

# Resistive Wall Mode Stabilization to Sustain High Normalized Beta at Low Internal Inductance in NSTX

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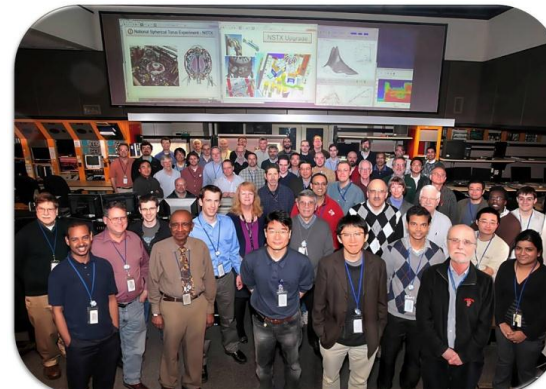
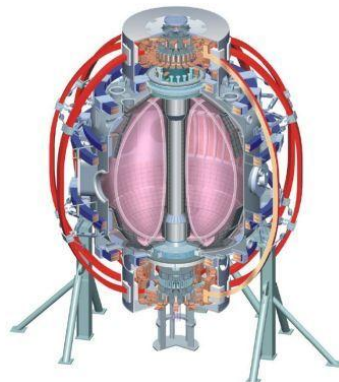
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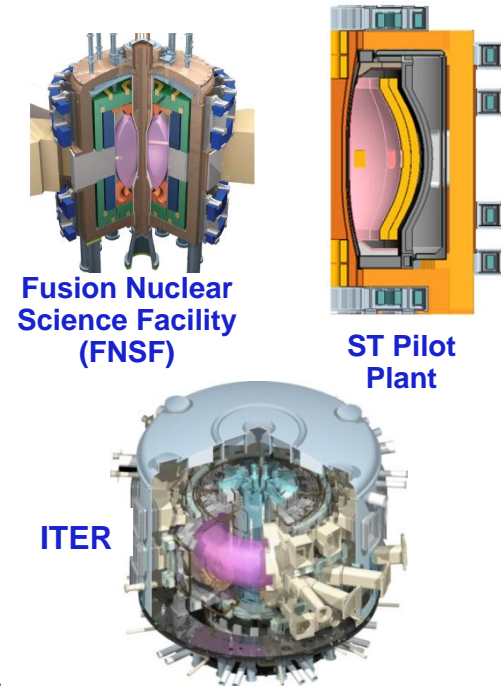
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IPP, Garching  
ASCR, Czech Rep

# NSTX is Addressing Global Stability Needs for Maintaining Low $I_i$ , High Beta Plasmas for Fusion Applications

## □ Motivation

- Maintain high  $\beta_N$  stability, validate predictive and control capability to allow confident extrapolation to ST fusion applications and ITER



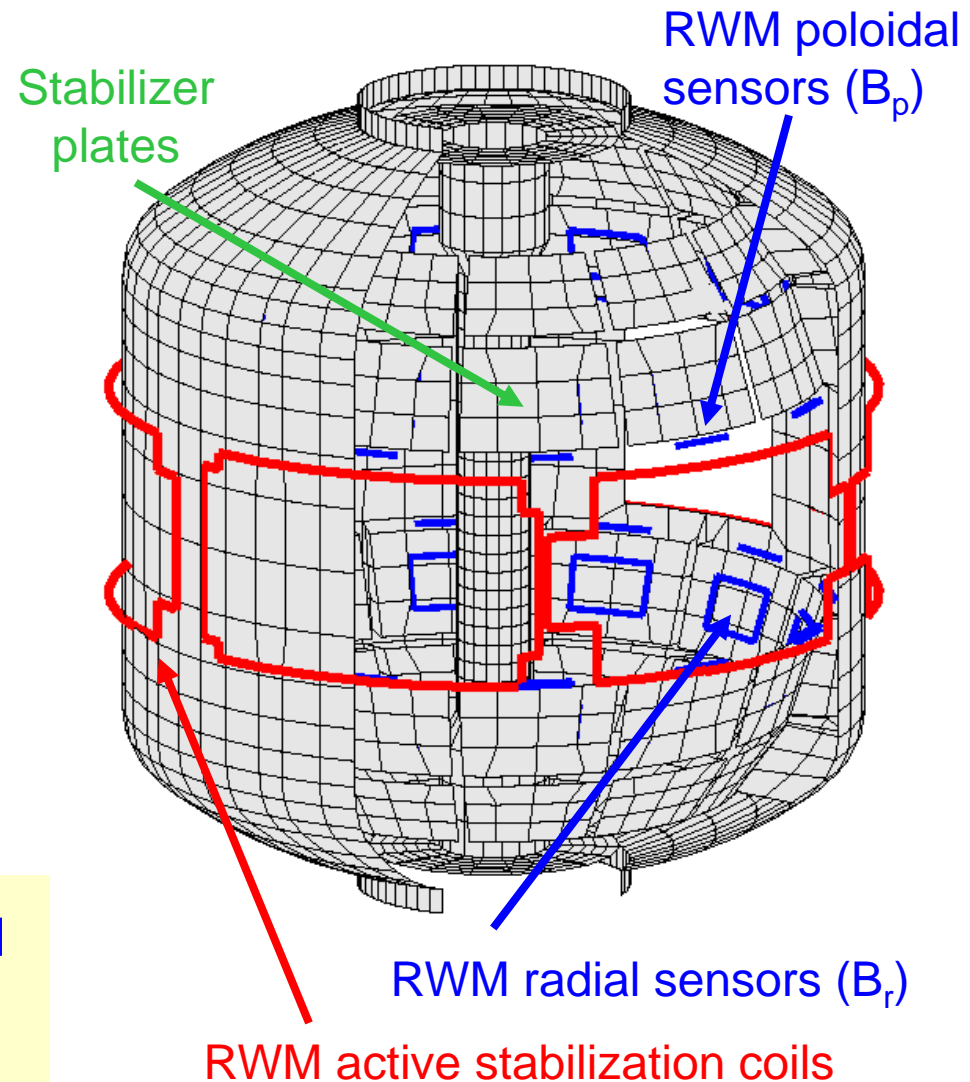
## □ Outline

- Resistive wall mode stabilization at low internal inductance,  $I_i$
- Analysis of RWM passive stability at low  $I_i$
- RWM active control advances to improve stabilization
- Model-based RWM state space controller use

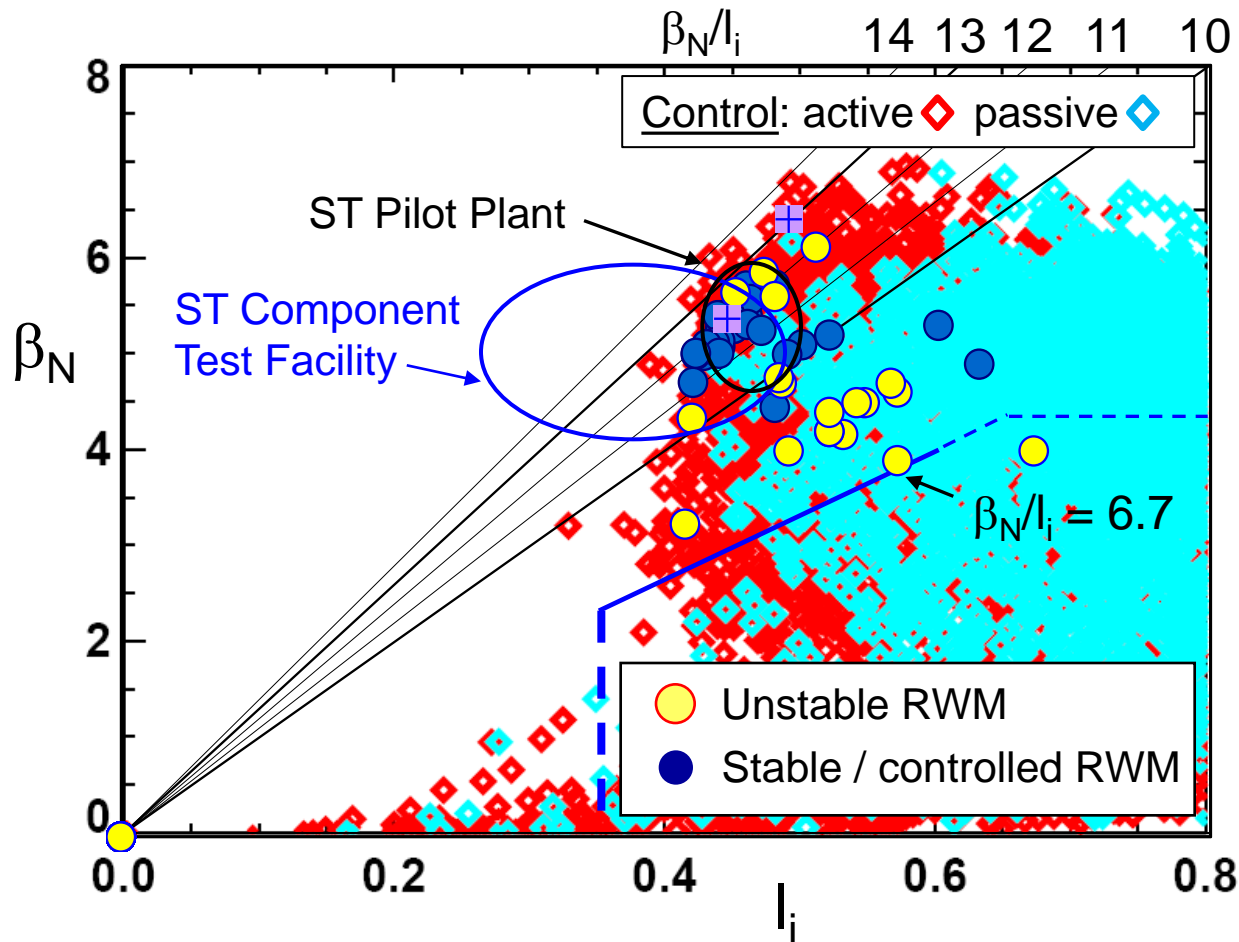
# NSTX is a spherical torus equipped to study passive and active global MHD control

- ❑ High beta, low aspect ratio
  - ❑  $R = 0.86$  m,  $A > 1.27$
  - ❑  $I_p < 1.5$  MA,  $B_t = 5.5$  kG
  - ❑  $\beta_t < 40\%$ ,  $\beta_N > 7$
- ❑ Copper stabilizer plates for kink mode stabilization
- ❑ Midplane control coils
  - ❑  $n = 1 - 3$  field correction, magnetic braking of  $\omega_\phi$  by NTV
  - ❑  $n = 1$  RWM control
- ❑ Combined sensor sets now used for RWM feedback
  - ❑ 48 upper/lower  $B_p$ ,  $B_r$

## 3D Structure Model



# Improvements in stability control techniques significantly reduce unstable RWMs at low $I_i$ and high $\beta_N$



## Initial experiments

- 48% disruption probability by RWM

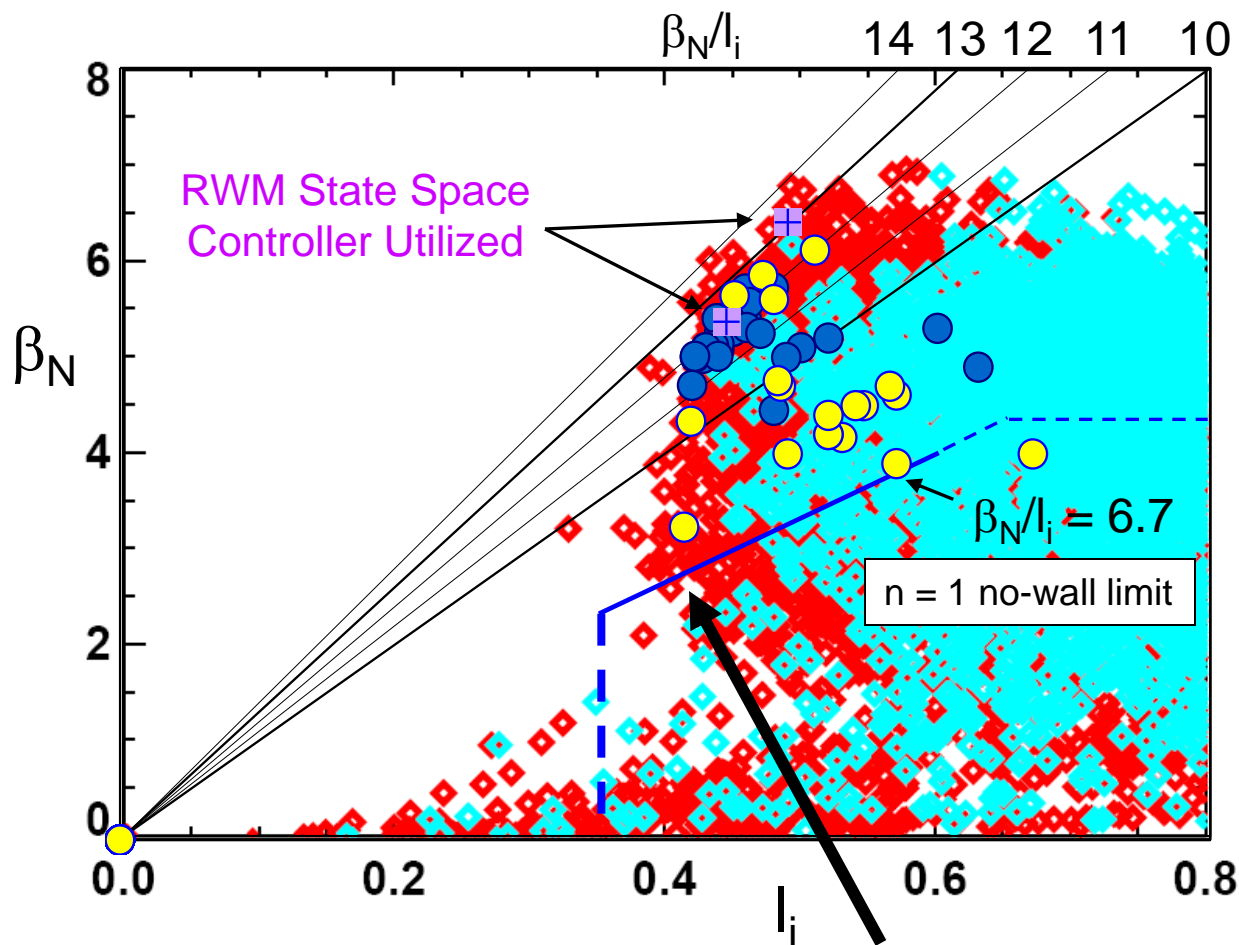
## Experiments with control enhancements

- Significantly reduced disruption probability with control enhancements
  - 14% of cases with  $\beta_N/I_i > 11$

## Plasma internal inductance ( $I_i$ ):

- Integral measure of the peakedness of the current profile
- Low  $I_i$  typical of non-inductive operation, and at high  $\kappa$  (for vertical stability)

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- Much higher probability of unstable RWMs at lower  $\beta_N$ , why??

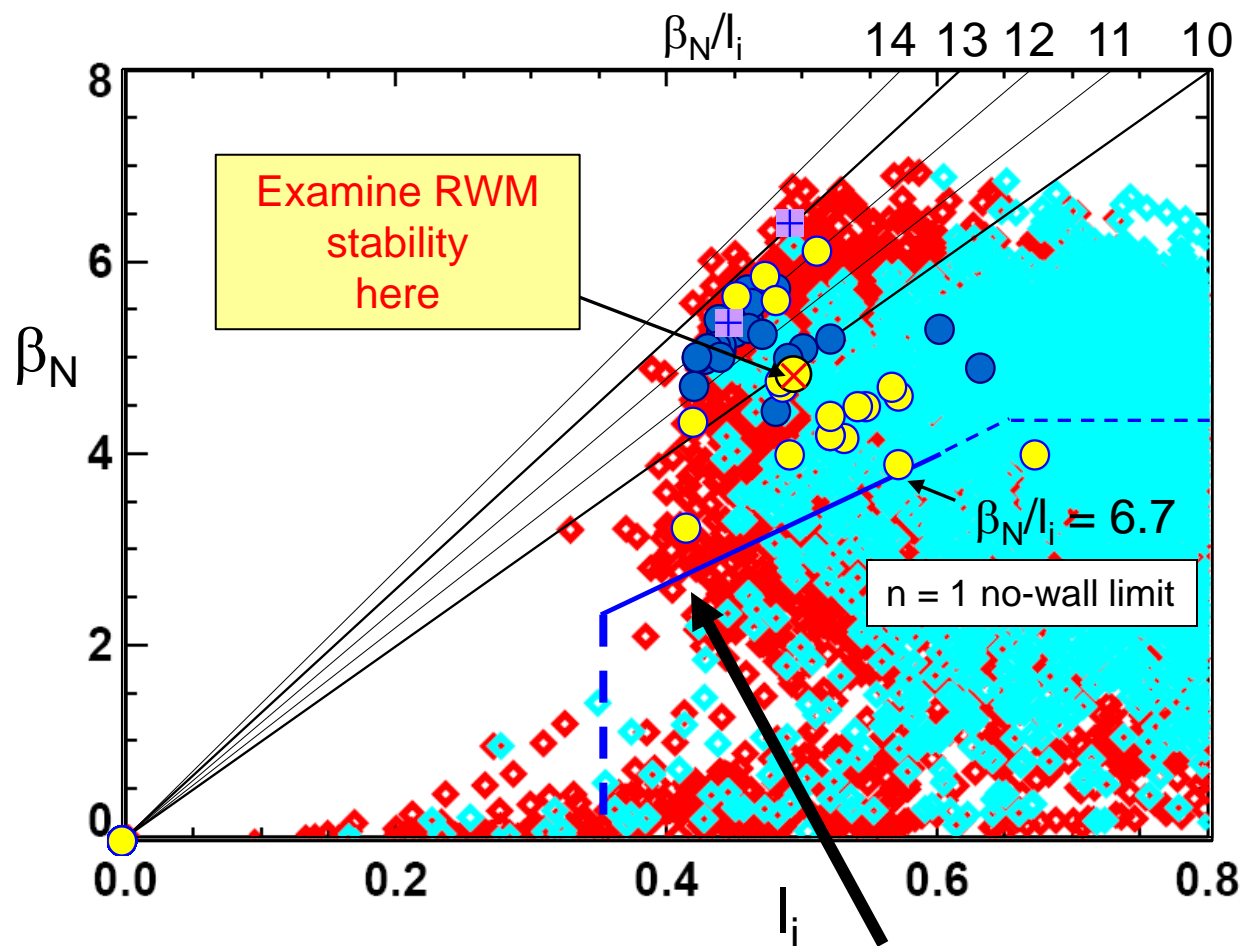
- Computed  $n = 1$  no-wall limit  $\beta_N/I_i \sim 6.7$  (low  $I_i$  range 0.4 – 0.6)

- Synthetic equilibria variation:  $n = 1$  no-wall unstable at all  $\beta_N$  at  $I_i \leq 0.38$  (current-driven kink limit)

- significant for NSTX-U, next-step ST operation



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Computed n = 1 no-wall limit  $\beta_N/I_i \sim 6.7$  (low  $I_i$  range 0.4 – 0.6)

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# Kinetic stability calculations show reduced stability in low $I_i$ target plasma as $\omega_\phi$ is reduced, RWM becomes unstable

## □ Stability evolves

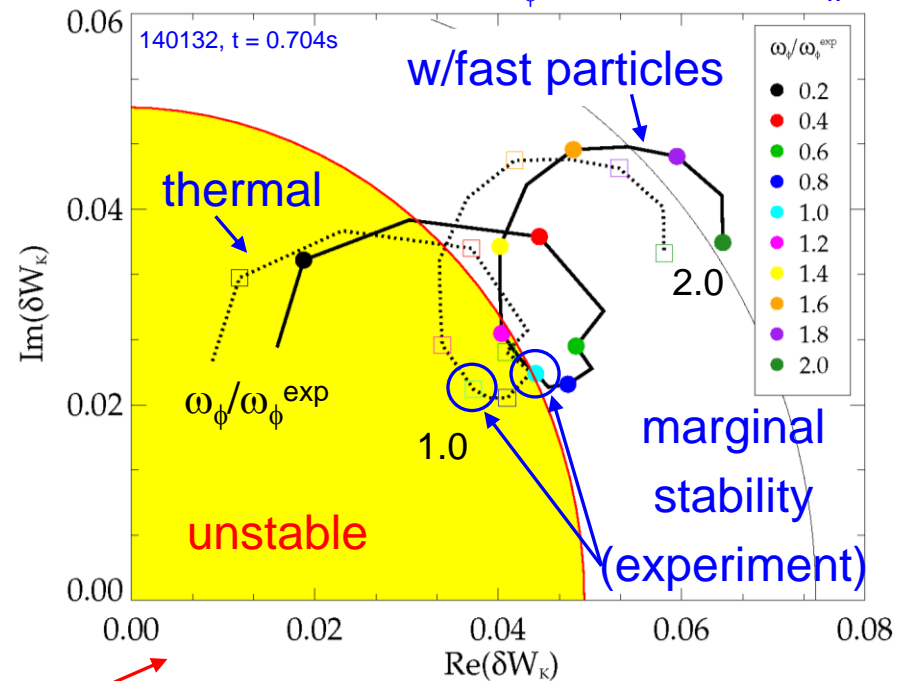
- Computation shows stability at time of minimum  $I_i$
- Region of reduced stability vs.  $\omega_\phi$  found before RWM becomes unstable ( $I_i = 0.49$ )

## □ Quantitative agreement between theory/experiment

- MISK, MARS-K, HAGIS codes being benchmarked (ITPA)
- MISK calculation of  $\omega_D$  improved

- Agreement between theory/experiment improved
- Best agreement with fast particle effects included

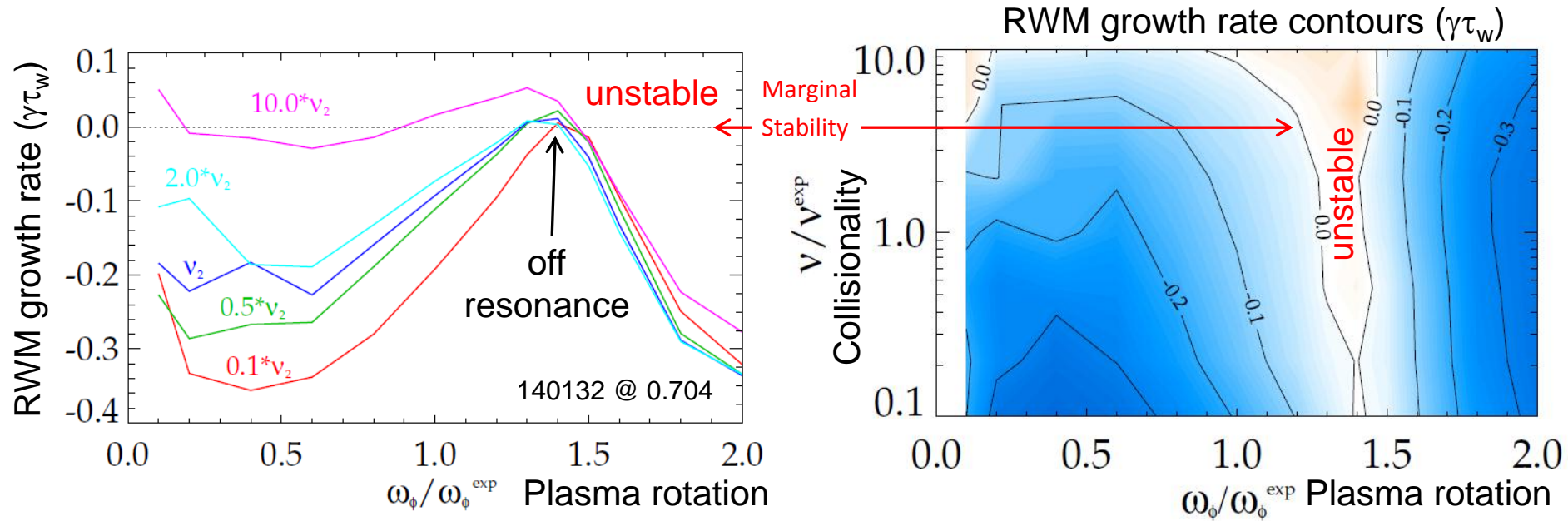
MISK code RWM stability vs.  $\omega_\phi$  (contours of  $\gamma\tau_w$ )



(more quantitative comparison to theory)

- J.W. Berkery, et al., PRL **104** (2010) 035003
- S.A. Sabbagh, et al., NF **50** (2010) 025020
- J.W. Berkery, et al., Phys. Plasmas **17**, 082504 (2010)
- S.A. Sabbagh, et al., IAEA FEC 2010, Paper EXS/5-5

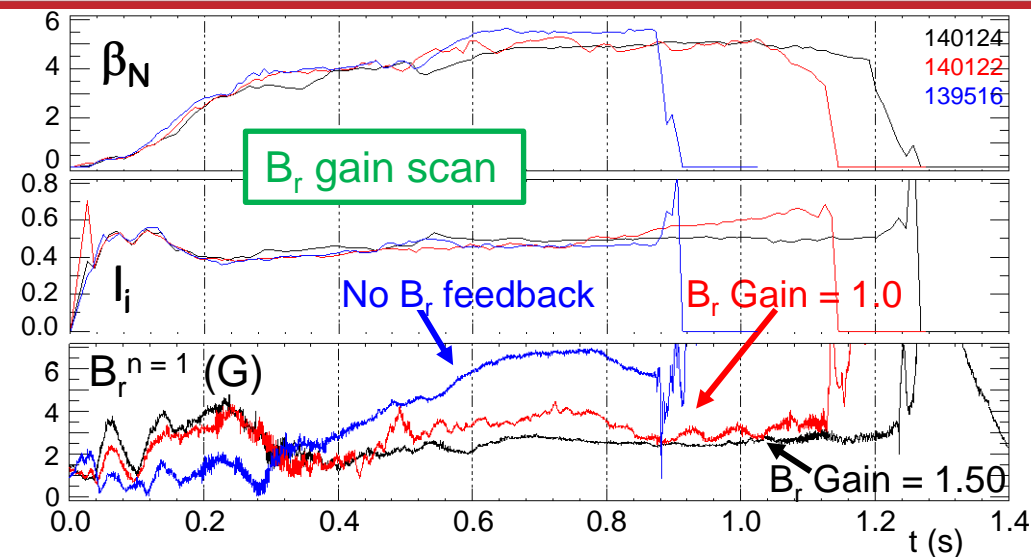
# Reduced collisionality ( $\nu$ ) is stabilizing for resistive wall modes, but only near kinetic resonances



- ❑ NSTX-tested kinetic RWM stability theory: 2 competing effects at lower  $\nu$ 
  - ❑ Stabilizing collisional dissipation reduced (expected from early theory)
  - ❑ Stabilizing resonant kinetic effects enhanced (contrasts early RWM theory)
- ❑ Expectations in NSTX-U, tokamaks at lower  $\nu$  (e.g. ITER)
  - ❑ Stronger stabilization near  $\omega_\phi$  resonances; almost no effect off-resonance
  - ❑ Plasma stability gradient vs. rotation increases
    - important to avoid unfavorable rotation, suppress transient RWM with active control

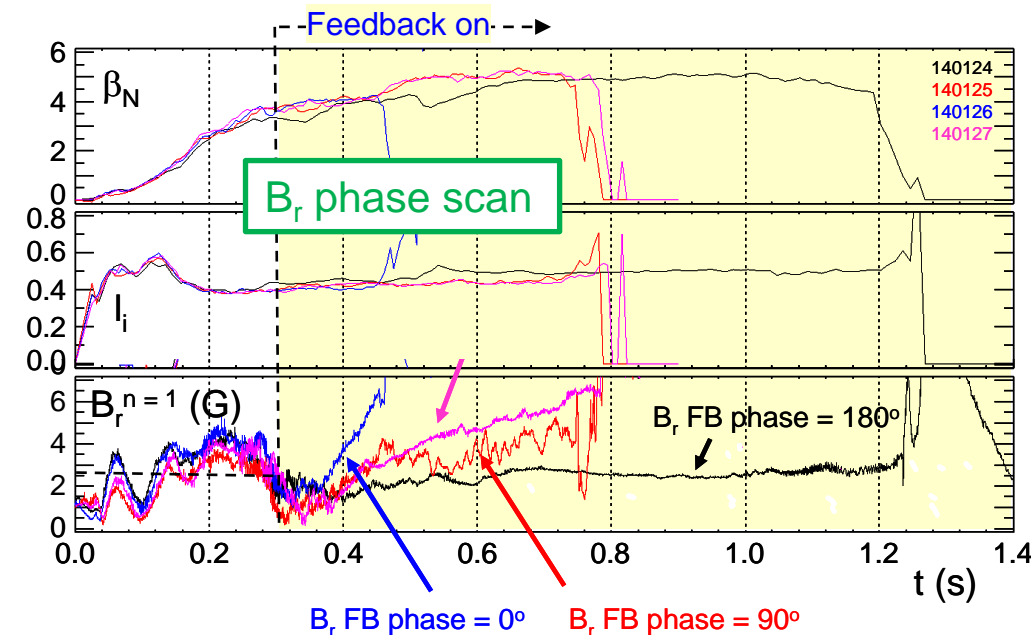
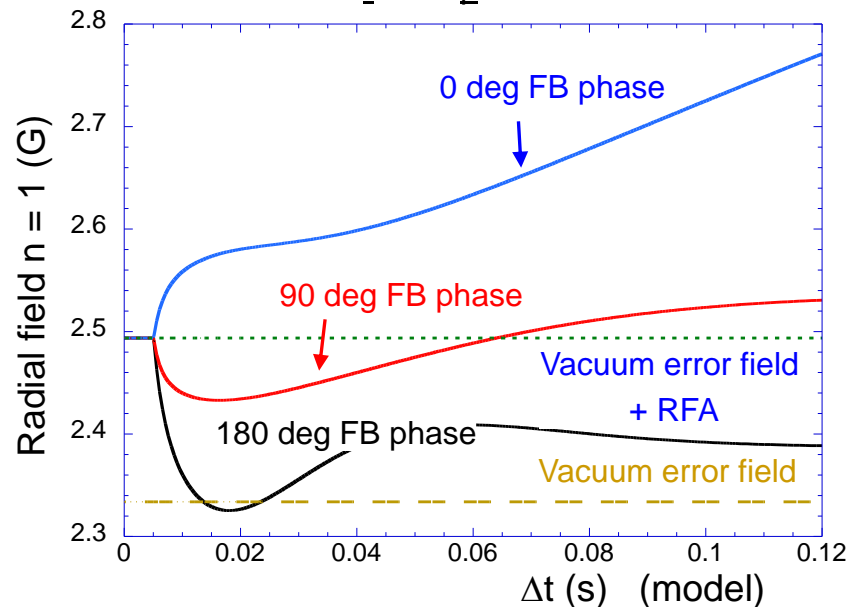


# Active Control: combined $B_r + B_p$ sensor feedback gain and phase scans produce significantly reduced $n = 1$ field

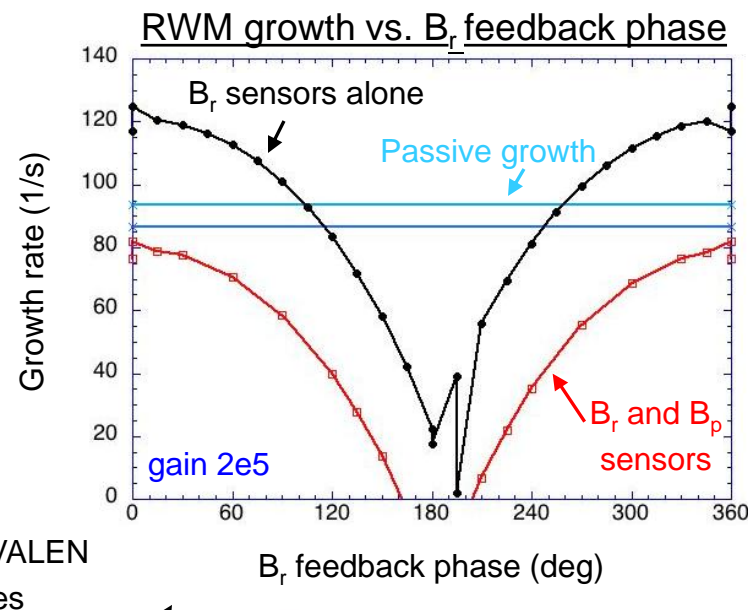
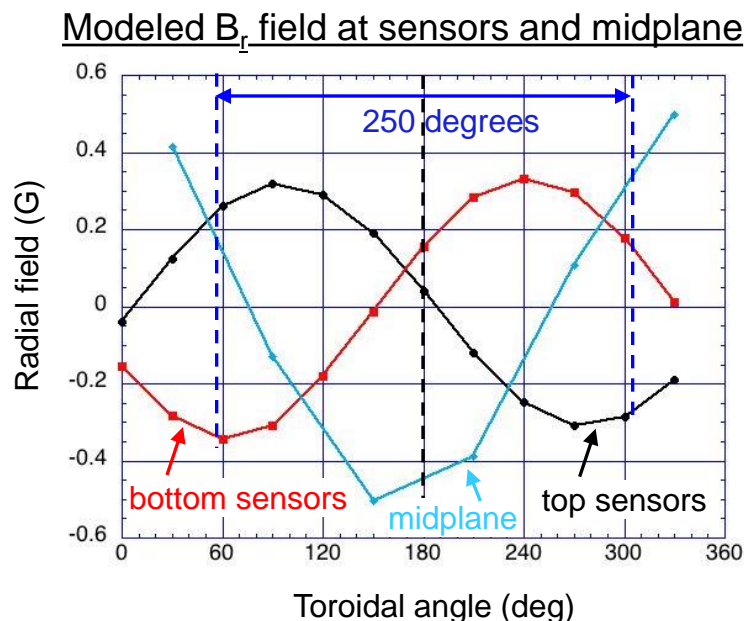


- Favorable  $B_p + B_r$  feedback (FB) settings found (low  $I_i$  plasmas)
  - Fast RWM growth  $\sim 2 - 3$  ms control by  $B_p$
  - $B_r$  feedback controls slower ( $\sim 10$  ms)  $n=1$  field amplification, modes
- Time-evolved theory simulation of  $B_r + B_p$  feedback follows experiment

## Simulation of $B_r + B_p$ control (VALEN)



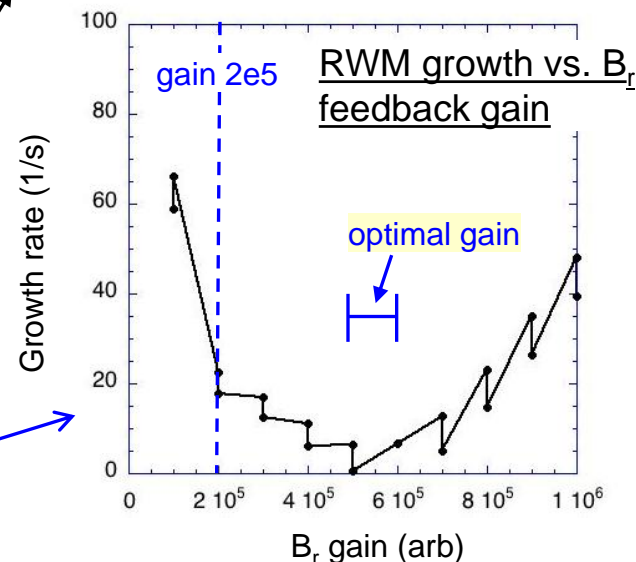
# RWM feedback using upper/lower $B_p$ and $B_r$ sensors modeled and compared to experiment



## Both $B_r$ , $B_p$ feedback contribute to active control

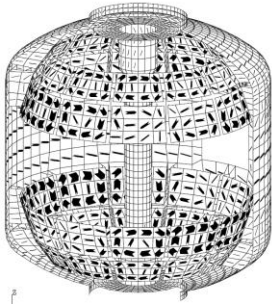
- $B_r$  mode structure and optimal feedback phase agrees with parameters used in experiment
- $B_r$  feedback alone provides stabilization for growth times down to  $\sim 10$  ms with optimal gain

## Theory shows optimal feedback phase used in experiments; gain used is near optimal



# Model-based RWM state space controller including 3D model of plasma and wall currents used at high $\beta_N$

Full 3-D model ~3000+ states

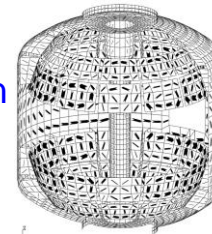


Balancing transformation

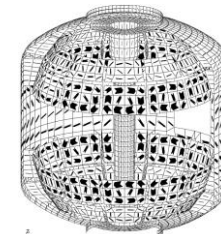
State reduction (< 20 states)

RWM eigenfunction (2 phases, 2 states)

$(\hat{x}_1, \hat{x}_2)$



$\hat{x}_3$



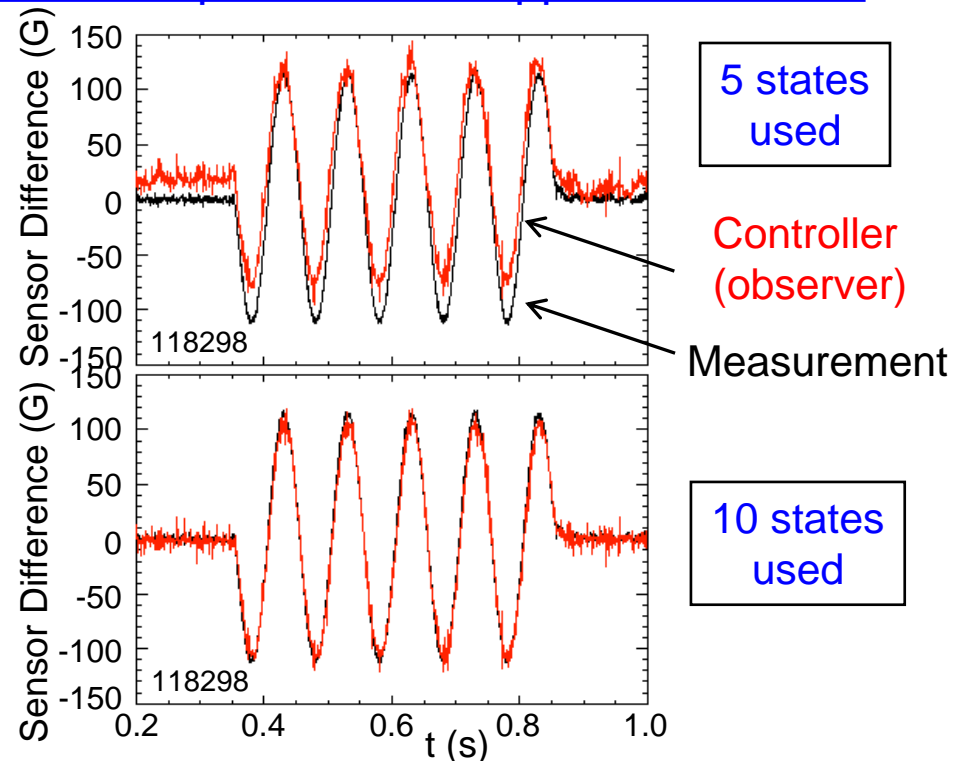
$\hat{x}_4$

...

- Controller models, can compensate for wall currents
  - Including mode-induced current
- Potential to allow more flexible control coil positioning
  - May allow control coils to be moved further from plasma, and be shielded (e.g. for ITER)
- Straightforward inclusion of multiple modes ( $n > 1$ ) in feedback

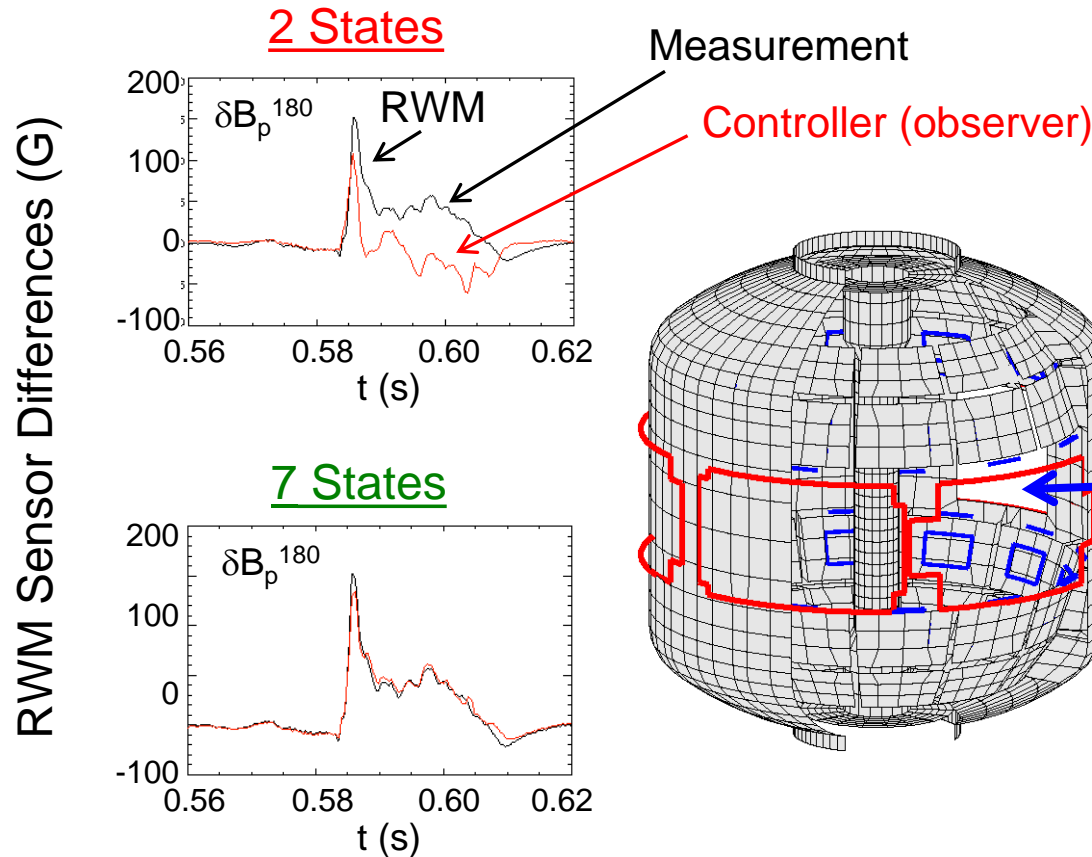
Katsuro-Hopkins, et al., NF 47 (2007) 1157

Controller reproduction of applied  $n = 1$  field

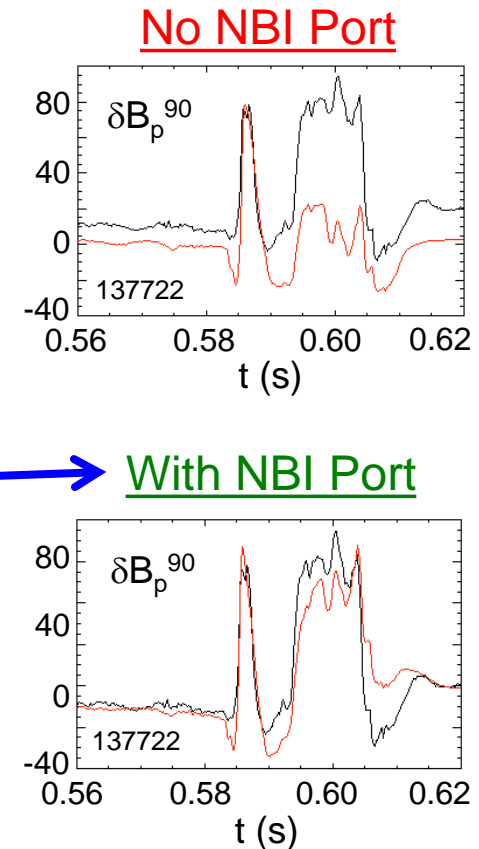


# Open-loop comparisons between sensor measurements and state space controller show importance of states and model

## A) Effect of Number of States Used



## B) Effect of 3D Model Used

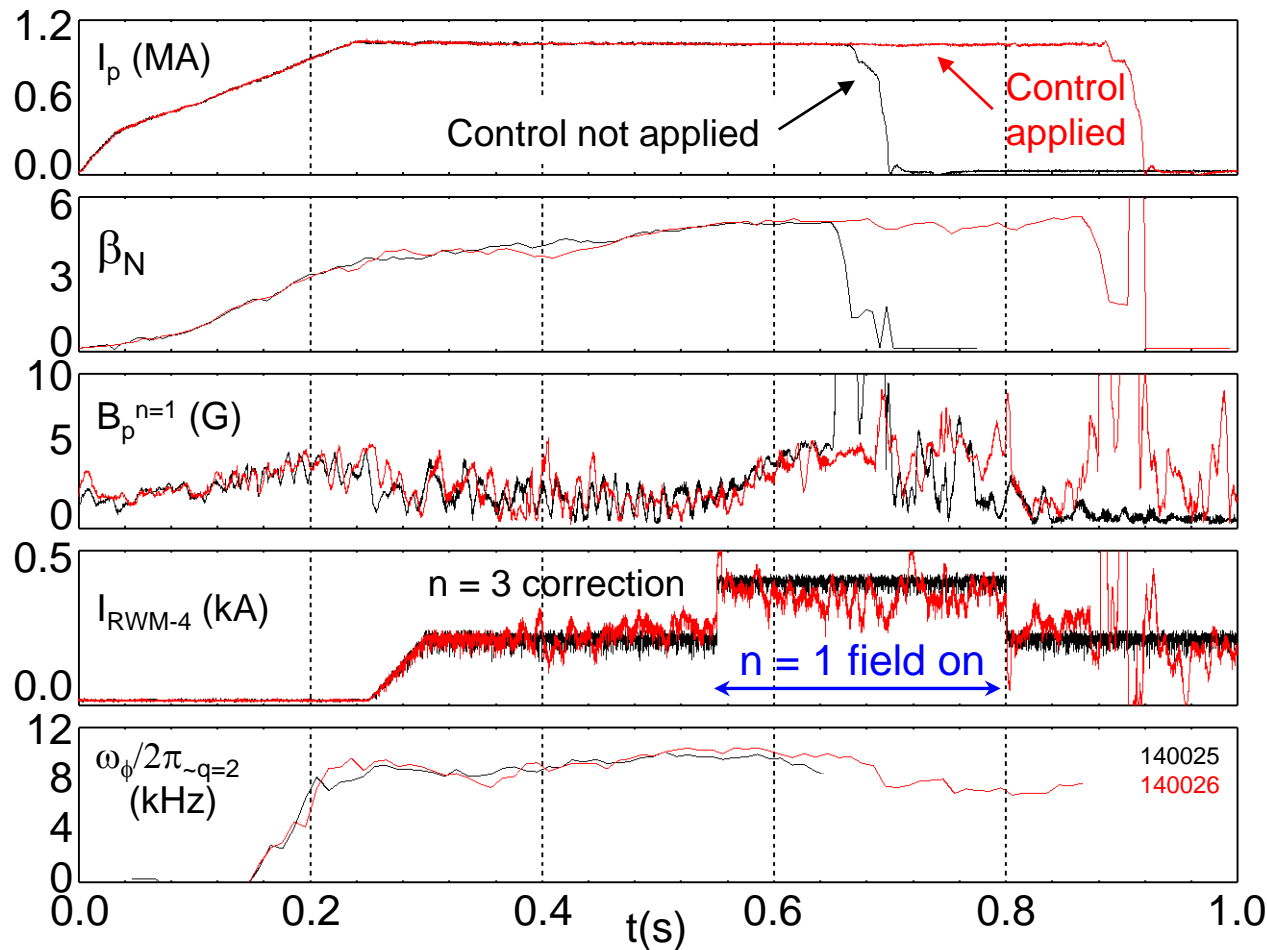


Improved agreement with sufficient number of states (wall detail)

3D detail of model important to improve agreement

# RWM state space controller sustains otherwise disrupted plasma caused by DC $n = 1$ applied field

## RWM state space feedback (12 states)



### □ $n = 1$ DC applied field test

- Generate resonant field amplification, disruption
- Use of RWM state space controller sustains discharge

### □ RWM state space controller sustains discharge at high $\beta_N$

- Best feedback phase produced long pulse,  $\beta_N = 6.4$ ,  $\beta_N/I_i = 13$



# NSTX is Addressing Global Stability Needs Furthering Steady Operation of High Performance ST / Tokamak Plasmas

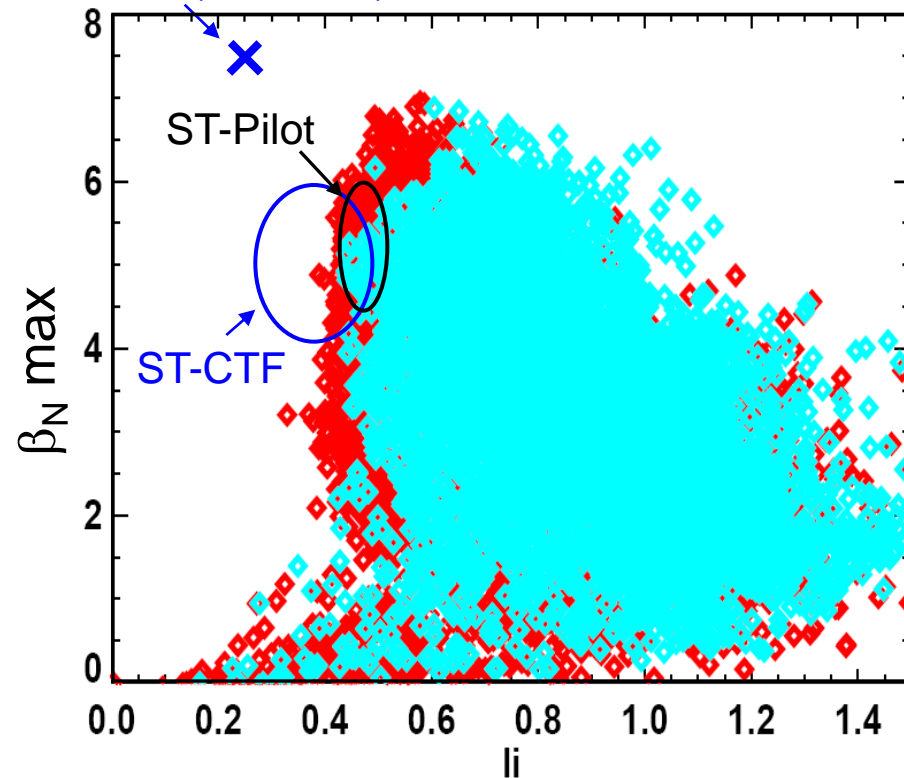
- ❑ Significant reduction in disruption probability in high  $\beta_N$  plasmas with reduced  $I_i$
- ❑ Quantitative agreement between RWM marginal stability and kinetic stabilization theory for low  $I_i$ , high  $\beta_N$  plasmas
- ❑ Use of combined  $B_r + B_\theta$  RWM sensor  $n=1$  feedback improves reduction of  $n=1$  field amplitude, improved stability
- ❑ RWM state space controller sustains low  $I_i$ , high  $\beta_N$  plasma
  - ❑ Potential for greater flexibility of RWM control coil placement and shielding in future burning plasma devices (e.g. FNSF, ITER)

# Supporting Slides

# Operation has aimed to produce sustained low $I_i$ and high pulse-averaged $\beta_N$

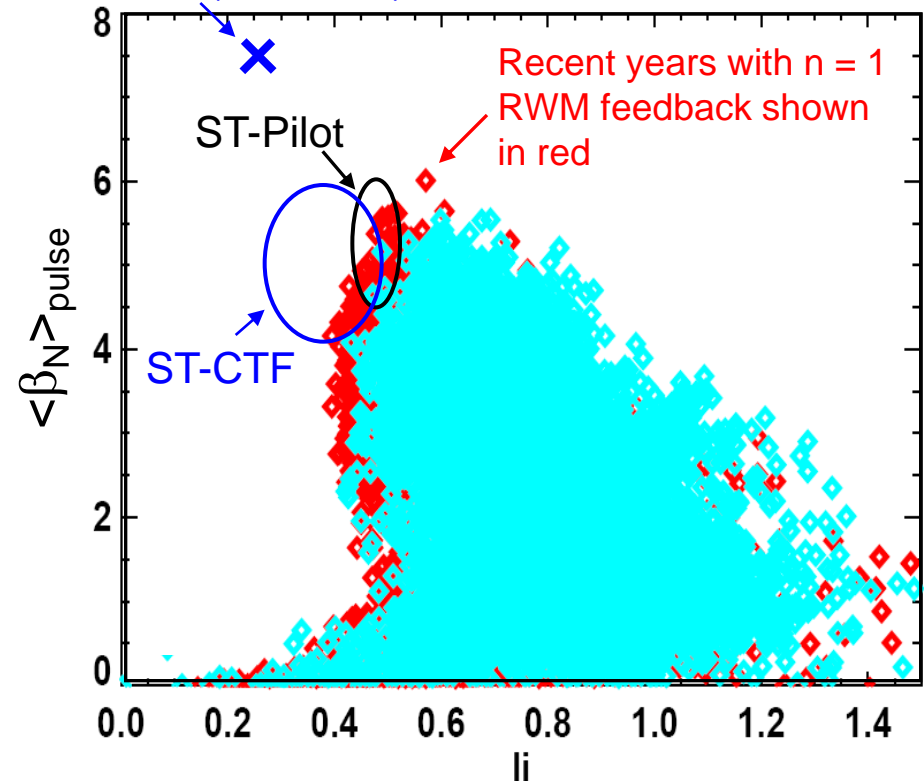
$\beta_N$  vs.  $I_i$  (maximum values)

ST-DEMO (ARIES-ST)



$\beta_N$  vs.  $I_i$  (pulse-averaged values)

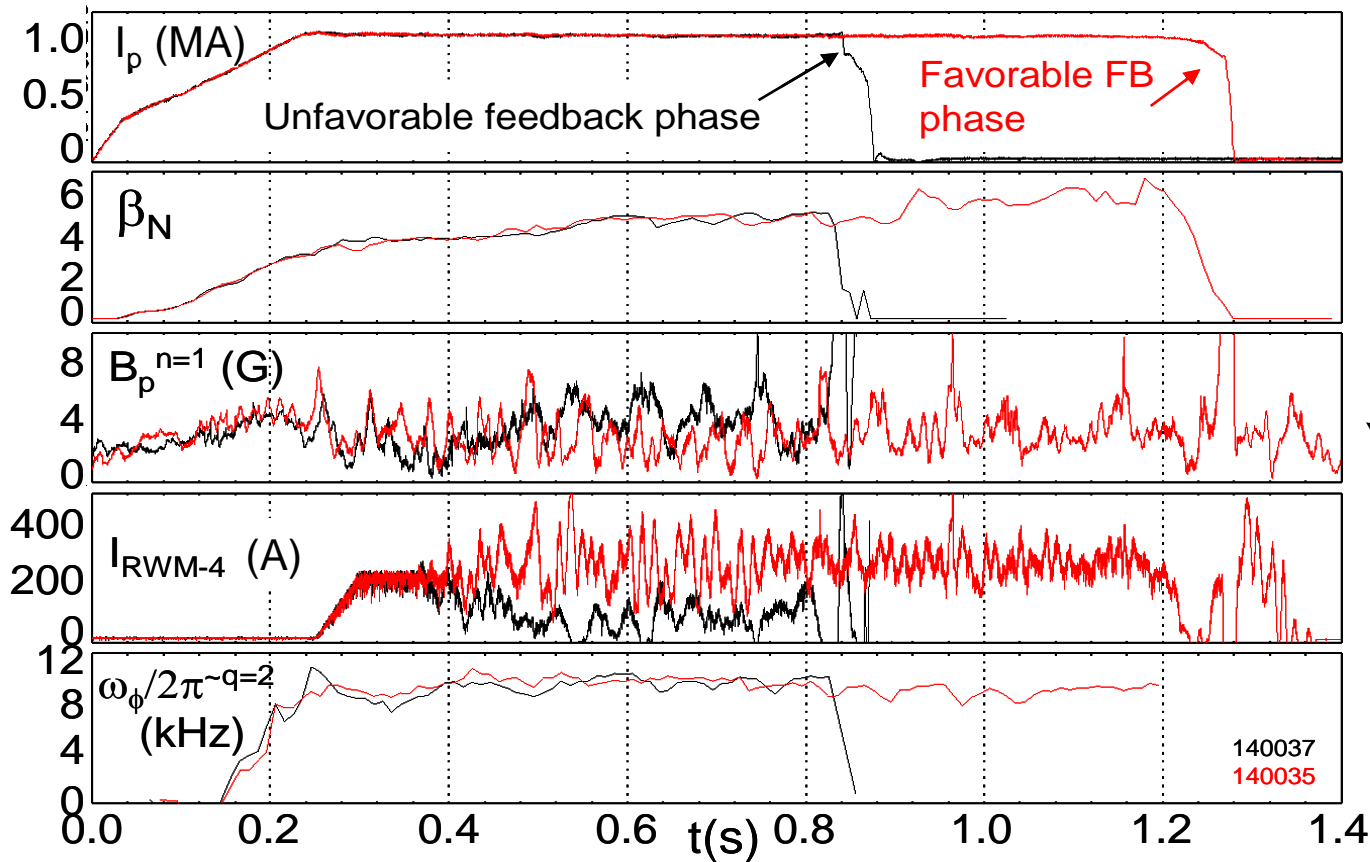
ST-DEMO (ARIES-ST)



- Plasmas have begun to reach low  $I_i$  and high  $\langle \beta_N \rangle_{\text{pulse}}$  suitable for next-step ST fusion devices
  - Some parameters (e.g. elongation  $> 3$ ) still need to be reached self-consistently

# NSTX RWM state space controller sustains high $\beta_N$ , low $I_i$ plasma

## RWM state space feedback (12 states)

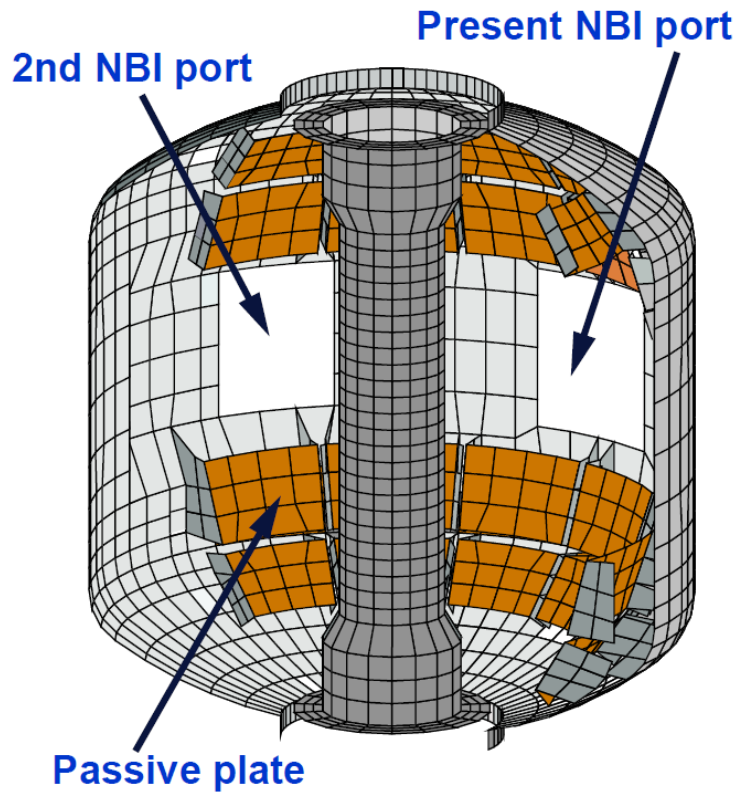


Successful First Experiments

- Feedback phase scan
- Best feedback phase produced long pulse,  $\beta_N = 6.4$ ,  $\beta_N/I_i = 13$

# Second NBI beam port in NSTX-U makes a small difference in with-wall limit

VALEN model of NSTX Upgrade passive conducting structure



VALEN computed RWM growth rate vs.  $\beta_N$

