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# Lithium Loaded Target Plate for driving NSTX toward high performance

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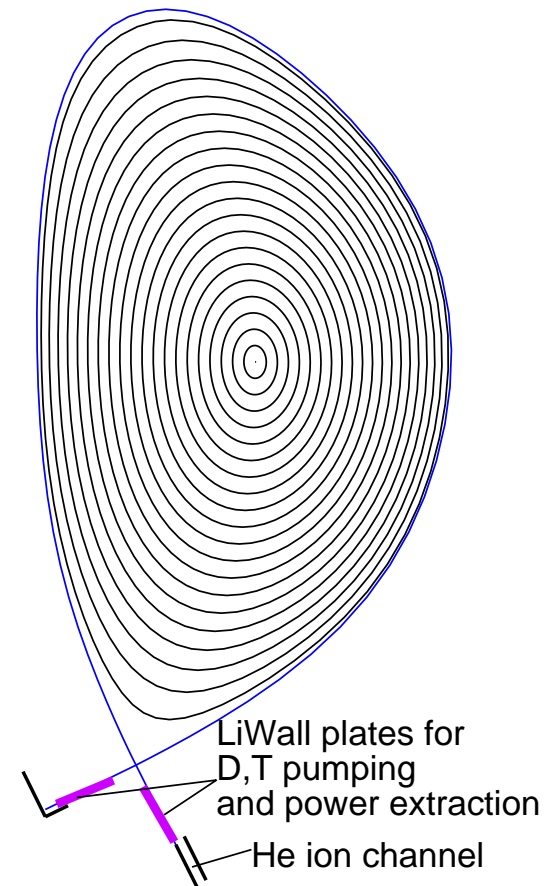
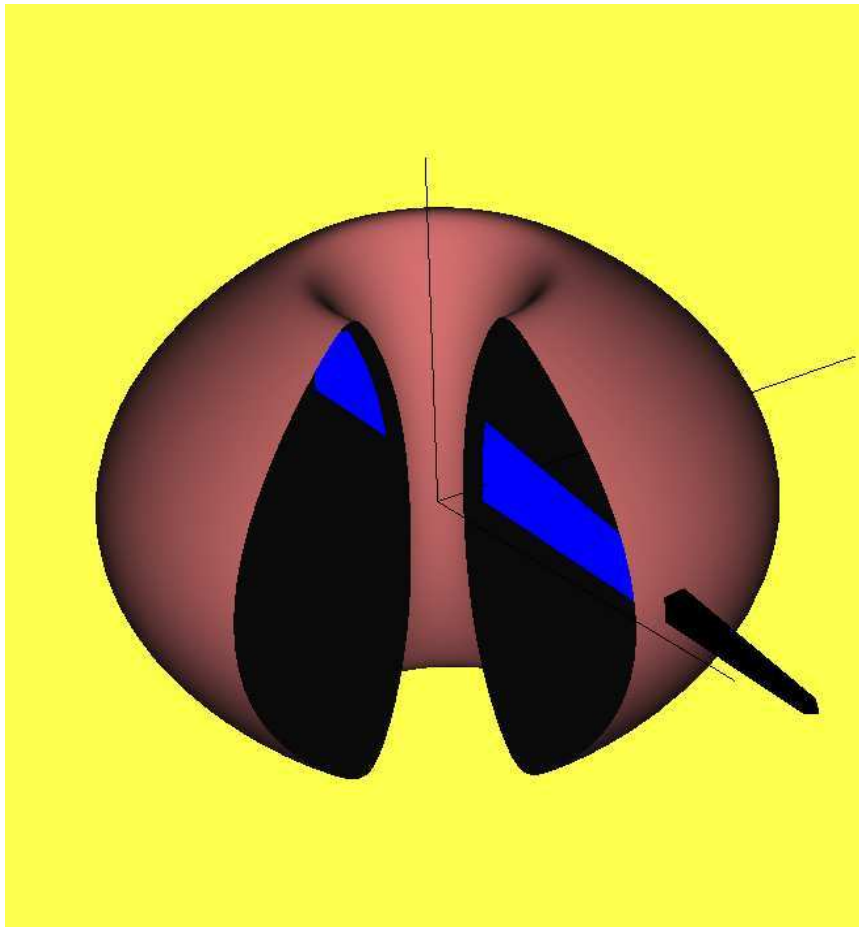
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# 1 The idea of the Lithium Wall Fusion (LiWF)

What will happen if: (a) Neutral Beam Injection (NBI) supplies particles into the plasma core, while (b) a layer of Lithium on the Plasma Facing Surface (PFC) absorbs all particles coming from the plasma ?

*(Assume that Maxwellization is much faster than the particle diffusion.)*



## 1 The idea of the Lithium Wall Fusion (LiWF) (cont.)

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The answer is very simple: because there is no cold particles in the system (other than Maxwellian)

**The plasma temperature will be uniform over entire cross-section. Plasma physics is not involved in this answer.**

Ion/electron temperature gradient instabilities (ITG,ETG) would be eliminated automatically.

In fact, any thermo-conduction losses would be eliminated. Energy from the plasma will be lost only due to particle diffusion.

**For the first time theory reveals a possibility of a regime which is insensitive to anomalous electrons**

In addition, LiWF regime is ELM-free, eliminates the thermo-force driving impurities from PFCs into the plasma, and is consistent with other aspects of a stationary plasma.

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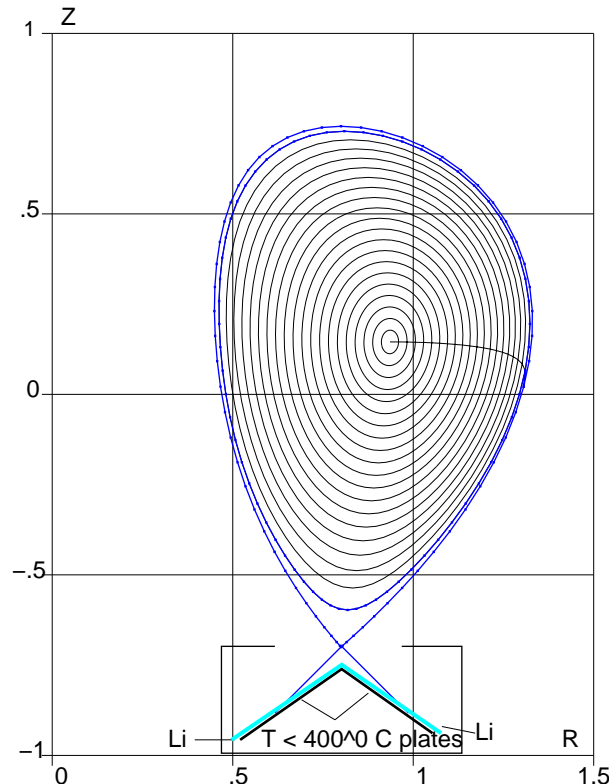
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### 3 The concept of the Pumping Lithium Divertor (PLD)

**PLD  $\equiv$  actively cooled plates with flowing  $h \simeq 0.1$  mm Li layer**



*Gravity, Marangoni effect, residual  $\mathbf{j} \times \mathbf{B}$  forces,*

$$V_g = \frac{\rho g h^2}{2\nu} \sin \theta = 0.049 \sin \theta \text{ [m/s]}, \quad (3.1)$$

$$V_M = \frac{d\sigma(T) h \nabla T}{dT \nu} = 0.8 h \nabla T \text{ [m/s]}$$

*are sufficient for replenishing Li surface.*

*Lithium can accept 5-10 MW/m<sup>2</sup> and keep  $T_{Li} < 400^\circ\text{C}$*

$$\chi_{Li} = 47.6,$$

$$\Delta T \text{ [}^\circ\text{C]} = 100 \frac{q}{4.7} \cdot h \left[ \frac{\text{MW}}{\text{m}^2} \cdot \text{mm} \right]. \quad (3.2)$$

*For any PFC (W,C,Li) power extraction is limited*

**by the coolant temperature,**

*rather than by the temperature of PFC surface.*

**No Li rivers, Li water-falls, evaporation, Li dust, pellets, LiLi trays, meshes, sponges, or thick ( $\geq 1$  mm) Li on the target plate**



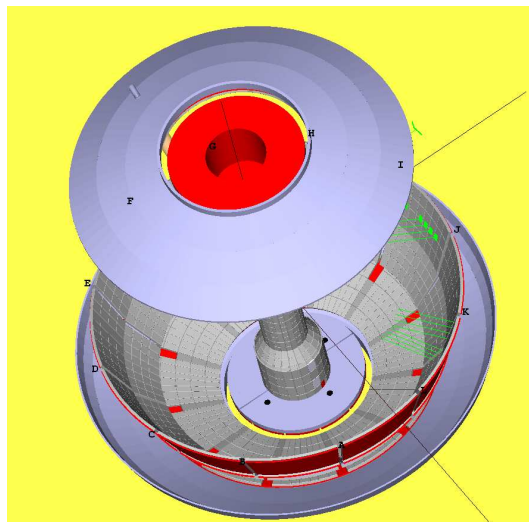
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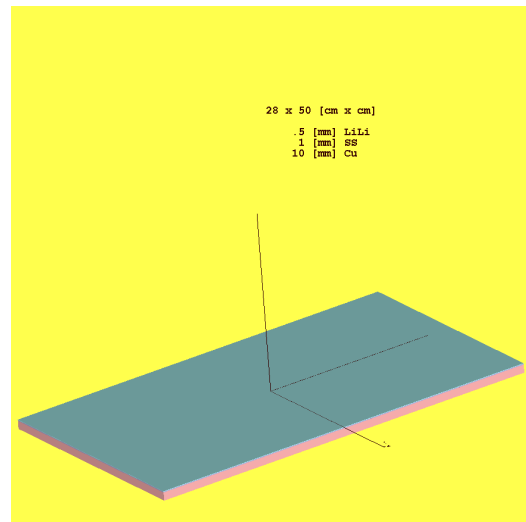
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## 4 Li/SS/Cu plate for NSTX

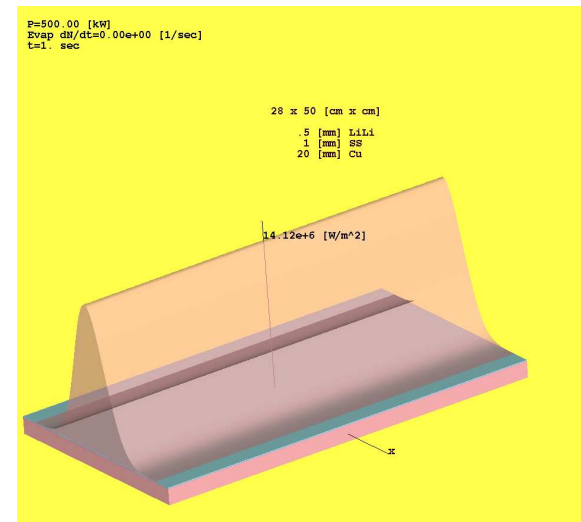
10000 active monolayers or  $\simeq 3\mu m \times 0.75 m^2$  of molten Li, needed for NSTX, can be provided by Lithium Loaded Target Plate (LLTP)



Li coated plate in low inner divertor



Li/SS/Cu (0.5mm/1mm/10mm) sandwich with a trrenched surface



Gaussian (8 cm wide) heat deposition profile

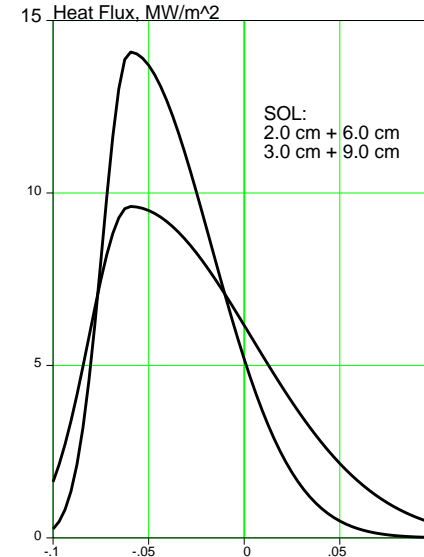
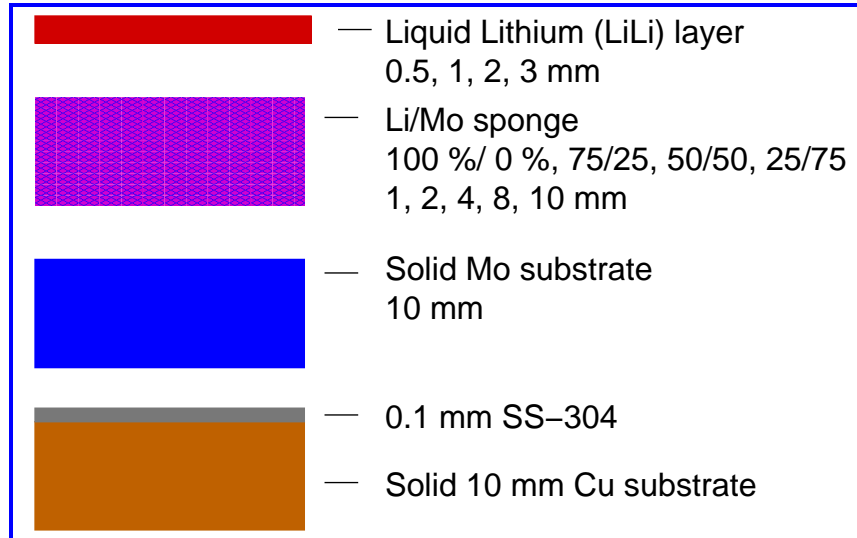
$$S \simeq 0.75 [m^2], \quad L_{SOL,m} = 2.5, \quad V_{Li} \simeq 0.35 [L], \quad M_{Li} \simeq 175 [g],$$

$$V_{Li,cm/sec} = (2 - 5) \cdot B_{tor} \frac{h_{Li,mm}^2}{0.01} \frac{0.1 I_{SoL,MA}}{w_{SOL} I_{ion}}, \quad I_{ion,MA} = \frac{(0.4 - 1) \cdot 10^{-3}}{1.6} \quad (4.1)$$

The simple Li/SS/Cu plate could be a real first step toward  
**PLD and LiWF regime**

## 4 Li/SS/Cu plate for NSTX (cont.)

Both Liquid Lithium (LiLi) and Li/Mo CPS were considered



Heat flux profile from the SOL

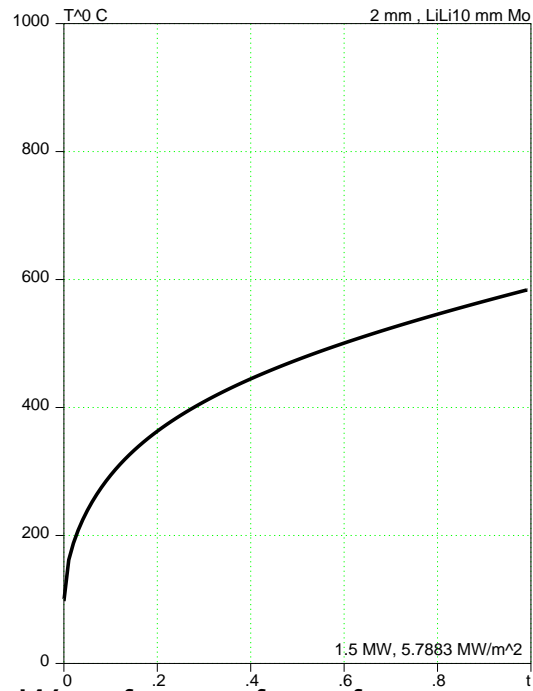
$$Q_{SOL} = Q_0 \exp \left[ - \left( \frac{x - x_0}{d(x)} \right)^2 \right], \quad \begin{cases} d = d_{out}, & x \geq x_0 \\ d = d_{in}, & x < x_0 \end{cases} \quad (4.2)$$

Characteristic scale lengths, mm

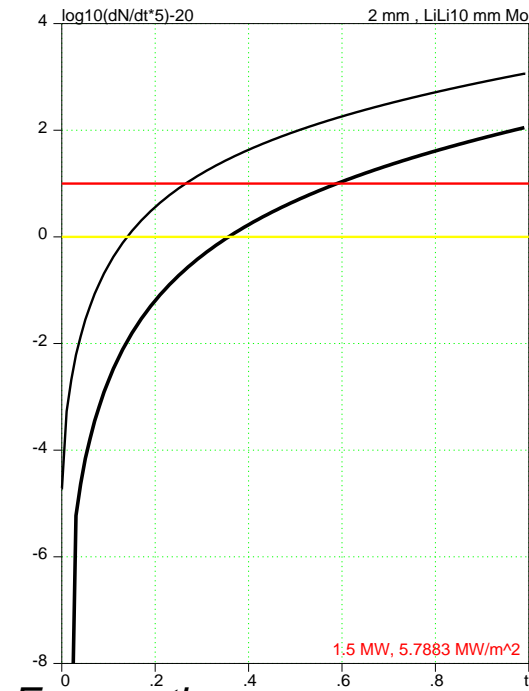
$d_{in}$	$d_{out}$	$\Delta_{LiLi}$	$\Delta_{Li/Mo}$	$\Delta_{SS}$	$\Delta_{Mo,Co}$	Li/Mo CPS
20,30	$3d_{in}$	0.5, 1,2,3	1,2,4,8,10	.1	10	4/0, 3/1, 2/2, 1/3, 0/4

## 4 Li/SS/Cu plate for NSTX (cont.)

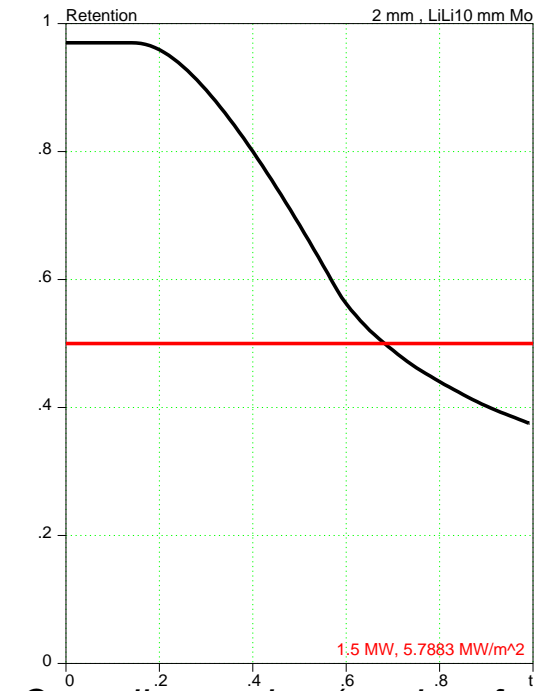
3-D Cbebm code (written for Marangoni effect) is used to simulate heating of Li surface



Waveform of surface temperature  $T_{Li}$



Evaporation  $\log_{10}(dN/dt) - 20$



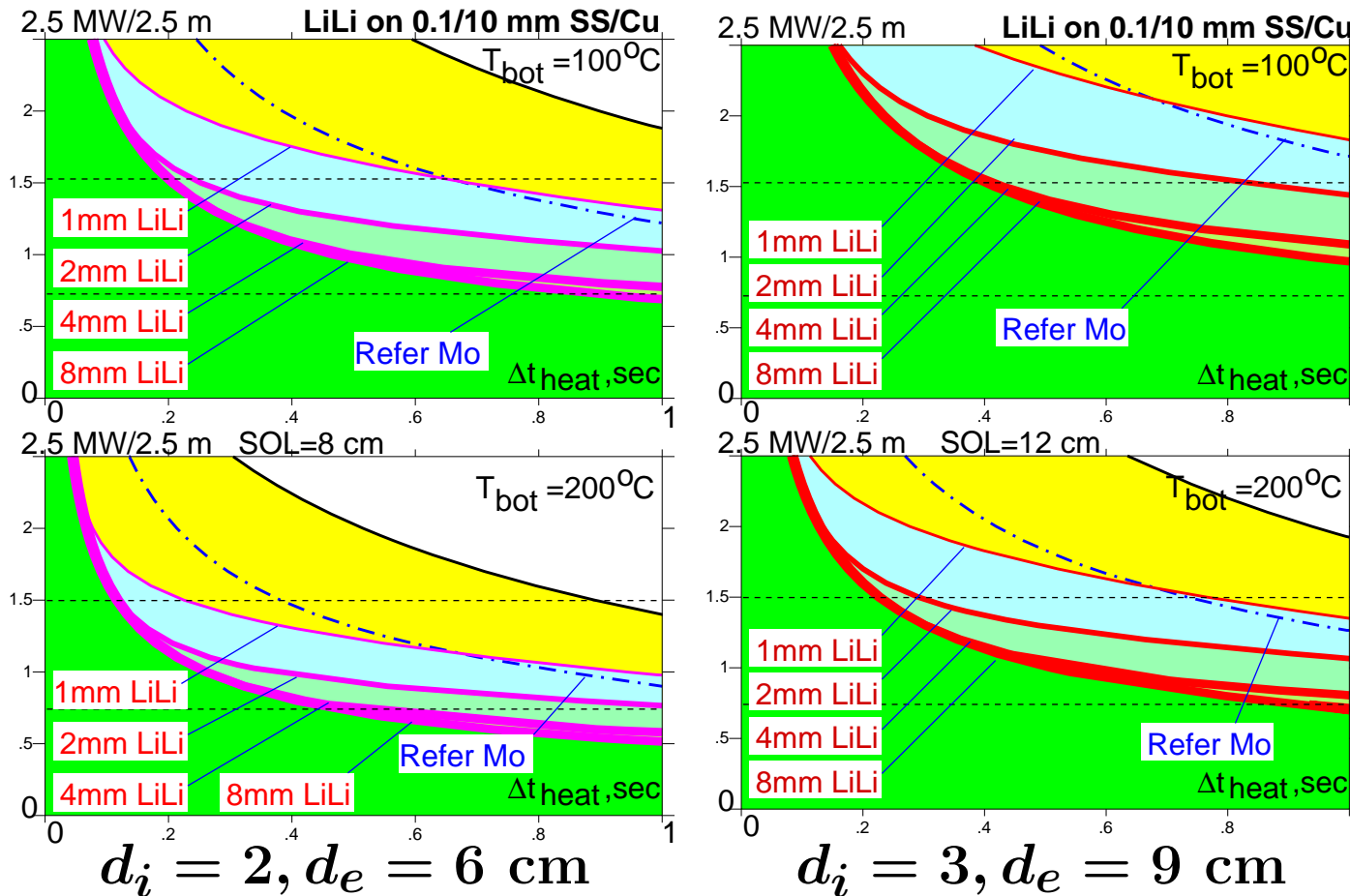
Overall retention (no data for  $T > 350^\circ C$ )

Evaporation limit,  $dN/dt \leq 10^{21}/sec$ , determines the operational

space  $P_{NBI}$  vs  $\Delta t_{NBI}$

## 4 Li/SS/Cu plate for NSTX (cont.)

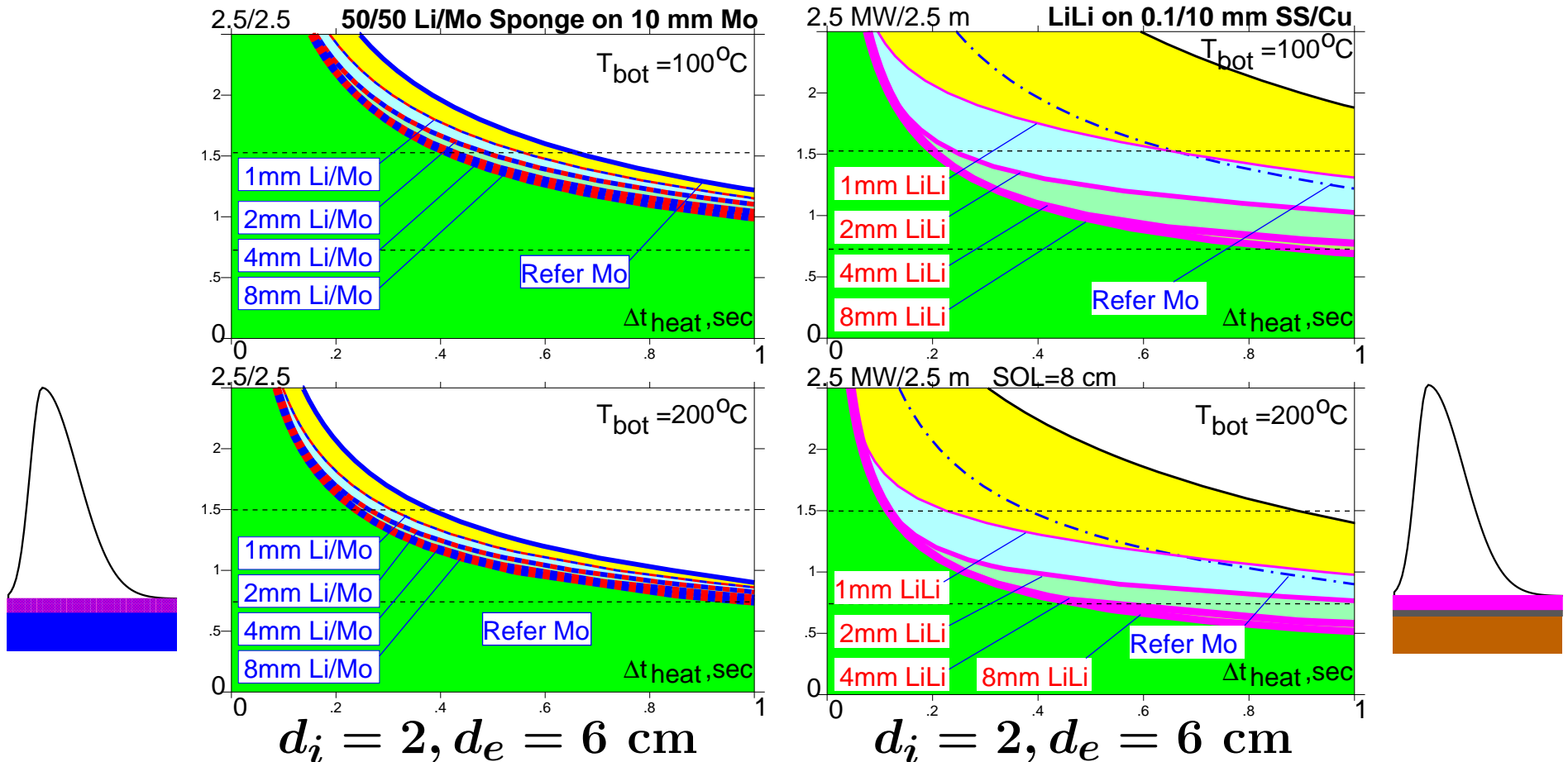
The plate 0.1-1 mm of Li on 0.1/10 SS/Cu provides the operational space for LiWall regime in NSTX



The heat flux profile in the SOL is a crucial unknown

## 4 Li/SS/Cu for NSTX (cont.)

**1/0.1/10 mm Li/SS/Cu plate outperforms 10 mm Li/Mo CPS**



**The plate also has fewer technology unknowns**

## 4 Li/SS/Cu plate for NSTX (cont.)

**Even short term experiments with LLTR can provide initial information on**

1. *effects of wetting, wicking, adhesion of Li with large metal surfaces in the plasma environment,*
2. *rate of passivation of Li surface in a specific NSTX device with C-walls*
3. *electric currents in the SOL*

The goal of experiments with LLTP is limited (1-2 campaigns), realistic and well specified:

1. *To clarify the system compatibility with molten Li using a simple Lithium Loaded Target Plate*
2. *To reproduce the T-11M (1998) level of plasma pumping using the LLTP in divertor configuration.*
3. *To collect sufficient information for redesigning the divertor area of NSTX for a long lasting PLD and other aspects of a LiWF regime.*

This approach will pave a way for

**Conversion of NSTX into ST0 in order to demonstrate the feasibility of the LiWF regime, by achieving  $\tau_{E,ST0} > 2\tau_{E,NSTX}$**

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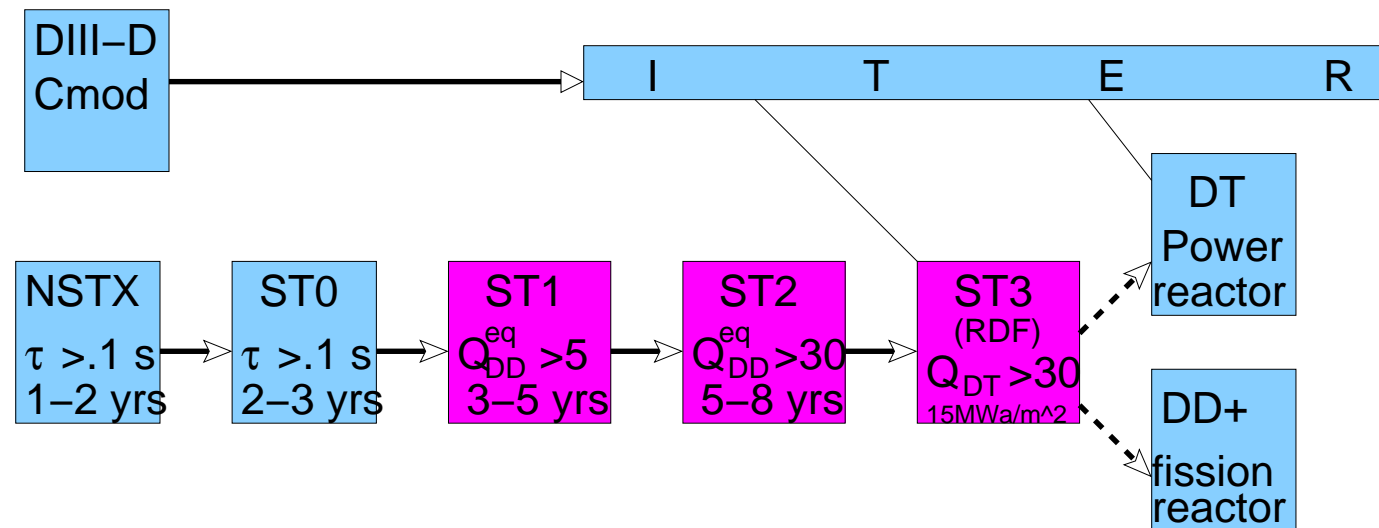
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## 5 Summary

**Installation of LLTP plate gives NSTX a chance to contribute to the Orbach/Bodman initiative for domestic fusion**

NSTX, and then ST0, are well suited to motivate the 3 Step Program for a Reactor Development Facility, with a clear strategic role for STs.



**The success of ST0 would be crucial for bootstrapping funding for domestic fusion and the ST program**

**The next ST1 machine ( $B = 1.5$  T,  $I_{pl} = 3 - 4$  MA,  $R_{outer} = 1.65$  m) can reach the ignition level of  $nT\tau_E$**