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Macroscopic Stability Physics (MHD) Research - Applied Stability Control Understanding

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Steven A. Sabbagh
Columbia University

For the NSTX Research Team

NSTX PAC Meeting

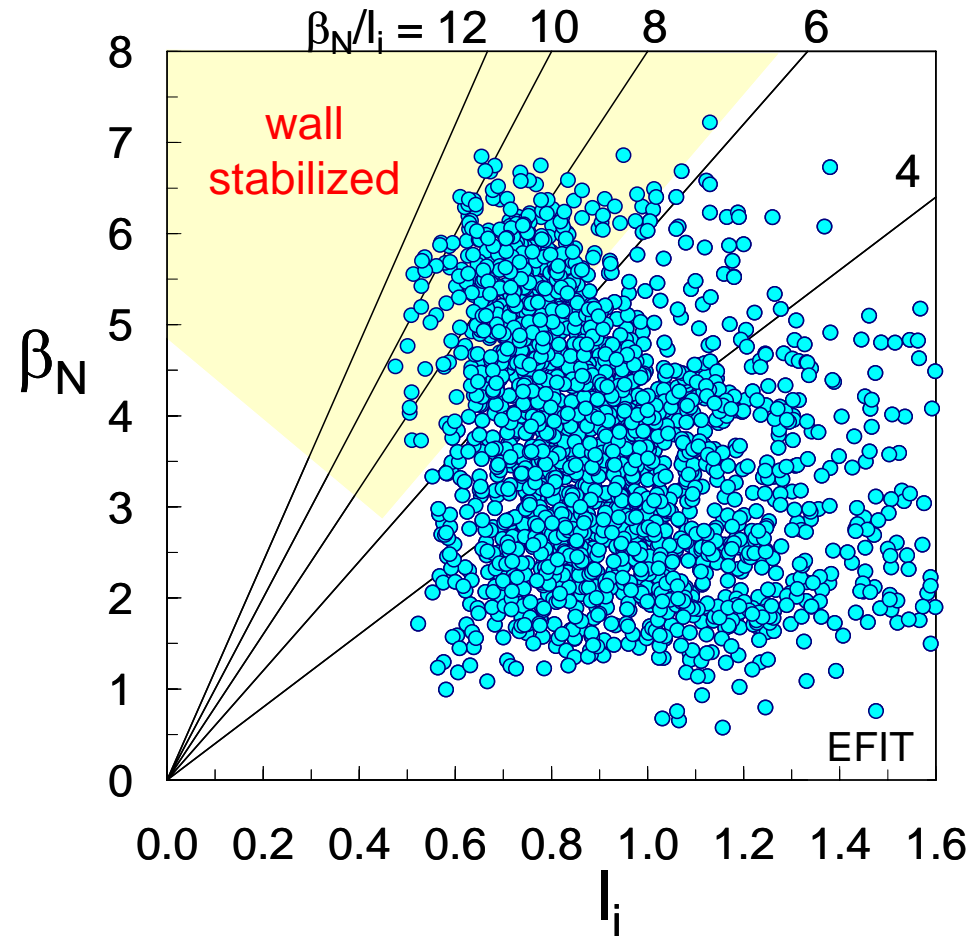
January 23, 2008

Princeton Plasma Physics Laboratory
Princeton, NJ

*Culham Sci Ctr
U St. Andrews
York U
Chubu U
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Advanced tokamak operation demonstrated in a mega-Ampere class spherical torus

- High β operational space
 - Ultra-high $\beta_t = 39\%$, near unity in core
 - Broad current and pressure profiles
 - $\beta_N > 7$, $\beta_N/I_i > 11$
 - Wall-stabilized, $\beta_N/\beta_N^{no-wall} > 50\%$ at highest β_N
- Future research moves to develop predictive capability and control for *steady-state operation near with-wall β limit*
 - Extrapolate to next step device with high confidence



S.A. Sabbagh, et al., *Nucl. Fusion* **46**, 635 (2006).

Continued development of active control tools / fundamental understanding will enable robust extrapolation to next step STs

- ❑ High plasma shaping ($\kappa \sim 3$), low I_i operation
 - ❑ Vertical stability, kink / ballooning mode stability, coupling to passive stabilizers
- ❑ Resistive wall mode (RWM) stabilization
 - ❑ Increase reliability of active control, investigate multimode RWM physics
 - ❑ Understand physics of passive mode stabilization
- ❑ Dynamic error field correction (DEFC)
 - ❑ Reduction of RFA, demonstrate sustained V_ϕ , preferably with V_ϕ profile control
- ❑ Tearing mode / NTM
 - ❑ Stabilization physics at low A , mode locking physics, resonant/non-resonant plasma viscosity
- ❑ Non-axisymmetric field-induced viscosity / plasma rotation control
 - ❑ Non-resonant and resonant, due to RWM, RFA, NTM, error fields, applied 3-D fields for ELM mitigation
 - ❑ Control: Source (2nd NBI, magnetic spin-up) and sink (non-resonant magnetic braking)
- ❑ Mode-induced disruption physics and prediction/avoidance

Stability group has responded to all PAC-21 recommendations

- ❑ PAC21-4a: pursue understanding non-axisymmetric field shielding
 - ❑ Resonant fields: Locked mode study results consistent with Cole theory (APS 07 invited talks). Plasma response needed to understand q scaling.
 - ❑ Non-resonant fields: shielding not needed for quantitative experimental agreement with theory
- ❑ PAC21-4b: encourages NTM studies, but not at expense of RWM
 - ❑ We did both: RWM study and ran two NTM XPs; start of an expanded study of mode seeding and stabilization in 2008
- ❑ PAC21-5: suggests NSTX contribution to ITPA disruption database
 - ❑ Present contributions to be supplemented by new diagnostic data in 2008
- ❑ PAC21-6: pursue optimal configuration / control, push to steady-state
 - ❑ Beta control in '09 – might not close loop in '08 with NBI, rt-MSE pushed out to '11, V_ϕ control pushed out to '11 minimum
- ❑ PAC21-36: RWM control system upgrades operation closer to β_N^{wall}
 - ❑ New “non-axisymmetric control coil” (NCC) planned 2011 upgrade; pursue advanced state-space control algorithm in 2010 timeframe



Access and maintain high β_t and β_N at high shaping

Contributes to: NHTX, ST-CTF development

Present status / issues

- Sustained $\kappa < 2.6$, $\delta < 0.8$; transient $\kappa = 3$ with record shaping factor, $S \equiv q_{95}(I_p/aB_t) = 41$
- Highest κ and S plasmas suffer β collapse at $\beta_N \sim 4 \sim$ computed ideal $\beta_N^{\text{no-wall}}$

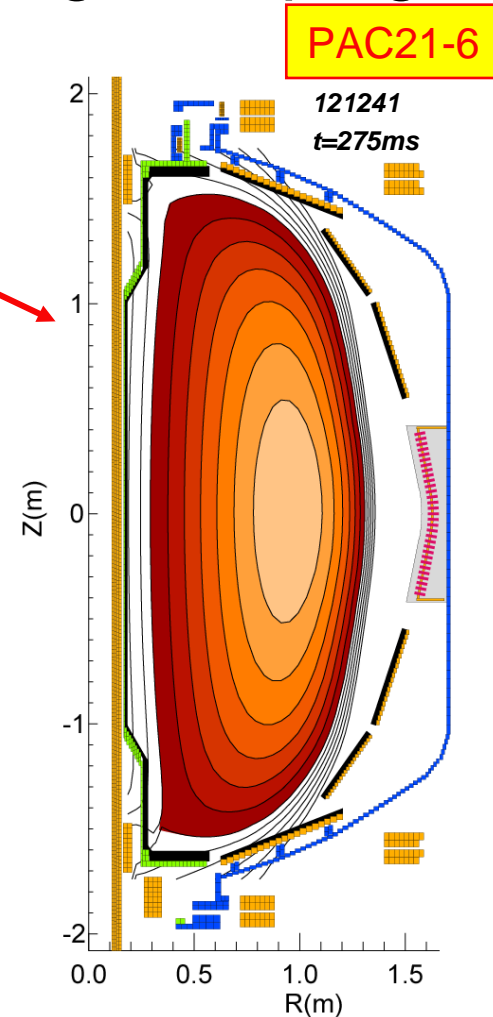
Plan summary 2008-2010

- Operate with upgraded control computer. Test β feedback control using diamagnetic loop sensor and NBI power.
- Research to extend high S plasmas into wall-stabilized, high $\beta_N > 6$ operating space, understand stabilization

Reduction to 2 year program provides only enough time for initial understanding of stability at extreme shaping; eliminates integration with full β and V_ϕ feedback

Plan summary 2011-2013

- Integration of β feedback into real-time EFIT, improve strike point control for Li divertor.
- real-time MSE w/real-time V_ϕ for real-time EFIT, β feedback using stability models; profile control with 2nd NBI



D.A. Gates, et al., *Nucl. Fusion* **47**, 1376 (2007).

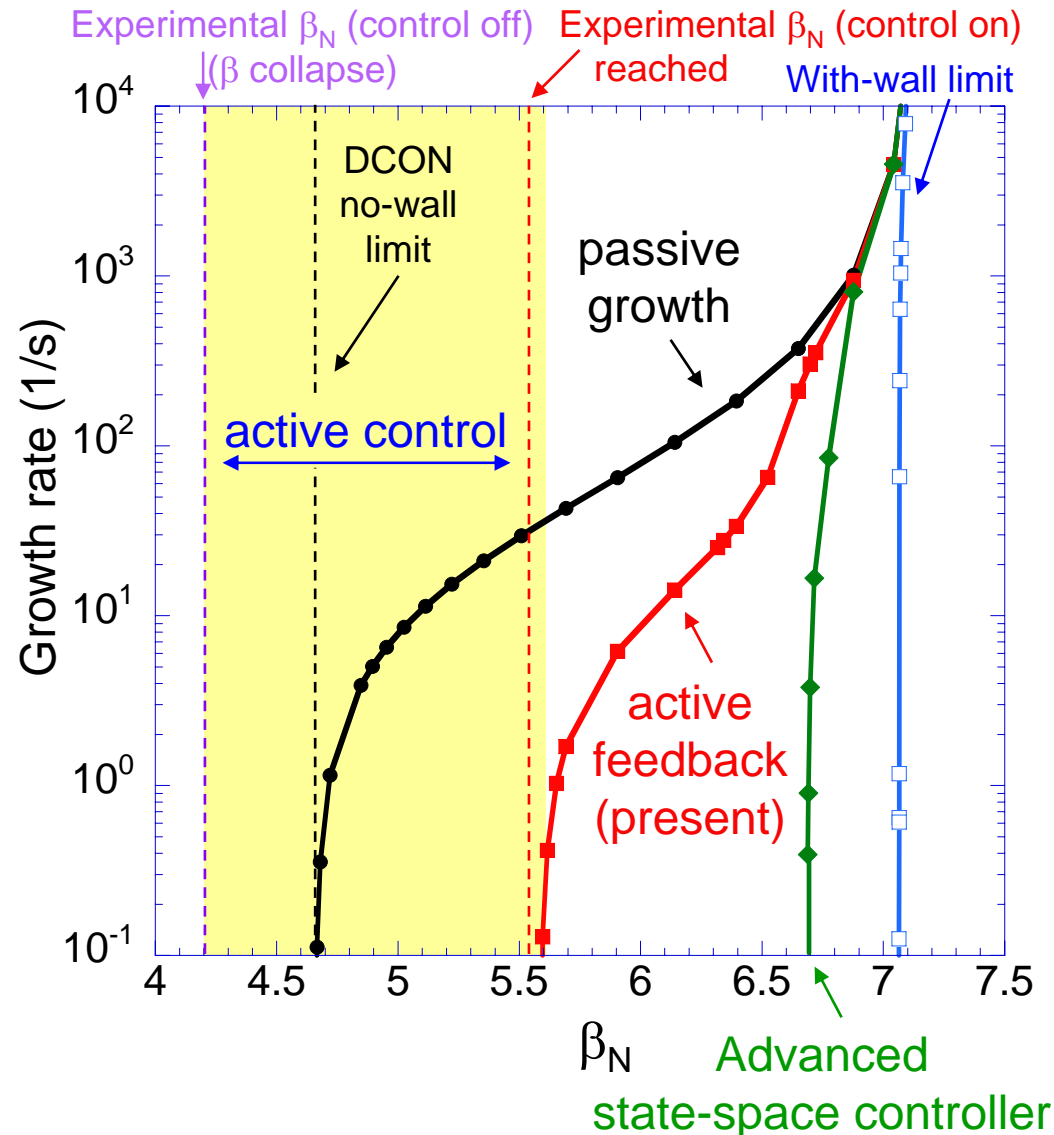
VALEN code reproduces RWM feedback performance

□ New model simulates experiment

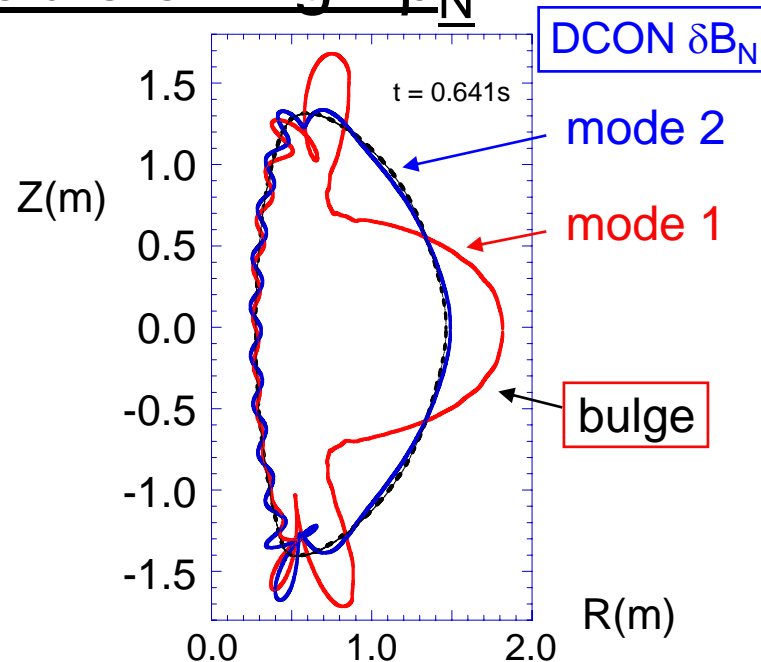
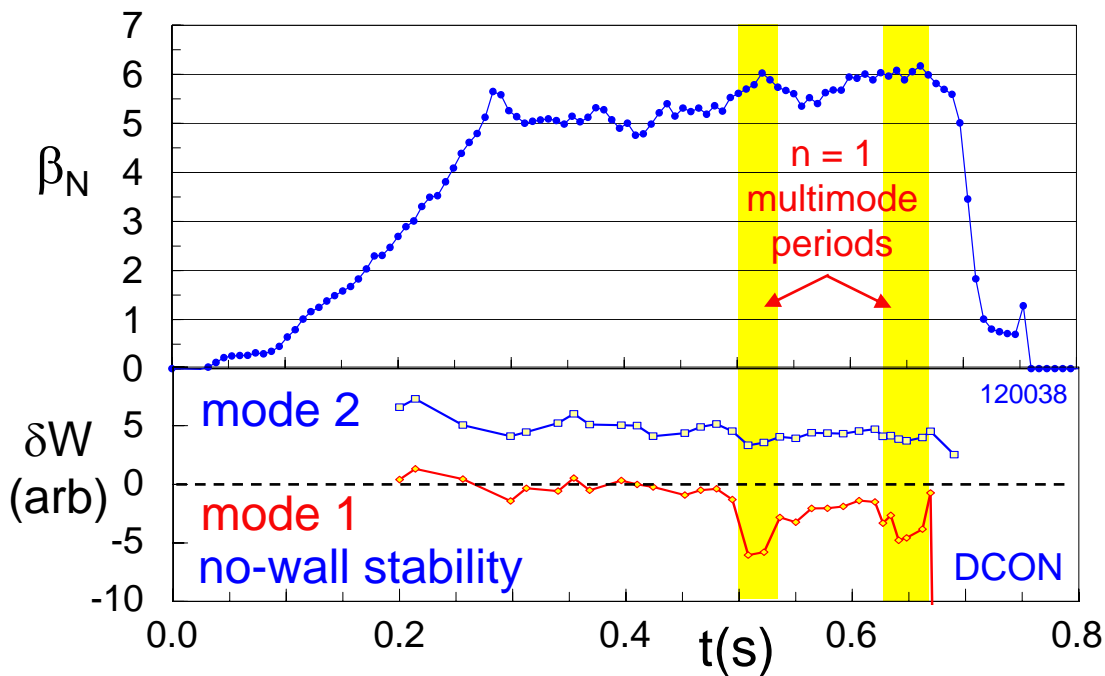
- Upper B_p sensors located as on device
- Compensation of control field from sensors
- Experimental equilibrium reconstruction (including MSE data)
- Proportional gain

□ Advanced control may greatly improve performance

- Advanced state-space controller with B_{pu} sensors may stabilize $\beta_N/\beta_{N,wall} < 95\%$ (benefit to ITER, next-step ST)
- Plan to test offline in 2008, implement for initial RWM control testing 09-10



Multimode theory applicable at high β_N



□ Boozer multimode criterion for $n = 1$ met at high β_N (PoP 10 (2003) 1458.)

□ $|\delta W|$ smallest for 2nd $n = 1$ eigenfunction

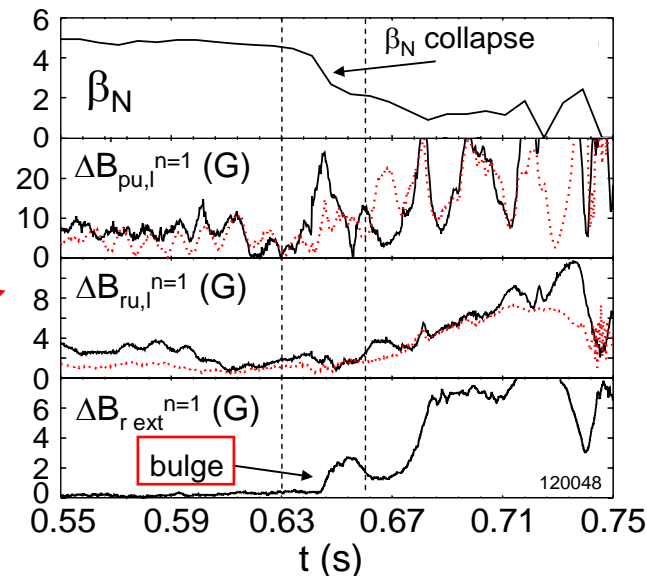
□ Multiple $n > 1$ RWM also observed in NSTX (Sabbagh, et al., Nucl. Fusion 46 (2006) 635.)

□ Multiple $n = 1$ modes may explain observations

□ Upper/lower sensor phases do not always match single mode

□ Poloidal deformation of mode during feedback (mode "non-rigidity")

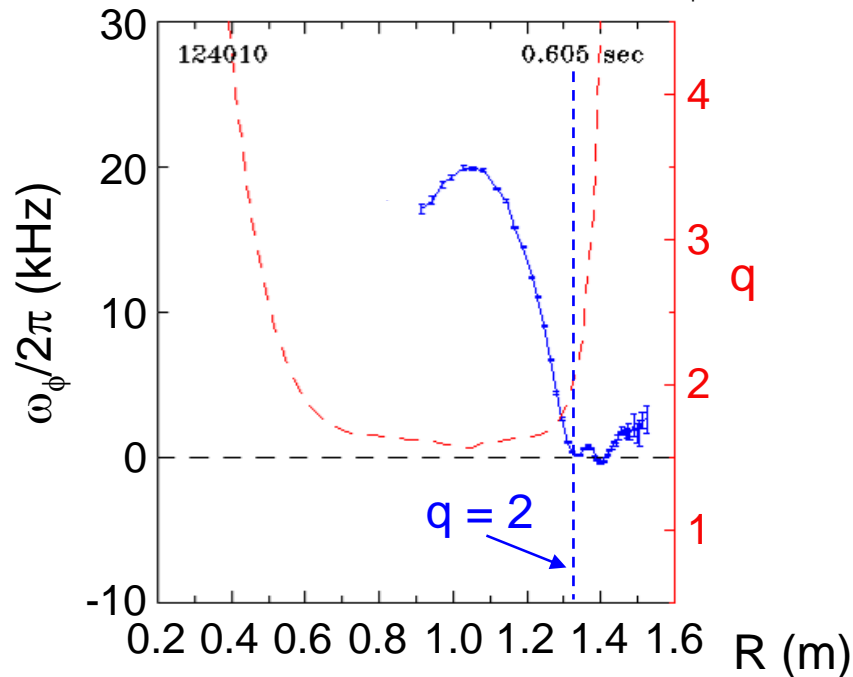
S.A. Sabbagh, et al., PRL 97 (2006) 045004.



Non-resonant magnetic braking allows V_ϕ modification to probe RWM critical rotation and stabilization physics

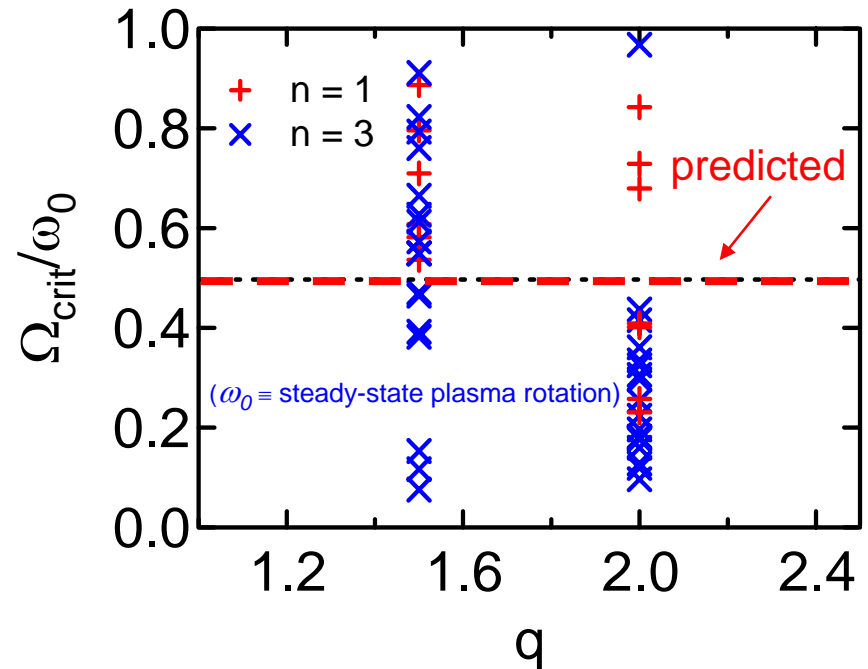
- Scalar plasma rotation at $q = 2$ inadequate to describe marginal stability

Plasma stable $\beta_N > \beta_N^{\text{no-wall}}$, $\omega_\phi^{q=2} = 0$



- Ω_{crit} doesn't follow simple $\omega_\phi/2$ rotation bifurcation relation

A.C. Sontag, et al., NF 47 (2007) 1005.



- Ion collisionality profile variation appears to alter Ω_{crit} profile
- RWM stability at low rotation profile not yet found unless (i) active RWM control applied, (ii) internal rotating mode with $\omega \sim \omega_\phi$ present

Significant progress planned for RWM stabilization research

Contributes to: ITPA MDC-2, USBPO coil design, 2008 Joule, 09-10 NSTX milestones

PAC21-36

Plan summary 2008-2010

- ❑ Determine RWM stabilization requirements for broader range of V_ϕ profiles
- ❑ Test present MHD theories against present profile database, new parameter scans (2009 milestone)
 - Kinetic modification of ideal MHD δW – Hu-Betti-Manickam code (ported locally, setting up test runs now)
 - Compare to latest MARS-F implementation (full kinetic effects modeled - Y. Liu)
- ❑ Continue proportional feedback parameter variation, all RWM sensors used for feedback
- ❑ Test advanced state-space active stabilization algorithms offline; implement and perform initial tests for RWM control
- ❑ Investigate multiple modes in stabilization, implement methods of decreasing possibility of RWM poloidal deformation
 - Multi-mode VALEN code now completed (running NSTX, DIII-D, and HBT-EP test cases now)
- ❑ Design high- n control coil (NCC) – 12 coils toroidally (n/a if research plan limited to 3 years)
- ❑ Implement initial non-magnetic (SXR) RWM sensors, study mode identification

❑ Reduction to 2 year program greatly compromises ability to reach conclusions in these areas, eliminates testing of state-space RWM control; eliminates study of passive stabilization under active V_ϕ profile control

Plan summary 2011-2013

- ❑ Start NCC use for RWM stabilization; $n > 1$ RWM, multimode study during $n = 1$ stabilization
- ❑ Implement real-time mode ID with non-magnetic RWM sensors; use in feedback control
- ❑ Examine greater range of V_ϕ profiles with 2nd NBI, greater non-resonant braking flexibility using NCC
- ❑ Examine RWM stabilization during V_ϕ profile feedback control



Dynamic Error Field Correction used to maintain V_ϕ , high β_N

Contributes to: ITPA MDC-2, 2010 milestone

Plan summary 2008-2010

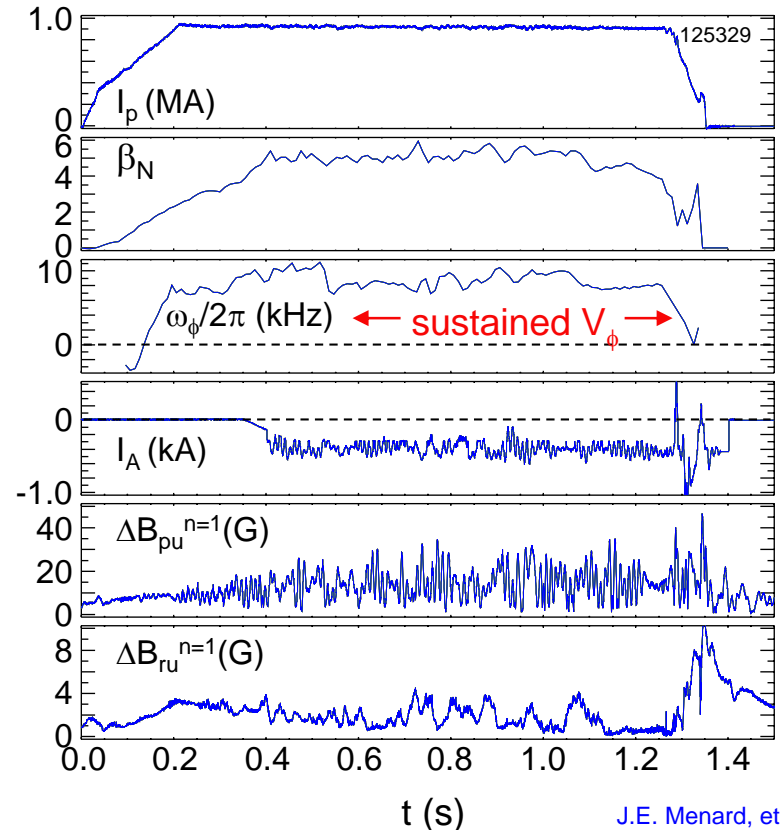
- Investigate sources / physics of dynamic error field, resonant field amplification
- Investigate correction of $n = \text{even}$ fields (primarily $n = 2, 4$, also $n = 6$ with limited scope)
- Feedback on upper and lower arrays of both B_r and B_p sensors

Reduction to 2 year program eliminates research/understanding from improved capability/flexibility of DEFC planned

Plan summary 2011-2013

- Independent control of $n \leq 6$ fields with expanded power supply capability (midplane coil) to sustain plasma rotation and high β_N
- Significantly expanded correction capability (e.g. ability to vary poloidal spectrum of field) using new NCC

$I_p = 0.9$ MA record pulse length
 $n = 1$ feedback (All RWM B_p sensors, optimal phase) and $n = 3$ error correction



J.E. Menard, et al.
APS DPP Invited
Talk (2007)

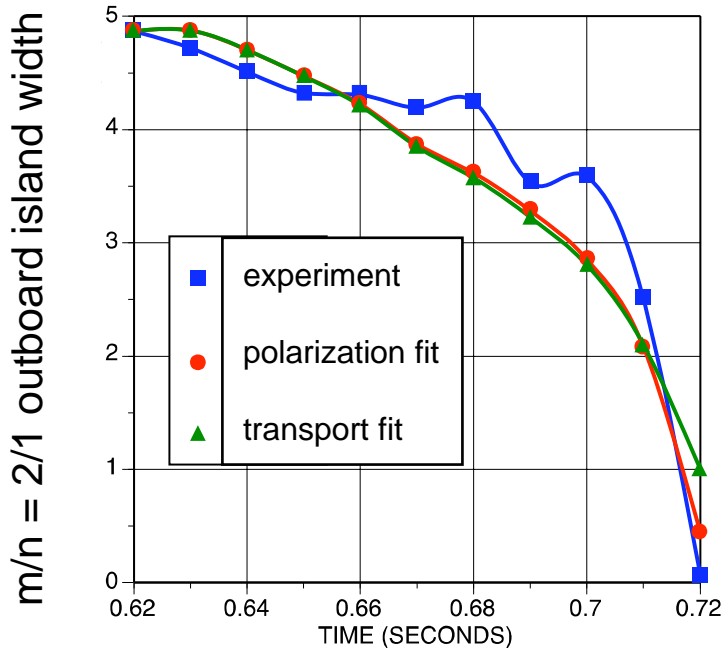


Comparison XPs to investigate A effects on tearing stability

PAC21-4b

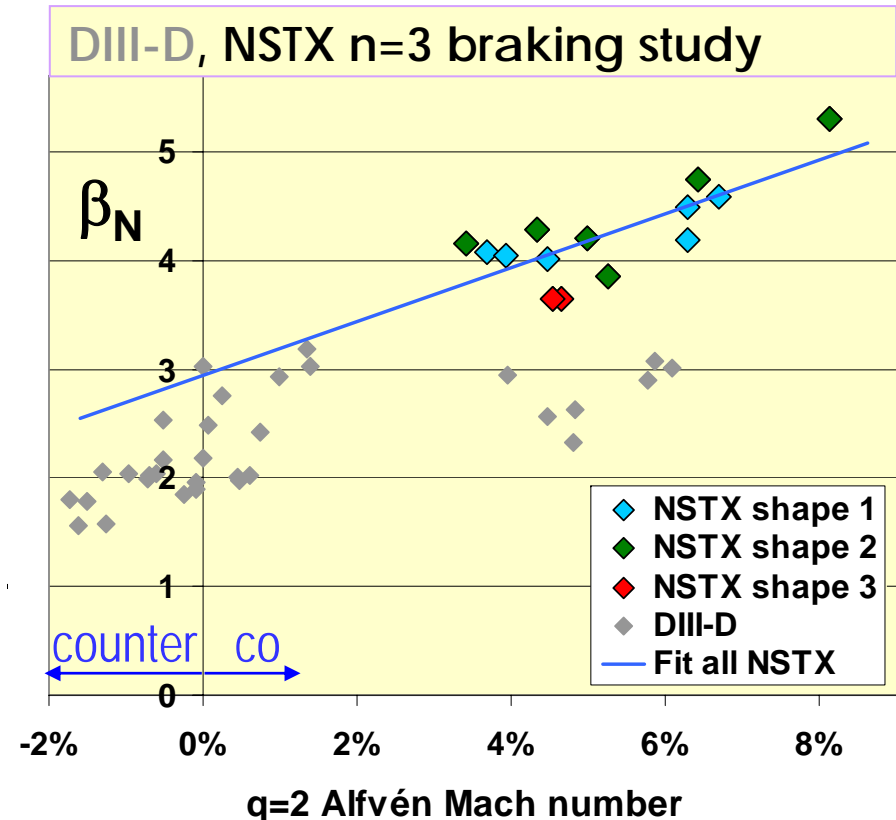
2/1 Marginal island width for stabilization

(R. LaHaye, et al., NSTX XP 739 (2007).)



2/1 onset β_N vs. toroidal rotation

(E.J. Strait, et al., NSTX XP 740 (2007).)



- $W_{\text{marg}}/\epsilon^{0.5}\rho_{\theta i}$ ratio ~ 2 in tokamaks
 - AUG, DIII-D, JET data for 3/2 mode
- First result shows $W_{\text{marg}}/\epsilon^{0.5}\rho_{\theta i}$ also ~ 2 for NSTX (2/1 mode) (!)
 - NSTX $W_{\text{marg}} = 3.5$ cm; DIII-D: 3.1 cm

- Similar % fall in β_N per % Mach
- Further analysis to examine A, V_{ϕ} shear effects



Investigate explicit A dependences on tearing stability

Contributes to: ITPA MDC-3, ITPA MDC-4, 2008 Joule, 2010 NSTX milestone

□ Plan summary 2008-2010

- Compare mode characteristics/stability of sawtooth seeded 2/1, 3/2 modes to spontaneous modes
 - Compare to higher A
 - Determine further seeding mechanisms
- Increase simulation capacity: modified Rutherford equation at low- A
 - PEST3, rDCON, NIMROD, M3D
- Begin NTM onset β_N experiments vs. ρ^* , ν^* , V_ϕ , V_ϕ shear

□ Reduction to 2 year program significantly limits scope of experiments, integrated comparison of theory/experiment

□ Plan summary 2011-2013

- Complete NTM onset β_N study vs. ρ^* , ν^* , V_ϕ , V_ϕ shear; stability near β_N^{wall}
- Comparison of NTM experiments to theory/simulation developed
- Develop discharges that avoid mode, minimize mode impact via profile control (e.g. q , V_ϕ , shear) with 2nd NBI, NCC
- Assess EBWCD results for potential stabilization



Control V_ϕ profile using knowledge of plasma viscosity

PAC21-4a

Significant interest in plasma viscosity due to non-axisymmetric fields

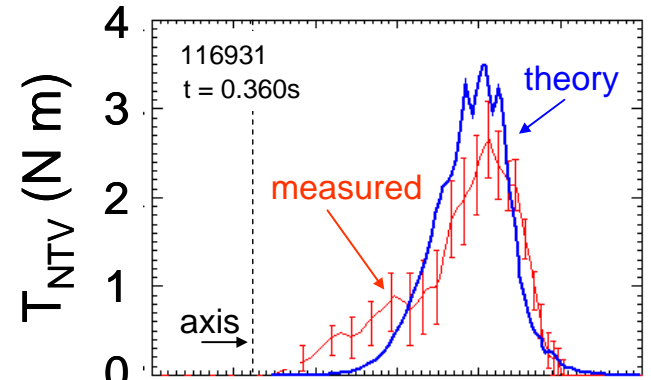
- Physics understanding needed to minimize rotation damping from ELM mitigation fields (ITER, etc.)
- New Neoclassical Toroidal Viscosity (NTV) investigations on DIII-D, JET, C-MOD, MAST, etc. following quantitative agreement on NSTX

Expand present studies on NSTX

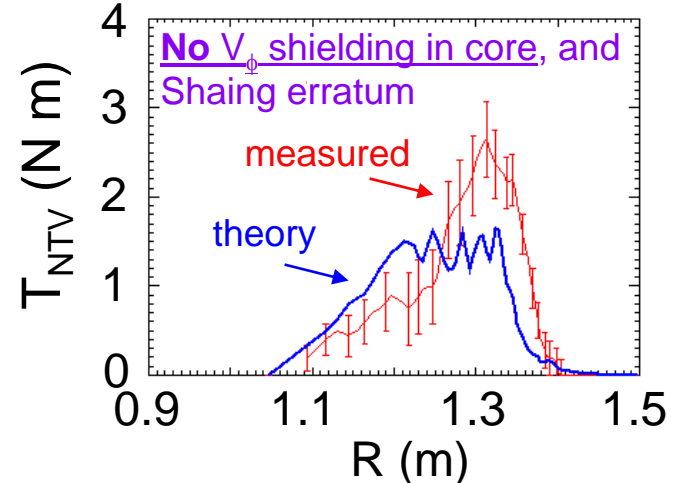
- New capability to examine larger n spectrum (e.g. $n = 2, 4$, limited capability for $n = 6$)
- Consider expansions of NTV theory
 - Saturation due to E_r , multiple trapping states, errata
 - No V_ϕ shielding needed in core when recent Shaing erratum included
- Examine NTV from magnetic islands
- Compare to kinetic modeling using new GTC-Neo upgrade (W. Wang)

Measured $d(I\Omega_p)/dt$ profile and theoretical NTV torque ($n = 3$ field) in NSTX

W. Zhu, et al., *Phys. Rev. Lett.* **96**, 225002 (2006).



New result



Control V_ϕ profile using knowledge of plasma viscosity

Contributes to: ITPA MDC-2, ITPA MDC-12, 2008 Joule, 2010 NSTX milestone

Plan summary 2008-2010

- Continue testing viscosity theory from resonant /non-resonant fields
 - Greater n spectrum (e.g. even parity fields)
 - Focus on key parameters: ν_i , q , β_N , V_ϕ , n ; joint experiments with other devices
 - Influence of magnetic islands (INTV)
- Expand analysis to further test theory
 - Impact of multiple trapping states
 - Time-evolved computations using GTC-Neo; examine saturation at low ν_i ; compare to present theory
- Influence NCC design (eliminated in 3 year termination plan)

□ Reduction to 2 year program limits research to study of NTV and INTV theories with present machine capability - no real-time V_ϕ control demonstration and verification

Plan summary 2011-2013

- 2nd beam line to vary torque at fixed power
- Use NCC to test viscosity theories ($n \leq 6$)
- Real-time V_ϕ control using CHERS sensors, basic sources and sinks of plasma toroidal momentum
- Real-time V_ϕ control using additional momentum sources
- Attempt momentum input with NCC via non-resonant NTV



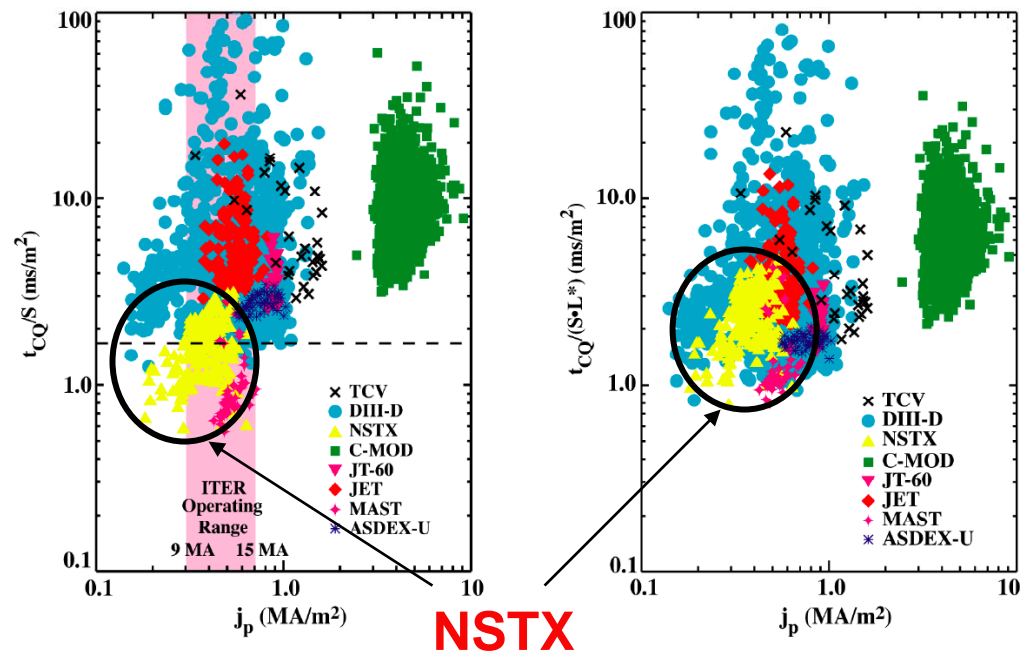
Analyze disruption characteristics, causal modes, avoidance

Contributes to: NHTX, ST-CTF development

PAC21-5

- ❑ Plan summary 2008-2010
 - ❑ Further characterize modes that lead to disruptions, operational boundaries
 - ❑ Current quench rate study (ITPA)
 - ❑ Greater halo current diagnostics in the lower divertor, for magnitude and toroidal peaking studies. Initial IR camera studies.
 - ❑ Studies of heat flux deposition patterns and timing vs. disruption type and discharge parameters (e.g. q_{95} , dr_{sep})
- ❑ Reduction to 2 year program eliminates disruption avoidance via plasma control; might drop entirely
- ❑ Plan summary 2011-2013
 - ❑ Precursor studies for disruption prediction
 - ❑ Detection of impending disruptions based on real-time measurements
 - RFA, stability model based detection

Area-normalized (left), Area and L_{ext} -normalized (right) t_p quench time vs. toroidal J_p (ITER DB)



J.C. Wesley et al., 21st IAEA Fusion Energy Conference (Chengdu, China 2006), paper IT/PI-21.

2008 Run Plan includes specific ITER support XPs

□ ELM Mitigation

Suggested by

- “Joint” ELM mitigation experiment to address “challenge” of demonstrating ELM mitigation with a single row of midplane coils

← USBPO,
ITER
Org.

□ Neoclassical Toroidal Viscosity (NTV) Study

- Several experiments following quantitative agreement on NSTX to best determine impact of non-axisymmetric fields from ELM/RWM coils, error fields on ITER rotation
- Experiments include $n = \text{even}$ fields, v_i variation, INTV

← USBPO,
ITER
Org.

□ RWM stabilization

- Experiment to artificially vary active feedback system latency to simulate the effect of greater time delays due to ITER blanket
- Experiment to simulate ITER port plug RWM stabilization coil geometry by eliminating current in one coil during feedback

← NSTX
PAC

← ITER
Org.



New capabilities planned for 2011 – 2013 research

(ALL Eliminated if operations limited to 3 years)

Capabilities

2nd NBI

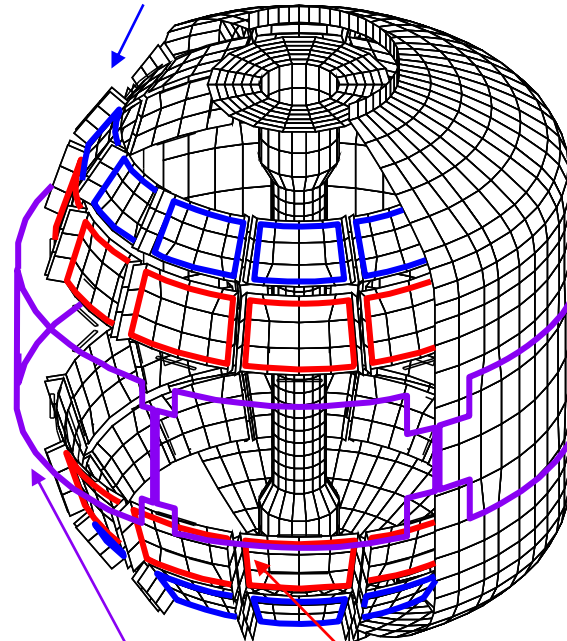
- q profile variation
- momentum source variation

Non-axisymmetric control coil (NCC) – at least four applications

- RWM stabilization ($n > 1$, higher β_N)
- DEFC with greater field correction capability
- ELM mitigation ($n = 6$)
- V_ϕ control increase; $n^\phi > 1$ propagation
- Non-magnetic RWM sensors; advanced RWM active feedback control algorithms (ITER, etc.)
- Alteration of stabilizing plate materials / electrical connections

Proposed Internal Non-axisymmetric Control Coil (NCC) (12 coils toroidally)

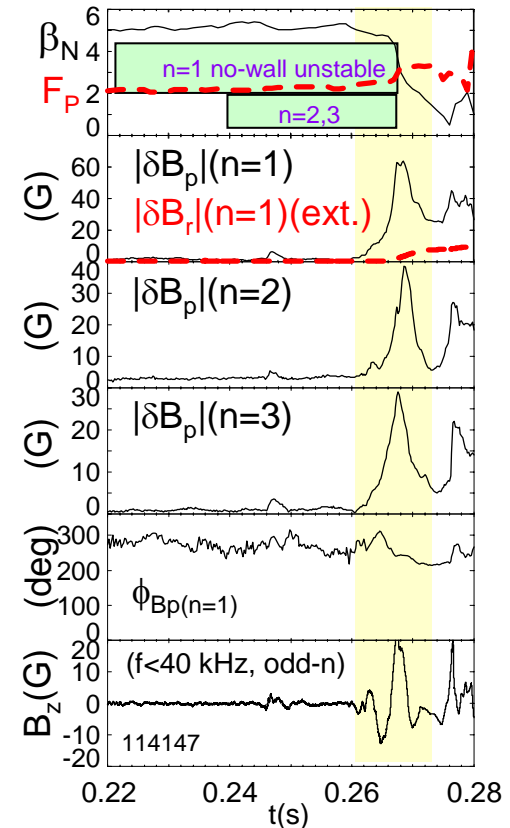
Secondary PP option



Existing coils

Primary PP option

RWM with $n > 1$ RWM observed



(Sabbagh, et al., Nucl. Fusion **46**, 635 (2006).)

U.S. leads the world in high β ST research and is in position to bridge the gap to next-step STs

❑ Macroscopic stability research direction

- ❑ Transition from establishing high beta operation to reliably and predictably sustaining and controlling it – required for next step device

❑ Research provides critical understanding for tokamaks

- ❑ Stability physics understanding applicable to tokamaks including ITER, leveraged by unique low- A , and high β operational regime
- ❑ Specific ITER support tasks on schedule for 2008 run

❑ NSTX provides access to best diagnosed high beta plasmas

- ❑ 2008-2010 Run: allows significant advances in scientific understanding of ST physics, supports ITER, and advances fundamental science
- ❑ Curtailed 2 Year Run: does not allow completion of key scientific investigations; significant loss of control demonstration/research
- ❑ 2011-2013 Run: allows demonstration/understanding of reliable stabilization/profile control - performance basis for next-step STs



Macroscopic Stability Research Timeline (2008-2010)

FY07 08 09 10 11 12 13 14 ^{V1.0}



Physics

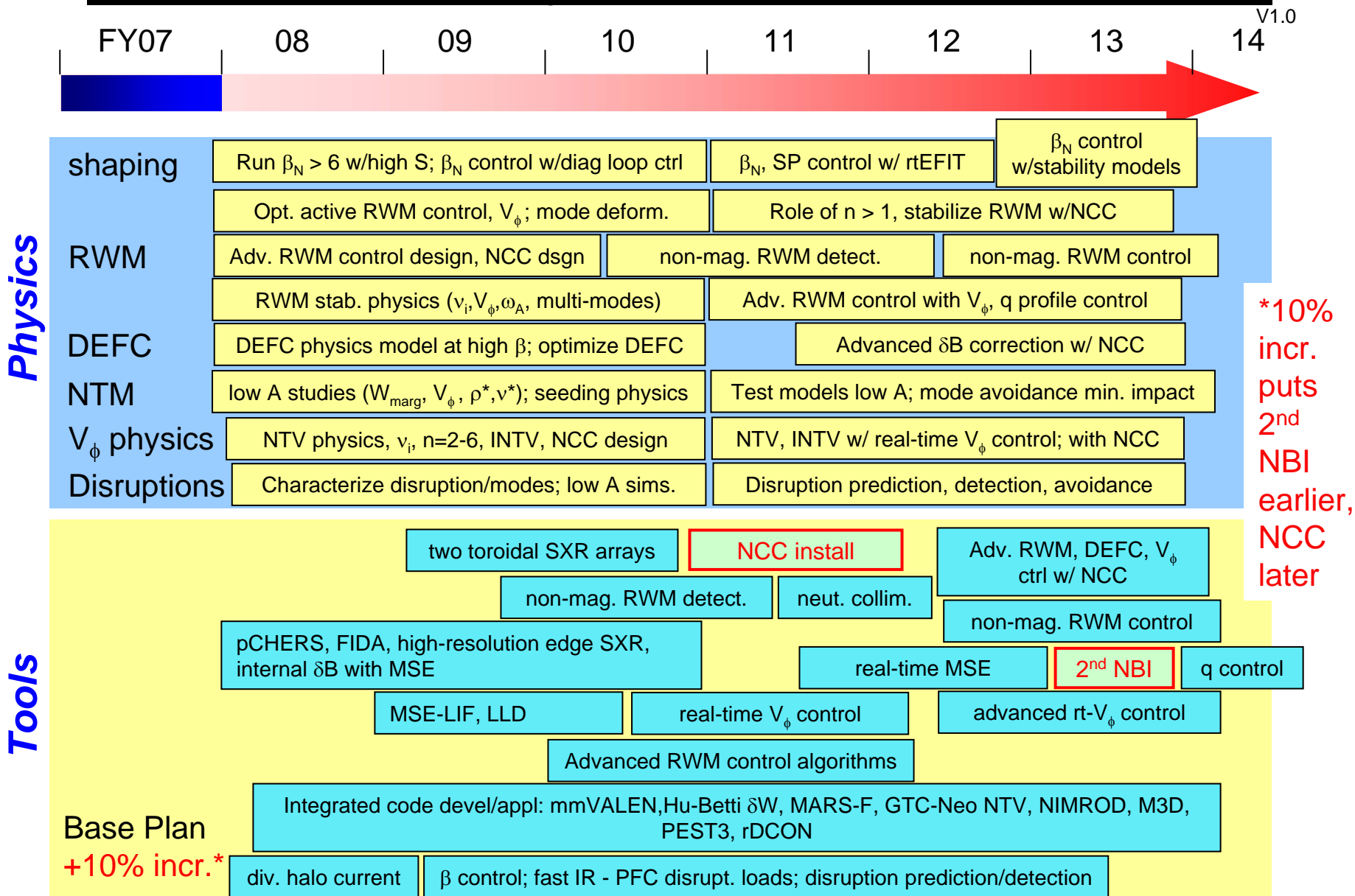
Physics	shaping	Run $\beta_N > 6$ w/high S; β_N control w/diag loop ctrl	
		Opt. active RWM control, V_ϕ ; mode deform.	
	RWM	Adv. RWM control design	non-mag. detect.
		RWM stab. physics (v_i, V_ϕ, ω_A , multi-modes)	
	DEFC	DEFC physics model at high β ; optimize DEFC	
	NTM	low A studies ($W_{\text{marg}}, V_\phi, \rho^*, v^*$); seeding physics	
	V_ϕ physics	NTV physics, v_i , n=2-6, INTV	
Disruptions	Characterize disruption/modes; low A sims.		

Tools

Tools		two toroidal SXR arrays	
		non-mag. RWM det.	
		pCHERS, FIDA, high-resolution edge SXR, internal δB with MSE	rt- MSE
		MSE-LIF, LLD	rt- V_ϕ
		Adv. RWM cntl. alg.	
	Base Plan	Code devel/appl: mmVALEN,,Hu-Betti δW , MARS-F, GTC-Neo NTV, NIMROD, M3D, etc.	
	+10% incr.	div. halo current	β control; fast IR, PFC loads



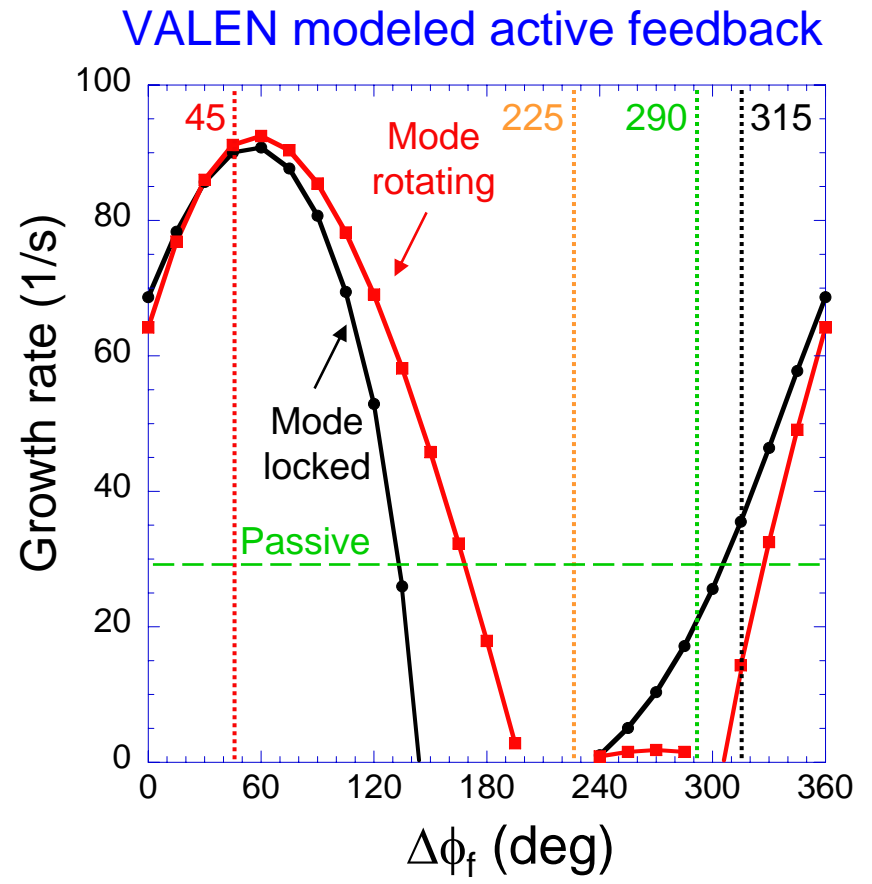
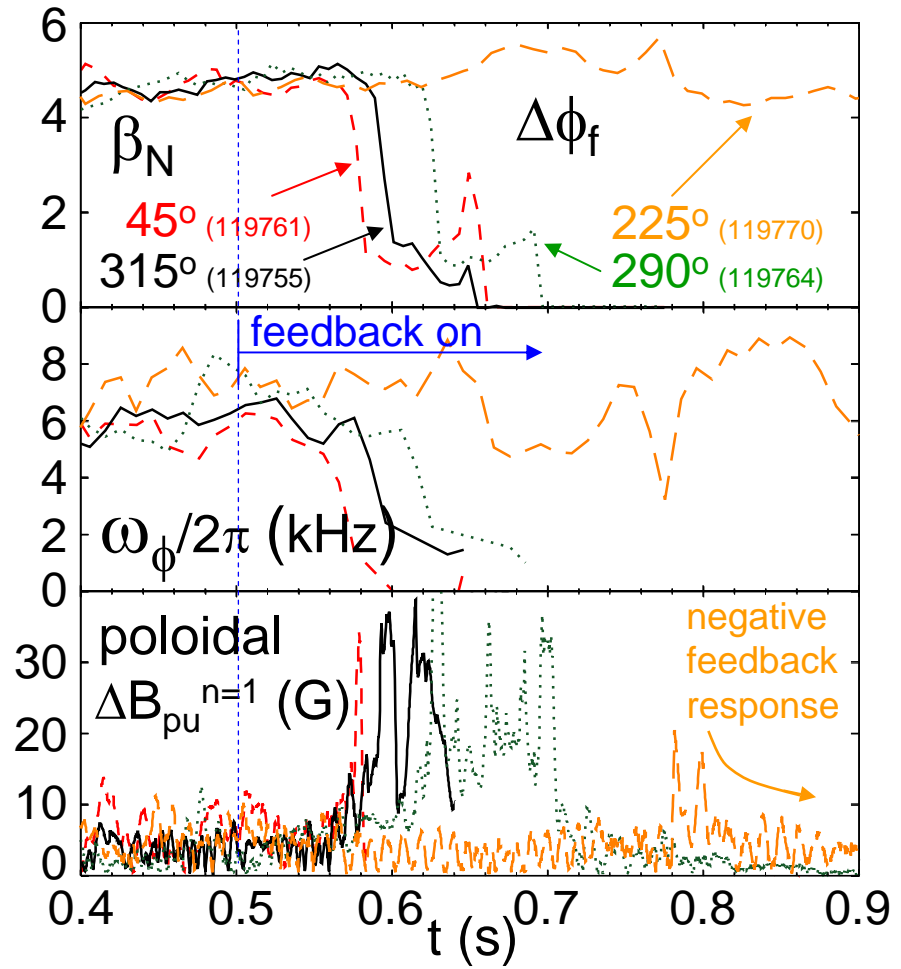
Macroscopic Stability Research Timeline (2008-2013)



Backup slides



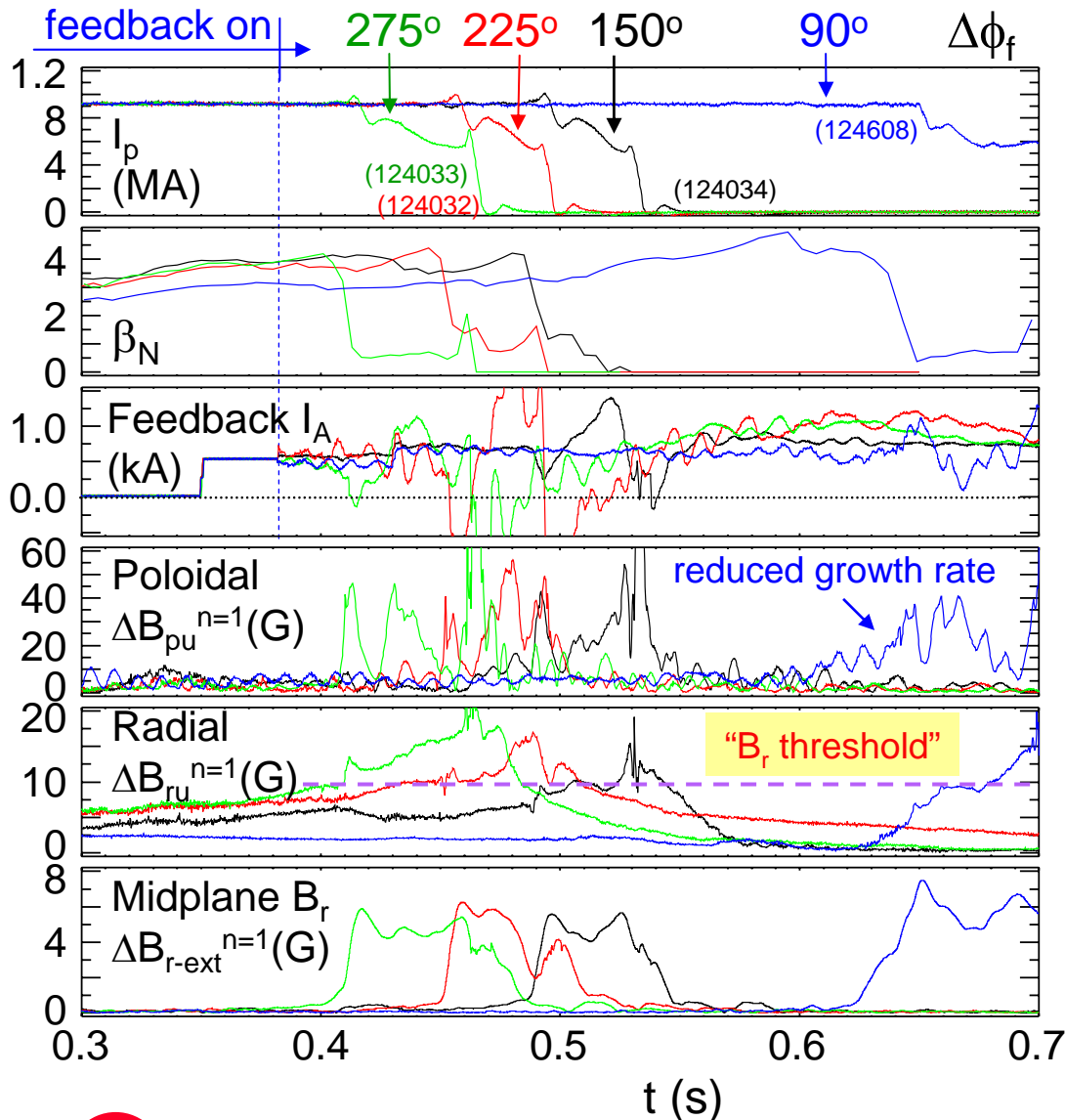
Varying relative phase shows positive/negative feedback



- ❑ Feedback control current has relative phase $\Delta\phi_f$ to measured ΔB_{pu}
 - ❑ Internal plasma mode seen at $\Delta\phi_f = 225^\circ$, damped feedback system response

- ❑ Phase scan shows superior settings for negative feedback
 - ❑ Agreement between theoretical and experimental feedback behavior

Combination of upper/lower B_p sensors reduces RWM growth



Feedback phase scan using B_{pu} and B_{pl}

Best phase shown 90°, not optimal configuration

Reduction in $\Delta B_{pu}^{n=1}$ growth rate

Spatial phase offset between upper/lower B_p sensor flux can improve feedback

Control using B_{pu} and B_{pl} also reduces ΔB_r

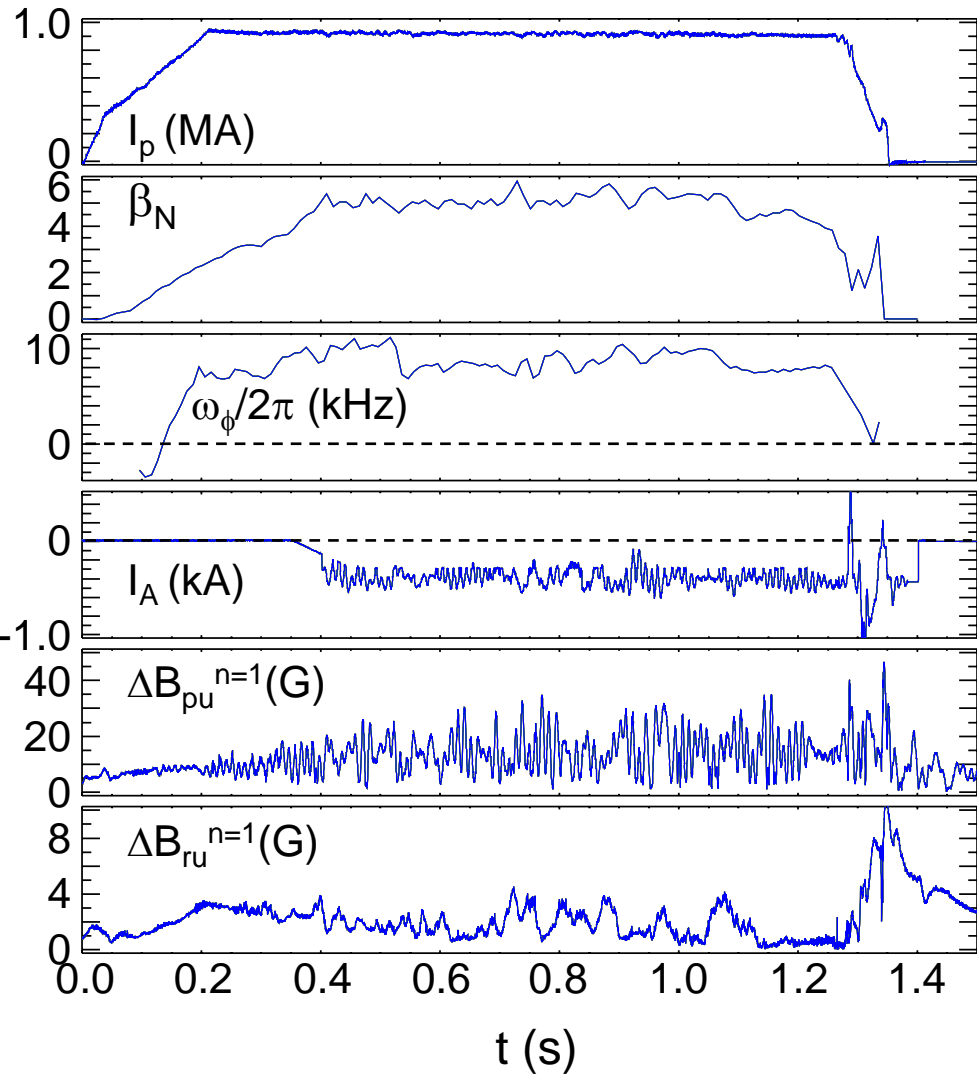
Correlation of β_N collapse and $\Delta B_{ru}^{n=1}$ amplitude

Attempted feedback on ΔB_r - RWM control not reliable

Reduced ΔB_r successfully

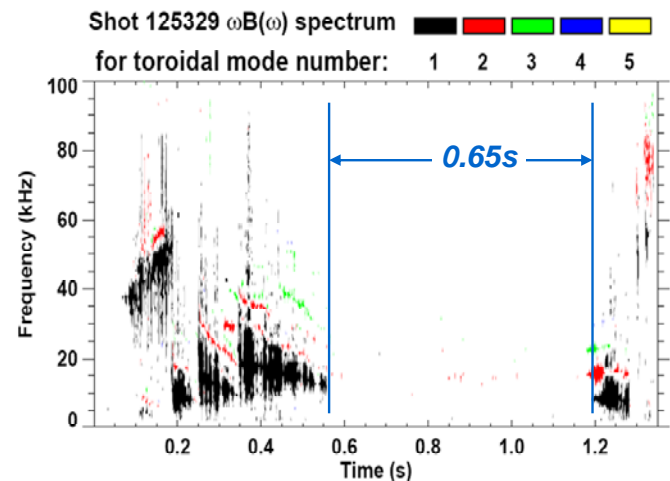
Fast $n = 1, 2$ RWM onset ($\gamma\tau_w \sim 1$) occurs

Feedback control modifications used successfully at moderate ω_ϕ



■ NSTX record pulse length at $I_p = 0.9$ MA

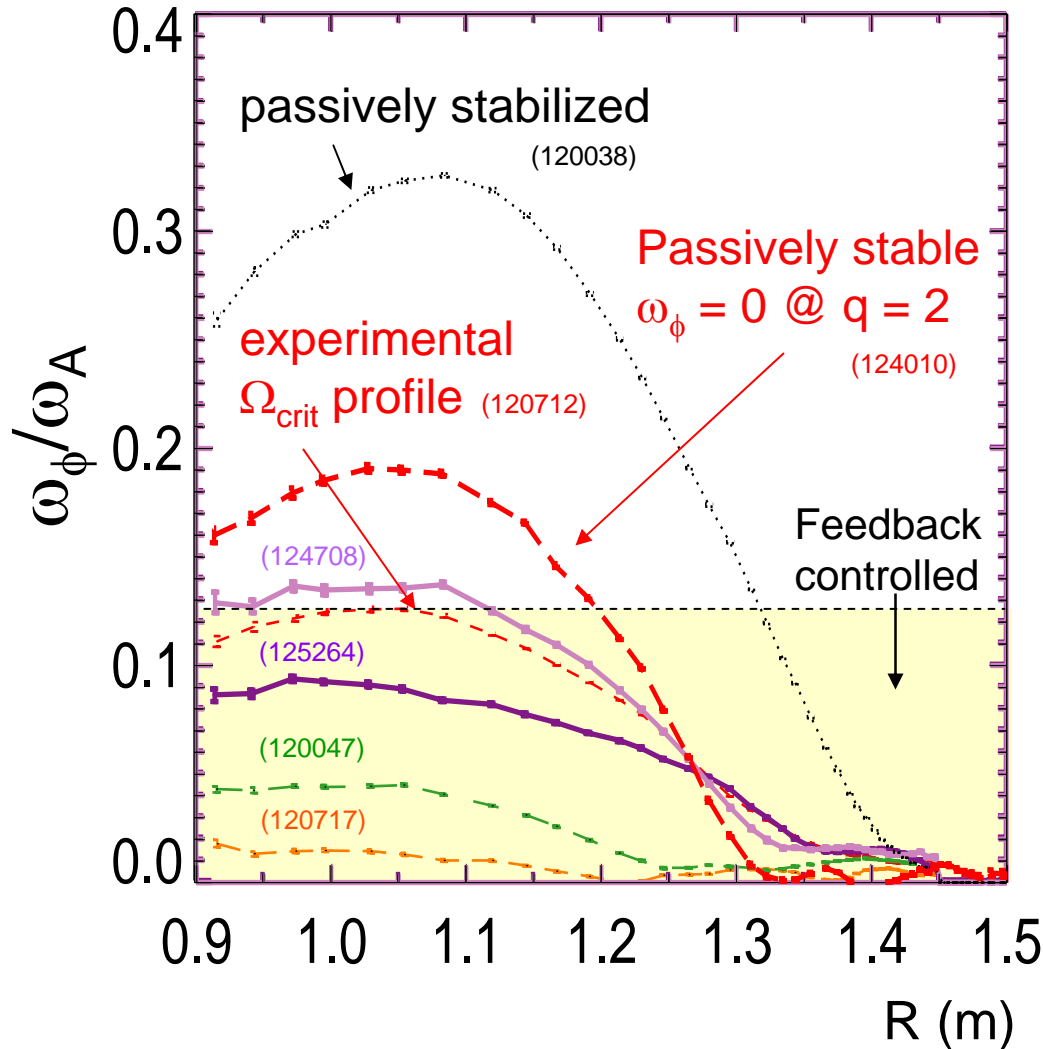
- Feedback used combined upper/lower B_p sensors with spatial phase offset
- Moderate, sustained ω_ϕ keeps ΔB_r in check
- $n = 3$ DC field phased for field correction, maintains ω_ϕ
- $n = 1$ rotation mode suppressed, helps maintain ω_ϕ



J.E. Menard, et al. APS DPP Invited Talk (2007)



Large variation of V_ϕ produced to study RWM passive stabilization physics



- High rotation typically stable, but not always
 - Apparent RWM observed at high core rotation
 - Need to understand instability mechanism

- Stability at intermediate rotation depends on profile
 - Passively stable plasma with $\omega_\phi = 0$ at $q = 2$ has slower edge and faster core rotation

- Compare experimental database to present theory
 - Hu-Betti kinetic δW theory
 - New MARS-F code (full kinetic model)