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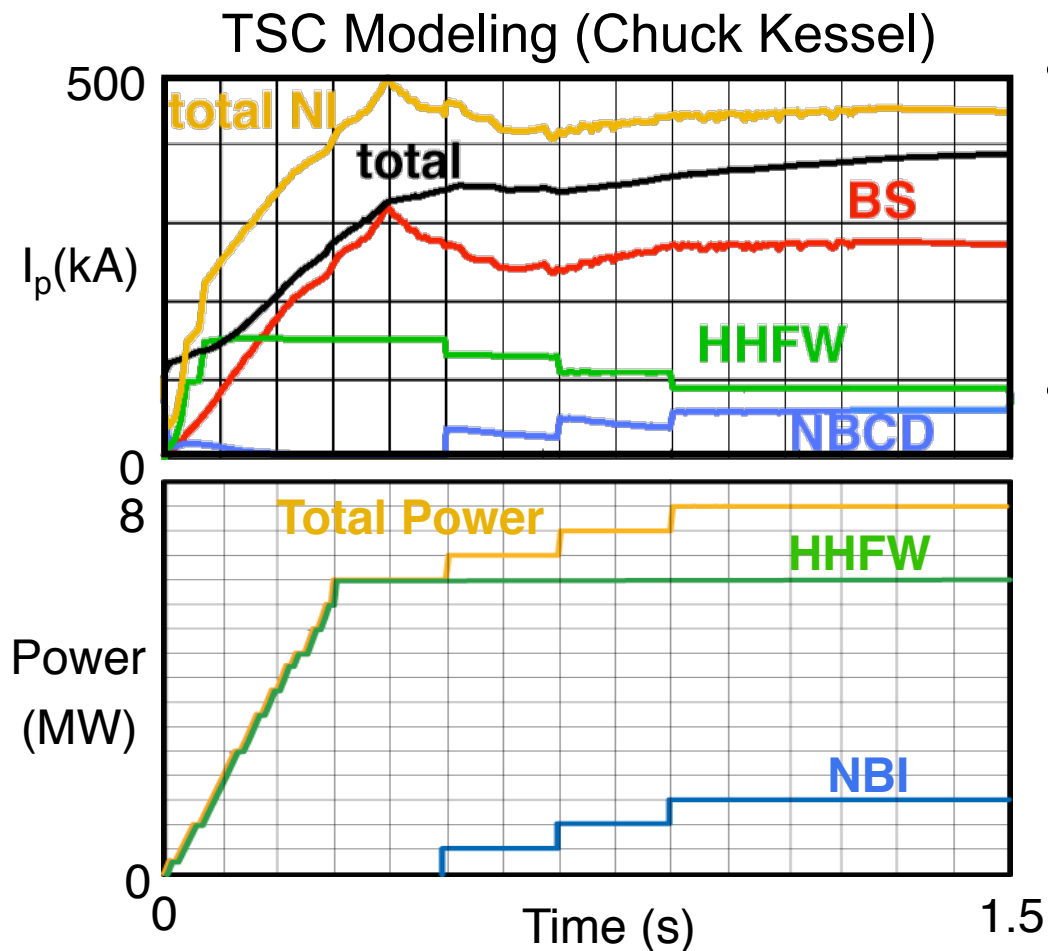
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XP-920: HHFW Heating of Low $T_e(0)$, I_p Plasmas

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Modeling Predicts 5-6 MW of HHFW Power Can Achieve Fully Non-Inductive I_p Ramp-up in NSTX

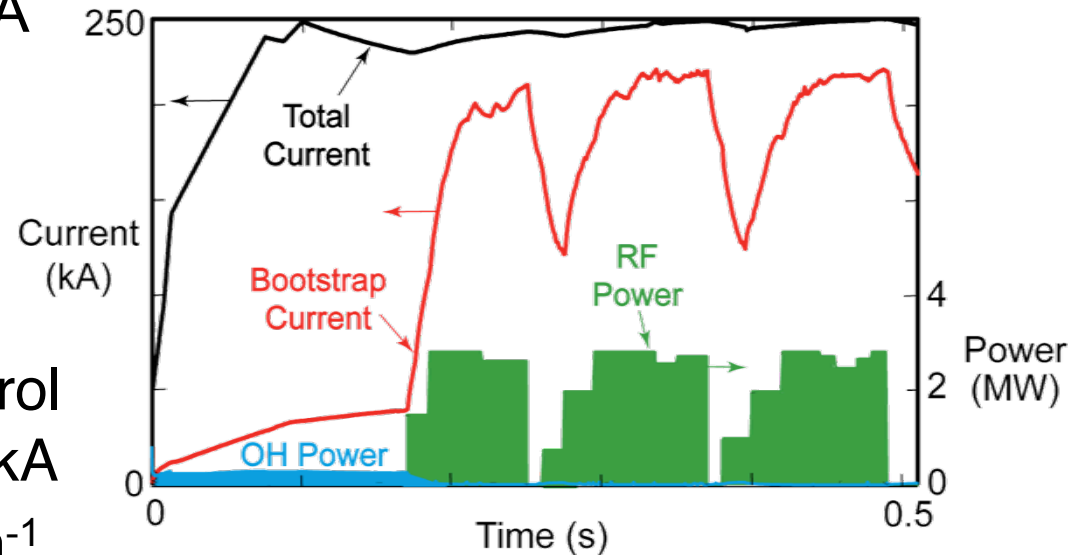


- Simulation of I_p ramp-up at $B_T(0) = 0.45$ T
 - HHFW provides heating & CD at low I_p and T_e
- HHFW-assisted I_p ramp-up started at 100 kA
 - 6 MW HHFW ($k_{||} = 8$ m⁻¹) Co-CD phasing
 - 6 MW NBI added when $I_p \geq 400$ kA (only 2-3 MW absorbed due to slow I_p ramp rate in 1.8 s plasmas)

HHFW Heating of Low I_p Plasmas Since 2005 Show Promise, But Also Problems with Plasma Control

2005: (XP-521)

- 85% bootstrap current in HHFW heated ($k_\phi = -14 \text{ m}^{-1}$) H-mode D_2 plasmas at $I_p = 250 \text{ kA}$
- Transiently produced $V_{\text{loop}} \leq 0$ and $dI_{\text{OH}}/dt \approx 0$



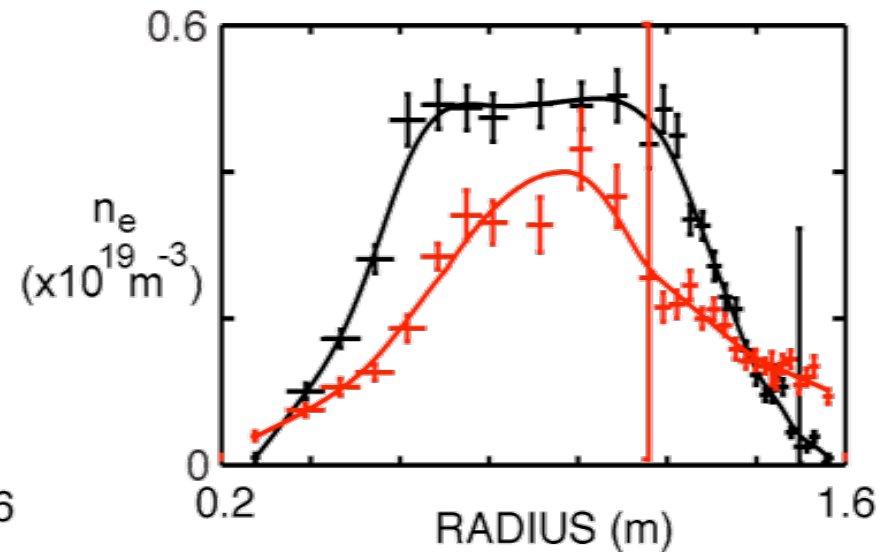
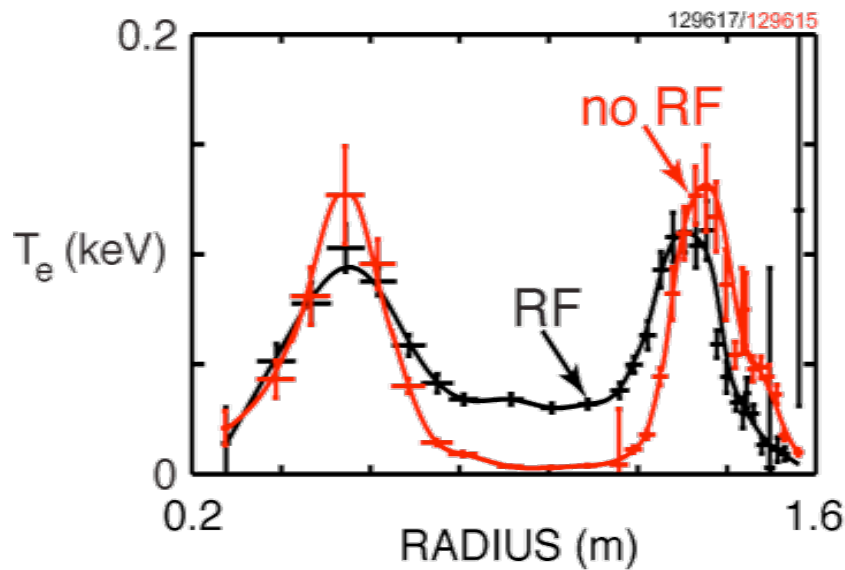
2007: (XP-731)

- Problem with rtEFIT control at $I_p = 250 \text{ kA}$, used 300 kA
- Many trips with $k_\phi = 14 \text{ m}^{-1}$
- Up to 2.7 MW of $k_\phi = -8 \text{ m}^{-1}$ heating, produced transient H-mode

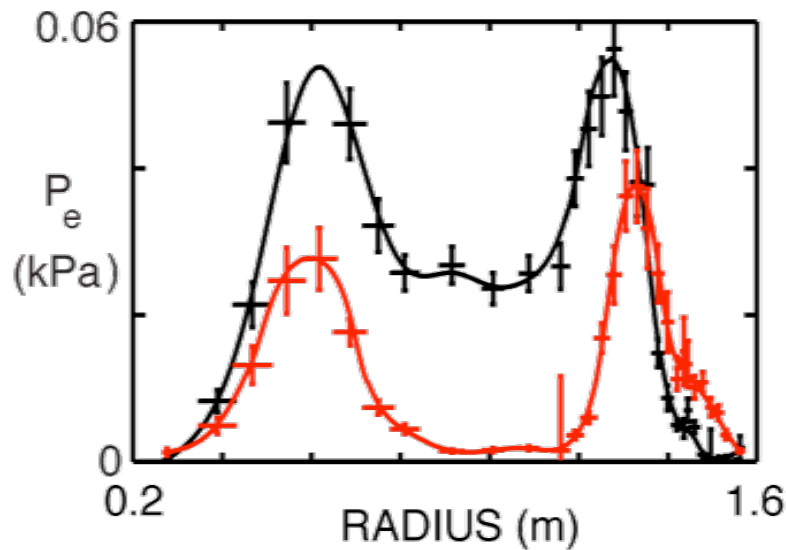
2008: (XP-817)

- Li conditioning reduced edge density, improving HHFW core heating, even in CHI start-up plasmas with $n_e(0) \sim 4 \times 10^{18} \text{ m}^{-3}$

XP-817: 550 kW $k_\phi = -8 \text{ m}^{-1}$ HHFW Heating 18 to 64 ms Increased $T_e(0)$ from ~ 3 to $\sim 33 \text{ eV}$

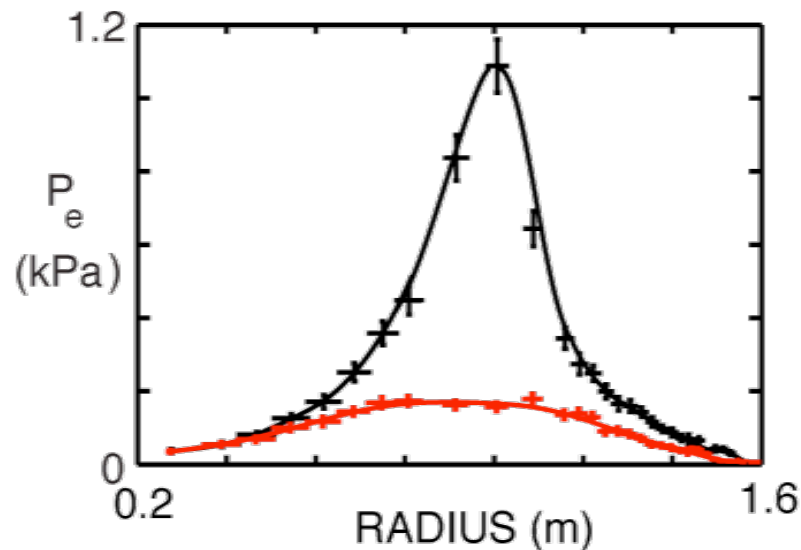
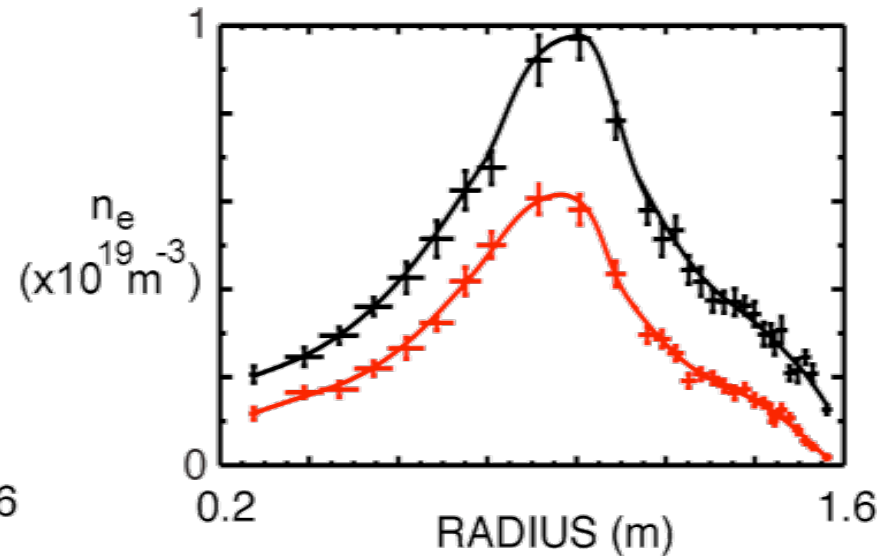
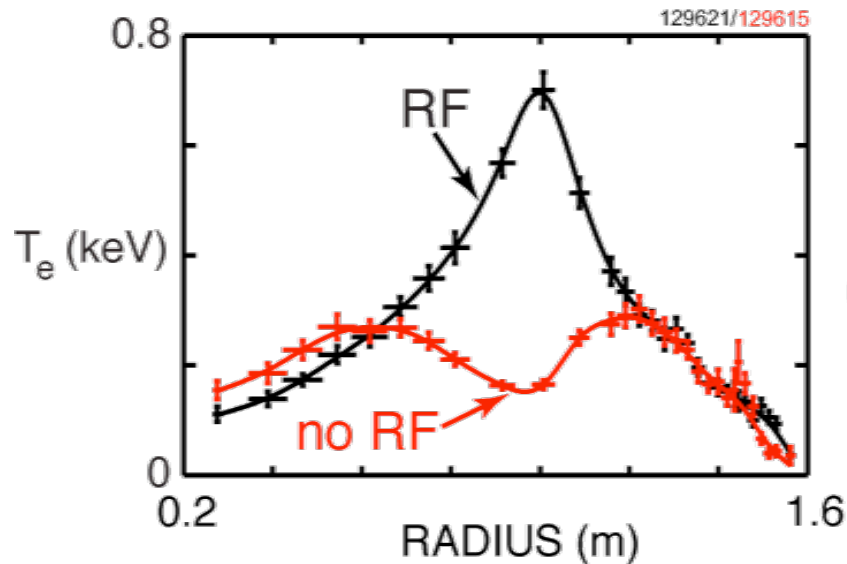


Time = 53 ms



- HHFW coupled from 18 to 64 ms
- T_e , P_e profiles remained hollow during HHFW heating pulse

XP-817: Good $k_\phi = -8 \text{ m}^{-1}$ HHFW Heating Core Electron Heating During I_p Ramp from 100-300 kA



Time = 120 ms

- $T_e(0)$ increased from 140 eV to 700 eV at $n_e(0) = 6 \times 10^{18} \text{ m}^{-3}$ and $9 \times 10^{18} \text{ m}^{-3}$, respectively

XP-920: Aims to Study HHFW Heating of $I_p \leq 200$ kA Plasmas; Develop HHFW Ramp-Up in SFSU TSG

- Experiment will benefit from improved coupling possible at low RF loading with the new double-fed HHFW antenna
- Based on our experience in 2008, the experiment will also benefit from lithium conditioning
- Considerable effort may be needed to establish a stable target discharge with $I_p < 200$ kA (Dave Gates & Dennis Mueller)
- Experiment has two phases and requires one day:
 - Setup $I_p = 200$ kA plasma, add ~ 3 MW of $k_\phi = -8$ & -14 m⁻¹ heating
 - If $I_p = 200$ kA plasma is stable, develop $I_p \leq 200$ kA plasma & add RF

Run Plan

- Before starting experiment rtEFIT control should be setup to operate at low I_p , with I_{pmin} set to 150 kA (I_{pmin} currently set to 200 kA)
- Setup 600 ms I_p flattop plasma, similar to 123712 ($B_T = 5.5$ kG, D_2), but with $I_p = 200$ kA, instead of 300 kA (**5 shots**)
 - Adjust outer gap to 5-10 cm and introduce lithium at 20 mg/min, adjust evaporation rate as necessary later for good RF coupling
- If the $I_p = 200$ kA plasma is stable add $k_\phi = -8$ m⁻¹ HHFW power, coupled from 100 – 600 ms. Increase RF power to ~ 3 MW (**5 shots**)
 - Adjust lithium evaporation rate, gas injection rate and outer gap to optimize HHFW heating efficiency
- Repeat with $k_\phi = -14$ m⁻¹ HHFW heating (**5 shots**)
- If the $I_p = 200$ kA is stable, adjust $I_p < 200$ kA, while still maintaining plasma position control so that the outer gap is 5-10 cm (**5-10 shots**)
- Add $k_\phi = -8$ m⁻¹ heating (**5 shots**) and then $k_\phi = -14$ m⁻¹ heating (**5 shots**)

Operational Requirements & Analysis

- Experiment should follow high power HHFW experiments: XP-944 (HHFW Edge Effects) and XP-946 (HHFW Heating and Current Drive Phase Scans)
- Experiment needs rtEFIT isoflux control of the outer gap
- NBI blip from source A @ 90 keV required for CHERS data acquisition from 580 to 600 ms
- MPTS is required for electron heating
- SOL reflectometer & ERD ion heating data required for edge power loss and coupling efficiency
- Analysis includes EFIT, TRANSP & GENRAY/CQL3D
- Results will be published in *Physics of Plasmas* or *Nuclear Fusion*