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Outline

- Role of HHFW in NSTX Program
- Status of HHFW Research & Response to PAC-27
- HHFW Research Plan for FY11-12
- Plans for FY13-14 Outage



HHFW heating & current drive (CD) being developed for non-inductive ramp-up, bulk heating & q(0) control



Near-term approach is to heat low-I_p ohmic plasmas and access 100% non-inductive current drive



January 27, 2011

HHFW double end-fed upgrade was installed in 2009, shifted ground from end to strap center to increase maximum P_{RF}



Designed to bring system voltage limit with plasma (~15 kV) to limit in vacuum (~25 kV):

> Increasing $P_{RF} \sim 2.8$ times

- Antenna upgrade was beneficial:
 - Reached arc-free P_{RF} ~ 4 MW after a few weeks of operation at the end of 2009 campaign
- In 2008-9, Li wall conditioning was observed to enhance HHFW coupling by decreasing edge density

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Increased lithium usage in 2010 significantly degraded performance of the upgraded antenna compared to 2009

- In 2010 RF plasma operations started after extensive Li injection, only reached arc-free $P_{RF} \sim 1.4$ MW; observed copious Li ejections during arcs
 - Before run quickly reached stand-off voltage ~ 25 kV during vacuum conditioning
 - After extensive Li injection difficult to reach ~ 15 kV in vacuum
 - Dust seen during plasmas & inside antenna





- For FY11-12 will return to FY09 Li levels:
 > should be okay for HHFW operations
- Tested prototype ELM/arc discrimination system in 2010:
 - Worked well on bench using recorded data, but tripped undesirably 20-30 times during real shot





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PAC27-28

Progress made in sustaining HHFW heating during I_p =300 kA RF-only H-mode plasma; $T_e(0)$ = 3keV with only 1.4 MW

- Low I_p HHFW experiments in 2005 could not maintain P_{RF} during H-mode
- Produced sustained RF-only H-mode in 2010:
 - Better plasma-antenna gap control than in 2005, due to reduced PCS latency
 - Modeling predicts I_{RFCD} ~ 85 kA, I_{Bootstrap} ~ 100 kA → f_{NI} ~ 60%
 - > High f_{NI} enabled by positive feedback between ITB, high $T_e(0)$ and RF CD
 - > $f_{NI} \sim 100\%$ requires $P_{RF} \sim 3$ MW, well below arc-free P_{RF} available in 2009
 - ➢ No q-profiles for these RF-only plasmas MSE-LIF will enable this in FY11-12



Adding NBI to P_{RF} = 1.4 MW, I_p = 300kA plasma resulted in lower f_{NI} due to RF absorption on fast-ions & higher n_e

- Density increased during HHFW heating probably due to fast-ion interaction with the antenna
- Much lower T_e(0) & higher n_e(0) than RF-only H-mode, resulted in I_{RFCD} ~ 20 kA
- 60% of P_{RF} to electrons, 40% to fast-ions
- ~ 50% of injected NBI fast-ions are promptly lost at this low current:
 - Predict ~ 80% will be confined using more tangential 2nd NBI in NSTX-U
- I_{Bootstrap} = 60-90 kA, I_{NBICD} = 50-70 kA
- f_{NI} ~ 50%

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DNSTX



Self-consistent finite-orbit-width (FOW) ORBIT-RF/AORSA in good agreement with FIDA data for NBI+HHFW plasmas



- HHFW interaction with NBI fast-ion can be significant
- At PAC-27 there was significant disagreement between modeling &
 FIDA fast-ion profiles
- Good agreement between modeling & FIDA has now been achieved by including additional iterations between
 AORSA & FOW ORBIT-RF
- Work has also begun on a full FOW correction to CQL3D which is expected to be completed in late 2011



M. Choi, GA

PAC27-28

3-D AORSA full-wave model with 2-D wall boundary predicts large E_{RF} following magnetic field near top & bottom of NSTX



Edge RF eigenmode looks similar to striated structures imaged by visible plasma TV



- In addition to RF power coupling to core, AORSA predicts some RF power propagates just inside LCFS as an edge localized RF eigenmode
- Beginning to make divertor tile current measurements to compare to theory



HHFW coupling & heating efficiency on NSTX-U

- B_T , I_p and P_{nbi} in NSTX-U will be 2x higher than NSTX, what are the implications for HHFW coupling & heating efficiency on NSTX-U?:
 - Higher B_T in NSTX-U moves fast-wave cut off towards or inside the separatrix, reducing surface wave losses
 - Higher n_e in NSTX-U may increase scrape off density, moving cut off outside separatrix and closer to wall, increasing surface wave losses although SOL width may shrink at higher I_p
 - Higher P_{nbi} in NSTX-U may cause more fast-ion interactions with antenna, requiring larger antenna-plasma gap
 - Larmor radius (and banana width at high I_p) will be smaller, helping to reduce antenna interaction
- Will revisit higher I_p RF+NBI H-mode plasmas in FY11-12, including discharges with P_{nbi} up to 6 MW
- AORSA modeling of NSTX-U scenarios is also planned for FY11-12

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HHFW research plan for FY11-12 – (1)

- Assess "clean" antenna performance in plasma early in FY11-12 campaign
- Operate antenna strap on test stand to increase standoff voltage in vacuum <u>1st Priority:</u>
- HHFW coupling at low I_p & during I_p ramp-up (with SFSU TSG): IOS-5.2 R(12-2)
 - \succ HHFW heating of low I_p plasma
 - Couple HHFW into CHI-initiated plasma
 - Assess I_p ramp-up of an inductively generated discharge to 250-400kA with HHFW + bootstrap current overdrive

2nd Priority:

- Prepare for NSTX-U by running RF+NBI high I_p H-modes at P_{nbi} up to 6 MW: IOS-5.2, TC-9, TC-14
 - Study interaction of fast-ions with antenna and antenna heating
 - Study surface waves at maximum available P_{rf} & P_{nbi}
 - > Study dependence of heating and CD efficiency on k_{\parallel} , outer gap, n_{edge}

ITPA task NSTX milestone



HHFW research plan for FY11-12 – (2)

<u>3rd Priority:</u>

- Many HHFW experiments not started in FY10 due to RF power limitations, if time permits complete these experiments: IOS-5.2, TC-9, TC-14
 - HHFW interactions with ELMs & SOL
 - Fast-ion interactions, rotation effects & RFCD

High Priority Modeling Activities:

- Complete full FOW CQL3D
- Benchmark core HHFW CD in RF+NBI H-mode against advanced RF codes upgraded to include interactions with fast-ions
- Use non-linear gyrokinetics modeling to identify source of transport during HHFW heating and compare to that during NBI heating:
 - Especially effect of varying collisionality (microtearing vs ETG)

New & Upgraded Diagnostics will Aid HHFW Research in FY11-12:

- MSE-LiF will provide q(r) without NBI heating
- Additional MPTS channels will improve RF modeling
- Tangential FIDA will improve the study of RF fast-ion interaction

ITPA task NSTX milestone



HHFW plan FY13-14 outage period

- Analysis and publication of low-I_p results, fast-ion interactions, and divertor heating by RF
- RF modeling NSTX-U plasma scenarios with the extended boundary AORSA-3D and full FOW CQL3D codes
- Implement design changes to antenna to minimize Li deposition inside the antenna box:
 - May install double "closed" shield configuration
- Assess possible change to six-strap antenna configuration:
 - ➤ Full control of spectral peak location (~3 -14 m⁻¹) through arbitrary phasing
 - Simplified, less crowded power distribution system
 - Frees up mid plane port space for possible CAE and/or EHO antenna
 - Would need to increase strap current and maximum system voltage by 40% to obtain same power
 - Run AORSA with six straps to see if there are any unexpected changes

Summary

- Sustained I_p = 300 kA RF-only H-mode with P_{rf} = 1.4 MW achieved f_{NI} ~ 60%; project f_{NI} ~ 100% achievable with P_{RF} ~ 3 MW
- In FY10 Li compounds inside antenna limited arc-free $P_{RF} \le 1.4 \text{ MW}$
- Improved agreement between modeling & FIDA for NBI+HHFW plasmas
- Modeling predicts edge RF eigenmodes may drive significant power flows
- FY11-12 experiments will focus on CHI-initiated, HHFW-heated plasmas and NBI+HHFW H-modes with P_{nbi} up to 6 MW



Backup Slides



NSTX HHFW antenna has well defined spectrum, ideal for studying phase dependence of heating



HHFW antenna extends toroidally 90°





 Phase between adjacent straps easily adjusted between Δφ = 0° to Δφ = 180°



Double-fed HHFW antenna operation improved with time in 2009, but degraded over time in 2010



- Operation spread out evenly over ~ 5 months
- Initial plasma conditioning, then majority of time on experiments
- Most operation in 2-3 MW
 range throughout campaign

- 2009 operation concentrated in last 4 weeks
- Majority of time spent on plasma conditioning
- Operation in 2.5-4 MW range, improved with time
- 2010 operation widely separated over ~4 months
- Almost all time spent on plasma conditioning
- Operation < 3 MW, deteriorated with time

TRANSP-TORIC simulation with no RF coupling losses predicts ~180 kA Bootstrap CD + ~120 kA RFCD





TRANSP-TORIC analysis of matched NBI+HHFW & NBI-only ELMfree H-modes predicts ~ 50% of P_{RF} is absorbed inside LCFS

- Fraction of P_{RF} absorbed within LCFS (f_A) obtained from TRANSP-calculated electron stored energy:
 - W_{eX} from HHFW+NBI H-mode
 - $W_{eR}-$ from matched NBI-only H-mode
 - $W_{eP}-$ using χ_{e} from NBI-only H-mode to predict T_{e} in HHFW+NBI H-mode
- $f_A = (W_{eX} W_{eR}) / (W_{eP} W_{eR}) = 0.53 \pm 0.07$
- TORIC used to calculate the power absorbed by electrons (P_{eP}) assuming 100% RF plasma absorption
- Electron absorption, P_{eA}= f_A × P_{eP} For P_{RF} = 1.9 MW:
 0.7 MW → electrons
 0.3 MW → ions





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CQL3D Fokker-Planck code predicts significant fast-ion losses in HHFW-heated ELM-free NBI H-modes

 Without fast-ion loss CQL3D predicts much higher neutron production rate (S_n) than is measured



• First-order finite-orbit width loss model being implemented in CQL3D



Slow fall of P_{RF} results in sustainment of high $T_e(0)$ and core electron heating even down to $P_{RF} < 1.4$ MW





- Slow transition to H-mode from L-mode as power is ramped to 3.7MW
- During slow ramp down of P_{RF}, the core temperature is maintained and broadened in radius even down to 1.36 MW
- Large ELM at even lower power strongly reduces the stored electron energy and marks the transition back to the L-mode

Divertor tile currents are used to track presence of RF fields (sheath) and driven currents



Tile I3, I4 Tile K3, K4



- Tiles in row 3 and 4 of divertor plate are instrumented with Rogowski sensors
- Bay I and K tiles are in line with "hot" zone for RF edge deposition



Divertor tile currents in row 3 show movement of RF hot zone across tiles as magnetic field pitch is increased



• $\Delta I_{\text{tile 3k}}$ decreases and $\Delta I_{\text{tile 3i}}$ increases as magnetic field pitch increases and RF spiral hot zone moves toward the center stack

Tile currents in row 3 are consistent with RF hot zone movement measured with the fast IR camera at Bay H



 Movement of RF hot zone with magnetic field pitch is relatively fast in the lower pitch range but slows considerably in the higher pitch range

NSTX