DIII-D Support for ITER Disruption Mitigation System Selection and Design

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DIII-D is Addressing Critical Issues for the ITER Disruption Mitigation System



- Preferable to avoid disruptions, but Disruption Mitigation System (DMS) needs to be available and reliable
 - General approach: Inject particles → radiate plasma's energy
- Timescale/scope of research set by ITER DMS Final Design Review in 2017
 - Current plan allocates space in three upper and one equatorial port
- DIII-D research is addressing all phases of the disruption
 - Choice/location/number of injectors
 - RE mitigation is still an unresolved issue for ITER



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DIII-D is Supporting the ITER DMS Design for All Phases of the **Disruption**



C.M. Greenfield/APS-DPP/Nov. 2013

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DIII-D is Evaluating Different Particle Delivery Methods for the ITER DMS



Goal of All Injectors is to Harmlessly Radiate Away the Energy Contained in the Plasma



Goal #2 Minimize radiation peaking (PFC melting)

 $P_{rad}^{\flat}(\theta, \Phi, t) = f(\theta, \Phi, r)$

• Experiment and theory are working together to explore

- Asymmetries due to injector location
- Asymmetries due to MHD instabilities
 - [V.A. Izzo, Phys. Plasmas 20, 056107 (2013)]



DIII-D Studying Sources of P_{rad} Asymmetry During Disruption Mitigation



 Injector # & distribution: No significant effect upon TQ P_{rad} asymmetry Thermal quench MHD:
 Does appear to affect TQ
 P_{rad} asymmetry



DIII-D Studied Effect of Injector Delay & Poloidal Location on Vertical Displacement Event (VDE) Mitigation

- ITER DMS Question: Will lack of lower injectors impede downward VDE mitigation?
- Timing Effect: Clear advantage to early VDE mitigation
- Location Effect: Only small advantage for valve in direction of VDE





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DIII-D Will Test Integrated Disruption Mitigation Scenarios that Simultaneously Meet ITER TQ & CQ Requirements

ITER DMS requirements:

Radiate 90% thermal energy Adiate 90% thermal energy Maintain CQ duration 35 ms < τ_{CQ} < 150 ms

- Conflicting requirements likely
 mean small operating space
- Minimal difference between DIII-D carbon wall & Be wall when using high-Z (neon) impurity injection





DIII-D is Supporting the ITER DMS Design for All Phases of the **Disruption**



Research Program Provides Multi-Layered Approach for Minimizing RE Threat





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Research Program Provides Multi-Layered Approach for Minimizing RE Threat





DIII-D is Investigating "Anomalous" RE Transport Physics and Feasibility of Suppressing RE Avalanche

- Theory: Increasing critical electric field (E_{crit}) enough to suppress avalanche requires ~100X n_{e,0} during CQ
 - Not compatible with other ITER operating limits
- Experiment: Enhanced dissipation (higher threshold E) observed
 - Lower n_e for RE suppression may be possible
- 20 Mode Measured 20 Hollmann & Parks, Nucl. Fus. (51),103026 201 -30 -0.5 0.0 0.5 1.5 2.0 1.0 Applied – E_{crit} (V/m)
- Anomalous RE dissipation measured in both post-disruption RE & ohmic flattop quiescent RE (QRE) beams



3 DIII-D is Exploring Scenarios for Dissipation of an Existing RE Beam



- Pre-emptive high-Z Massive Gas Injection provides fast RE shutdown
- DIII-D has demonstrated 4× dissipation increase with small quantities of high-Z gas
- DIII-D results suggest that gas or pellet injection systems sized for ITER disruption mitigation will also be effective for 'post-emptive' RE dissipation
- Future DIII-D studies with Ar, He and Xe needed



Future Work Will Asses Controllability and Termination Physics of RE Beams



- Assess vertical controllability of RE beam
 - What are true limits?
 - Improved controllability by preplacing plasma at neutral point before disruption?



Pursue physics basis for onset of RE final loss and assess impact to vessel

- Thermal Footprint
- Magnetic \rightarrow Kinetic energy transfer







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