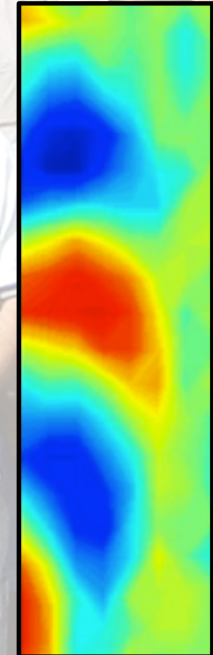
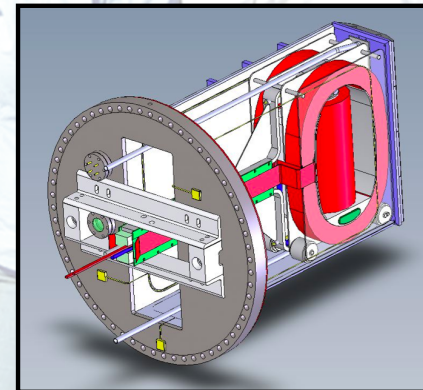
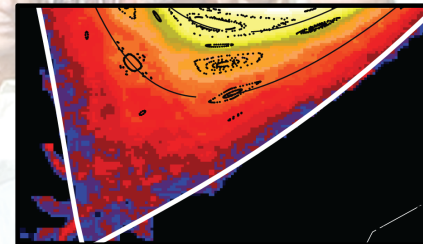
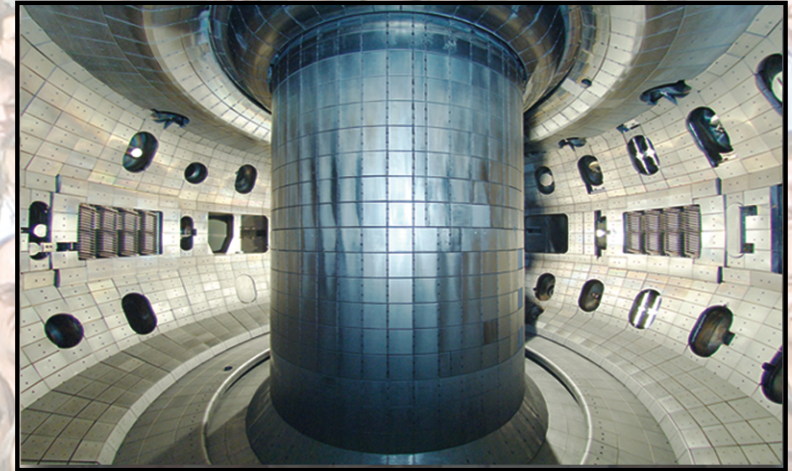


Opportunities for DIII-D/NSTX Collaborations in 2011-13

by
M.R. Wade

Presented at
**Princeton Plasma Physics Laboratory
Princeton, NJ**

October 19, 2011



DIII-D 2012 FWP Proposed 13 weeks of operation – Present FES Guidance is for 3 weeks

- Plan calls for:
 - 3 weeks in Oct.-
Nov 2011
 - 10 weeks in Apr-
Jul 2012

PROPOSED DIII-D FY2012 OPERATIONS SCHEDULE																											
Oct							Nov							Dec							Jan						
S	M	T	W	T	F	S	S	M	T	W	T	F	S	S	M	T	W	T	F	S	S	M	T	W	T	F	S
						1			1	2	3	4	5					1	2	3	1	H	3	4	5	6	7
2	3	4	5	6	7	8	6	7	8	9	10	11	12	4	5	6	7	8	9	10	8	9	10	11	12	13	14
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30	31																										
Feb							Mar							Apr							May						
S	M	T	W	T	F	S	S	M	T	W	T	F	S	S	M	T	W	T	F	S	S	M	T	W	T	F	S
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5	6	7	8	9	10	11	4	5	6	7	8	9	10	8	9	10	11	12	13	14	6	7	8	9	10	11	12
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Jun							Jul							Aug							Sep						
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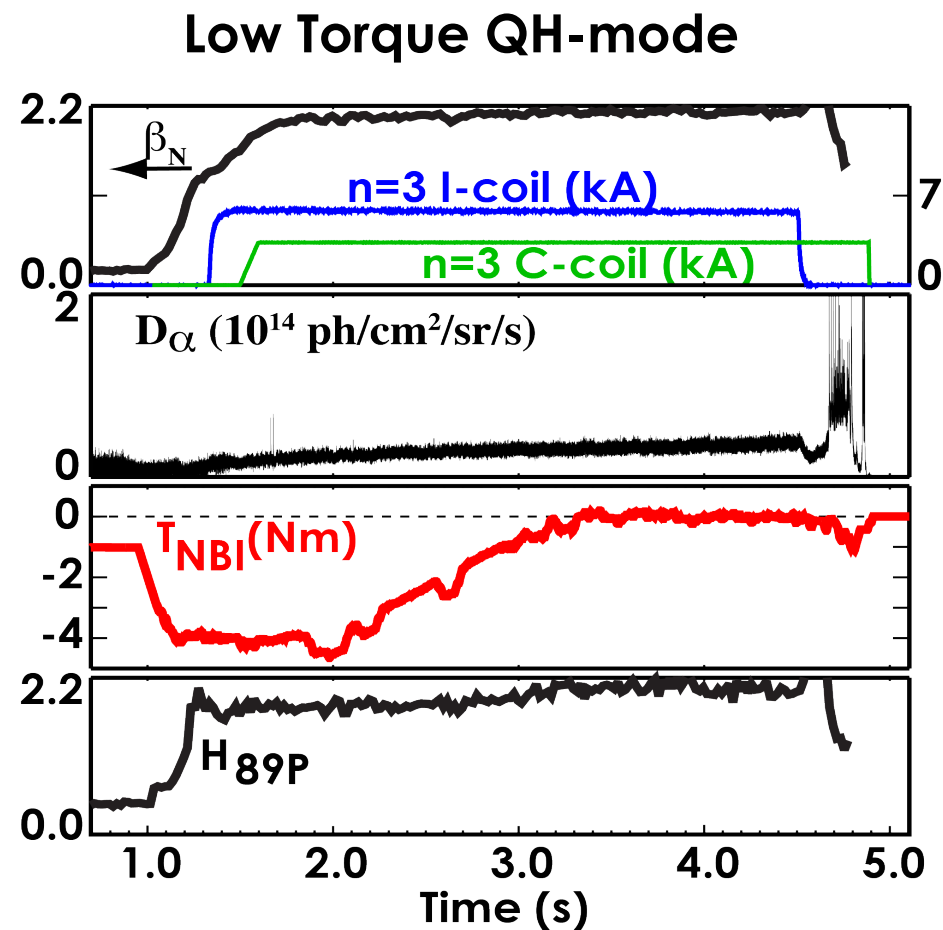
Plasma physics
 Startup
 Option
 Vent

Priorities of October 2011 Run Plan

- **QH-mode performance extension (co-Ip operation)**
- **Steady-state with off-axis NBI**
- **ITER baseline scenario assessment**
- **TBM experiment**
- **Pellet Pacing**
- **Runaway Electron Control**

QH-mode performance extension

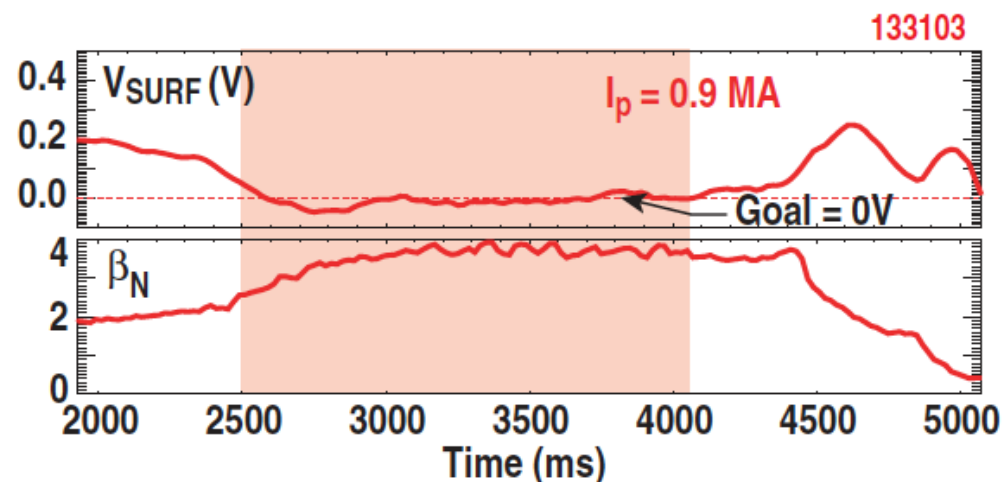
- Recent experiments have demonstrated capability to utilize NTV from non-resonant 3D fields to sustain QH-mode at zero NBI torque
- Near-term goals:
 - Demonstrate QH-mode access with low torque startup
 - Utilize larger 3-D fields from I- and C-coils (both at 7 kA) to push QH-mode to higher fusion gain ($\beta_N H/q_{95}^2$)
 - Larger NTV will allow larger co-NBI power (and presumably higher β)



3 days allocated (Garofalo, Fenstermacher, Doyle)

Steady-State with off-axis NBI

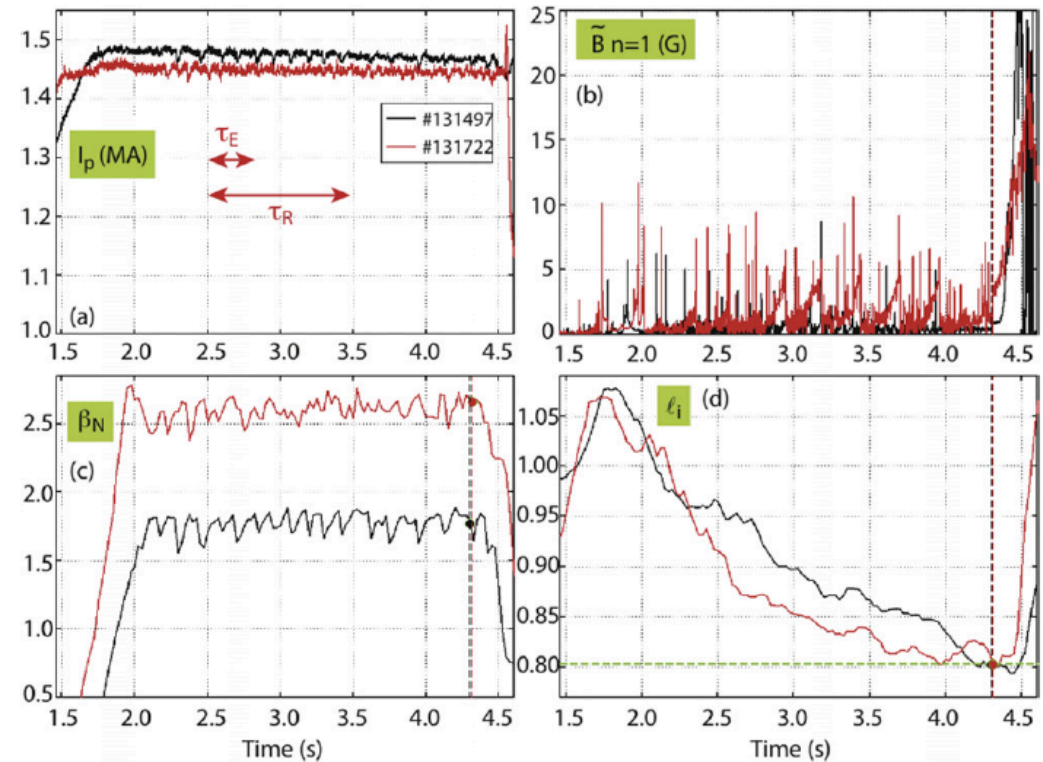
- **Experiments in 2007 have demonstrated > 1 s fully non-inductive operation at $\beta_N \sim 4$ with $q_{\min} \sim 1.5$**
 - Conducted with on-axis NBI
 - Limited by available NBI duration
- **Experiments in FY11 with off-axis NBI focused on $q_{\min} \sim 2$**
 - $\beta_N \sim 3$ achieved, limited by confinement
- **Near-term goal is to revisit 2007 $q_{\min} \sim 1.5$ experiments with off-axis NBI**
 - Additional off-axis CD (NBI and EC) and longer NBI pulse limits should allow ~ 3 s



- **2 days allocated (Holcomb)**

Assessment of Long-pulse Stability and Confinement Characteristics of ITER Baseline Scenario

- Performance consistent with ITER needs for $Q = 10$ demonstrated in ITER Baseline Scenario discharges worldwide
- Detailed examination of data set indicates tearing mode onset related to evolution of current profile (not pressure)
- Near-term goal is to characterize sensitivity of NTM stability and transport to long-pulse (8-sec flattop) evolution of current profile
 - If NTMs limit duration, determine ECCD requirements for long-pulse requirements

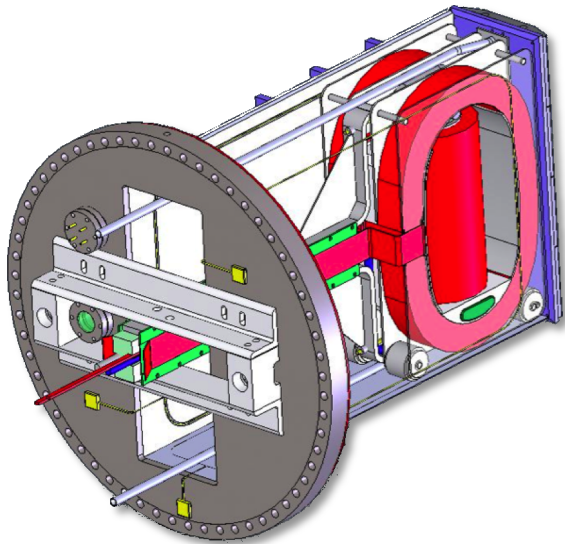


- 3 days allocated (Jackson, Solomon, Turco)

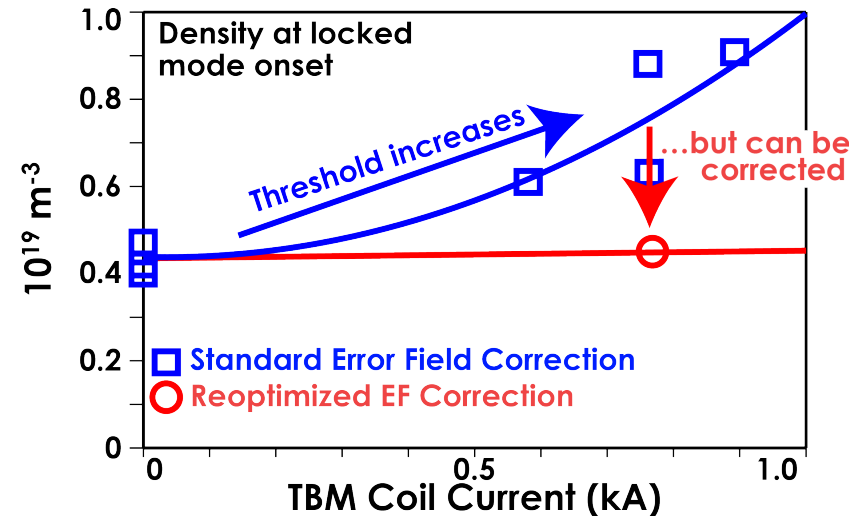
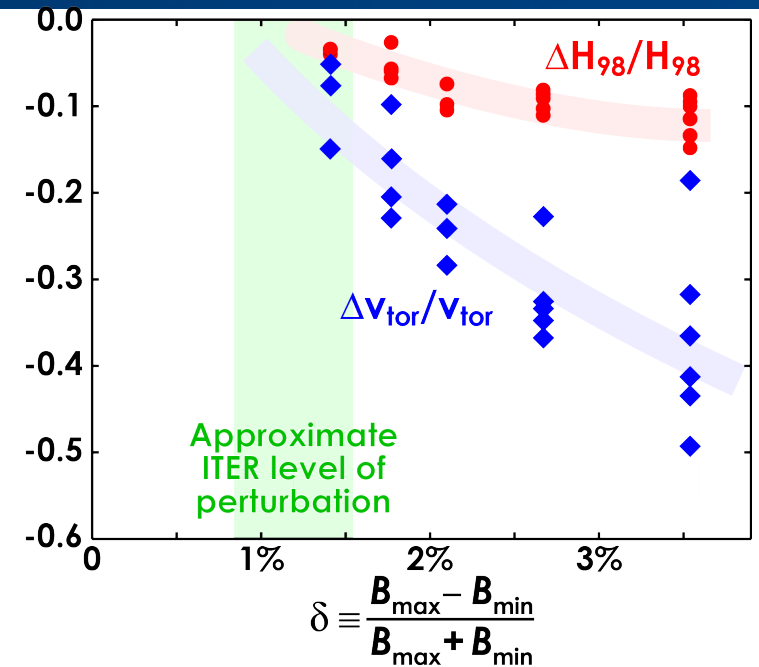
Test Blanket Module Simulation Experiments

Experiments in 2009 showed:

- Little effect on most quantities of interest
- Modest effect of TBM on rotation/ confinement
- Significant n=1 response of plasma to TBM field



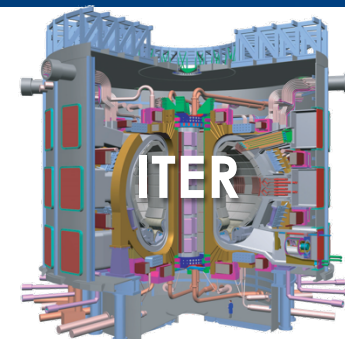
2011: Evaluate effect of compensation TBM-induced n=1 error field on rotation and confinement (Schaffer, Snipes, Reimerdes)



DIII-D Has Three Major Research Themes

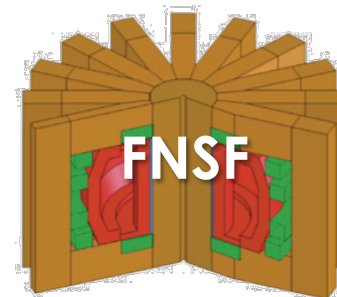
Ensuring the success of ITER

Critical issues for the design and operation of ITER



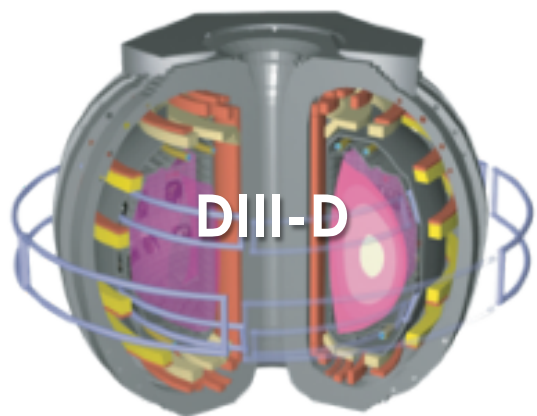
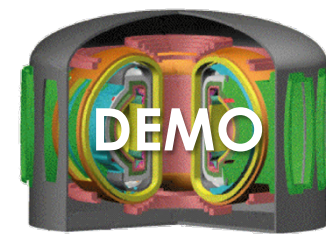
Improving predictive capability

Validation studies of physics-based models of burning plasma behavior



Developing sustained operational scenarios

Advanced inductive, steady-state



Research Goal is To Provide Physics Basis for Design/Operational Decisions Future Devices

Burning Plasma Conditions

- Dominantly electron heated
- Low torque
- Low fuelling & collisionality
- Energetic ions

Stable Operation at High β

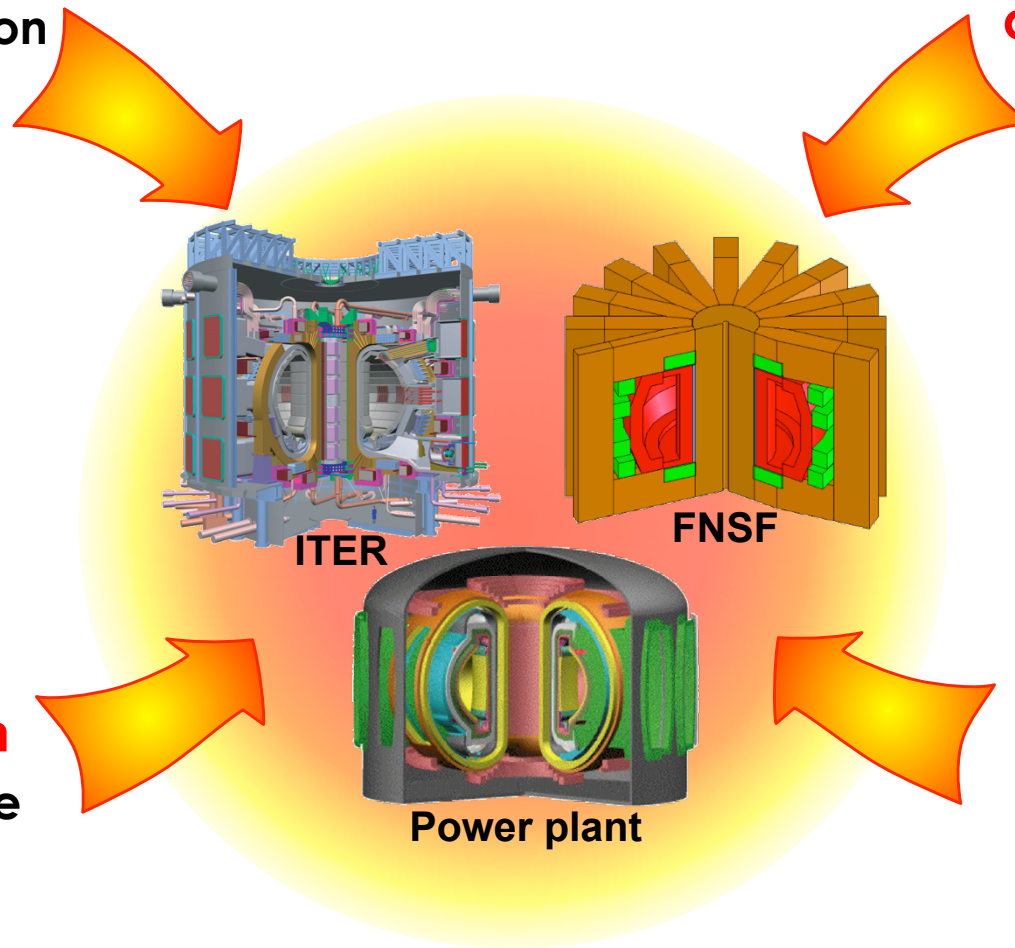
- Good passive stability
- Effect of energetic ions
- Active control through heating & 3D tools
- Event prediction, detection & control

Steady State Plasma Operation

- Fully non-inductive
- Self driven well aligned currents
- High confinement
- Configuration control

High Fluence Boundary Solution

- Spread heat
- Cool exhaust to avoid erosion
- Compatible with high performance core



Support for ITER is the Major Focus of DIII-D Research

ITER Timeline

Design & Construction

Initial Operation 2019

Burning Plasma

World Fusion Community Prepares for ITER Operation

Resolve short-term design issues for ITER

ELM control
Disruption mitigation
Startup, shape and position control
Hydrogenic retention & dust control

Resolve medium-term design issues for ITER

H-mode access in H₂ and He
Magnetic field asymmetries & 3D effects
Heating & current drive requirements

Address operational issues for commissioning and high-gain operation

Fast-ion instabilities
3D field effects
Operational scenarios

Integrated plasma dynamics and control

Strong electron heating $T_e \sim T_i$
Low external torque operation
Profile control, Divertor control

DIII-D priorities developed in consultation with ITER Organization

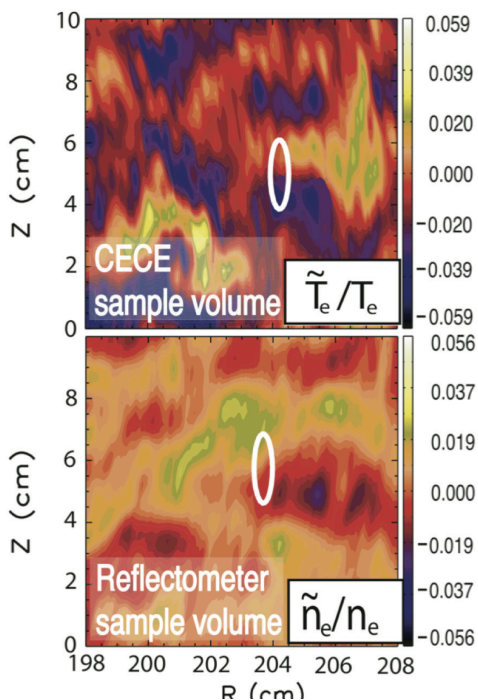


Strong Partnership Between Theory and Experiment is a Central Theme of the DIII-D Research Program

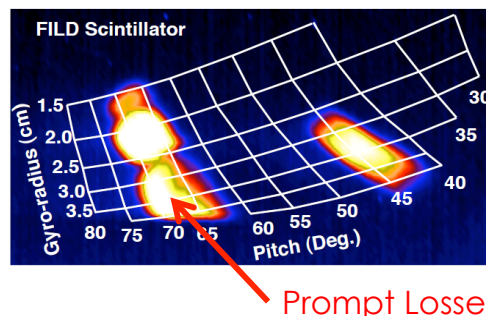
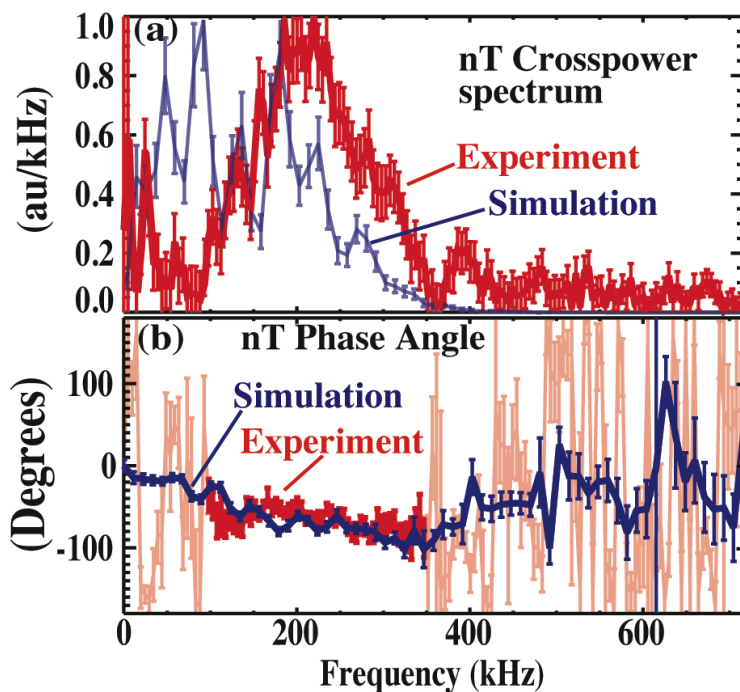
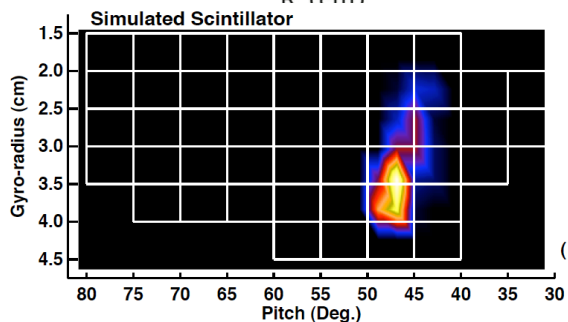
Theory

Experiment

Turbulence & Transport



Energetic Particles



A.E. White,
APS Invited
Talk 2009

M.A. Van
Zeeland,
APS Invited
Talk 2010

Anticipated Upgrades in 2012-13

- **Heating and Current Drive**
 - 7th (2012) and 8th (2013) gyrotron (4.2 MW ECH/ECCD)
 - Real-time steering of EC mirrors (2012)
 - FW assessment in Dec 2011 → upgrades??
 - NB pulse length extension (2012-2013)
- **Particle Delivery Systems**
 - 30 Hz D₂ pellet pacing (2012)
 - Rupture disk (2012-2013)
 - 50 Hz LiD pacing (2013 incremental)
- **Diagnostics**
 - IRTV/periscope (2012)
 - 3D Magnetics (2013)
 - Helium Line Ratio for ne and Te (2012)
 - TALIF neutral measurement prototype (2013)
 - Swing Langmuir probe on inner wall (2012)

DIII-D 2012 Experimental Planning Schedule

- **Early December: Research Council will meet to discuss DIII-D rolling 3-year program and develop priorities for the 2012 experimental campaign in the context of this plan.**
 - Task Forces and High Priority Working Groups announced
- **January 10-12, 2012: Research Opportunities Forum (ROF)**
 - Open Forum to discuss experimental proposals
 - Web Site should open in mid-December and be open through beginning of ROF
- **Jan 12 - ~ Feb 15, 2012:**
 - Experimental groups to discuss and prioritize proposals in each area
 - Research Council discusses overall allocation of run time
 - DIII-D management, based on Research Council advice, determines allocation of run time

DIII-D Physics Groups

Area	Responsibilities (not necessarily exclusive)
ITER Physics (Strait)	ELM control; Disruptions; NTM control; ITER scenarios; Error-fields; ITER-specific research (e.g., He/H ops, TBM)
Steady-state Integration (Luce)	Fully non-inductive development, hybrid/advanced inductive development; core-edge integration; RWMs
Fusion Science (Petty)	Transport; Energetic Particles; General stability; H&CD
Plasma Control and Operations (Humphreys)	Physics Operations; Model-based control
Integrated Modeling (Prater)	Pedestal Physics; Model validation
Boundary Plasma (Leonard)	SOL Physics; Boundary-Material Interfaces; New Divertor Geometries

High Impact Areas of Opportunity for 2012-2013

- **3D Physics and ELM Control (Wade)**
- **Disruption Mitigation, Avoidance, and Characterization (Hollmann)**
- **Transport Modeling (Petty/Burrell)**
- **Off-axis NBI and steady-state integration (see earlier slide)**
- **Non-inductive startup (??)**
- **Plasma Control (Humphreys)**

3D Physics and ELM Control

- **While much has been learned about RMP ELM suppression, a first-principle model has not emerged.**
 - Significant opportunities exist for new insights/discoveries
- **Still remains an open physics issue for ITER design**
- **Specific areas of need are:**
 - Assistance in defining a compelling 3D magnetics diagnostic
 - Modeling of DIII-D data with state-of-the-art computational tools (e.g., IPEC, XGC0, VMEC)

Aggressive Research Program is Envisioned to Provide Input for Critical Decisions on Future ITER and DIII-D RMP Coils

	2011	2012	2013
Tools			Prototype RMP coil
Diagnostics	Edge TS, Tang. X-pt. SXR, B probes	IR/visible periscope New diagnostic TBD	3-D magnetics

Near-Term activities focused on improving the physics understanding of edge transport and ELM suppression

Plan:

2011. Initial tests of models/hypotheses

Conduct comparative n=2 studies with Asdex-Upgrade

Measure turbulence response to RMP with edge turbulence diagnostics

Use expanded diagnostic set to carry out rigorous tests of promising models

2012. Demonstrate cross-machine capability to suppress ELMs in similar regimes

New 3D Coil Physics Assessment

Provide input for ITER RMP coil decision



Improved Understanding of the Plasma Response to 3-D Magnetic Perturbations Will Benefit ITER

	2011	2012	2013
Tools			Prototype RMP coil
Diagnostics	Edge TS, Tang. X-pt. SXR, Main ion CER, B probes	IR/visible periscope New diagnostic TBD	3-D magnetics

- Effects can be beneficial or detrimental
 - RMP ELM suppression, NTV rotation drive are examples of benefits
- Key is understanding the plasma response

The plan leverages experience in the stellarator community, an existing suite of 3-D MHD codes, and an expanding set of 3-D diagnostics:

2011. Measure $n=3$ plasma response to test ideal/resistive models

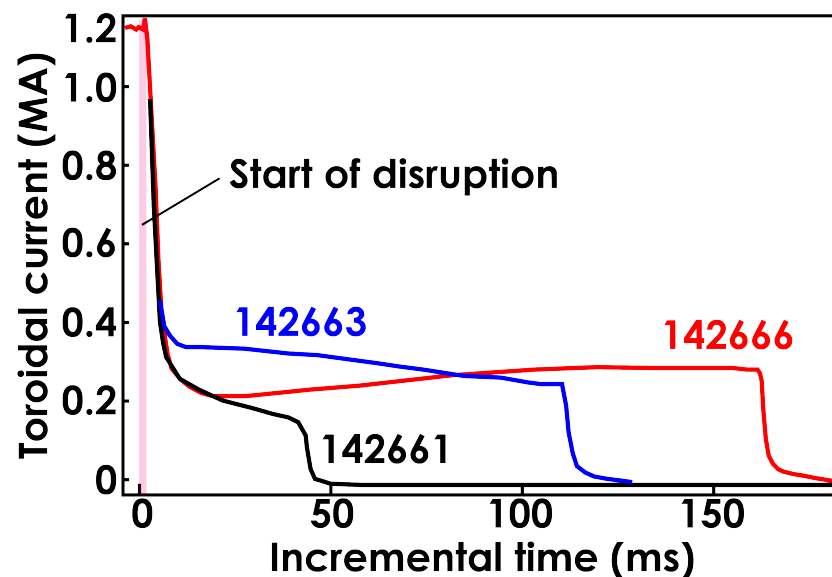
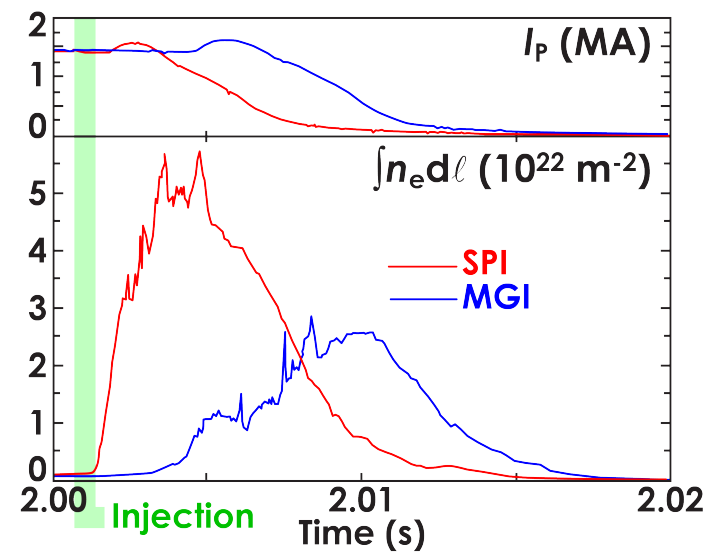
Evaluate impact of reoptimized error field correction on TBM effects

2012. Assess plasma response to applied error fields with multiple modes

2013. Utilizing expanded 3-D magnetics, quantify plasma response to a wide range of externally imposed 3-D fields

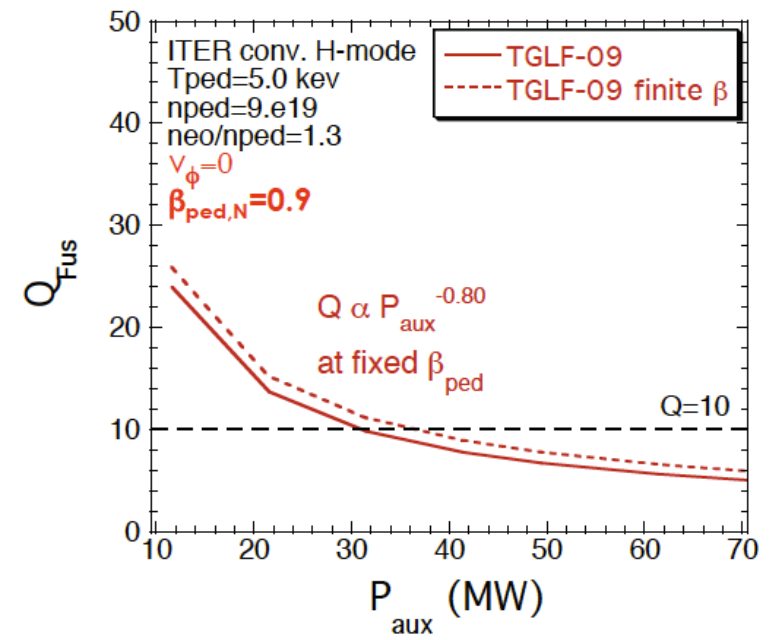
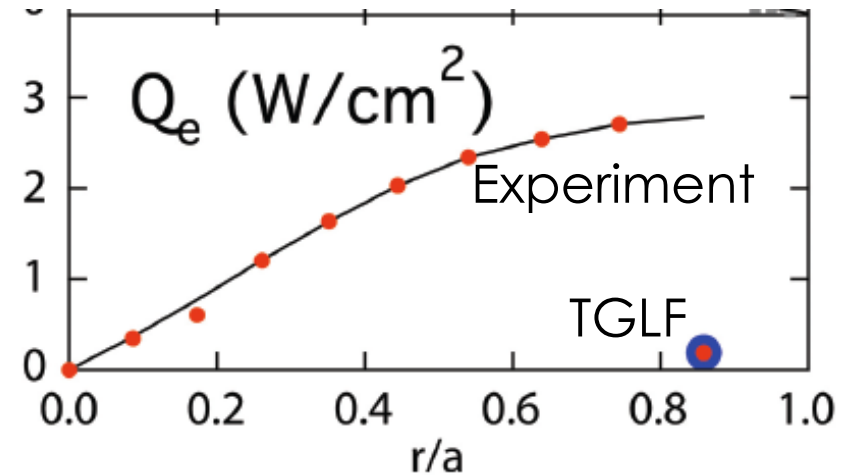
Disruption Mitigation, Avoidance, and Characterization

- **Avoiding and/or mitigating disruptive effects is recognized as a critical enabling element in the success of the ITER Research Plan**
- **Mitigation:**
 - Significant effort in recent years (massive gas injection, runaway channel control); some success, extrapolation uncertain
 - New schemes may still be needed
- **Avoidance:**
 - Ultimate need of any tokamak is to demonstrate robust avoidance
 - Limited effort to date; significant opportunity space in MHD detection and control
- **Characterization:**
 - ITER very interested in non-axisymmetric effects including halo currents/heat loads



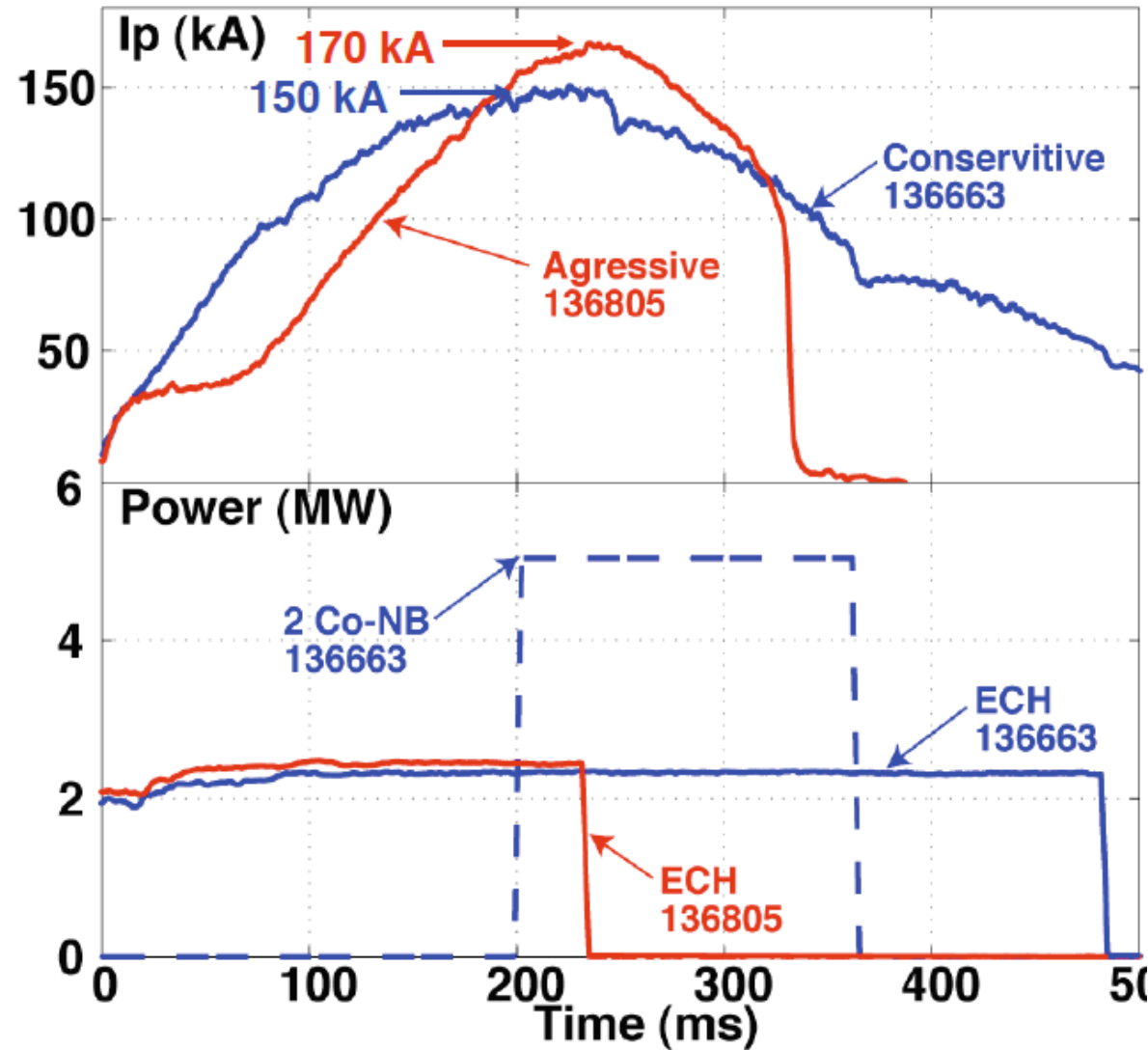
Transport Modeling

- Large amount of data taken over past few years using extensive set of diagnostics
 - Low-k, high-k δn_e , Low-k δT_e , $\alpha_{<\delta n_e \delta T_e>}$
- Comparisons show reasonable agreement between GYRO/TGLF for $\rho < 0.6$, **but disagreement for $\rho > 0.7$**
 - More cases available for comparison
- Recent TGLF modeling of ITER shows strong degradation of Q with auxiliary power
 - Analysis indicates this is due to profile stiffness
 - Lots of data collected in 2011 – limited transport analysis/modeling done to date



Solenoidless Startup

- > 150 kA plasma current produced without any assistance from DIII-D coils in “center stack”
- Need path to get ~ 300-350 kA before NBCD can take over
- Primary local champion (J. Leuer) is retiring in Dec 2011
 - Opportunity for run-time if compelling experiment is put forth



Transport and Confinement Opportunities

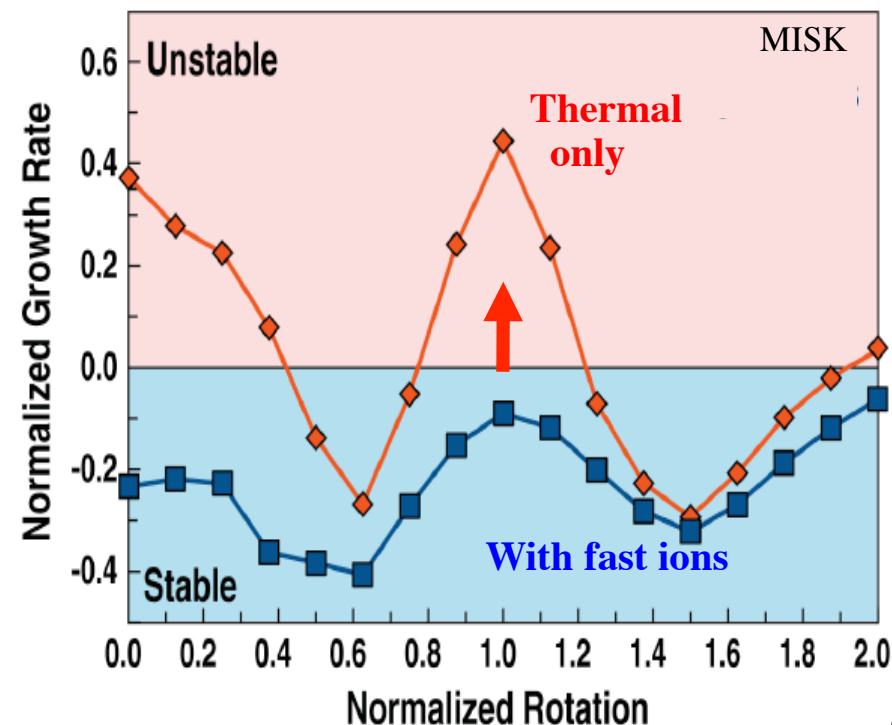
- **Model Validation (discussed previously)**
- **Test of Profile Stiffness / Critical Gradients (discussed previously)**
- **Particle/Impurity Transport (Petty)**
 - Due to lack of resources (i.e., people), currently an area with significant opportunities on DIII-D
- **Transport Analysis of Advanced Tokamak plasmas (Luce)**
 - Also lacking critical level of support

General MHD/Stability Opportunities

- **Resistive Wall Mode (RWM) Physics and Stabilization (Okabayashi)**
- **Disruption Avoidance (Strait)**
- **Plasma Response to Error Fields (Buttery, La Haye)**

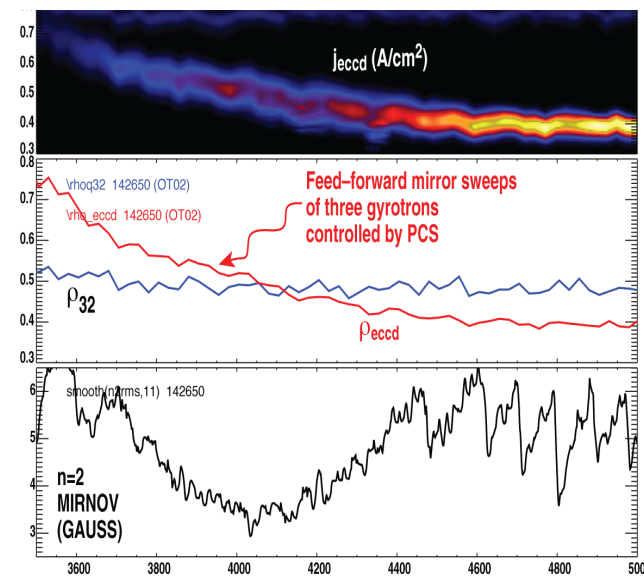
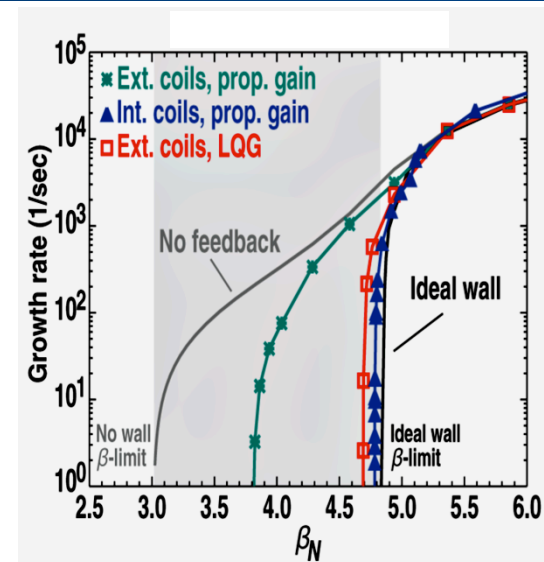
Variable NB Aiming and Torque Should Provide Interesting Tests of RWM Fast-ion Stabilization

- **Rotation and Fast-Ion Distribution (Spatial and Velocity) can be systematically varied using off-axis and co/counter NBI**
 - Should allow detailed tests of kinetic stabilization of RWMs by fast ions
 - Also will provide platform for RWM feedback stabilization in reactor-like conditions (low torque, more isotropic fast ion distribution)



Individual Tools for Instability Control are Nearly in Place; Next Step is Integrated Disruption Avoidance

- **RWM control:** New model based 'state space' controller suggests stabilization may be possible with external coils
- **NTM control:** Utilize new steerable ECH to develop 'black box' tearing control
- **Major challenge is to develop robust disruption avoidance algorithms that are applicable over a range of operating space. Requires:**
 - Routine implementation of stabilization tools
 - Real-time detection of instabilities
 - Hierarchical control logic to handle range of possible events

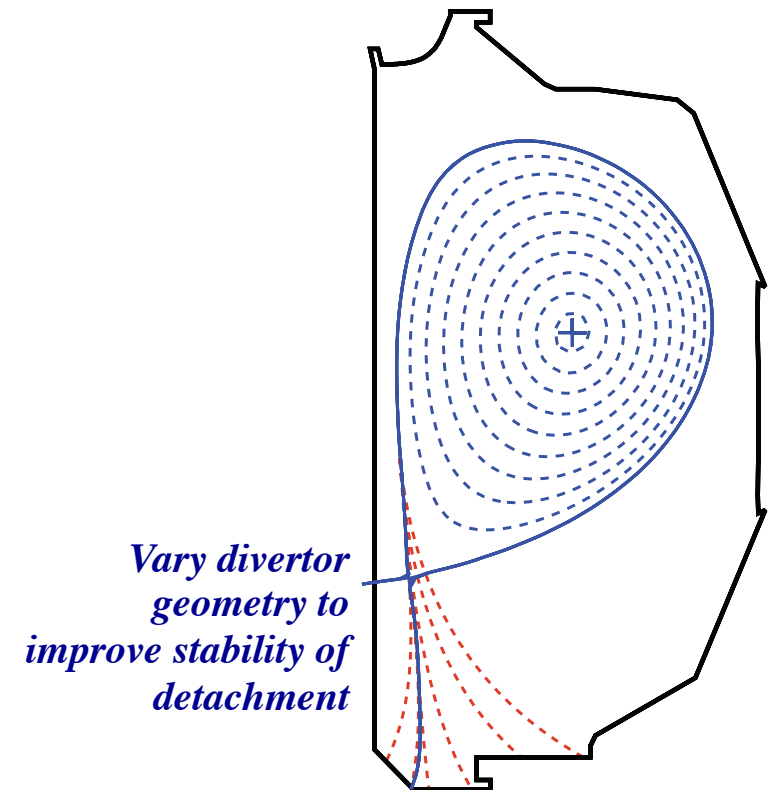


Boundary/Pedestal Physics Opportunities

- **Basic SOL/Erosion Studies (Leonard)**
 - Takes advantage of extensive diagnostic set and ability to expose sample materials in a controlled setting
- **Heat Flux Control/Divertor Geometry Studies (Petrie, Leonard)**
- **Pedestal Structure (Groebner)**

Divertor Experiments in 2012-13 to Focus on Physics of Detachment Onset and Configuration Optimization

- **Self-Consistent Heat and Particle Flux Control Solutions are required for next-step devices**
 - Yet, detachment thresholds, stability & asymmetry not well understood
- **DIII-D Near-Term Emphases:**
 - Determine controlling physics for detachment onset in standard configurations
 - Assess ability to reduce heat flux and improve stability of detachment using advanced configurations



New Diagnostics

Flow: 2D coherence imaging 2011

IR & Visible: Periscope 2012

Neutrals and 2013

2011 JRT Has Provided the Basis for More Coordinated Effort on Understanding Pedestal Structure

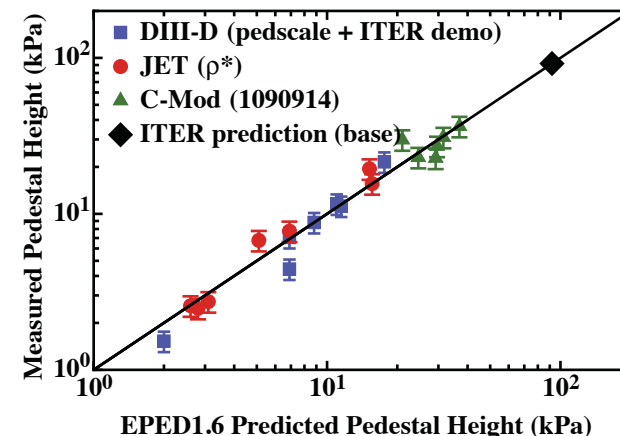
	2011	2012	2013
Diagnostics	Edge Thomson upgrade X-pt SXR Lithium beam upgrade	Line ratio (n_e & T_e)	Neutrals /TALIF

Present status:

- Significant progress made as part of 2011 JRT
- Strong collaboration between NSTX and DIII-D exists now

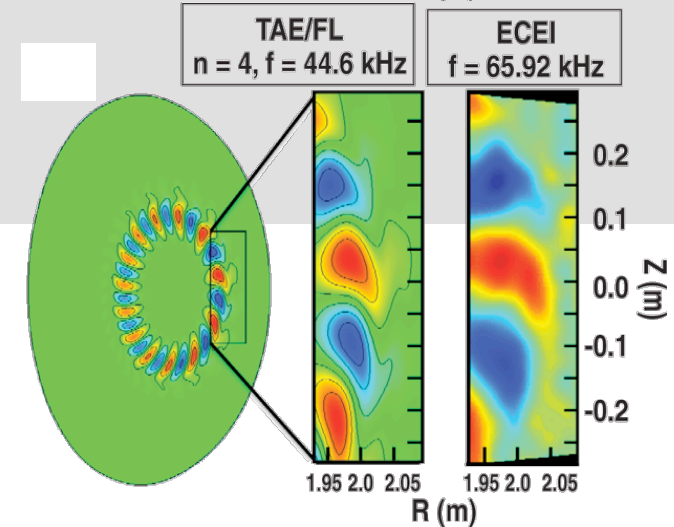
Future potential opportunities:

- Compare evolution of Enhanced Pedestal H-mode on NSTX and VH-mode on DIII -D
 - Identify mechanisms leading to enhanced performance
- Assess impact of 3D fields on pedestal structure
 - RMP, QH-mode, ...



Excellent Set of Diagnostics/Tools Should Provide Basis for Highly Detailed Energetic Particle Studies

	2011	2012	2013
Tools	Off-axis NBI	6 (3) MW FW	
Diagnostics	ECEI FILD-2 FIDA imaging upgrade BES upgrade NPA upgrade	Fast FILD upgrade	



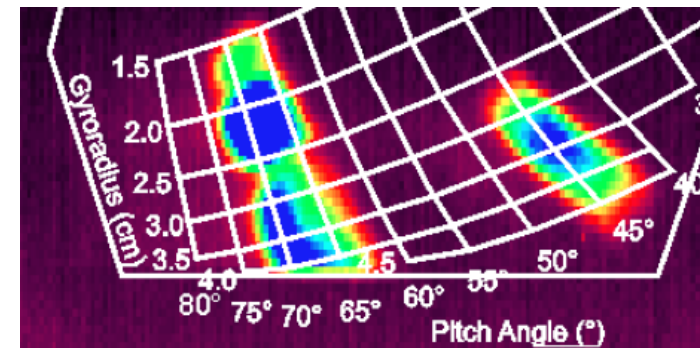
Present status:

- Modes have been observed/identified, and their role on fast ion transport reasonably well understood

Plan:

- 2012.** Interplay between OANB and sawteeth
Electron thermal and EP transport by RSAE/TAE
- 2013.** Super-Alfvénic ions driven by RF tail
AE validation experiments TBD

Energy and Pitch of Loss

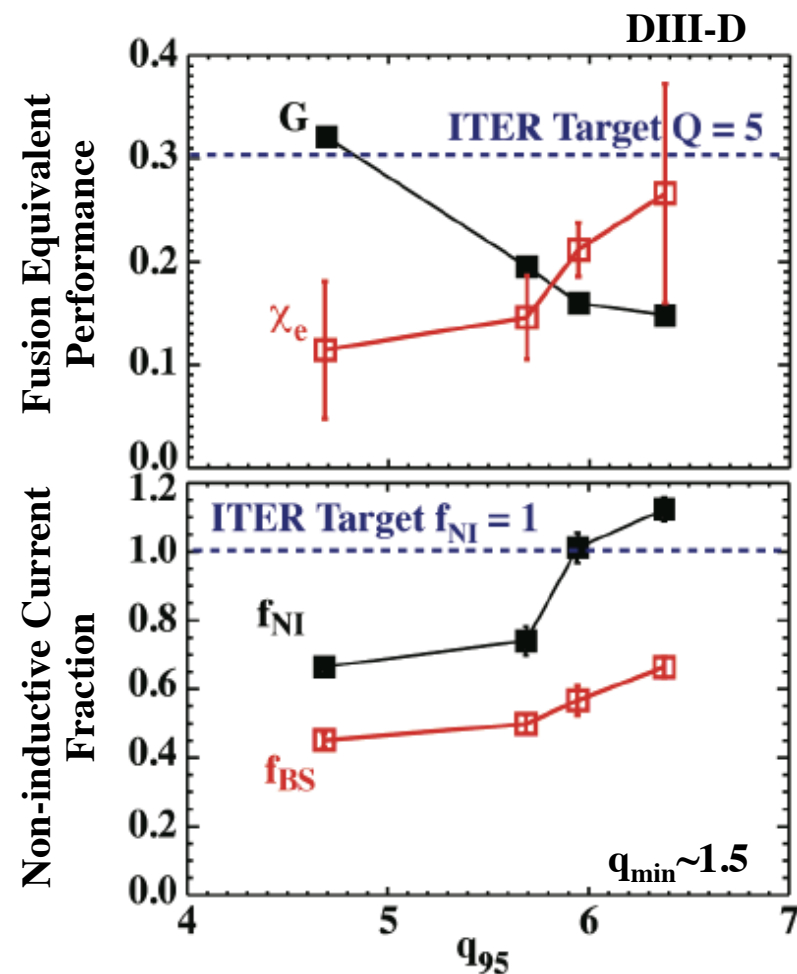


Advanced Scenario Development Opportunities

- **Assessment of ITER Steady-State Scenarios (Luce)**
- **Demonstration of high performance solutions needed for FSNF (Luce)**
- **Transport analysis of relevant regimes (discussed previously)**

Near-Term ITER Focus is Providing Physics Basis for Upgrade/Operational Decisions for Steady-State Ops

- Trade off between fully non-inductive and Q=5 performance goals
 - Observed in DIII-D q_{95} scan →
- DIII-D will identify route to raise performance:
 - Vary current drive deposition (2012)
 - Increased q_{min} radius & value expected to reduce transport by weakening shear
 - Optimization of shape consistent with ITER PF set (2012)
 - Define physics access requirement & develop feedback control (2013)
 - Determine how to deploy ITER's current drive systems & phase II requirements



DIII-D Is In Unique Position to Test Regimes Needed for FNSF and DEMO

- **Goal: Establish operating scenarios and design basis**
 - While FNSF parameters demonstrated in DIII-D, need to understand optimization in terms of performance and machine design
- **Issues:**
 - q profile → Heating systems, RWM control
 - Plasma shape → Coil specifications
 - Requirements for high $\beta_N=5$ (FNSF: $\beta_N=4$) for self-driven (FNSF: fully non-inductive) goals
→ Continuous vs pulsed operation

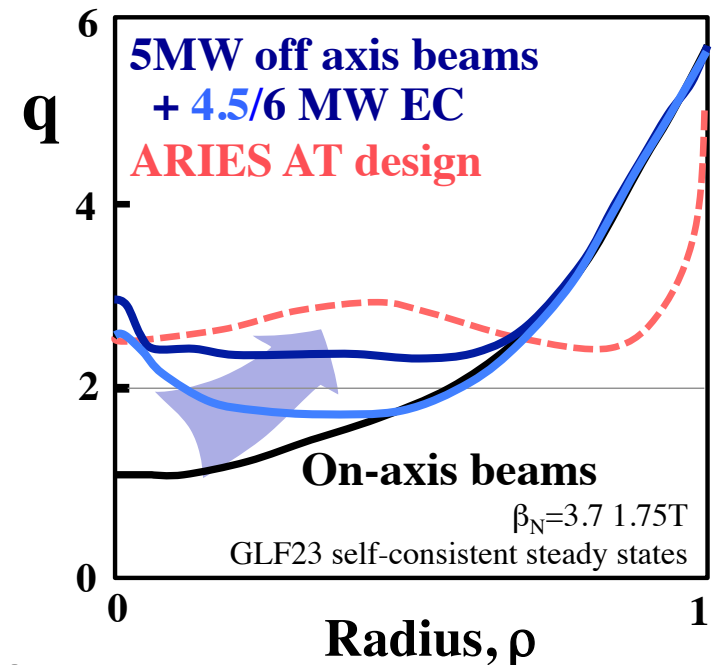
Plan: First explore off-axis current regimes:

2011: Explore physics of $q_{\min} \geq 2$ regimes

- Assess current drive, transport & stability
- q profile tailoring for optimal current drive

Current profile evolution & feedback control

2012-13: Shape optimization



2nd off-axis beam assessment 2012

Other Opportunities to Support ITER

- **Alternate ELM control techniques (JRT 2013) (Fenstermacher)**
- **Improved error field correction understanding (Buttery)**
- **Non-nuclear operating scenarios (Gohil)**

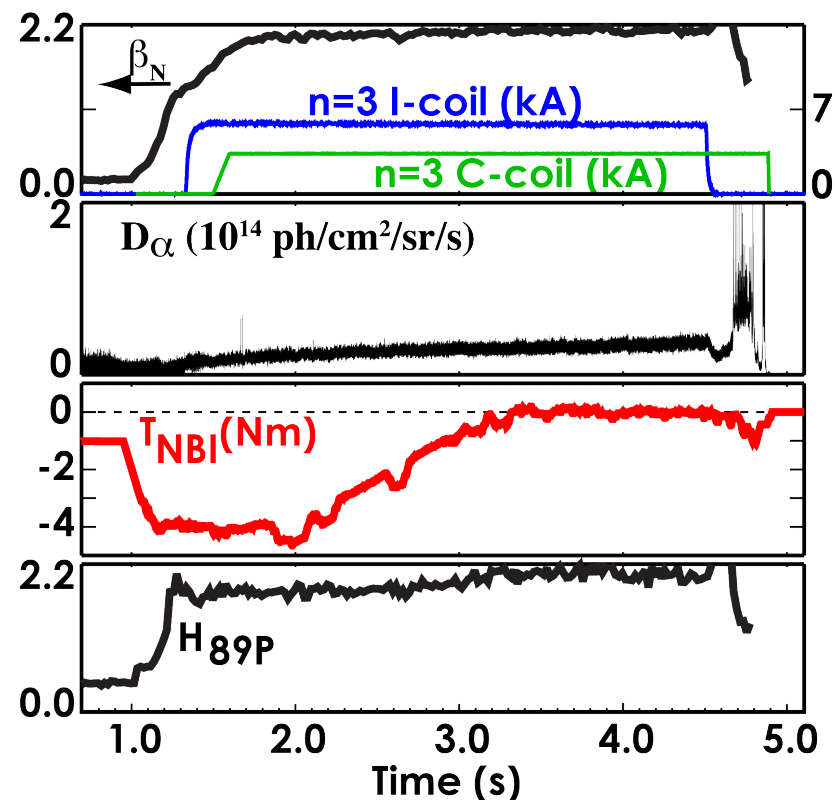
Research Will Focus on Extrapolability of Using Non-Resonant Fields to Access QH-mode in ITER

Present status:

- QH-mode maintained with no NBI torque using non-resonant fields to drive NTB torque

Plan:

2011. Extend operation to small co-NBI torque using full non-axisymmetric coil set
2012. Test theory of NTV-driven shear at edge and assess extrapolability to ITER 180
2013. Demonstrate NTV-driven QH-mode in ITER-like conditions



Provide input for ITER RMP coil decision

New Pellet Injection Capabilities Will Enable Testing Critical Issues for Pellet Pacing in ITER

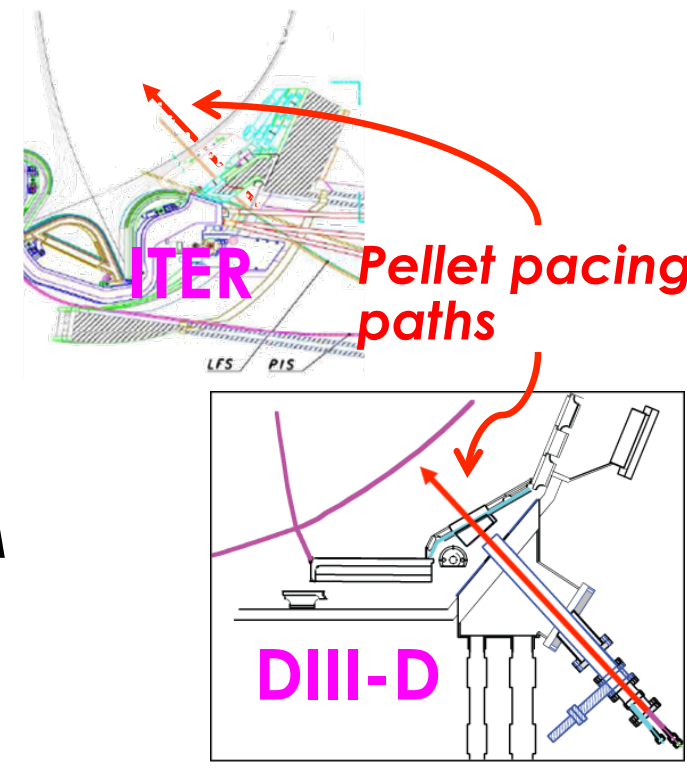
	2011	2012	2013
Tools	30 Hz pellet injector, new guide tubes		>50 Hz LiD injector

Present status:

- 14 Hz pellets → ~25 Hz ELMs
- $\Delta W_{ELM}/W$ decreases by ×3-4

Plan:

- 2011.** Experiments in ITER-like shape with 30Hz pellets
Experiments to combine HFS fueling with LFS ELM pacing
- 2012.** Modify pellet size as necessary to explore minimum required pellet size/penetration
Continue ITER extrapolation experiments **180**
- 2013.** Experiment on high rep-rate pellet ELM pacing



Provide input for ITER RMP coil decision

DIII-D Will Continue to Assess and Qualify Scenarios for the Non-Nuclear Phase of ITER Research Plan

	2011	2012	2013
Tools	Off-axis NBI	6.7 (4.8) MW ECH 6 (3) MW FW	8.2 (6.0) MW ECH
Diagnostics	ECEI Main ion CER CER poloidal rotation upgrade Thomson scattering upgrade	UF-CHERS	

Present status:

- Previous isotope studies focused on L-H threshold

- P_{th} increases by $\sim \times 2$ from D to He
- Energy confinement reduced by $\sim 30\%$

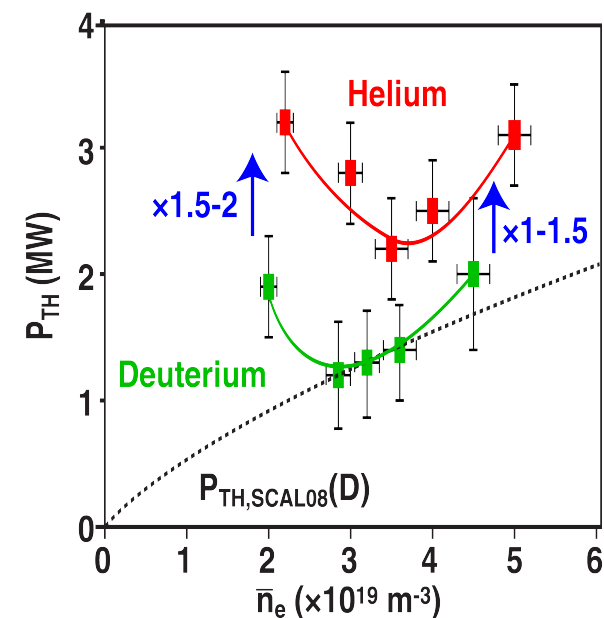
Plan - Study with hydrogen isotopes and helium:

2011. Evaluate existing H and He data

2012. Establish conditions and power requirements for type I ELMs

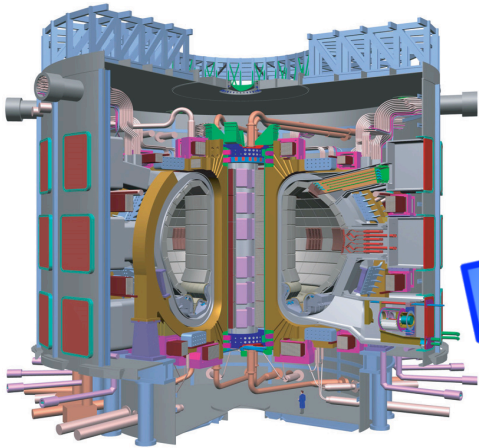
Evaluate τ_e and plasma performance

2013. Demonstrate ELM suppression in He **188(I)**



Utilizing Strengths of Devices and Personnel, U.S. Poised to Make Significant Contributions Towards Fusion Energy

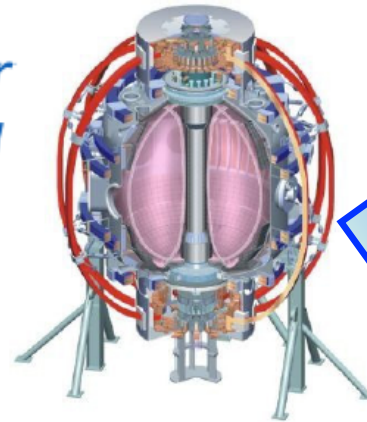
High Energy Gain



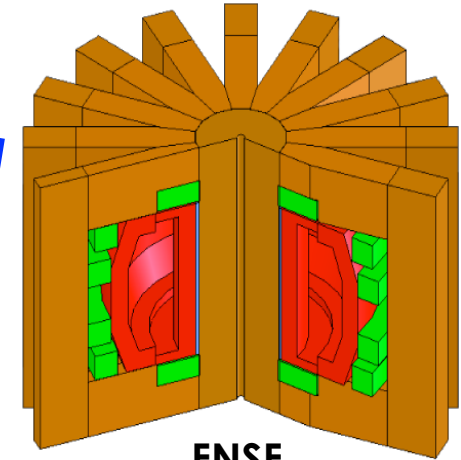
ITER

NSTX

*Alcator
C-Mod*



Steady-State
Nuclear Materials
Fuel Cycle



FNSF

Steady-State

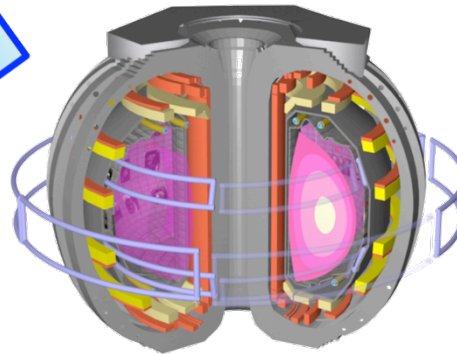


EAST

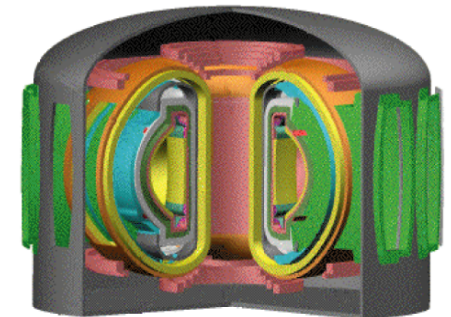


KSTAR

DIII-D



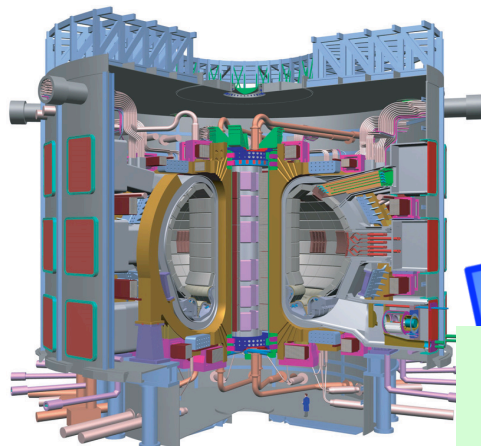
Net Electric



Power Plant

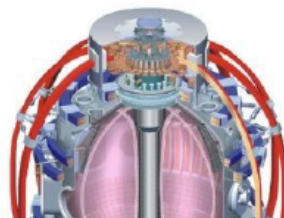
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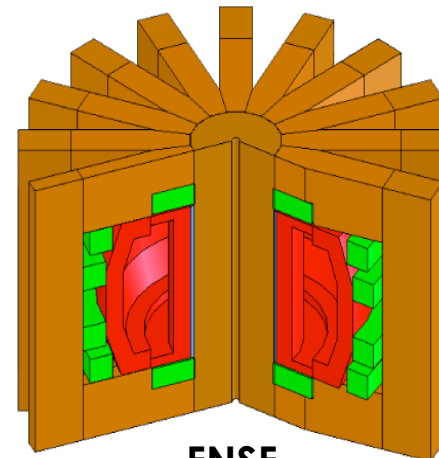
ITER

NSTX



Alcator
C-Mod

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FNSF

DIII-D Welcomes Your Ideas, Ingenuity, and Expertise - the Driving Force Behind This Progress

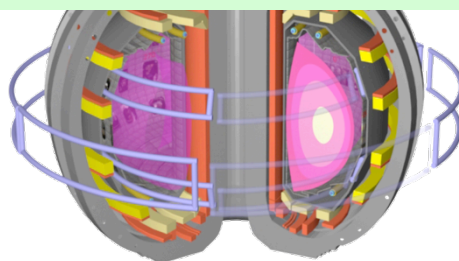
Steady-State



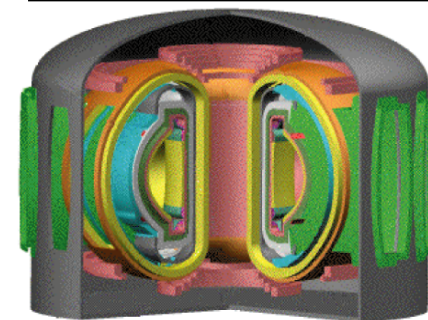
EAST



KSTAR



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