Models and Workflows for Heating and Current Drive and Synthetic Diagnostics in the IMAS Integrated Modelling Framework

PPPL seminar, 7 February 2022

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M. Schneider – PPPL Seminar – 7 February 2022

Outline

- The ITER mission goals and Research Plan
- The IMAS Integrated Analysis and Modelling Suite
- Synthetic Diagnostics in IMAS
- Workflows and Bayesian technique platforms
- H&CD models and workflow in IMAS
- Summary



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ITER mission goals

ITER shall demonstrate the scientific and technological feasibility of fusion energy:

Pulsed operation:

 $Q \ge 10$ for burn of 300-500 s, with inductively driven current. \rightarrow Baseline scenario 15 MA / 5.3 T.

Long pulse operation:

 $Q \sim 5$ for long pulses up to 1000 s, supported by non-inductive current drive. → Hybrid scenario ~ 12.5 MA / 5.3 T.



Steady-state operation:

 $Q \sim 5$ for long pulses up to 3000 s, with fully non-inductive current drive \rightarrow Steady-state scenario \sim 10 MA / 5.3 T.

H&CD systems and diagnostics in the ITER Research Plan

- The ITER Research Plan defines the strategy to achieve its mission goals throughout the scientific and technical exploitation of the tokamak and its ancillary systems.
- It will unfold in four stages:





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The ITER Integrated Modelling & Analysis Suite (IMAS)

- IMAS is the collection of physics software that will be used to support ITER operations and research as defined in the ITER Integrated Modelling Programme
- It uses standard Interface Data Structures (IDS):
 - For access to experimental and simulated data defined in collaboration with ITER Members
 - For exchanging data between components in an integrated modelling environment
- It is suitable for any fusion device
- It will contain components enabling high physics-fidelity predictive simulations of ITER plasmas
- It will be used for ITER data processing and analysis



The IMAS Data Dictionary

• Core	charge_exchange	dataset_description
• Edge	edge_profiles bremsstrahlung_visible	summary
 Electro-Magnetics 	edge_sources	transport_solver_numerics
Physics phenomena	gas_injection magnetics	numerics
Fuelling	pellets mhd ntms spectrometer mass	temporarv
H&U	disruption radiation spectrometer visible	dataset fair
 Other plant systems Diagnostics 	turbulence mhd linear	controllers
 Diagnostics Data management 	ec_launchers	
Data management	gyrokinetics sawteen waves ic_antennas	puise_scriedule
spectrometer uv	core transport	amns_data
speciforneter_uv	distributions nbi	san
bolometer pl_acti	core profiles wall soft x rays	
langmuir_probes	core instant changes cryostat	
hard_x_rays		The dictionary
polarimeter ^{pf}	_passive equilibrium ^{mse} interferometer	evolves with the
barometry	em_coupling iron_core reflectometer_profile	development of
camera vi	sible coils non axisymmetric camera ir	
	thomson scattering	the IIVI platform.
spectror	neter_x_ray_crystal	
	ece calorimetry neutron_diagnostic	

Towards a high-fidelity plasma simulator

- IMAS will contain a plasma simulator that integrates free-boundary evolution, core-edge-SOL transport, divertor physics and PFC models to allow high fidelity physics simulations.
- Diagnostic and actuator models will be used together with control algorithms to simulate the behaviour of the Plasma Control System.
 Synthetic Diagnostics workflow



Criteria for models to be in IMAS

An IMAS model exchanges IDSs exclusively + an optional xml code parameter file:



ids4,ids5 = model(ids1,ids2,ids3,xml_codeparam)

- Associated development:
 - Extension of the IMAS Data Dictionary (some IDSs are too basic or not existing)
 - Population of the Machine Description DB with the geometry of ITER diagnostics

PPPL possible contributions

- TRANSP physics components should exchange IDSs without any layer of data conversion or translation
- To help with planning, it would be good to communicate the schedule for creating new IMAS actors, e.g.:
 - TORAY-GA, GENRAY, CQL3D, MMM, GLF23, TGLF, NEO already extracted
 - TEQ, ISOLVER and reduced SOL model will be extracted

... All these components to be made IMAS-compliant

Models in IMAS



Ready

IMAS scenario database

~1800 simulations for core and/or edge scenarios, among which 680 are active

---> Default call equivalent to:

20

15

5

0

11 KeV

scenario summary -c shot, run, database, ref name, ip, b0, fuelling, confinement, workflow

Pulse	Run	Database	Reference	Ip[MA]	B0[T]	Fuelling	Confinement	Workflow
100001	2	ITER	ITER-full-field-H	-15.0	-5.3	Н	L-mode	METIS
100002	1	ITER	ITER-half-field-H	-7.5	-2.65	Н	L-mode	METIS
100003	1	ITER	ITER-third-field-H	-5.0	-1.8	Н	L-H-L	METIS
100007	1	ITER	ITER-intermediate-3T-H	-8.5	-3.0	Н	L-H-L	METIS
100008	1	ITER	ITER-intermediate-3.3T-H	-9.5	-3.3	Н	L-H-L	METIS
100009	1	ITER	ITER-intermediate-4.5T-H	-12.5	-4.5	Н	L-mode	METIS
100013	1	ITER	ITER-PFP01-1.8T-H	-5.0	-1.8	Н	L-H-L	METIS
100014	2	ITER	ITER-PFPO2-1.8T-H-0.5*n_GW-NBI_530keV_9.4MW	-5.0	-1.8	Н	L-H-L	METIS
100015	1	ITER	ITER-PFP02-1.8T-H-0.9*n_GW-NBI_745keV_22.3MW	-5.0	-1.8	Н	L-H-L	METIS
100501	3	ITER	ITER-nonactive-H	-7.5	-2.65	Н	L-H-L	CORSICA
100502	3	ITER	ITER-nonactive-H	-7.5	-2.65	Н	L-H dithering	CORSICA
100503	3	ITER	ITER-nonactive-H	-7.5	-2.65	Н	L	CORSICA
100504	3	ITER	ITER-nonactive-H	-9.6	-3.25	Н	L	CORSICA
100505	3	ITER	ITER-nonactive-H	-12.7	-4.7	Н	L	CORSICA
100506	3	ITER	ITER-nonactive-H	-15.0	-5.3	Н	L	CORSICA
100507	3	ITER	ITER-nonactive-H	-5.0	-1.77	Н	L-H-L	CORSICA
101000	50	ITER	<pre>PFPO-2 tf=tE,2NBI,highTped,postST</pre>	-7.5	-2.65	Н	H-mode	ASTRA
101001	50	ITER	<pre>PFPO-2 tf=tE,2NBI,highTped,preST</pre>	-7.5	-2.65	Н	H-mode	ASTRA
101002	50	ITER	<pre>PFPO-2 tf=tE,2NBI,lowTped,postST</pre>	-7.5	-2.65	Н	H-mode	ASTRA
101003	50	ITER	<pre>PFPO-2 tf=tE,2NBI,lowTped,preST</pre>	-7.5	-2.65	Н	H-mode	ASTRA
101004	60	ITER	PFPO-2 tf=2tE,2NBI	-7.5	-2.65	Н	H-mode	ASTRA
101005	60	ITER	PFPO-2 tf=tE,2NBI	-7.5	-2.65	н	H-mode	ASTRA
101006	60	ITER	PFPO-2 tf=0.5tE,2NBI	-7.5	-2.65	Н	H-mode	ASTRA
101007	40	ITER	PFPO-2 H-5MA-20EC-10NBI Pr=0.3(tF/tE=2)	-5.0	-1.8	Н	H-mode	ASTRA
101007	41	ITER	PFPO-2 H-5MA-20EC-10NBI Pr=0.3(tF/tE=1)	-5.0	-1.8	Н	H-mode	ASTRA
101007	42	ITER	PFPO-2 H-5MA-20EC-10NBI Pr=0.3(tF/tE=0.65)	-5.0	-1.8	Н	H-mode	ASTRA

DINA-JINTRAC free boundary core-edge ITER DT scenario 15 MA / 5.3 T



 $1e_{20}$ t = 426.9 s t = 426.9 sTools are available to list and $n_e(0)$ visualise IMAS scenarios $n_i(0)$ [^{1.0} [⁻u]_{0.5} $n_T(0)$ $n_D(0)$ n_{He4}(0) Predictive TRANSP modelling $n_{Be}(0)$ 0.0 t_{slice} of ITER discharges 0.00 0.25 0.50 0.75 1.00 0.00 0.25 0.50 0.75 1.00 p/po p/po

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IMAS Machine Description database

Machine Description available for H&CD systems, many diagnostics, wall, magnetics and coils:

Defeult			
md_summ	ary -c pbs,ids,description		
PBS	IDS	DESCRIPTION	SHOT/RUN
PBS PBS-111 PBS-111 PBS-115 PBS-15 PBS-15 PBS-15 PBS-55.15 PBS-55.15 PBS-55.15 PBS-55.15 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17 PBS-55.17	IUS pf active tf coils_non_axisymmetric coils_non_axisymmetric coils_non_axisymmetric coils_non_axisymmetric pf_passive bolometer spectrometer_x_ray_crystal spectrometer_visible spectrometer_visible spectrometer_visible ecc ecc ecc ecc ecc ecc ecc e	<pre>DESKUPTION PF/CS Coil System, TF busbars (equivalent) and Virtual Coils TF Coil System, TF busbars (equivalent) and Virtual Coils TF Coil System Ex-Vessel Coils (EVC) Systems (ELM) In-Vessel Coils (IVC) Systems (ELM) Priodical Coils (IVC) Systems (ELM) Newsel (VV), Triangular Support (TS) and Divertor Inboard Rails (DIR) from IDM Vacuum Vessel (VV), Triangular Support (TS) and Divertor Inboard Rails (DIR) from DINA PP pinholes and collim., Div. collim., VV collim. (S50 channels) Core X-Ray Spectrometer (XRCS) Charge Exchange Recombination Spectroscopy (CXRS) Edge Charge Exchange Recombination Spectroscopy (CXRS) Core Charge Exchange Recombination Spectroscopy (CXRS) Pedestal Electron Cyclotron Emission (ECE) - Radial 0-mode Electron Cyclotron Emission (ECE) - Radial X-mode Electron Cyclotron Emission (ECE) - Oblique 0-mode Electron Cyclotron Emission (ECE) - Oblique X-mode Ion Cyclotron (IC) antennas Toroidal Interfero-Polarimeter (ITP) Density Interfero-Polarimeter (ITP) Density Interfero-Polarimeter (ITP) Density Interfero-Polarimeter (ITP) Density Interfero Bission (HBB) - H beams 870 keV - off-off Heating Neutral Beams (HMB) - H beams 870 keV - off-off Heating Neutral Beams (HMB) - H beams 870 keV - off-off Heating Neutral Beams (HMB) - H beams 10 keV - off-off Heating Neutral Beams (HMB) - D beams 1 MeV - off-off Heating Neutral Beams (HMB) - D beams 1 MeV - off-off Heating Neutral Beams (HMB) - D beams 1 MeV - off-off Heating Neutral Beams (HMB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O beams 1 MeV - off-off Heating Neutral Beam (MB) - O f</pre>	SH017K00 111001/3 111002/1 111003/1 115002/1 115003/1 115003/1 115005/2 150601/2 150505/2 150505/2 150501/2 150501/2 150601/3 150601/3 150601/3 150601/3 150601/3 150601/3 130000/120 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 130000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000/250 150000000000000000000000000000000000
PBS-16 PBS-16.FC PBS-16	wall wall pf_passive	First wall and divertor geometry for PFPO and FPO phases First Plasma Protection Components (FPPC) Blanket Module Panel (BMP)	116000/2 116612/1 116001/1

The MD database provides the geometry of the plant systems to be used as input of simulation codes.



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 Development and/or IMASadpation of SD models available/developed in PPPL

ITER (Synthetic) Diagnostics







Bolomet Determination of the spatial distribution of the radiated power in the plasma and divertor using tomographic reconstruction TOFU_bolo

Bolometric Systems



Spectroscopic Instruments and NPA Systems

Determination of plasma composition, density, particle fluxes, ion temperature, fuelling ratio, plasma rotation, current density; **CASPER, CXRS, XICSRT**





Neutron and Fusion Products Diagnostics



Plasma-Facing and Operational Diagnostics

Assist the machine protection and operation, especially the main chamber and divertor state (temperature, pressure, erosion, dust and tritium monitoring)



Central information on SD development for ITER

The webpage showing the status of the SD development for ITER is here: <u>https://confluence.iter.org/display/IMP/Synthetic+Diagnostics</u>

Confluence Spaces - People Que	istions Calendars Create ···		Q. Search 🕜 🛠	* 🐵			
Integrated Modelling	Pages / Integrated Modelling Home Page / Physics Components & Wor	Status and Plans for	Synthetic Diagnostic development				
	Synthetic Diagnostics	The following table describes the current status or the plans for developing SD models for each phase of the ITER Research Plan. This table is aimed to be ALIVE, i.e. it will regularly be updated as the status of a specific SD model evolves.					
Pages Poge Questions Calendars	Related sub-pages: • Synthetic Diagnostics Meetings • Synthetic Diagnostics Table	Color code: • Black: we have a plan! • Green: the model is already av • Red: there is no plan or the su	railable in IMAS and tested ggestion for the development has not been confirmed				
SPACE SHORTCUTS File lists Meeting notes	Table of Contents: • Categorization • Input data for SD models • Criteria for SD models to be in IMAS • IMAS Python Workflow for Synthetic Diagnostics	Diagnostics have either a Primary, Ba diagnostics matrix flow down can be Note: even when there is already a p undertaken.	Ickup or Supplementary role for a specific measurement. Only primary measurements are indi found here: Diagnostics Matrix flow down. Ian for SD development, every contribution is welcome as we need flexibility for choosing SD	cated in parentheses in the second column, as a raw indication. The complete models according to the specific Design, Control or Physics study to be			
How-to articles	 Status and Plans for Synthetic Diagnostic development Running a SD model in IMAS 	Category	Diagnostic list and IRP staging	SD Status or Plan			
I roubleshooting articles		Magnetic Diagnostics	First Plasma:	First Plasma:			
PAGE TREE Getting Started	Categorization	(magnetic coils, magnetic loops and halo sensors)	 55.A0: Magnetics System Electronics & Software 55.A1: Continuous External Rogowski coils (l_{tor}) 	 55.A0: not needed 55.A1: being covered by work on a prototype data analysis platform (J. 			
Getting Help	This page is aimed at describing the strategy and status of the developme when the actual diagnostic will start operating:	General purpose:	 55.A3/A4/A9: Outer Vessel coils (B_T) 	Svensson, L. Appel et al.) 55.A3/A4/A9: idem			
IMAS Infrastructure ITER Computing Cluster	 First Plasma (hydrogen): basic set of diagnostics for magnetics, pl PFPO-1 (hydrogen, helium): add subset for measurements of plasmost control of diagnostics of diagnostics. 	Determination of plasma equilibrium, current and stored energy, control of plasma shape	 S5.A5/A6: Steady-State sensors (suppl.) S5.A7/AD/AE/AI: Flux loops (dz/dt, plasma gaps, D_{sep}, divertor channel location, V_{loop}, B₀/×B₀>: RVM and error fields) 	 55.45/A6: idem 55.47/AD/AE/AI: idem 			
Physics Components & Workflows	FPO (D, DT): complete set including DT fusion products (for burning The categories of disappoties are listed below:	and position Measured parameters:	 55.AA/AB/AC: Inner Vessel Coils (I_p, dz/dt, plasma gaps, D_{sep}) 55.AF/AG/AH: Diamagnetic Sensors (β_p) 	 55.AA/AB/AC: idem 55.AF/AG/AH: idem 			
ASTRA Energetic Particle Stability Workflow (EP-WI JINTRAC	Magnetic Diagnostics are inset Delow. Magnetic loops and halo ss Determination of plasma equilibrium, current and stored ene Control of plasma shape and position Neutron and Fusion Products Diagnostics (neutron flux monitor.	Plasma current, plasma position, plasma shape, loop voltage, plasma energy, locked modes, low (m,n) MHD modes, sawteeth,	PFPO-1: • 55.A8: Fibre Optic Current Sensors (suppl.) • 55.A3: High Frequency Sensors (B ₈ / <b<sub>p>: complex, at wall, fishbone, TAE)</b<sub>	PFPO-1: • 55.A8: being covered by A. Goussarov in WPPrIO • 55.AJ: being covered by work on a prototype data analysis platform (J. Svensson, L. Appel et al.)			
SMITER SOLPS-ITER Synthetic Diagnostics	Measurement of fusion power, fusion products and fast ion Optical Systems (TR Systems (Thomson Scattering, polarimetry): Measurement of core and edge temperature and density pre Bolometric Systems (port-plugs, divertor, vacuum vessel); Determination of the spatial distribution of the radiated polarity provided to the radiated polarity of the radiated polarity of the spatial distribution of the spatial distribution of the spatial distribution of the spatial polarity polarity of the radiated polarity polarity of the radiated polarity polarity of the spatial distribution of the spatial distreb	disruption precursors, halo currents, toroidal B-field, static PF and TF error fields, high frequency macro-instabilities (fishbones, TAEs)	 55.AL/AC: Divertor colls (divertor channel location) 55.AL/ Divertor shutts (suppl.) 55.AN/AP: Divertor & Blanket Rogowski colls (current distribution in div. cassette, poloidal current in one sector) 	 55.AL/AC: idem 55.AL/ will be covered by intern working with T. Ravensbergen (September 2021) 55.AN/AP: being covered by work on a prototype data analysis platform (I. Svensson, L Appel et al.) 			
Synthetic Diagnostics Meetings Outbutic Diagnostics Table	 Spectroscopic Instruments and NPA systems (H-alpha, Visible, X Determination of plasma composition, density, particle fluxe Microwave Diagnostics (ECE: reflectometry, interferometry): 	Neutron and Fusion Products Diagnostics	PFPO-1:	PFPO-1:			
Software Development Parallel Workflows	Determination of the plasma position, through measuremen Plasma-Facing and Operational Diagnostics (Visible, IR cameras, Assist the machine protection and operation, especially the r	(neutron flux monitors, neutron cameras, neutron and gamma-ray spectrometers, fast ion loss	 5.5.83: Microtission Chambers (suppl.) 55.84: Neutron Flux Monitor Systems (fusion power, total neutron flux) 55.88: Neutron Activation System (first wall neutron fluence) 55.8C: Divertor Neutron Flux Monitors (fusion power, total neutron flux) 	 55.83: being developed by A. Kovalev 55.84: developed by A. Kovalev 55.88: may be covered by ITPA-EP? 55.8C: developed by A. Kovalev 			
➤ Code Camps & Meetings	A SD model can be used for three different types of application:	detectors) <u>General purpose:</u> Measurement of fusion power, fusion products and fast ico	SS.BV: Neutron Calibration 2.5 MeV SS.B1: Neutron Facility Area FPO: SS.B1: Padial Neutron Camara RNC (fusion power density, neutron/or source profile)	SS.BV: may be covered by ITPA-EP? SS.BT: not needed FPO: SS.B1: ran possibly be covered by ITPA-EP + Eurofusion MRDHO?			
		losses Measured parameters:	 South reader reaction camera Avic (taskin power density, neuron/α southe profile) SS.82: Vertical Neutron Camera Avic (fusion power density, neuron/α source profile) SS.87: Radial Gamma Ray Spectrometer RGRS (E_{max,runaway}, I_{nunaway}, α density profile) 	 55.82: idem 55.87: idem 			

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Examples of IMAS-adapted Synthetic Diagnostics for ITER

Model	Modelled diagnostic or signal	Input IDSs from scenario	Input IDSs from Machine Description or upstream code	Output IDSs
CASPER	Generic light spectrum for visible spectrometry	equilibrium core_profiles edge_profiles	spectrometer_visible nbi	<pre>spectrometer_visible</pre>
CXRS	Fit to create synthetic CXRS signal from spectrum	The start	<pre>spectrometer_visible</pre>	charge_exchange
DIP_TIP_POP	Toroidal Interfero-Polarimeter Density Interfero-Polarimeter	equilibrium core_profiles	interferometer	interferometer
	Poloidal Polarimeter		polarimeter	polarimeter
ECRad	Electron Cyclotron Emission	equilibrium core_profiles	ece	ece
REFI	LFS and HFS reflectrometers	equilibrium core_profiles	reflectometer_profile	reflectometer_profile
Refractometer	Refractometry channel of the HFS reflectometer	equilibrium core_profiles	refractometer	refractometer
TOFU_bolo	Bolometers	edge_sources wall	bolometer	bolometer
DNFM	Divertor Neutron Flux Monitor	equilibrium distribution_sources	neutron_diagnostic	neutron_diagnostic
NFM	Neutron Flux Monitor	equilibrium distribution_sources	neutron_diagnostic	neutron_diagnostic
XICSRT	X-ray Core diagnostic	equilibrium core_profiles	<pre>spectrometer_x_ray_crystal</pre>	<pre>spectrometer_x_ray_crystal</pre>

Density Interfero-Polarimeter

- 55.FA Density Interfero-Polarimeter (DIP), measures $\int n_e dl$
- Python model by A. Medvedeva



out_interferometer = dip_tip_pop(equilibrium,core_profiles,interferometer_md)



Toroidal Interfero-Polarimeter

- 55.C5 Toroid. Interfer. Polarim. (TIP), measures $\int n_e dl$, $\delta n_e/n_e$, $\delta T_e/T_e$
- Python model by A. Medvedeva





out_interferometer = dip_tip_pop(equilibrium,core_profiles,interferometer_md)

Poloidal Polarimeter

- 55.C6 Poloid. Polarim. (POP), measures q profile
- Python model by A. Medvedeva



out_polarimeter = dip_tip_pop(equilibrium,core_profiles,polarimeter_md)



PFPO-2

ICRH, NBI

ECH - 20 (+10) MM

ECH - 6.7 M

of tokamak core components CH, ICRH, NBI + fi

(+10) MW + 100

Reflectometry

PFPO-2 55.F9 (HFS) and 55.F2 (LFS) PFPO-CH, ICRH, NBI + fu Measure core/edge n_e profiles, $\delta n_e/n_e$, $\delta T_e/T_e$ ECH - 6.7 M ECH - 20 (+10) MM ICRH, NBI of tokamak core components REFI model developed by V. Nikolaeva GHz O-mode Input: shot 134173, run 106, time slice 296.9 s F: 90 - 140 **Scenario DB** ITER Baseline 5.3T 15MA - cutoff frequencies (with relativistic effect) Mach. Descr. DB E: 60 - 90 250 upper X-mode U: 40 - 60 core_profiles Fce reflectometer_profile O-mode Ka: 26.5 - 40 equilibrium lower X-mode 200 K: 18 - 26.5 [ZH] 150 Ku: 12 - 18 X-mode 2 <u>e</u> 100 REFI **REFI** workflow 50 Physics effects Machine Description DB Scenario DB Input LoS, frequencies 0 IDS: equilibrium, reflectometer_profile Relativistic effect m (+ sensitivity to thermal n_e, T_e, B profiles $R_{Los}[m]$ turbulence model displacement) Time-Frequency analysis Cutoff positions Reflectometry signal 17.5 X-mode 400 SD measurement + hardware noises + O-mode 15.0 Beat Frequency [MHz] 0 007-007-200 12.5 Filter, unwrap phase Data processing Fast Fourier Transform 7.5 Je l n_e(r) reconstruction X-mode U band Data analysis $\delta n_e/n_e$ spectrum 5.0 -400 To do 2.5 40.0 42.5 45.0 47.5 50.0 52.5 55.0 57.5 60.0 IDS of REFI SD Output Probing frequency [GHz] to IMAS 0.0 · Beat frequency \rightarrow tof delay \rightarrow radial locations 5 8 RLos [m] Probing frequency \rightarrow density

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Refractometer



refractometer = sd.slice_xml_wrapper(equilibrium,core_profiles,refractometer,xml_filename)

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ECE synthetic diagnostic for ITER with ECRad

- 55.F1 Electron Cyclotron Emission (ECE)
- Measures T_e profile and $\delta T_e/T_e$

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- ECRad model developed by S. Denk, adapted to IMAS with A. Medvedeva
- First tests for 1.8 T and 2.65 T PFPO scenarios done to predict the ECE system operation and radial resolution

(radial and oblique ECE channels 123-353 GHz, O- and X-mode)









JINTRAC 7.5 MA / 2.65T Hydrogen



M. Schneider – PPPL Seminar – 7 February 2022

Bolometers



(Divertor) Neutron Flux Monitors

- 55.BC: DNFM developed by A. Kovalev
- Fortran and Python versions, all in IMAS



- 55.B4: NFM developed by A. Kovalev
- DNFM and NFM measure the total neutron flux and fusion power:
 - DNFM more sensitive to vertical plasma shift
 - NFM more sensitive to horizontal plasma shift
- → To be combined to deliver a measurement with less systematic error.



DNFM + NFM EQ#1 & uncertainty analysis



X-Ray Crystal Spectrometer Core

- 55.E5 X-Ray Crystal Spectrometer Core, measures T_i and v_{tor}
 XICSRT python code developed by N. Pablant, IMAS-adapted
 - by E. Bourcart and Z. Cheng

spectrometer_x_ray_crystal

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fter



• x_ray_matrix_signal (Python, E. Bourcart)

PFPO-1

ECH - 6.7 M Integrated commis of tokamak core CH, ICRH, NB

• Minerva XICS (Java, A. Langenberg)

Visible Light Spectrum

 CASPER: CAmera & SPectroscopy Emission Ray-tracer: generates the light spectrum for visible spectroscopy and cameras synthetic diagnostics





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- Current status:
 - CASPER provides light spectrum for VSRS and CXRS
 - H-alpha and Divertor Impurity Monitor to be added (with RTM calculation)

Charge Exchange Recombination Spectroscopy

- 55.E1 / 55.EC / 55.EF Core / Edge / Pedestal Charge Exchange Recombination Spectroscopy
- Measure T_i , Z_{eff} , He, impurity profiles, toroidal and poloidal rotation
- Python fit_CXRS model developed by A. Shabashov:
 → reconstruct plasma parameters from spectrum fit





 On-going comparison between sightline and simplified fibre source models





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Workflow for Spectrometry Modelling



Workflow for Synthetic Diagnostics (still under developement)

Workflow Parameters (standalone)		Magnetic Diagnostics					GUI adapted from				
Input User Path	public										
Input DB	iter	_ Neutron Diagnostics (Fusion Products)	H&CD WORKTIOW, WITH								
Input #Shot	134174	- 55.B4 Neutron Flux					extended features				
Input #Run	117	- 55.BC Divertor Neutron Flux		-	Time Base	OAC					
Output User Path	default	Optical Systems / IR Systems				Thi	e illustratas th	h nood			
	default	- 55.C5 TIP	dip_tip	•	Time Base	÷ 1111		e neeu			
оптрит рв		- 55.FA DIP	dip_tip	•	Time Base	to r	nake the work	flow			
Output #Run	118	- 55.C6 PoPoLa	pop		Time Base	1		1.			
Start Time [s]	20.0	Bolometric Systems		•		100	is generic and	το			
End Time [s]	140.0	- 55.D1 PP pinholes			Time Base	ext	ract them as a	n			
Time Step [s]	2	- 55 D1 PP collimators		•	Time Pase	in al		••			
Load	Load latest	- 55.D1 Divertor collimators	- 55.DI PP COllimators								
Save	Run	- 55.D1 VV collimators	Time Base	pac	скаде						
Save as Pestore Default		Spectroscopic Instruments and NPA Systems					Now footural				
Save as Rescore Default		- Generic Light Spectrum									
Exit		- 55.E6 VSRS	Time Base			🗡 (CO	mplex_mode.p	oy)			
		- 55.E1 CXRS Core		•	Time Base						
		- 55.EC CXRS Edge		¥	Time Base						
		- 55.EF CXRS BES		Model	s	Save	Restore default	Exit			
		- 55.E2 H-alpha									
		- 55.E4 DIM	DIP_TIP (tip_sel)		ip_sel)	plot_on	1				
		Microwave Diagnostics				n_points	3 256				
		- 55.F9.40 Refractometer	- 55.F9.40 Refractometer DIP_TIP (dip_sel)		Lp_sel)	noise	0.00001				
		Plasma-Facing and Operational Diagnosti	Plasma-Facing and Operational Diagnostics		(102	norse	0.000001				
		- (tba)		FOF (pop_	_361)						
		Edit Code Parameters	Show I	Flowchart							

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Independent time base management for each SD model



the H&CD workflow and all other IMAS Python workflows.

Example of using different time bases for SD models

- DINA-JINTRAC scenario with free boundary core-edge-SOL transport
- DT, 15 MA / 5.3 T, L-mode
- Results read from the interferometer IDS output by the diagnostic workflow (where DIP and TIP results are merged).



Bayesian inference with the Minerva framework

- Minerva is a platform for building large scale scientific models and performing corresponding inference using those models
 - Modular, traceable, versioned
 - Single or combination of diagnostics without information loss





[O. Ford. "*Tokamak Plasma Analysis through Bayesian Diagnostic Modelling*", PhD thesis, University of London, 2010]

- Accounts for uncertainties from measurement, systematic errors and model
- Based on Bayesian probability theory: uncertainties = probability distributions
- Used for forward analysis and experimental design
- Minerva installed in ITER cluster and reads IDSs

Synthetic Diagnostics in Minerva

- First focus on magnetics, VSRS, H-alpha, interferometer, X-ray spectrometer
- Associated Machine Description data (being populated):
 - Coils, flux loops, Rogowski loops:
 - * input = magnetics, pf_active, pf_passive, equilibrium
 - output = magnetics
 - EFIT++ reconstruction:
 - * input = pf_active, pf_passive, wall, tf
 - output = pf_passive, equilibrium
 - VSRS, H-alpha:
 - * spectrometer_visible
 - interferometry:
 - * interferometer
 - X-ray spectrometer (edge/core/survey):
 - * x_ray_crystal_spectrometer
- Near-term application:
 - Assessment of diagnostic coverage for L-H transition in PFPO



Model to compute toroidal flux loops, saddle loops and pickup probes.



Bayesian techniques with Integrated Data Analysis (IDA)

- Development of a workflow to combine the signals from ECE, TS and interferometry
- First prototype by R. Fischer using the TIP model from A. Medvedeva
- Adaptation of physics models to be compatible with IDA iteration loops:

→ Separate methods between initialisation steps and evaluation of SD signals

→ Future IMAS development planned for 2022 on "persistent actors" that would facilitate this



Initialisation of static variables	Defines input scenario, generates time loop, initialise all static data	<pre>init_static()</pre>
Initialisation of dynamic variables according to scenario time step	Execution within the time loop: get_slice, put_slice, etc. (e.g. read equilibrium for this time slice)	<pre>init_dynamic()</pre>
Evaluation of SD signal to be iterated in the IDA loop	Execution within the IDA convergence loop (adjustment of core_profiles quantities)	evaluate()

init_static()
 init_dynamic()

evaluate()

Page 37

Outline

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Reminder on the H&CD workflow

- Written in Python but orginally based on the Kepler version developed by T. Johnson
- Contains IMAS-adapted models to simulate all ITER H&CD sources:

	ECRH	ICRH	NBI	Nuclear reactions	
Wave or particle source	GENRAY GRAY TORBEAM	CYRANO LION PION TOMCAT	BBNBI NEMO	AFSI SPOT (α)	Validation of H&CD models in TRANSP
Fokker- Planck	Ø	FOPLA PION ASCOT SPOT	ASCOT SPOT RISK	ASCOT SPOT	NUBEAM, TORAY- GA, CQL3D could be part of it!

- The core of the workflow has an interface similar to actors (IDSs+codeparams) to make it easy to plug into other workflows
- Includes a Tk GUI for standalone H&CD simulation

GUI to configure the H&CD workflow

GUI dynamically built from code-specific parameters files (xml validated through xsd files)

	HCD W	RKFLOW		+ - • ×	Choic	e of F	180	D codes for ea	ach source	
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The universal algorithm of the H&CD workflow



Synergy between NBI and ICRH for ITER Helium scenario

- 2.65 T / 7.5 MA H-mode scenario (JINTRAC)
- 20 MW ICRH, 43 MHz, N=1 H (CYRANO/FOPLA)
- 33 MW NBI, 870 keV (NEMO/RISK)





 Fraction of RF power absorbed off-axis by H beams (higher collisionality)

 → Less energetic fast H distribution, leading to reduced neutron production
 [A. Polevoi et al, NF (2021)]



Study of ECH absorption profiles in 7.5 MA / 2.65 T scenarios



- X3 parasitic absorption at the edge: can be compensed by either increasing B-field or switching to O-mode polarization
- Excellent agreement between TORBEAM (solid) and GRAY (dashed).

[M. Schneider et al, NF (2021)]

H&CD modelling for an ITER 15MA / 5.3T DT scenario

 Input scenario from IMAS scenario database: ITER DT 15 MA / 5.3 T (from METIS)



- ICRH modelling: 20 MW:
 - ◆ 40 MHz, for N=1 D(+Be)

✤ 53 MHz for N=2 T heating



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Results for NBI (33MW) + alphas (96MW) + ICRH (20MW)



Weak RF-α and RF-NBI synergy (<5% ICRH)

[M. Schneider et al, sub. to NF (2021)]

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- Dominant electron heating (alphas)
 Significant core ion heating (~40%) due
 - to combined ICRH, NBI and α heating

Page 45

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Summary

- IMAS supports the ITER Research Plan by providing a standard for integrated modelling delivering a high level of modularity and flexibility
- Scenarios are available for each phase of the ITER Research Plan
- A Machine Description database is available to describe ITER plant systems
- IMAS will provide a high-fidelity plasma simulator including self-consistent calculation of freeboundary equilibrium + core-edge transport → to be used for predictive and interpretive modelling
- Progress in synthetic diagnostics development:
 - for generating synthetic data in IMAS (python workflow under development)
 - in preparation for ITER data analysis and interpretation using Bayesian modelling
 - for supporting controller development in connection with the PCS Simulation Platform
- The H&CD workflow is as an essential element of any high-fidelity plasma simulator, enabling the modelling of the synergy between different H&CD sources
- Other workflows being prepared for ITER operation using IMAS include those for interpretive equilibrium reconstruction and stability (including EPs) assessment.

PPPL could contribute through the use of these H&CD and SD models and workflows, and through the IMAS adaptation of TRANSP and its modules.