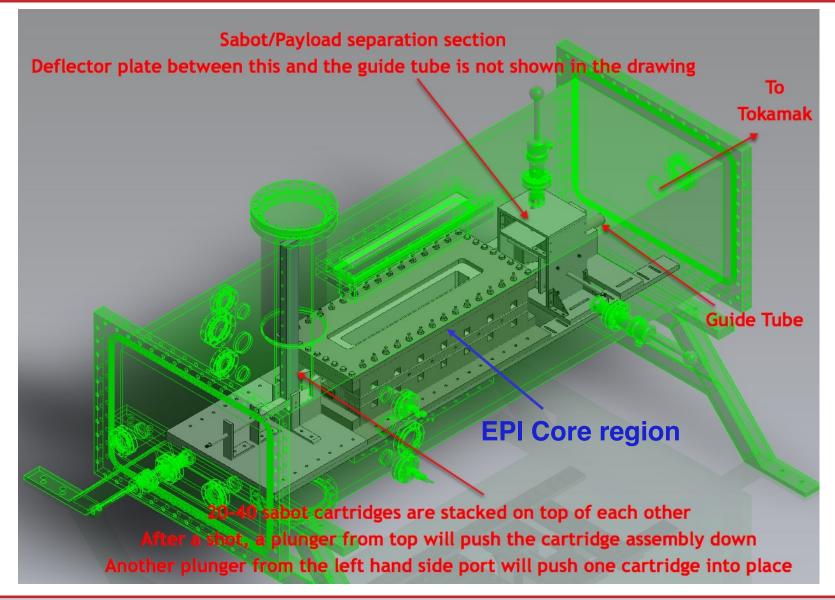




Maximum velocity limited by power supply limits

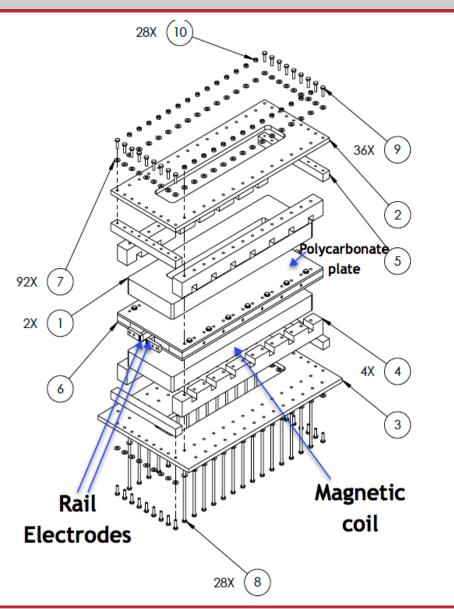


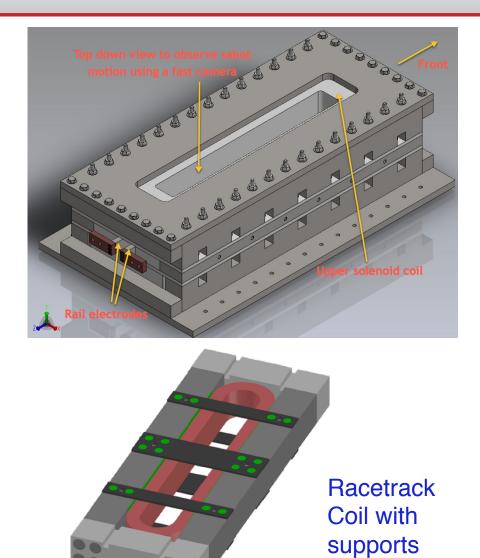
#### EPI-3 Conceptual Layout (KSTAR configuration) Vacuum Chamber Dimensions (1.5m x 0.6m x 0.5m)





## EPI-2 Core Components (KSTAR Configuration)





# All key components of EPI-2 (except vacuum operation) have been tested

