

# 3D modeling of toroidal asymmetry due to localized divertor nitrogen puffing on Alcator C-Mod

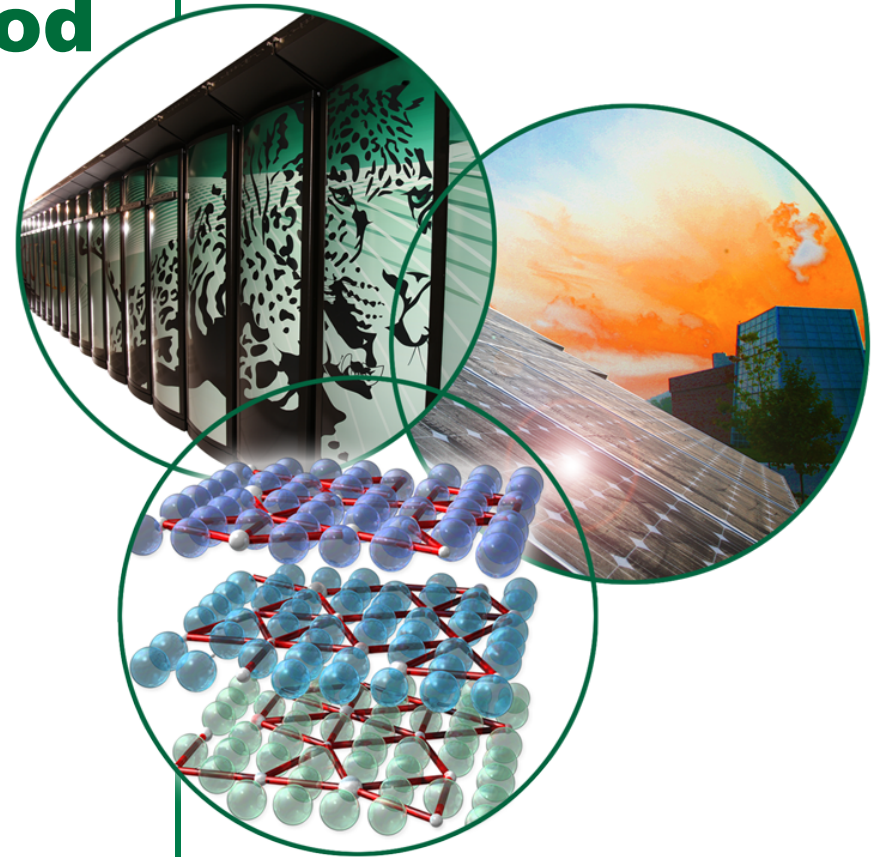
J.D. Lore<sup>1</sup>, M.L. Reinke<sup>2</sup>, B. LaBombard<sup>2</sup>,  
B. Lipschultz<sup>3</sup>, R. Pitts<sup>4</sup>

<sup>1</sup>Oak Ridge National Laboratory, Oak Ridge TN, USA

<sup>2</sup>Plasma Science and Fusion Center, MIT, Cambridge MA, USA

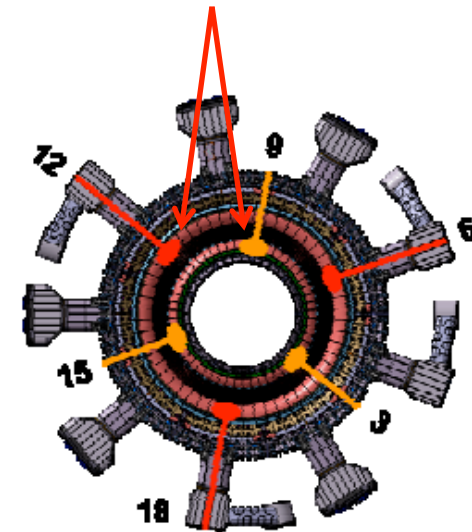
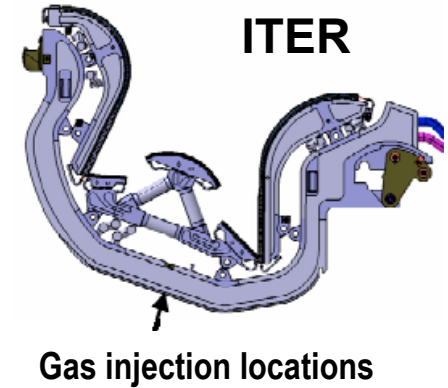
<sup>3</sup>University of York, York, UK

<sup>4</sup>ITER Organization, St Paul Lez Durance, France



# ITER requires partially detached divertor plasmas

- During inductive operation at  $Q=10$ , ITER must run with partially detached divertor plasmas
- A set of divertor gas valves will be used to maintain the radiated power fraction
  - Toroidally localized injection may lead to asymmetry in radiated power, detachment, heat flux
- Experiments were run on Alcator C-Mod to investigate potential asymmetry
  - Clear toroidal variation in radiated power, impurity line emission, divertor conditions measured with a single divertor puff
  - Experiments led to increasing the number of injection locations from 3 to 6 in ITER
- 3D modeling using the scrape-off-layer transport code EMC3-EIRENE is in progress to model these experiments
  - Goal: Validate model on C-Mod, then run predictive simulations for ITER

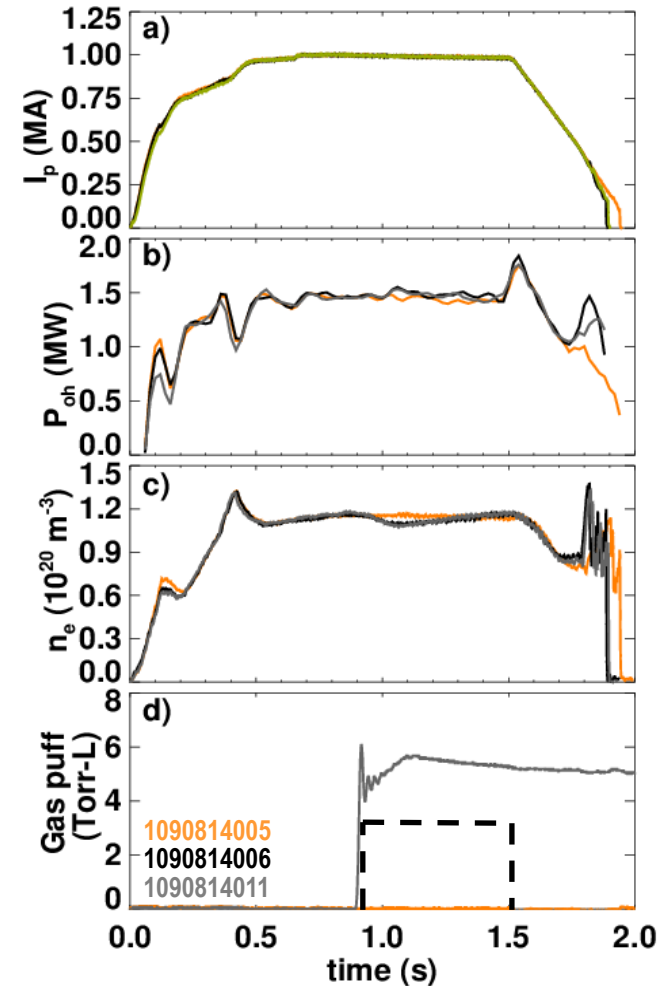
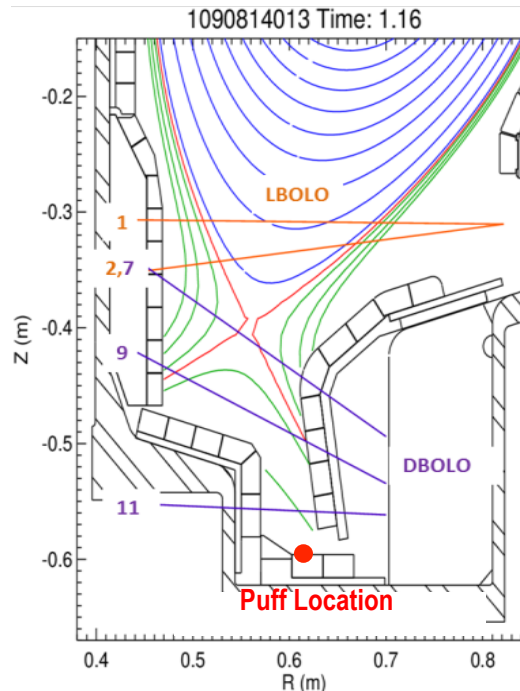
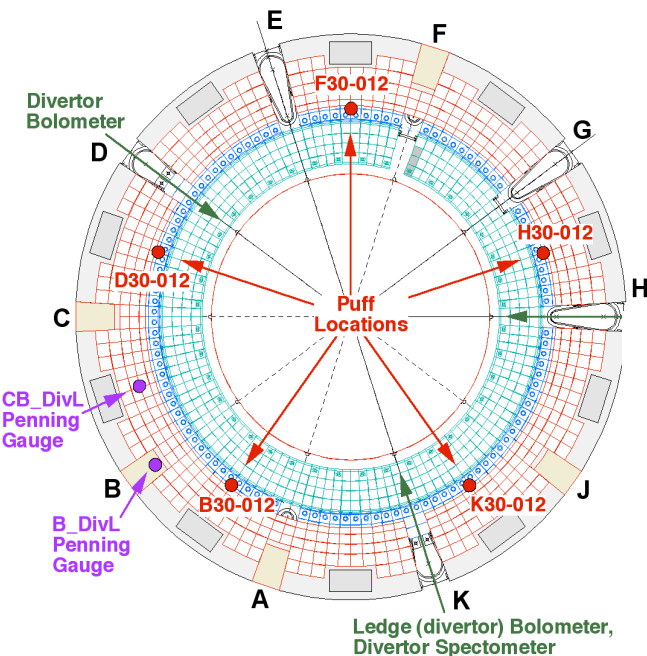


# Outline

- **C-Mod experiments with divertor gas injection**
- **The EMC3-Eirene Code**
- **Experimental and simulated trends in divertor pressure, radiated power, and nitrogen line emission**
- **Predicted asymmetry in divertor heat flux due to a single gas injection location**
- **Summary**

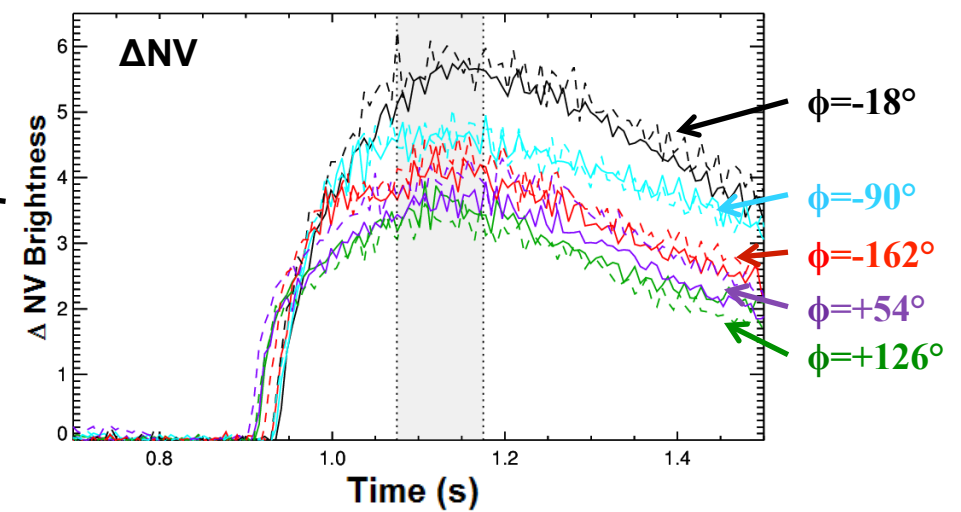
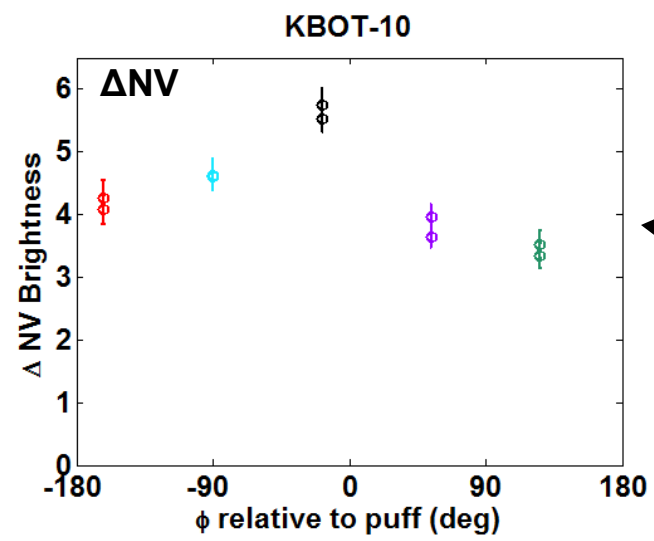
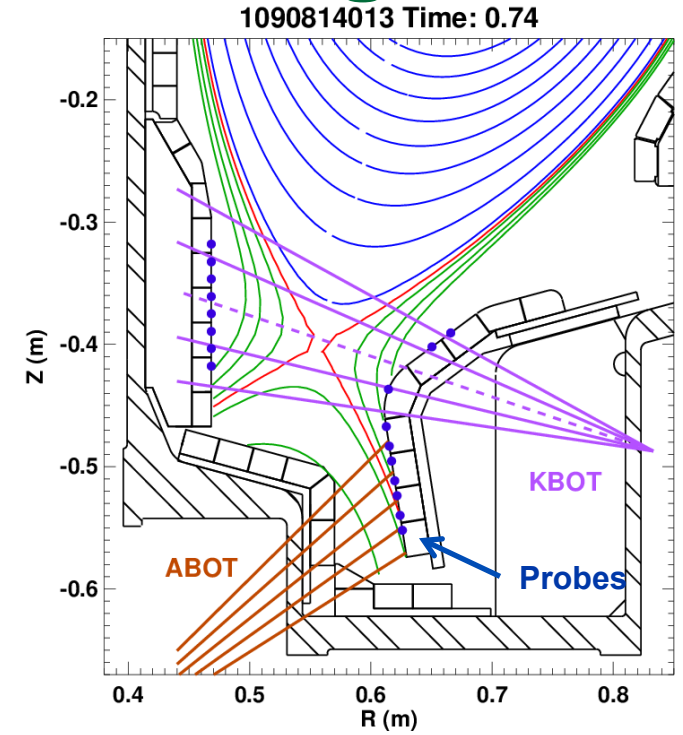
# C-Mod Experiments were Performed with Toroidally Localized Divertor Gas Injection

- Set of 10 reproducible discharges: 1090814(006-016)
  - Ohmic L-Mode,  $I_p=1\text{MA}$ ,  $B_t=5.4\text{T}$ ,  $n_e\sim 1.1\text{e}20\text{ m}^{-3}$ ,  $q_{95}\sim 3.75$
  - Divertor is in the high recycling regime
- $\text{N}_2$  injected into divertor at  $\sim 0.9\text{s}$  through a single valve each shot  $\rightarrow$  gas location shifts relative to diagnostics
  - Gas location analogous to ITER conditions



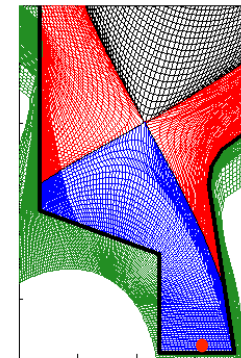
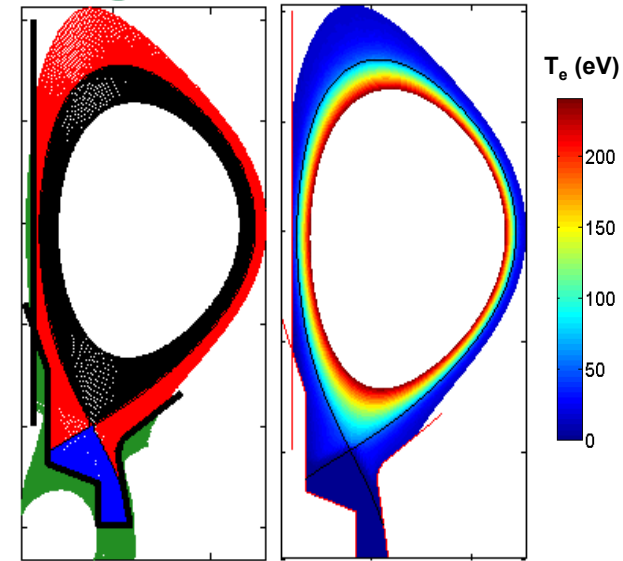
# Reproducible toroidal asymmetry is measured in edge and divertor diagnostics

- Experiments are well diagnosed, with many divertor and SOL views to constrain and validate modeling
- Toroidal modulation measured in nitrogen line emission,  $P_{\text{rad}}$ , and divertor electron pressure

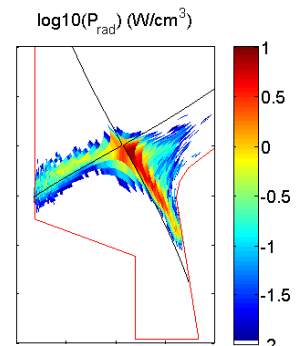
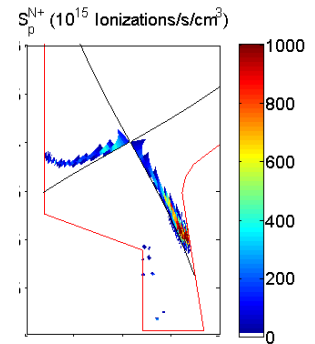


# Experiments are modeled using the 3D EMC3-Eirene code

- The EMC3-Eirene code<sup>1</sup>
  - 3D fluid plasma model (EMC3) coupled to kinetic neutral transport and PSI (EIRENE)
  - Classical parallel transport with prescribed anomalous cross-field diffusivities
  - Trace fluid impurity model ( $T_a = T_i, n_a Z_a \ll n_i$ ) with feedback to main plasma through electron energy loss
  - Outputs: 3D neutral and fluid plasma quantities, surface loads on to PFCs
  - No cross-field drifts or kinetic corrections
- Simulation of N puff experiments
  - High resolution, full toroidal grid with single N<sup>0</sup> puff in divertor
  - Inputs:  $P_{\text{core}} = 1.25\text{MW}$ ,  $n_{\text{core}} = 1e20\text{m}^{-3}$ , constant cross-field diffusivities, N<sup>0</sup> strength from puff calibration,  $R_{\text{imp}} = 0.5$
  - PFR is largely transparent to N<sup>0</sup>, ionization occurs near separatrix
  - Impurity radiation largest in flux tubes connecting to the divertor near the outer strike point

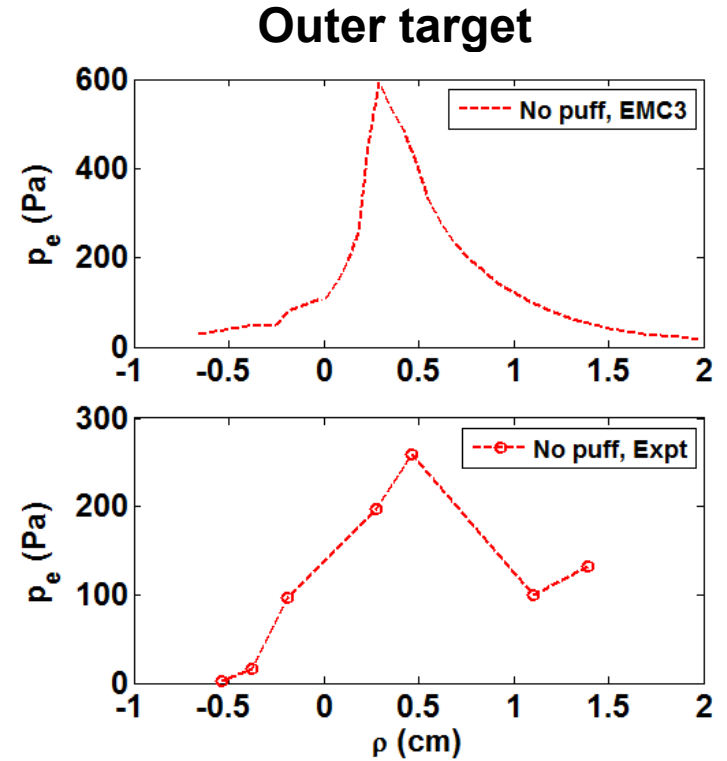
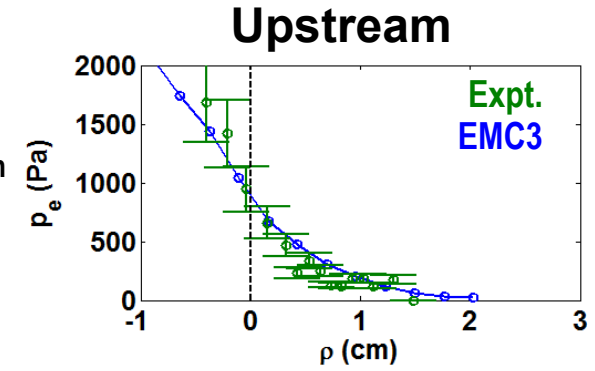


Puff Location



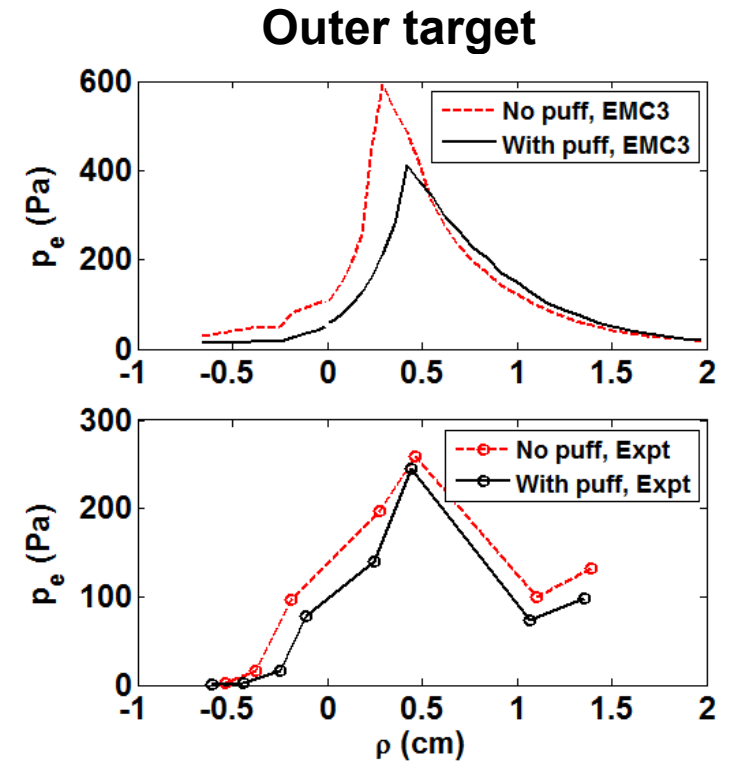
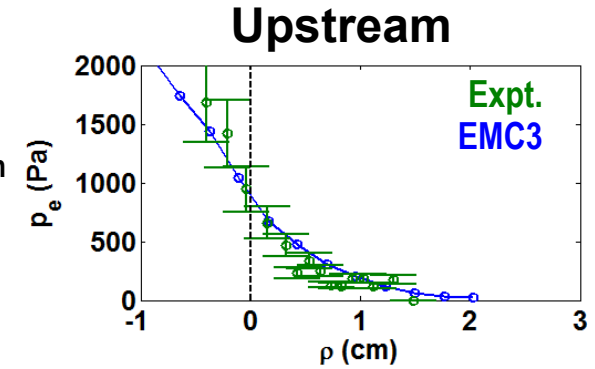
# Impurity puff results in net pressure drop with toroidal modulation

- Upstream pressure approximately matched with constant cross-field coefficients
  - Downstream pressure within  $\sim 2x$ , radially varying coefficients could be used to better match



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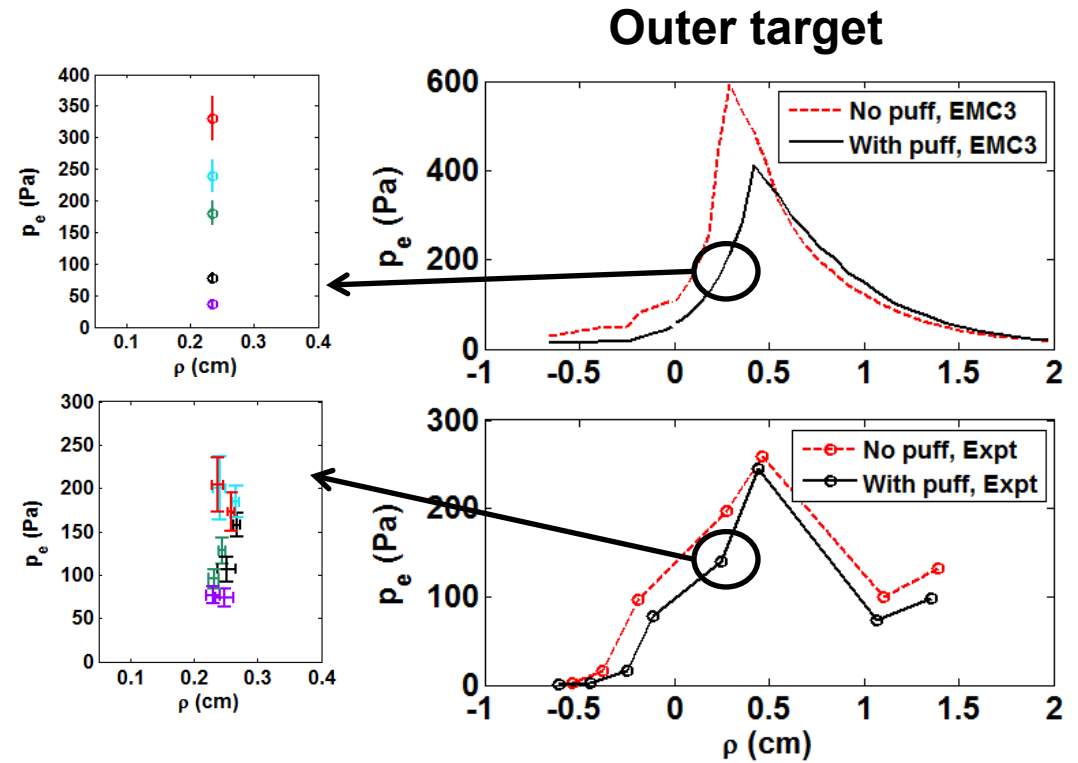
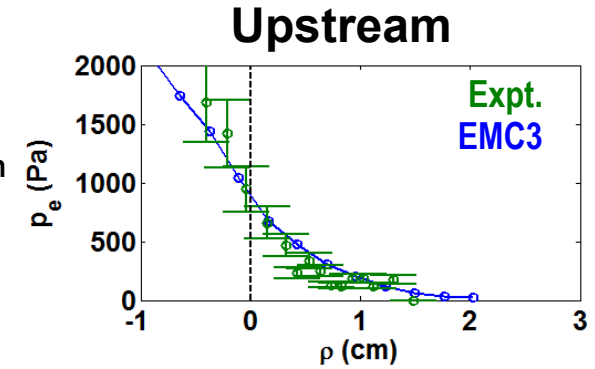
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  - Net ( $n=0$ ) pressure drop in experiment and model





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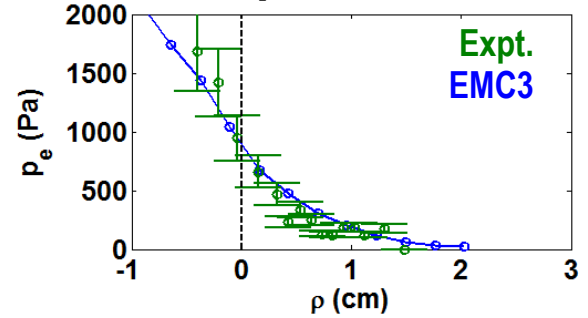
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- Repeatable toroidal modulation in measured pressure near outer strike point



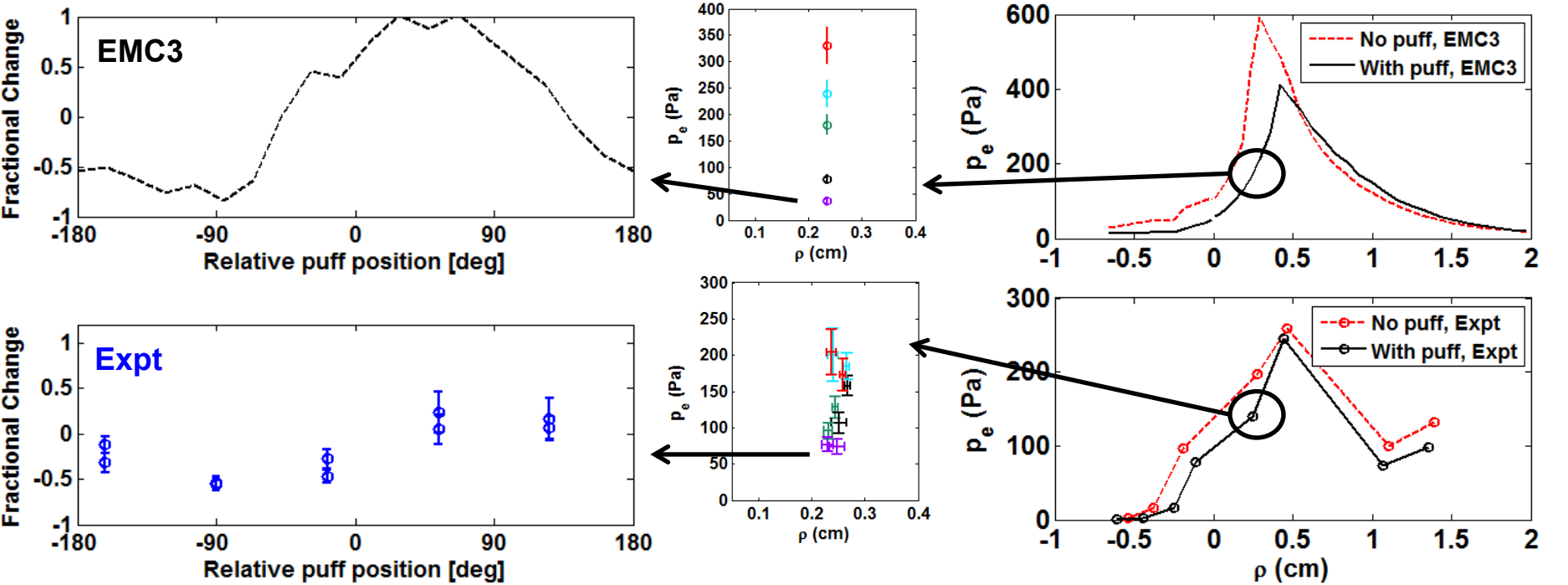
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  - $n \approx 1$  toroidal variation in pressure qualitatively captured by model

**Upstream**

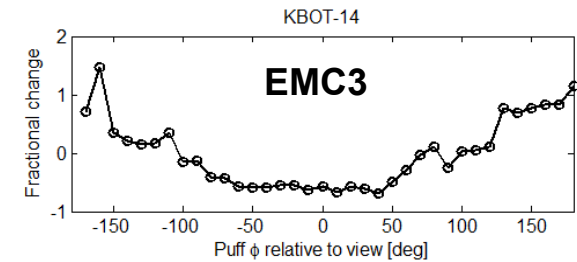
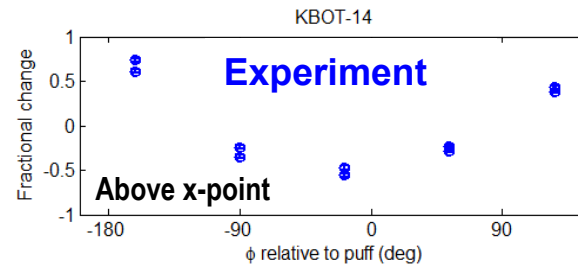
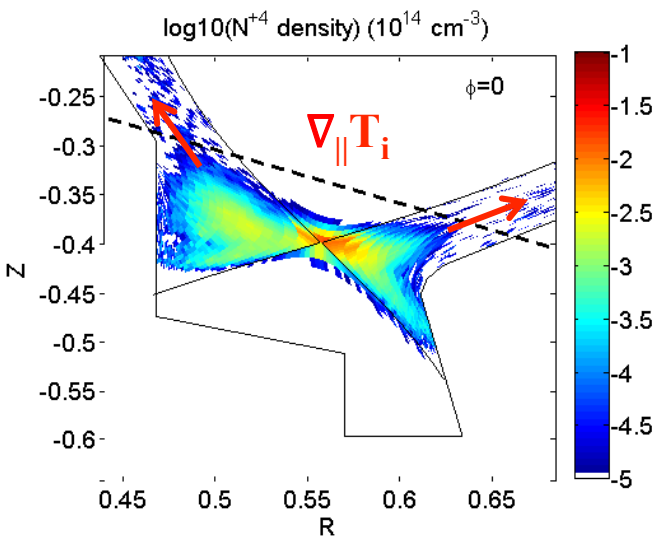


**Outer target**



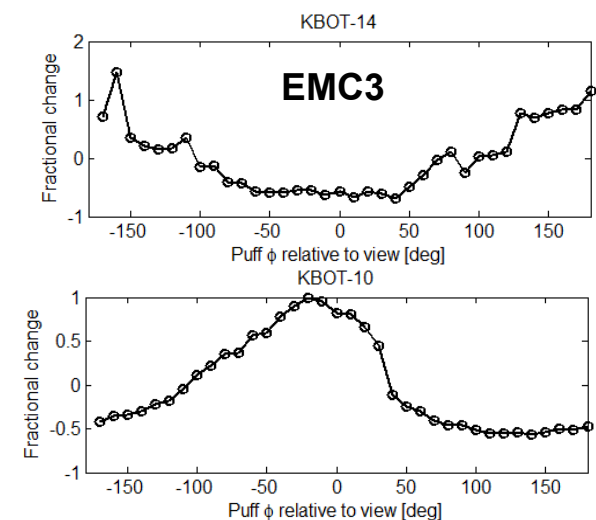
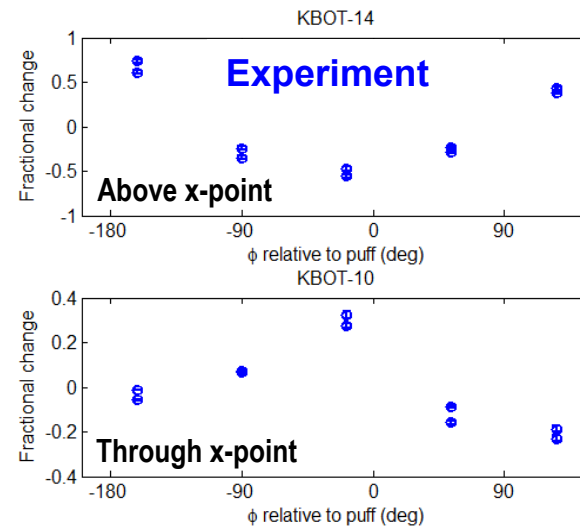
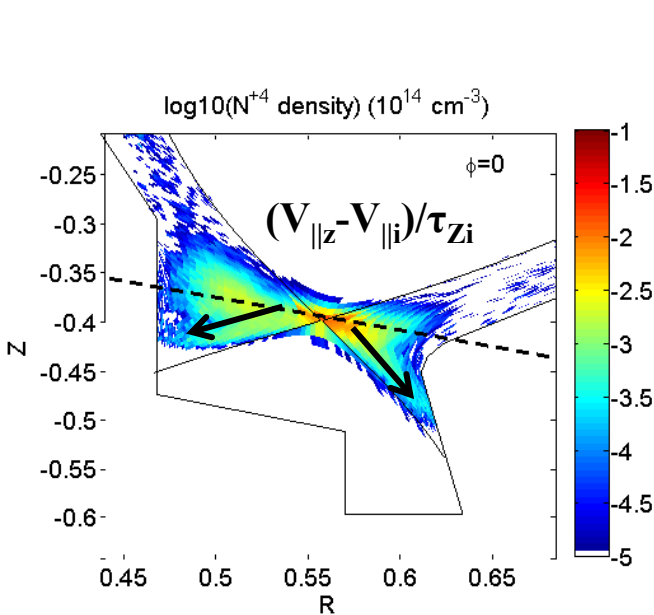
# Trends in N line emission captured near x-point, PFR may require cross-field drifts

- Clear toroidal asymmetry in NV emission in chords viewing near x-point
- Above x-point temperature gradient force pushes impurities upstream into view, results in inverted profile



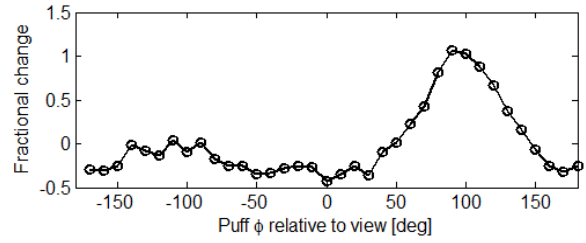
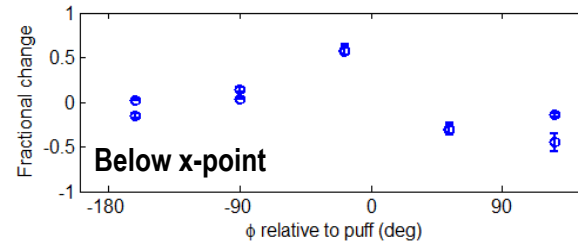
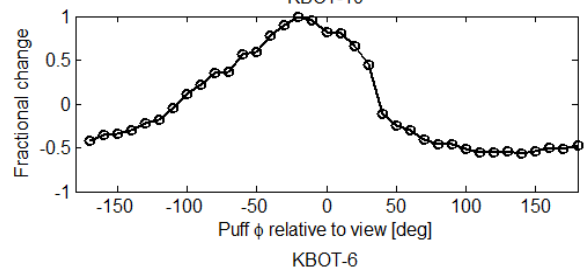
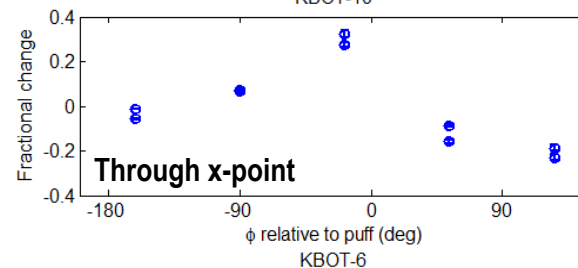
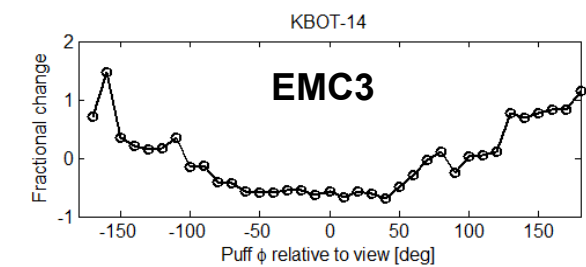
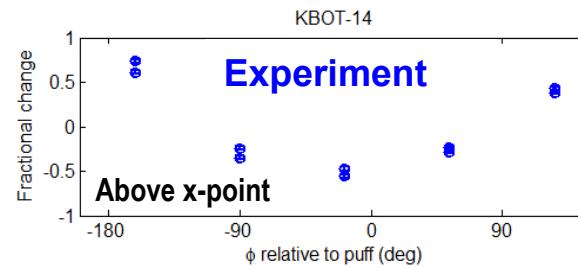
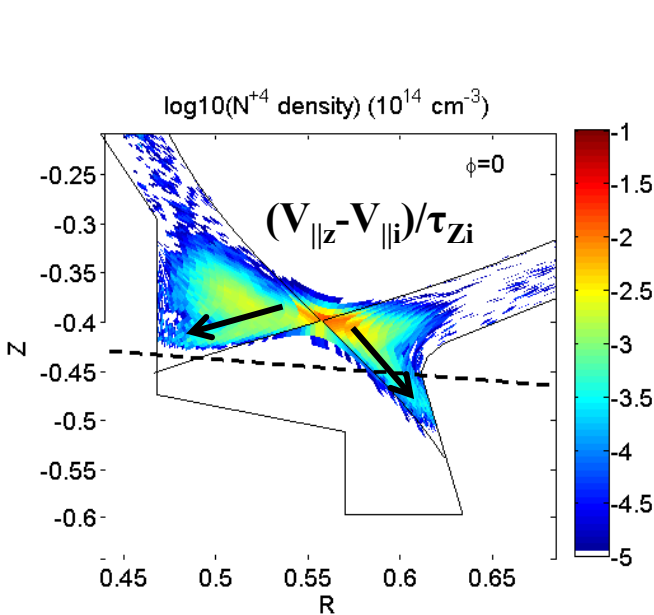
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- Toroidal behavior seems to be well described by parallel impurity forces in SOL near x-point



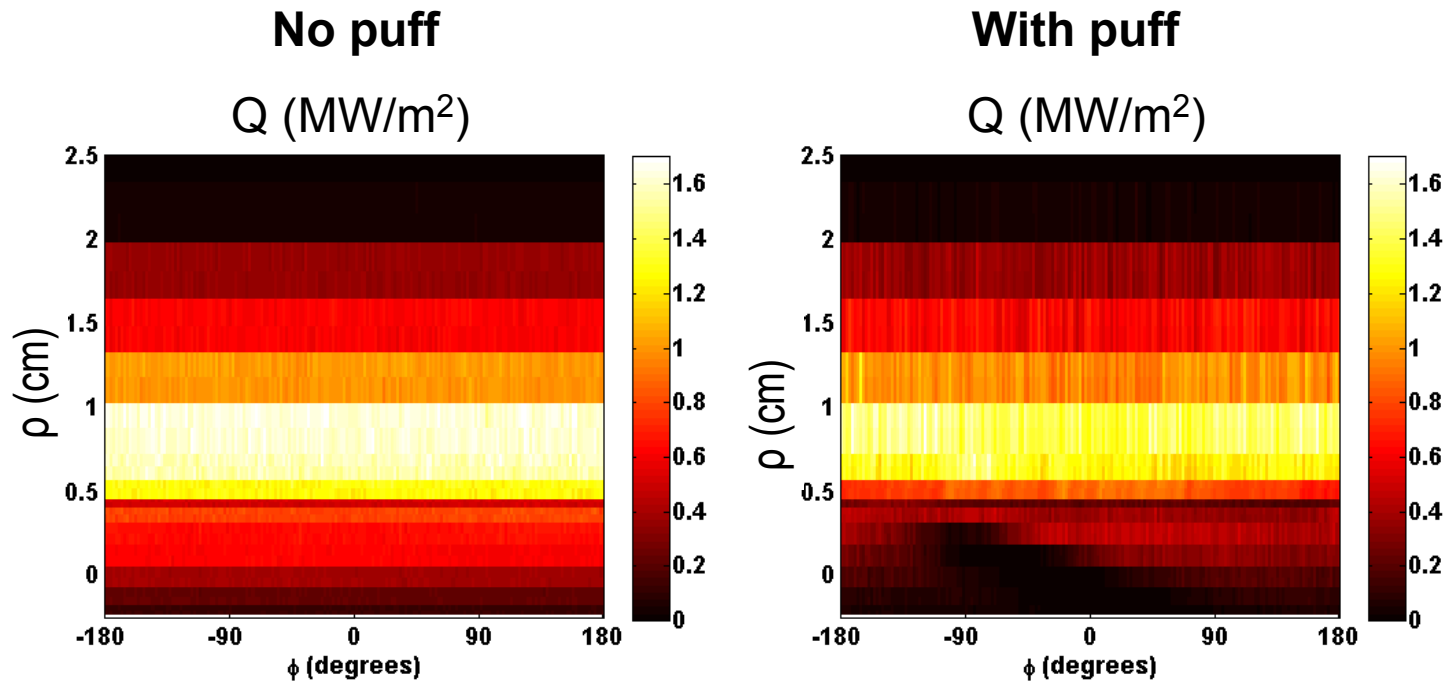
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- Toroidal behavior seems to be well described by parallel impurity forces in SOL near x-point
- Below x-point: Friction dominates in model, results in in downstream peaking. Experiment peaked at puff loc.
  - Cross-field drifts may be required to capture impurity behavior in PFR, experiments have demonstrated importance [1,2]



# Toroidally asymmetry in target heat flux predicted near outer strike point

- Impurity radiation results in net reduction in power carried by plasma to targets
  - $P_{\text{targ}}^{\text{plasma}}$  reduced from 930kW to 730kW ( $P_{\text{in}}=1.25\text{MW}$ )
- Toroidal asymmetry in  $P_{\text{rad}}$  results in toroidal asymmetry in heat flux near outer strike point
  - Toroidal extent will depend on machine size, divertor geometry



# Summary

- **C-Mod experiments were performed to assess toroidal asymmetry caused by local divertor impurity injection for ITER**
- **The 3D edge transport code EMC3-Eirene has been applied to model these experiments**
  - Validation will give confidence in predictive simulations for ITER
- **Measured net reduction and toroidal variation in divertor pressure at OSP are qualitatively reproduced**
- **Modeled toroidal asymmetry in NV emission near x-point have similar trends as experiment, however cross-field drifts are likely required to match behavior in PFR**
- **Toroidal asymmetry in target heat flux is predicted with a single divertor injection location**
- **Ongoing work: Continue investigation of diagnostic data and model validation, quantify effect of multiple injection locations, determine scaling with machine size**

# Extra slides



# EMC3-Eirene fluid equations

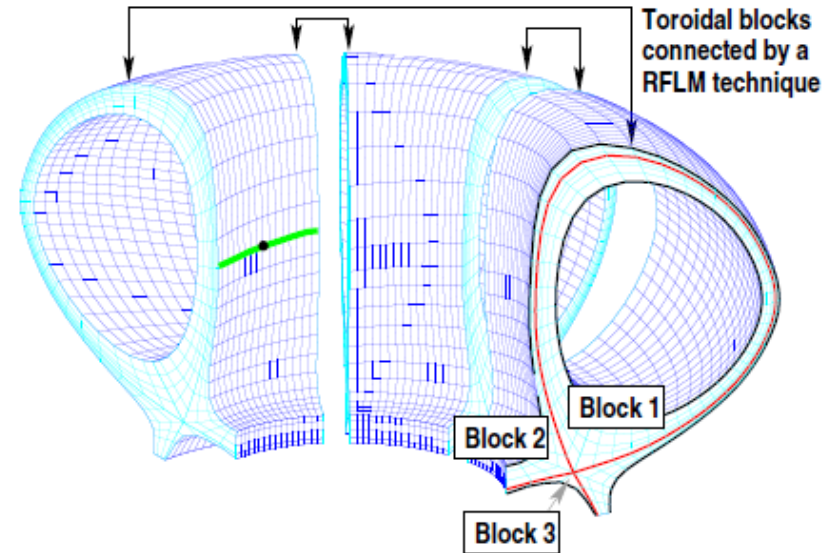
$$\nabla \cdot (n_i V_{i||} \mathbf{b} - D \nabla_{\perp} n_i) = S_p$$

$$\nabla \cdot (m_i n_i V_{i||} V_{i||} \mathbf{b} - \eta_{||} \nabla_{||} V_{i||} - m_i V_{i||} D \nabla_{\perp} n_i - \eta_{\perp} \nabla_{\perp} V_{i||}) = -\nabla_{||} p + S_m$$

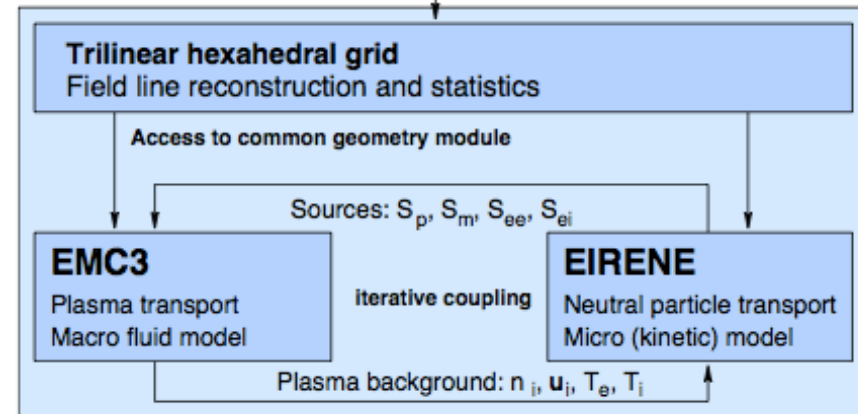
$$\nabla \cdot (\frac{\xi}{2} n_e T_e V_{e||} \mathbf{b} - \kappa_e \nabla_{||} T_e - \frac{\xi}{2} T_e D \nabla_{\perp} n_e - \chi_e n_e \nabla_{\perp} T_e) = -k(T_e - T_i) + S_{ee} + S_{imp}$$

$$\nabla \cdot (\frac{\xi}{2} n_i T_i V_{i||} \mathbf{b} - \kappa_i \nabla_{||} T_i - \frac{\xi}{2} T_i D \nabla_{\perp} n_i - \chi_i n_i \nabla_{\perp} T_i) = +k(T_e - T_i) + S_{ei}$$

Sources from plasma-neutral interactions provided by Eirene  
Trace impurity model



Field line integration

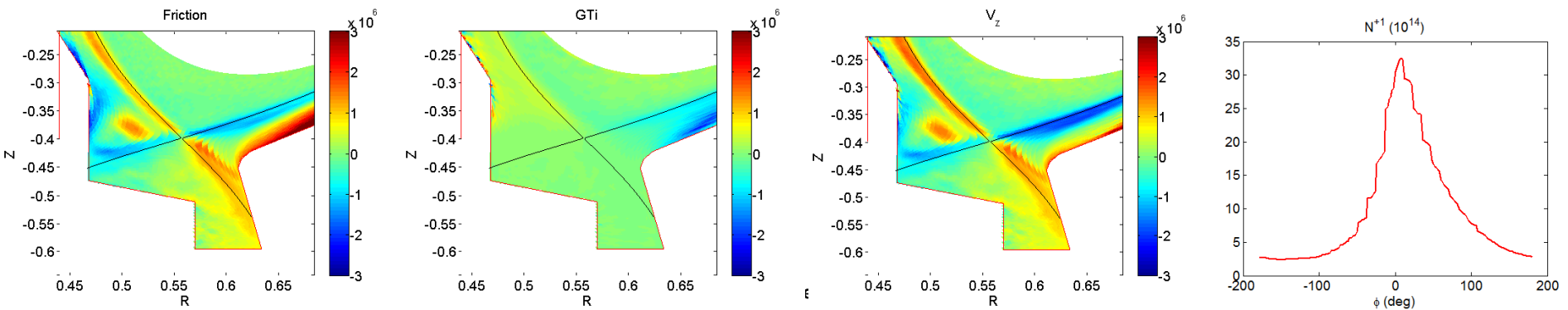


# Impurity forces in EMC3

$$V_z - V_i \sim \frac{T_i^{3/2}}{n_e} \nabla_{\parallel} T_i$$

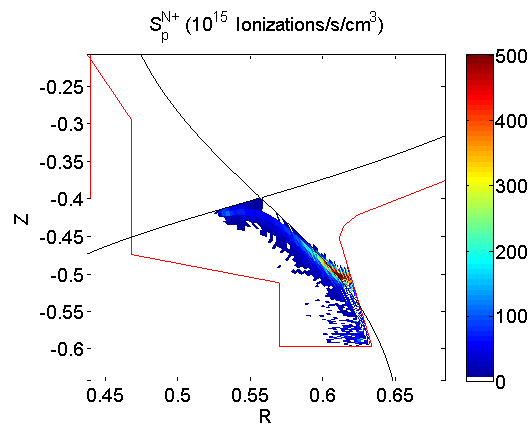
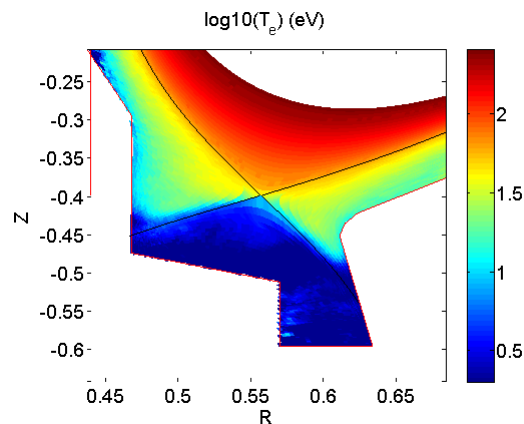
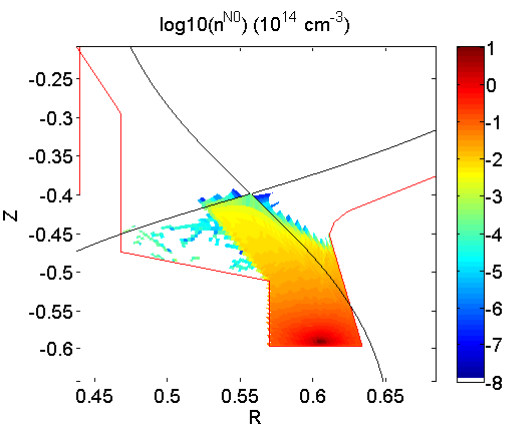
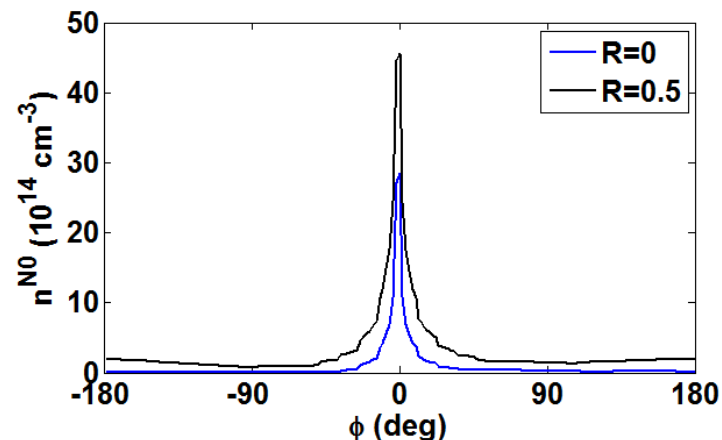
$$0 = \underbrace{-\frac{1}{n_z} \nabla_{\parallel} p_z}_{\text{Impurity pressure gradient}} + \underbrace{m_z \frac{(V_i - V_z)}{\tau_{Zi}}}_{\text{Main ion friction}} + \underbrace{eZ_z E_{\parallel}}_{\text{Electro-static}} + \underbrace{\alpha_{Ze} \nabla_{\parallel} T_e}_{\text{Electron Temp. Gradient}} + \underbrace{\beta_{Zi} \nabla_{\parallel} T_i}_{\text{Ion Temp. Gradient}}$$

- Dominant forces are friction and ion temperature gradient, and independent of Z
  - Divertor temperature is low, friction dominates in PFR
  - Non-recycling neutrals are trapped in PFR
- Experiments have shown evidence of drifts in SOL and PFR, not accounted for in this model



# Impurity ionization in PFR

- Gas injection is deep in divertor
- Plasma is nearly transparent to neutrals, ionization occurs near separatrix where  $T_e > 10\text{eV}$



# Comparison to ledge bolometers

- Similar trends in LBOLO, large discrepancies in DBOLO
  - Strike point position is critical for DBOLO

