

# NSTX Program Update: NSTX upgrade discussion 5 year plan schedule

College W&M **Colorado Sch Mines** Columbia U Comp-X **General Atomics** INEL Johns Hopkins U LANL LLNL Lodestar MIT **Nova Photonics** New York U **Old Dominion U** ORNL PPPL PSI **Princeton U** SNL Think Tank, Inc. UC Davis **UC** Irvine **UCLA** UCSD **U** Colorado U Maryland **U** Rochester **U** Washington **U Wisconsin** 

NSTX Program Update

# J. Menard, M. Ono

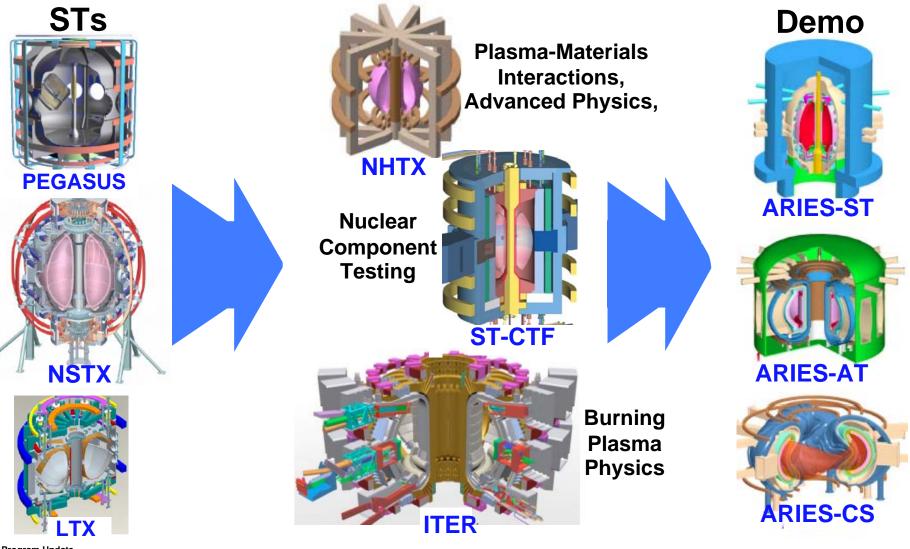
May 28, 2008 NSTX Team Meeting Princeton Plasma Physics Laboratory

Culham Sci Ctr U St. Andrews York U Chubu U Fukui U Hiroshima U Hyogo U Kyoto U Kyushu U Kyushu Tokai U NIFS Niigata U **U** Tokyo **JAERI** Hebrew U loffe Inst **RRC Kurchatov Inst** TRINITI **KBSI** KAIST ENEA, Frascati CEA, Cadarache IPP, Jülich **IPP, Garching** ASCR, Czech Rep **U** Quebec

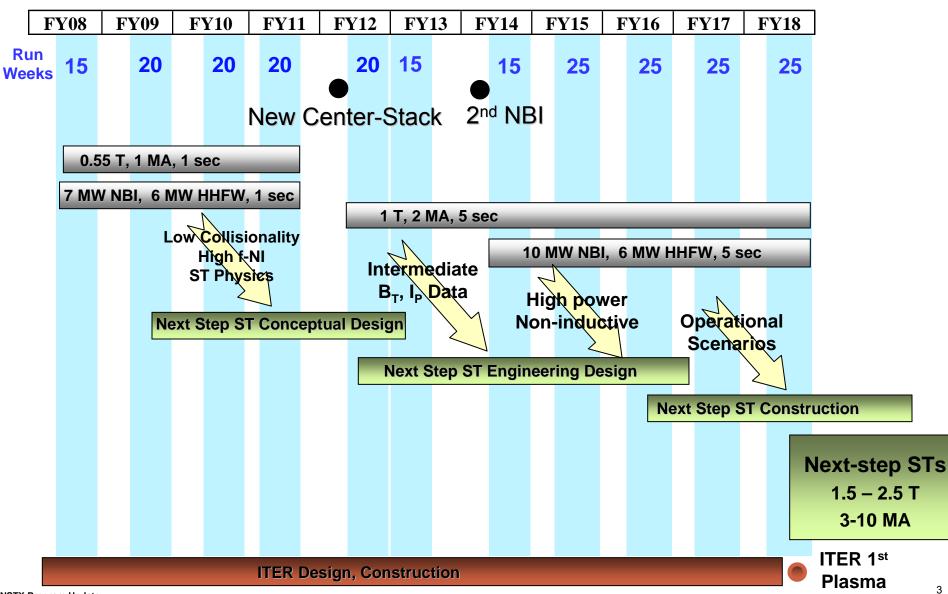
# NSTX Research Program Contributes Strongly to US and World Fusion Development



#### **ST offers compact geometry + high** $\beta$ for attractive fusion applications



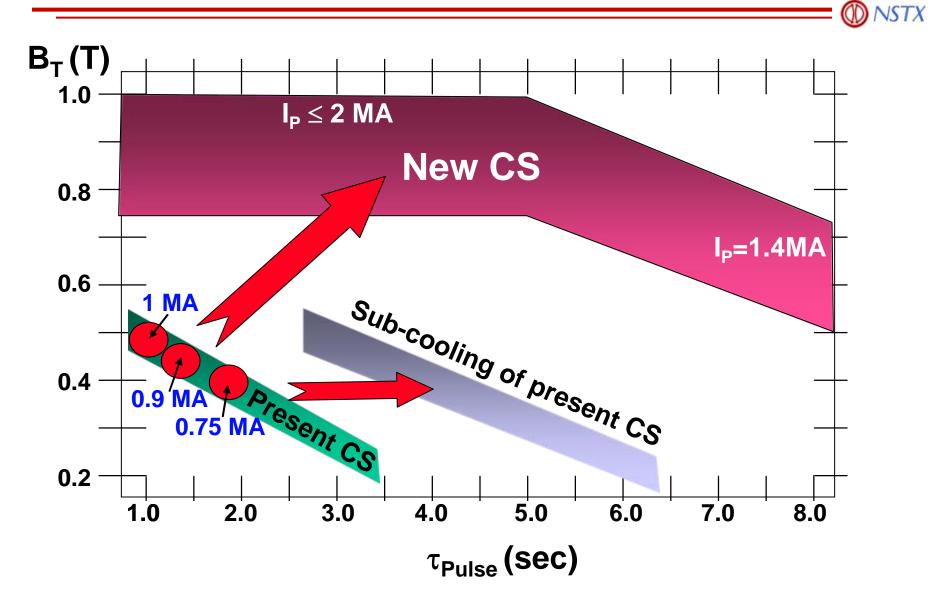
Proposed NSTX 10 year plan supports ST community goal of constructing and operating a next-step ST during the ITER era



NSTX Program Update

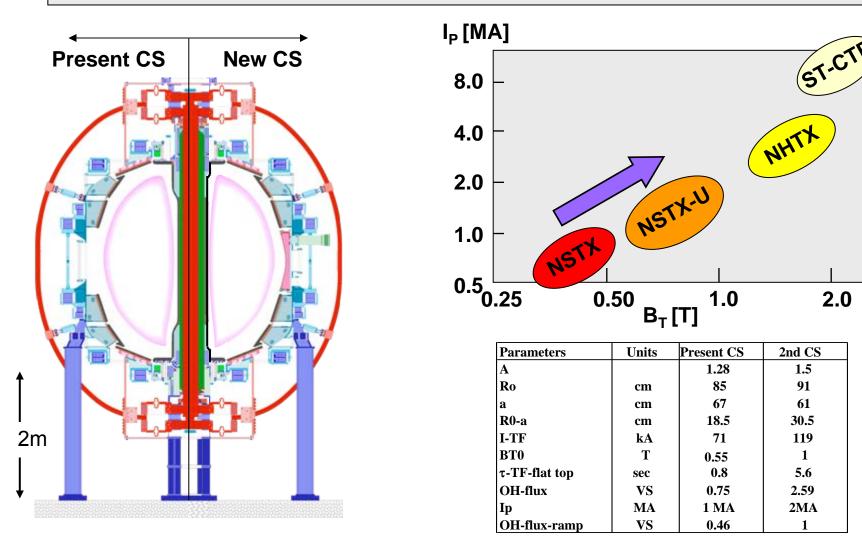
**D**NSTX

# New CS would greatly expand performance range of NSTX



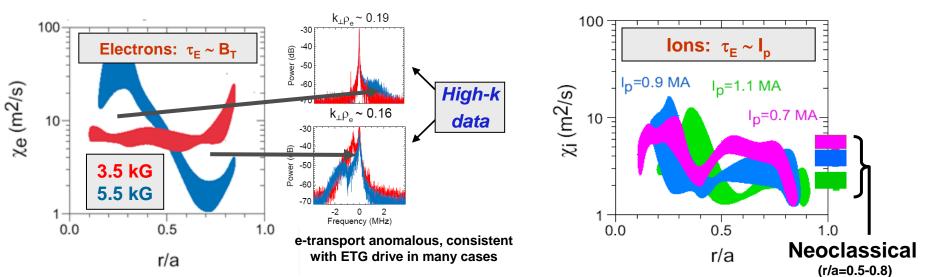
# New NSTX center stack would enable significant step toward parameter regime of next-step STs

• NSTX-U would operate within a factor of 2 in  $B_T$  and 2-4 in  $I_P$  of next-step STs • NSTX-U would have higher aspect ratio = 1.5-1.6 similar to next-step STs



**NSTX** 

## New center stack would greatly expand understanding and performance of ST plasmas



Access higher temperature, lower collisionality plasma

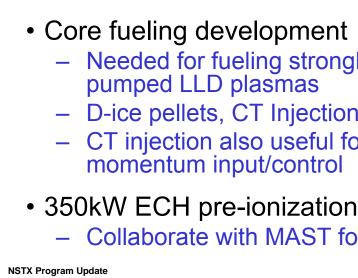
– Reduced  $v^*$  impacts transport, MHD, boundary physics, fast-ion-driven instabilities

#### • Improve understanding of transport and turbulence:

- Assess if electron  $\tau_E \sim B_T$  is result of low  $B_T$ , high  $\beta$ , suppressed ion transport, other
- Assess if near-neoclassical ion transport is maintained at lower v\*, higher B

•Assess heating, CD, start-up, ramp-up closer to parameters of next-step STs:

- Higher field, current  $\rightarrow$  improved fast-ion confinement, higher T<sub>e</sub>  $\rightarrow$  higher NBICD
- NBI  $v_{fast} / v_{Alfvén}$  lower  $\rightarrow$  fast-ion instabilities reduced and variable over wider range
- HHFW surface waves reduced  $\rightarrow$  improved power coupling
- Higher B<sub>T</sub>, T<sub>e</sub> aids plasma start-up (CHI, guns, PF)



 350kW ECH pre-ionization for improved CHI, plasma gun, and PF start-up Collaborate with MAST for EBW start-up and current drive?

Needed for fueling strongly-

LLD for sustained pumping and high heat flux

If LLD unsuccessful, implement cryo-pumps

- **NSTX** is only **ST** that can study ELM control at  $\beta_N > \beta_{N-no-wall}$ 

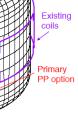
- EF correction w/  $n \le 6$ ,  $\Omega_{\phi}$  control ( $n \le 6$ ; n > 1 propagation)

– ELM mitigation w/ n up to 6, RWM control w/ n > 1 & higher  $\beta_N$ 

- D-ice pellets, CT Injection
- CT injection also useful for

Interna coil

option



7

Secondary

P option

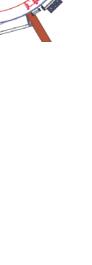
DNSTX

# FY14-18: 2<sup>nd</sup> NBI in combination with new CS would greatly expand understanding and performance of ST plasmas

- Assess full non-inductive regimes of next-step STs
  - Test prediction of higher CD efficiency from larger  $R_{TAN}$
  - Test off-axis NBI-CD for maintaining  $q_{min} > 1$
  - Test ability to avoid NTMs with  $q_{MIN} > 2$  operation
    - Higher field increases q at fixed  ${\rm I_P}$
  - Optimize mix of BS and NBI-CD for full-NI operation
    - Higher  $B_T$  lowers  $\beta_N$  at fixed  $I_P$  and  $f_{BS} \rightarrow$  expanded operating range
    - Compare full-NI operation near no-wall and ideal-wall  $\beta_{\text{N}}$  limits

– 5s TF  $\rightarrow$  can sustain full non-inductive operation for  $\tau_{flat} >> \tau_{CR}$ 

- Test NBI current ramp-up at higher  $B_T$ ,  $I_P$ , and  $P_{NBI}$
- Assess high-power scaling of confinement at high  ${\rm B_{T}},\,{\rm I_{P}}$
- Develop long-pulse divertor at high heat flux up to  $20 MW/m^2$
- Greatly expand research flexibility by varying:
  - q-shear for transport, MHD, fast-ion physics
  - Heating, torque, and rotation profiles
  - $\beta$ , including higher  $\beta$  at higher I<sub>P</sub> and B<sub>T</sub>
  - Fast-ion f(v\_{\parallel},v\_{\perp}) and \*AE instabilities
    - 2<sup>nd</sup> NBI more tangential like next-step STs



JSTX

New 2<sup>nd</sup> NBI

R<sub>TAN</sub>=110,120,130cm



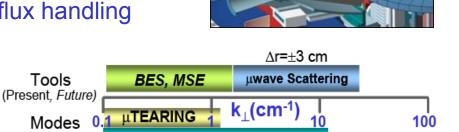
- 1. Increase and understand beam-driven current at lower  $n_e,\,\nu^{\star}$
- 2. Increase and understand H-mode confinement at low  $\nu^{\star}$
- 3. Demonstrate & understand non-inductive start-up & ramp-up
- 4. Sustain  $\beta_N$  and understand MHD near and above no-wall limit

### Near-term upgrades will continue to support highest research priorities identified previously for FY08-10

- 1. Implement liquid lithium divertor for pumping, and investigate other potential benefits:
  - Improved confinement
  - Reduction/elimination of ELMs
  - Compatibility of LLD with high flux expansion
  - Longer-term: steady-state high-heat-flux handling
- 2. Implement BES to complement existing high-k scattering diagnostic
  - Measure full wavenumber spectrum of turbulence
  - Determine modes responsible for anomalous transport of energy & momentum

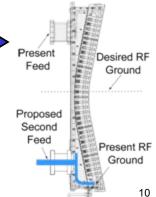
Tools

- 3. Upgrade HHFW system for higher  $P_{RF}$  + ELM resilience
  - Determine if HHFW can ramp-up I<sub>P</sub> in H-mode (BS+RF overdrive)
  - Determine if HHFW can heat high- $\beta_N$  advanced H-mode scenarios
  - HHFW/ICRF also important for NHTX/CTF/ITER



Liquid Lithium

**Divertor (LLD)** 



ETG

# Firm run-time guidance is not yet available, but we expect increased run-time and additional (incremental) milestones

**OD** NSTX **FY2008 FY2010 FY2009** Expt. Run Weeks: 15 20 (?) 20 (?) 1) Transport & Turbulence Study turbulence regimes responsible for ion and electron Measure poloidal rotation at energy transport low A and compare w/ theory Assess sustained operation 2) Macroscopic Stability Understand physics of RWM above the no-wall limit at stabilization & control as a reduced collisionality function of rotation 3) Boundary Physics Assess H-mode characteristics as a function of collisionality and **Study variation and control** lithium conditioning of heat flux in SOL Study how *j*(*r*) is modified by 4) Wave-Particle Interaction super-Alfvénic ion-driven modes Characterize HHFW heating, CD, and ramp-up in deuterium H-mode Accelerate high-power HHFW 1yr Integrate MHD mode modification of Test predictive capability of modej(r) into optimized operation induced fast-ion redistribution/loss 5) Start-up, Ramp-up, Sustainment **Couple inductive ramp-up** Investigate methods for solenoid-free Test non-inductive current to CHI plasma current initiation using induction generation using plasma guns from the outer poloidal field coils 6) Scenario Integration & Control Perform high-elongation wallstabilized operation at lower n<sub>a</sub> "JOULE" Milestones: TBD **Rotation and momentum** Particle control and hydrogenic

fuel retention

**NSTX Program Update** 

transport & stability physics

# 5 year plan document completion schedule

🔘 NSTX

- NSTX 5 year plan review will be week of July 28, 2008
  - 2 major themes: (1) physics from new CS, (2) impact of reduced  $v^*$ 
    - Up to 10× lower v\*, similar  $\rho^*_{th}$ , smaller  $\rho^*_{fast}$ , lower v<sub>fast</sub>/v<sub>Alfvén</sub>, 5s pulses, etc...
    - You should also consider diagnostic impact
      - For example, need to move MPTS sightline & dump, and measure up to  $T_e(0)$  = 10keV

Keep discussion of merits of 2<sup>nd</sup> NBI, but be clear this is at end of 5yr plan
Provide strong <u>scientific</u> justification for all major upgrades

Start NOW! **Incorporate new CS, low**  $v^*$  into chapters **June 9-13** EPS here Complete draft plan edits complete June 16, 2008 • June 23, 2008 5 year plan text complete Week of July 7 Dry runs of presentations Week of July 14 **Final presentation material ready** Week of July 28 2009-13 Five year plan review

Reduced  $\nu^{*}$  and normalized n\_e represent the largest gap between present and next-step ST operating scenarios

### Next-step ST's: $v^*$ will be 10-100× lower, $n_e/n_{GW}$ 2-4× lower

#### Reduced density and collisionality impact all topical science areas:

- Transport & Turbulence
  - Underlying instabilities (micro-tearing, TEM, and ETG) scale differently versus  $v^*$
  - If  $T_e(r)$  is determined by critical  $\nabla T_e$ , H-mode confinement may be reduced at reduced  $n_e$
- Macroscopic Stability
  - RWM critical rotation and viscous torques may increase at lower  $v_i$
- Boundary Physics
  - ELM  $\Delta W$  increases with reduced  $v_{e}^{*}$  could impact confinement, plasma purity, divertor
  - Detachment schemes for heat flux reduction more challenging with reduced SOL  $\nu$
- Wave-Particle Interaction
  - AE avalanches more easily triggered at reduced n<sub>e</sub> possible fast-ion redistribution/loss
- Start-up, Ramp-up, Sustainment
  - NBI-CD and RF-CD efficiency for ramp-up are increased at reduced  $n_{\rm e}$ , increased  $T_{\rm e}$
- Scenario Integration and Control
  - Steady-state scenarios rely on reduced  $n_e/n_{GW}$  to increase NBI-CD to achieve 100% NI-CD
- Expect higher  $B_T$  and  $I_P$  will increase  $T_e$  and  $T_i$
- Expect LITER + LLD and/or cryos will pump D to control and reduce density
  - Need to understand LLD operation, Li transport in SOL and to core, etc.
  - Separate dependence on collisionality vs. Lithium

(OD) NSTX