



Heating, Confinement and Stability Studies in NSTX

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presented for the

NSTX Research Team

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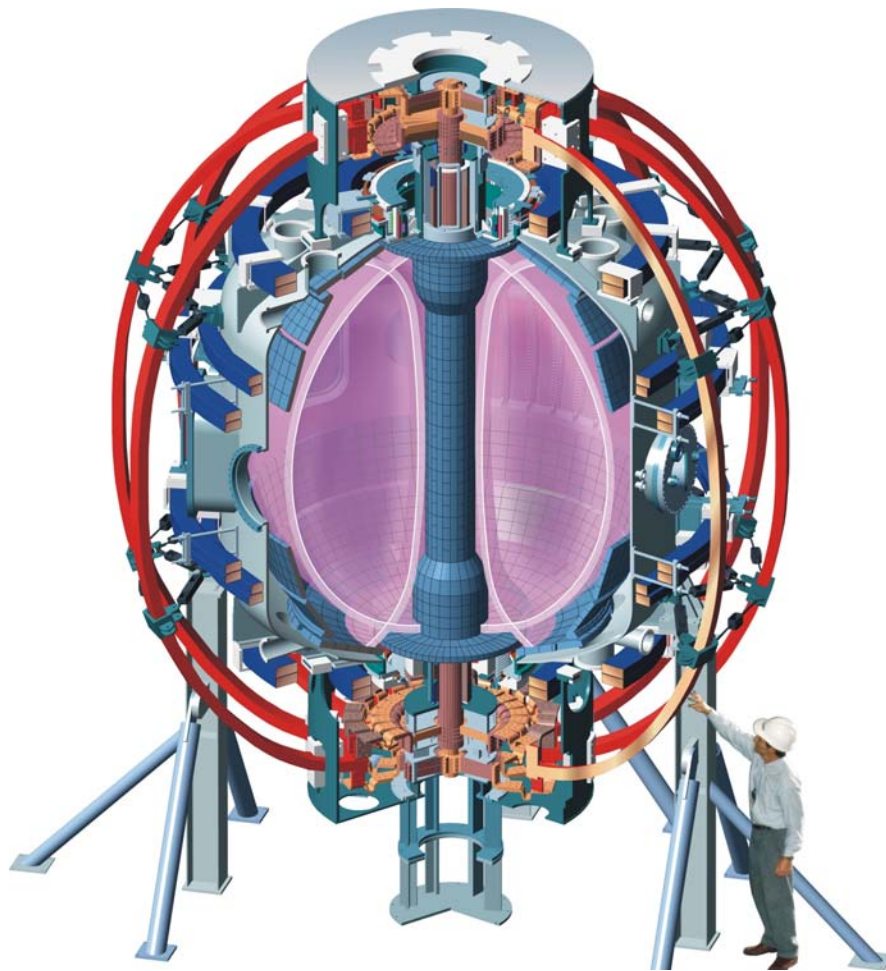


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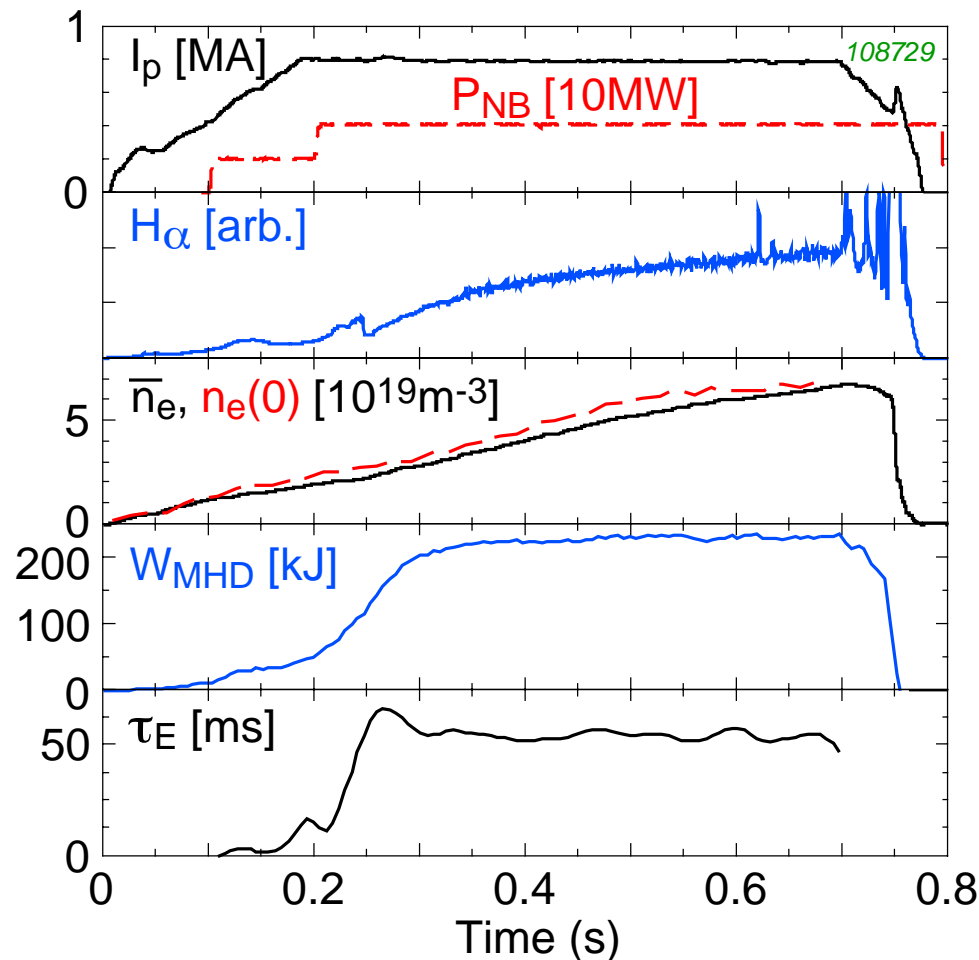
Experimental Capabilities Were Improved for 2002 Campaign



Capabilities *(this year)*

<i>PFC bakeout</i>	350°C
<i>Gas fueling</i>	HFS
Aspect ratio	1.27
Elongation	2.5
<i>Triangularity</i>	0.8
Plasma Current	1.5MA
<i>Toroidal Field</i>	0.6T
<i>NBI (100kV)</i>	7 MW
HHFW (30MHz)	6 MW
<i>- full antenna phase control</i>	
<i>Pulse Length</i>	1s
<i>Reduced PF error field</i>	

High-Field-Side Gas Injection Improved Reproducibility and Longevity of H-mode



- HFS injector gives large initial flow then continuing lower flow
 - contributes to density rise
- LFS fueling with rate similar to HFS produces
 - Delayed transition
 - Shorter H phase
- Can also get H-mode by loading walls with D_2 gas
- Confinement similar in all cases

Several Factors Contributed to Sustained Higher β Operation



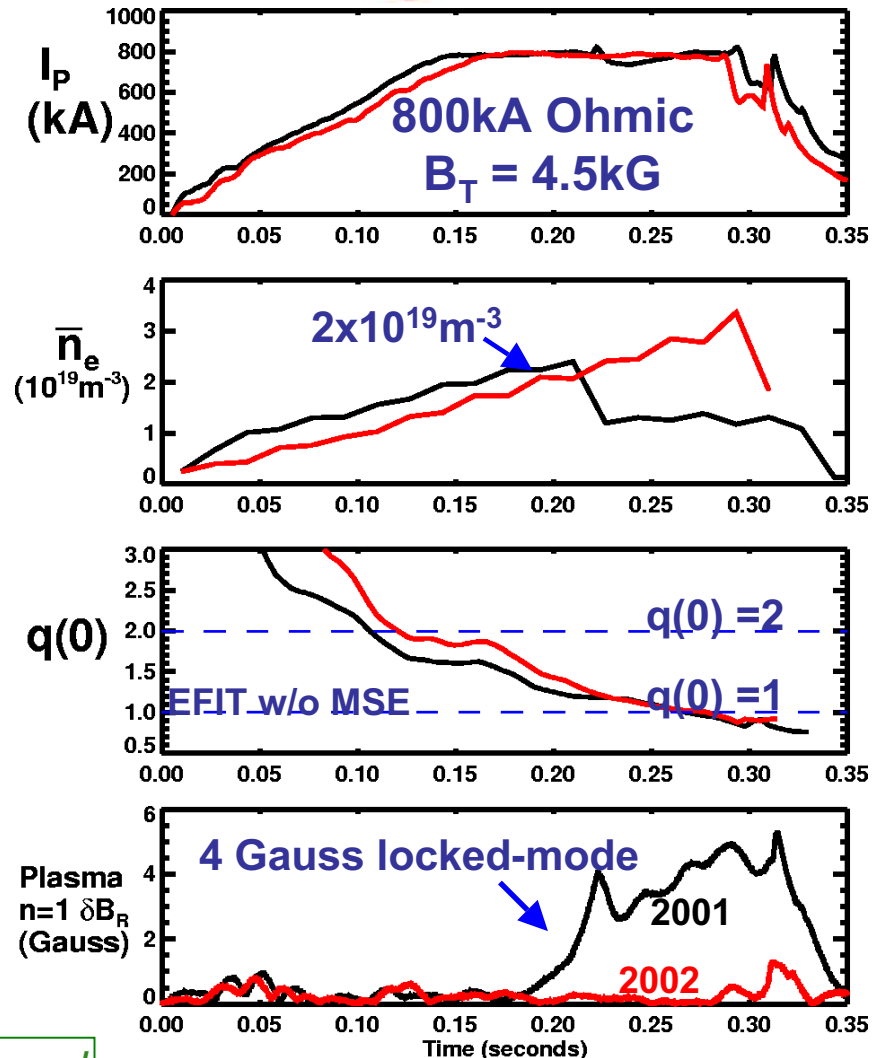
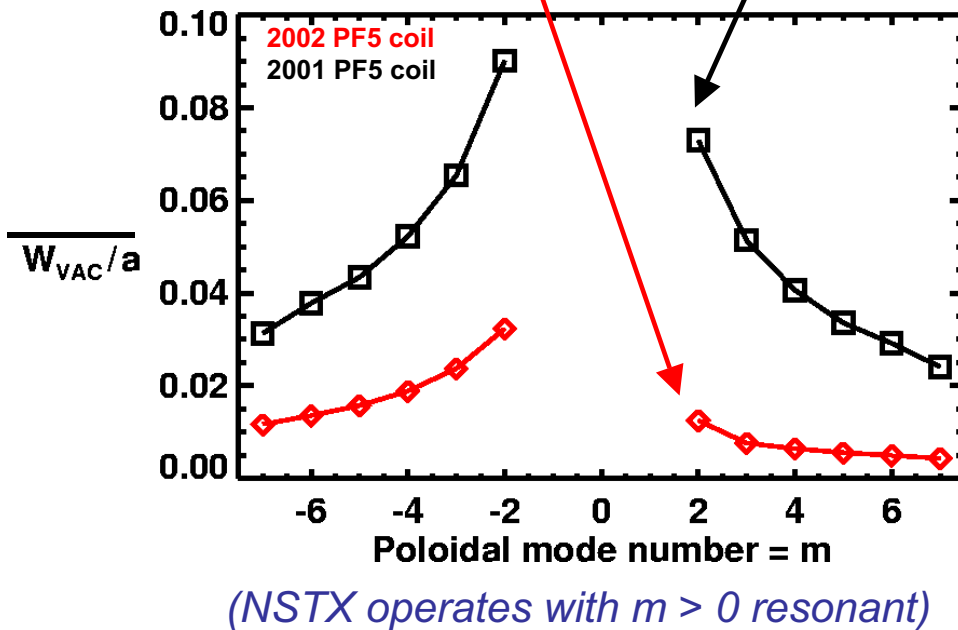
- Reduction of static error field
 - Reduced incidence of locked modes at low β
 - Reduced rotation damping
- Maintaining $q_{\min} > 1$ for longer
 - Previous high- β plasmas collapsed when $q_{\min} \leq 1$
 - Higher initial T_e & purity increased conductivity
- H-mode broadened profiles

Reshaping & Realignment of Outer PF Coil

Reduced Error Field & Mode Locking



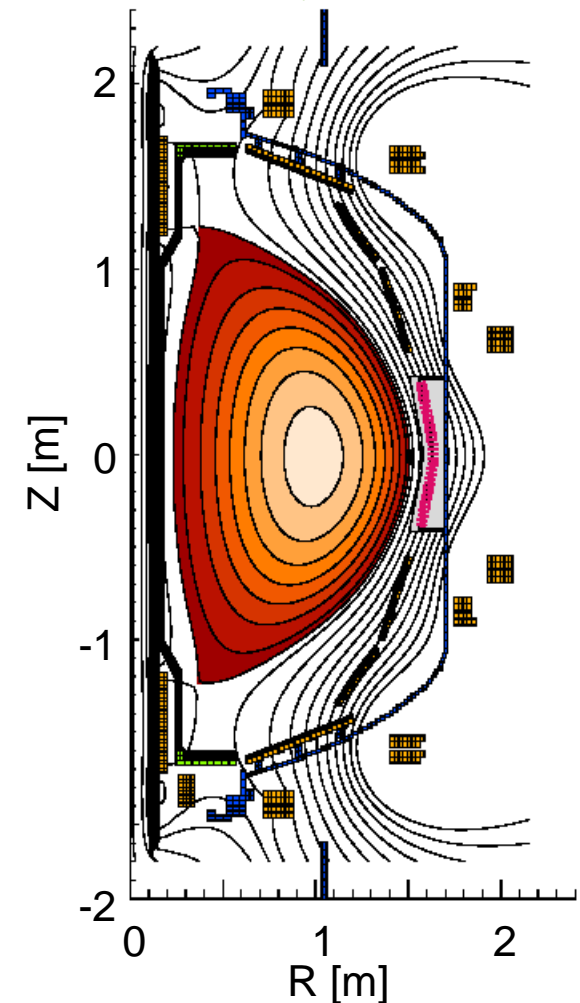
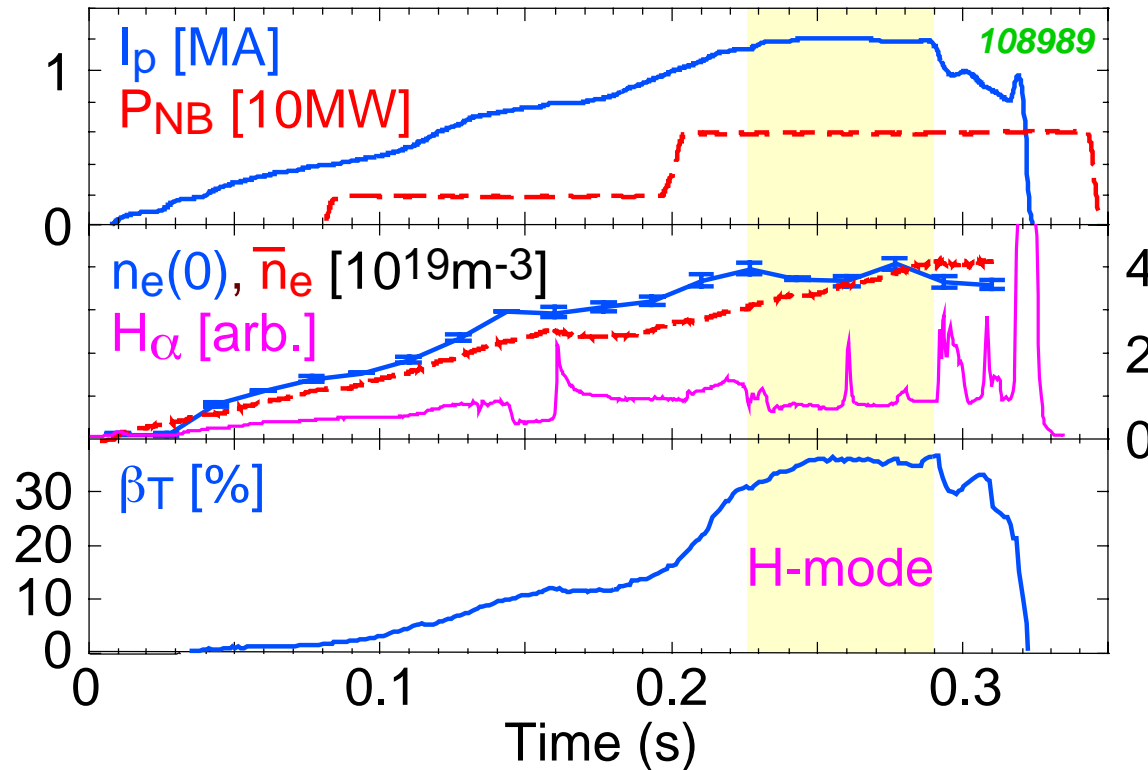
- PF5 (vertical field) coils found to generate large $n=1$ δB_r
- Coils re-shaped prior to '02 run
- Vacuum island widths now **reduced to $< 1\text{cm}$** (from $\sim 5\text{cm}$)



Achieved Substantial Progress in β_T

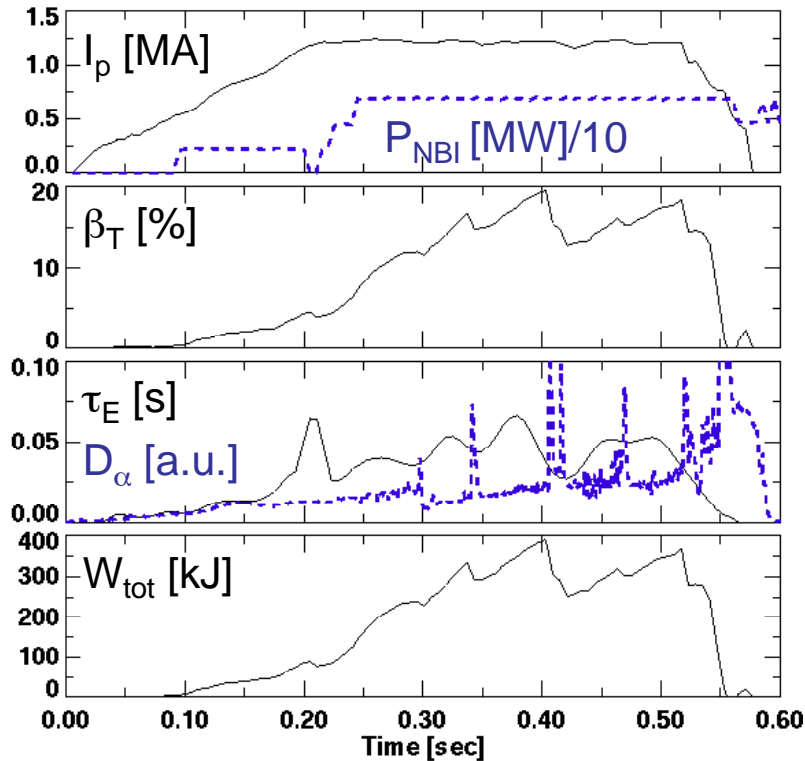


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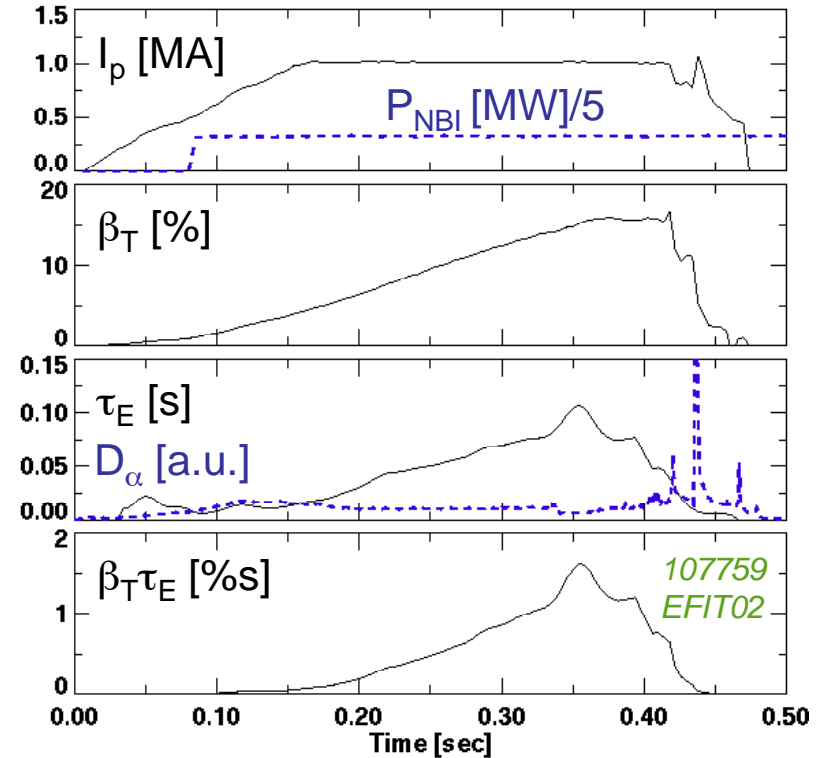
- $\beta_T = 35\%$ (EFIT with ϕ_{dia} and p_e profile)
- $B_T = 0.3T$, $A = 1.4$, $\kappa = 2.0$, $\delta = 0.8$
- $I_i = 0.6$, $q_0 \approx 1.4$

Highest Stored Energy and $\beta_T \times \tau_E$ Achieved at Higher Toroidal Field



- $W_{tot} = 390\text{kJ}$
- $B_T = 0.55\text{T}$, $\beta_T = 20\%$

- Also achieved $\beta_N \times H_{89L} > 12$ for $8 \times \tau_E$
- Higher B_T extends pulse (at lower β)

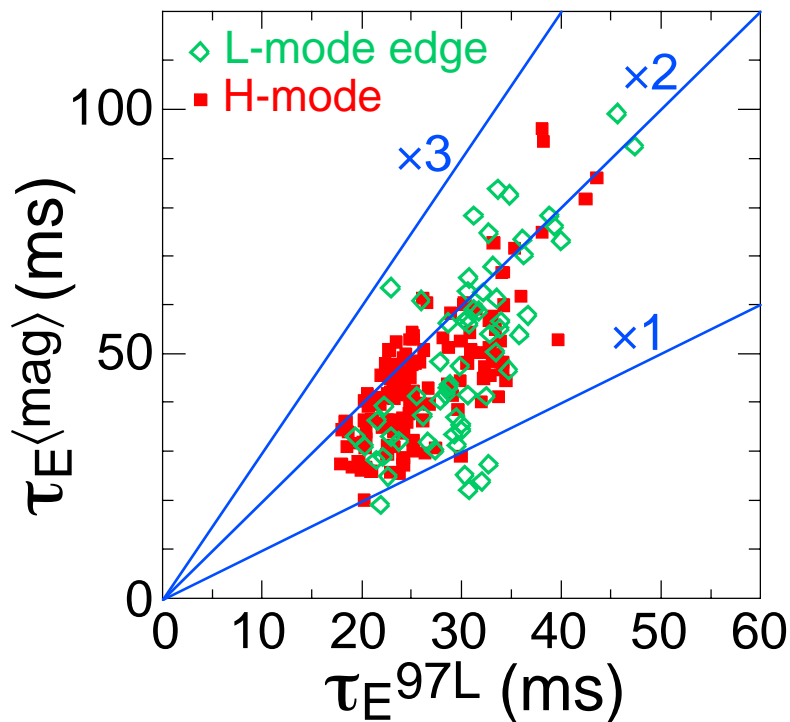


- $\beta_T \times \tau_E = 1.2\% \cdot \text{s}$
- $B_T = 0.4\text{T}$

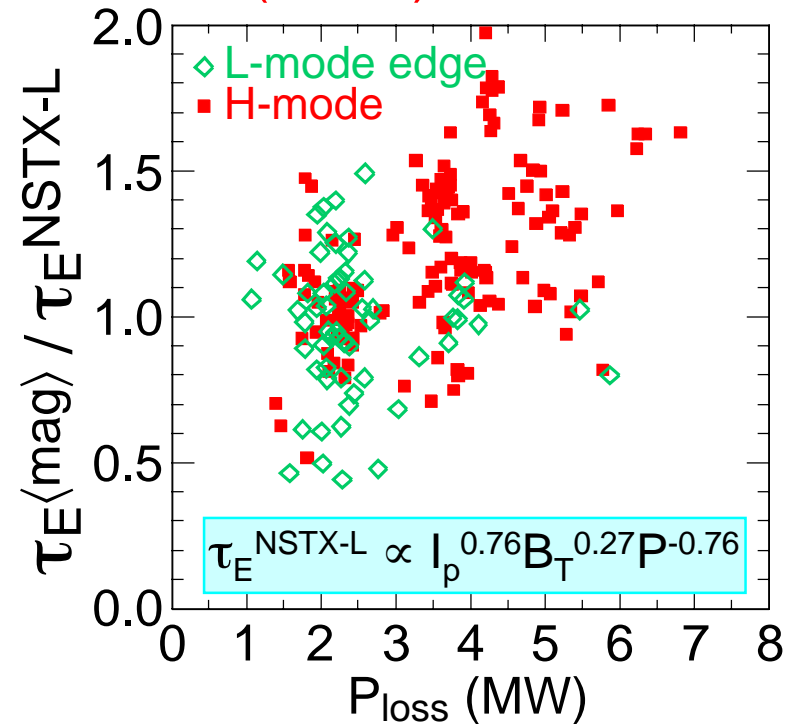
Global Confinement with NBI Continues to Exceed Standard Tokamak Scalings



Both L and H -mode plasmas can exceed ITER-97L scaling



Weaker power degradation in H-mode ($\sim P^{-0.5}$) than L-mode



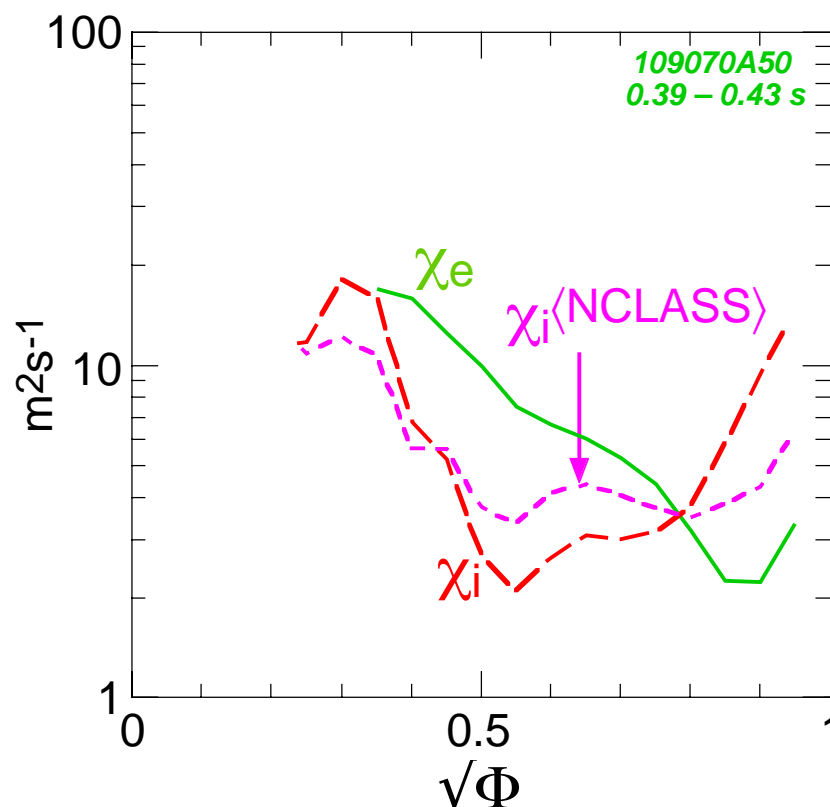
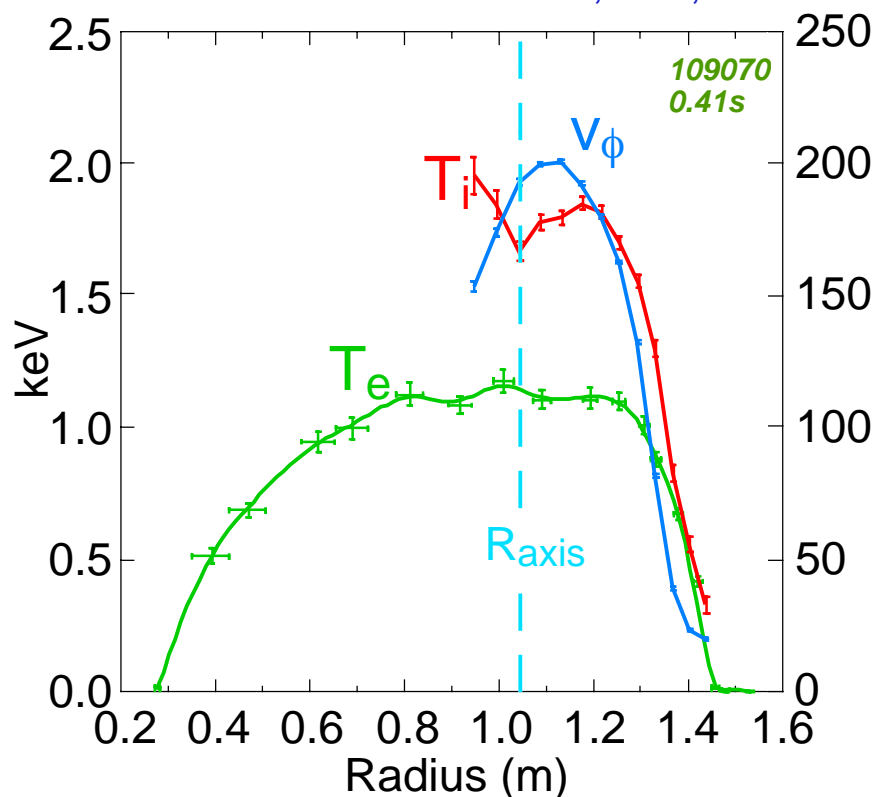
- Confinement times from EFIT determination of W_{tot}
 - Times near peak W_{tot} ; include NB *injected* and Ohmic power

Profile Analysis Confirms Low Ion Transport and Shows Unusual Features



$T_i > T_e$ throughout core
although classical $P_{b,i}/P_{b,e} \approx 0.7$

$\chi_i \sim \chi_i^{(NCLASS)} < \chi_e$
Diffusivities decrease with r/a



- In some discharges T_i locally exceeds $\chi_i = 0$ prediction

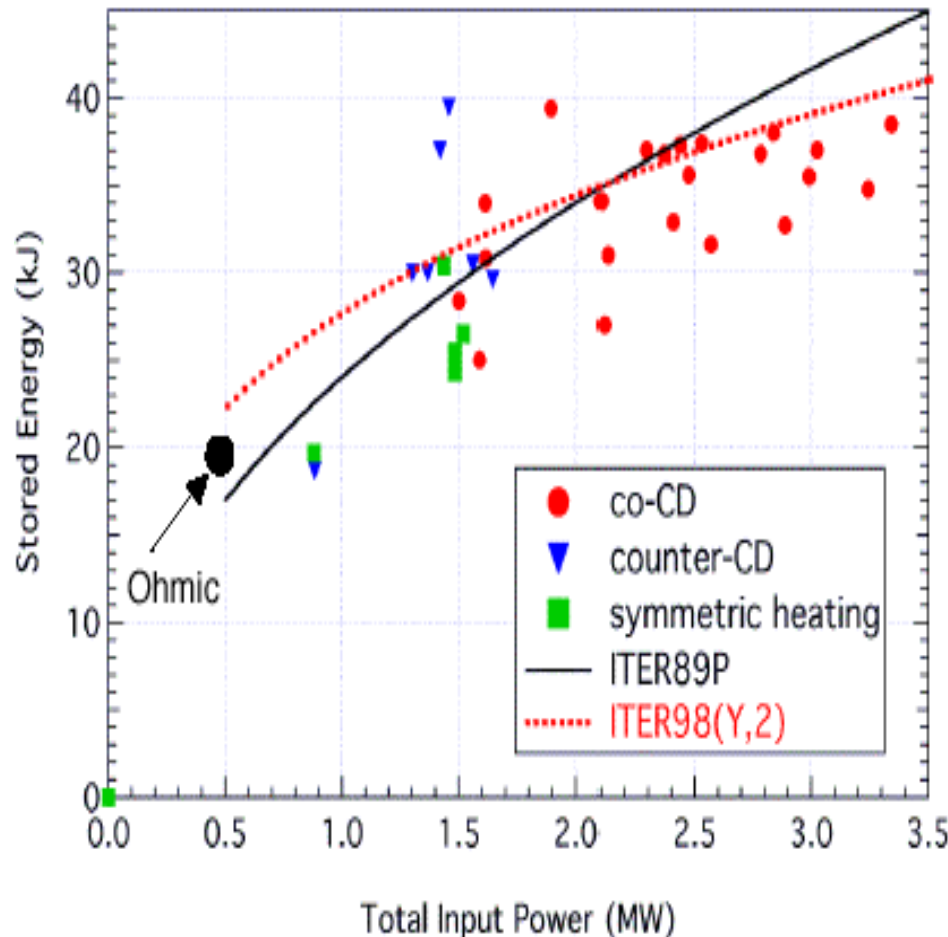
Summary



- New facility and diagnostic capabilities added
 - Routine H-mode operation
- Significantly increased β
 - β_T , β_N , β_P , $\beta_T\tau_E$, $\beta_N H$, W_{tot} reached new levels
- Good global confinement observed with NBI heating
 - H-modes have more favorable power dependence
 - Single parameter dependences remain to be determined
- Ion confinement appears to be very good

Following talks will elaborate on several of the interesting trends in NSTX performance

HHFW Heating Not Yet as Effective as NBI



- Compare measured plasma energy increase with ITER scaling for plasma conditions
 $I_p=0.5\text{MA}$, $B_T=0.45\text{T}$, $H=1$
- Current-Drive antenna phasings $\Rightarrow \mathbf{k}_{\parallel} \approx \pm 7\text{m}^{-1}$;
Heating phasing $\Rightarrow \mathbf{k}_{\parallel} \approx 14\text{m}^{-1}$
- Heating quite variable
- Power was limited by lower antenna standoff
 - feedthroughs now improved