

Increased CHI Startup Currents through Imposed Non-axisymmetric Perturbations

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Perturbations Crossing Flux Surfaces Give Cross-field Current Drive Effect

- The HIT-SI data show that perturbations generate a cross-field viscosity in the electron fluid. The viscous force per unit area between adjacent equilibrium flux surfaces is simply $(\delta B_{\perp})^2/2\mu_0$, where δB_{\perp} is the magnetic perturbation perpendicular to the flux surface.*
- The j/B profile is flattened
- Model agrees with tokamak rotation, disruption ramp down rates and allowable field-errors.

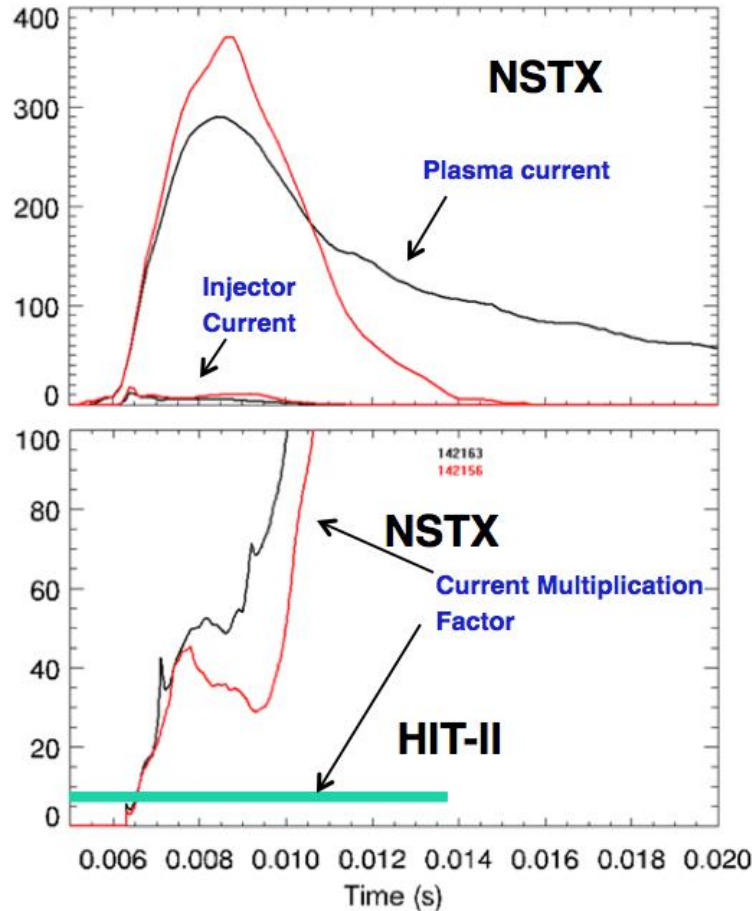
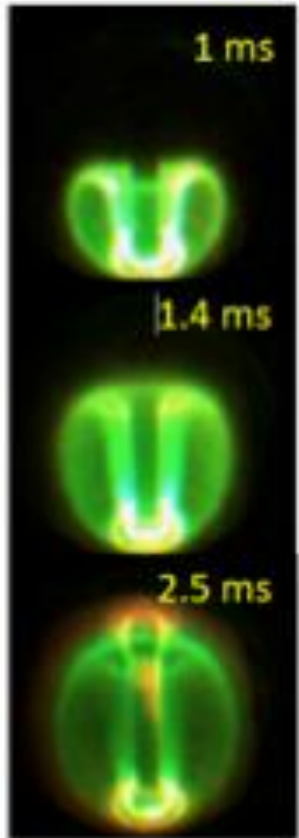
*T. R. Jarboe *et al.*, Nucl. Fusion, **52**, 083017 (2012)

XP: Apply Asymmetric Fields on Transient CHI to Produce Flux Amplification (FA)

- Previous experiments depended on non-axisymmetric *instabilities* to produce FA (e.g. SSPX)
- Imposed Dynamo Current Drive (IDCD: Jarboe NF 2012) applies non-axisymmetric fields to a *stable* equilibrium producing high- β spheromaks (HIT-SI: Victor PoP 2014)
- Hollow NSTX Transient CHI plasmas allow imposed perturbations to flatten edge J/B; IDCD scaling for NSTX CHI startup-up plasmas show scaling > 1 MA
- Asymmetric fields allow conversion of injected toroidal flux ($d\Phi_{\text{TOR}}/dt = -V_{\text{CHI}}$) to poloidal flux, increasing I_p
- **Proposal:** Use RWM coils to produce asymmetric fields on Transient CHI start-up plasmas to evaluate current penetration, and flux amplification (higher I_p)
 - 0.5 or 0.25 days; can run during Transient CHI operations

Backup Discussion

CHI Start-up Plasma Poloidal Flux is Limited to Initial Injector Flux Value



- Axisymmetric reconnection produces ~ 200 kA closed I_p
- High current amplification (I_p/I_{CHI}) achieved, but flux amplification (FA) requires non-axisymmetric motion

An effect of perturbations on the current profile and how to use it

By
Tom Jarboe and the HIT team
To
The NSTX team

Outline

- Perturbations flattening the j/B profile and cause rotation
- Agreement with tokamak data
- NSTX-U tests

Perturbations crossing flux surfaces give cross-field current drive effect

- The HIT-SI data show that perturbations generate a cross-field viscosity in the electron fluid. The viscous force per unit area between adjacent equilibrium flux surfaces is simply $(\delta B_{\perp})^2/2\mu_0$, where δB_{\perp} is the magnetic perturbation perpendicular to the flux surface.*
- The j/B profile is flattened
- In the externally-driven regions dynamo force brakes electrons so the force is in the direction of the current giving plasma velocity in that direction
- In the dynamo driven region the force is with the electron flow resulting in plasma flow against the current

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The cross-field current drive is consistent with observations on tokamaks.

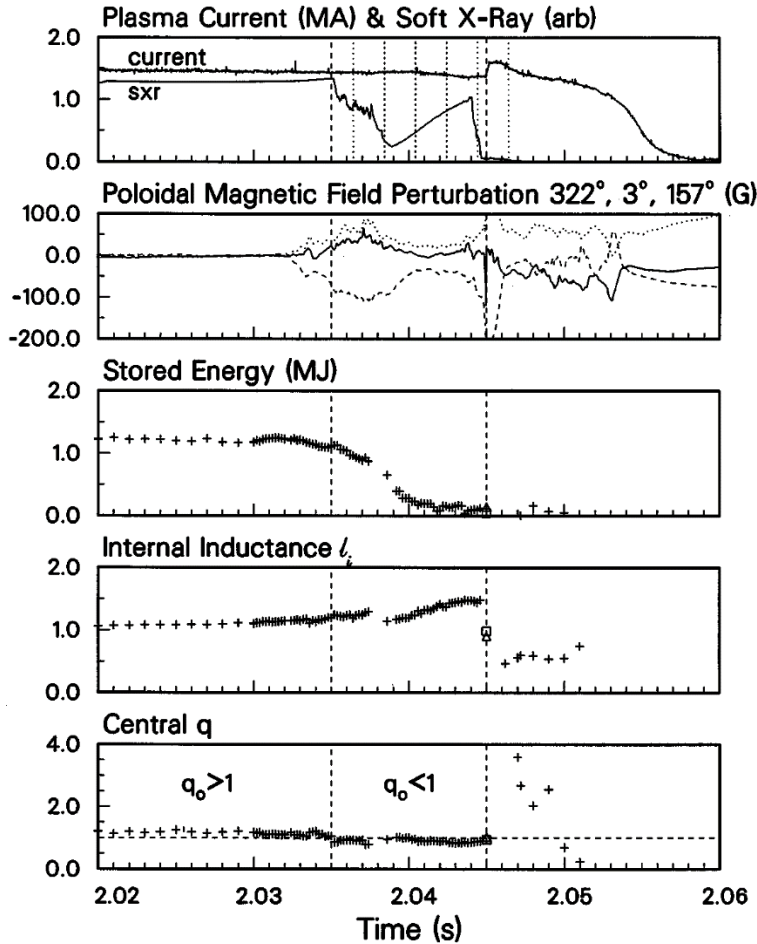
- Plasma rotates with current in normal tokamak and against the current when LHCD is used in the edge*
- Low level perturbations have a large effect on performance probably because they flatten j/B that flattens q -profile, on a tokamak, and decreases shear
- Maxwell stress tensor analysis for flux surface (given in the Imposed Dynamo Current Drive (IDCD) paper**) yields the δB_{\perp} required to drive the current:

$$\frac{(\delta B_{\perp r m s})^2}{\mu_o} \geq (\eta j_{\parallel} - E_{\parallel}) n e r; \quad \frac{\mu_o}{4 \pi} \frac{d(l_i I)}{d t} \approx -E_{\parallel}; \quad \delta B_{\perp r m s} \geq \left(\frac{\mu_o^2}{4 \pi} \frac{\Delta(l_i I)}{\Delta t} n e r \right)^{1/2}$$

*J. E. Rice *et al.*, Nucl. Fusion **49** 025004 (8pp) (2009)

T. R. Jarboe *et al.*, Nucl. Fusion, **52, 083017 (2012)

IDCD equation agrees with radiative disruption* perturbations on DIII-D



- Disruption created by argon injection.
- First: cold edge peaks the current until it is unstable. Instability cools plasma.
- Second: 1.5 MA profile is flattened in 1.2 ms

$$\Delta (l_i I) / \Delta t = 0.92 GA / s$$

IDCD requires δB of 190 G (at 2.045-6 s)

- Third: 10 ms current quench.

$$\Delta (l_i I) / \Delta t = 0.08 GA / s$$

IDCD requires δB of 60 G. (at 2.046-56 s)

*P. L. Taylor *et al.*, *Phys. Rev. Lett.*, **76**, 916-919 (1996)

On a reactor and ITER the perturbation levels required to drive the current are a little higher than considered acceptable (confirming the effect). They are small.

Parameter	Present tokamaks	ARIES-AT	ITER
I_{tor} (MA)	4.5	12.8	15.
Temp. (keV)	2	18	8.1
a (m)	1	1.3	2
$\tau_{\text{L/R}}$ (s)	15	605	454
$\delta B_{\perp \text{rms}} / B$	0.0004	0.0001	0.0001

- If they can drive the current they flatten the j/B profile which flattens the q -profile leading to poor performance.

Summary of the impact of perturbations flattening the j/B profile on a tokamak

- Low-level (10^{-4}) perturbations seriously damage performance. Uniform j/B gives low β .
- With an extensive low- j/B edge plasma or locked modes, higher levels (10^{-2} - 10^{-3}) cause a disruption.

Solutions

- Drive the edge current high and impose a perturbation profile that sustains the desired reversed-shear current profile.
- Solves the sustainment problem. (400 times more efficient than RF)
- High edge current prevents the edge from using perturbations to drag down the current in disruptions.

Possible related experiments on NSTX-U

- At $n = 2.5 \times 10^{19} \text{ m}^{-3}$, $T = 30 \text{ eV}$, $a = 0.6 \text{ m}$, $\delta B_{\perp} = 50 \text{ G}$
 - $I = 1.5 \text{ MA}$ can be sustained
 - $di/dt = 80 \text{ MA/s}$ ramp up or down, depends on edge current.
- Use RWM coils to augment CHI startup.
- Tie down the edge current with an argon puff and then demonstrate control of ramp down using RWM coils.
- Do not tie down the edge and generate halo currents.
- For current drive: The edge currents can be driven by CHI and the perturbations imposed by RWM coils. The only restriction for current drive is that j/B must be monotonically decreasing from the edge.
- Drive the edge with CHI for disruption control.