

# Impact of convective transport on ITER edge-plasma properties\*

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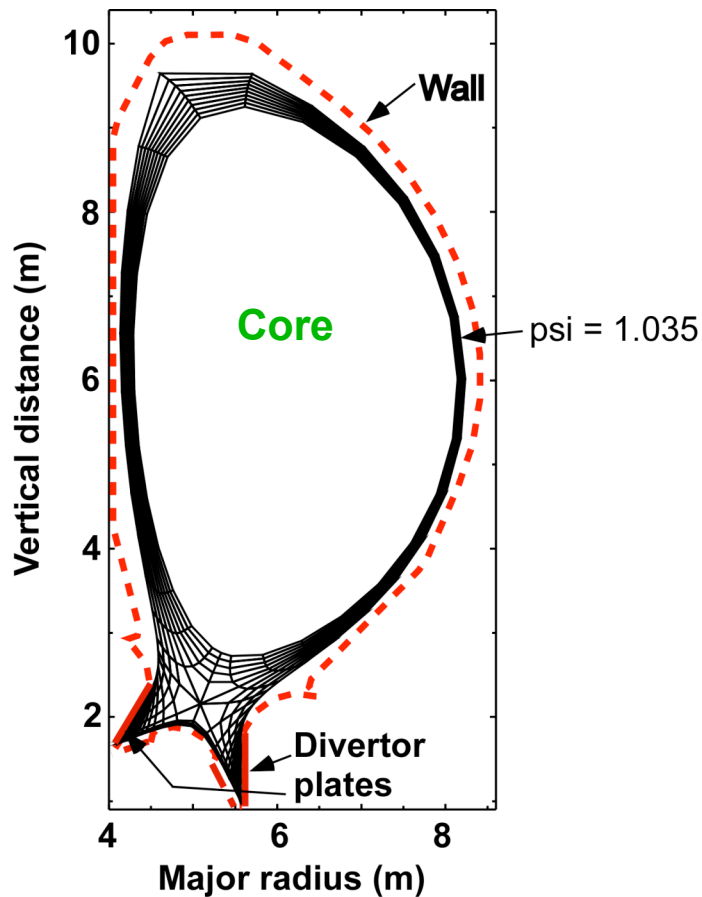
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# ITER utilizes a single-null divertor with steeply-inclined divertor plates



Poloidal cross-section showing edge-plasma region

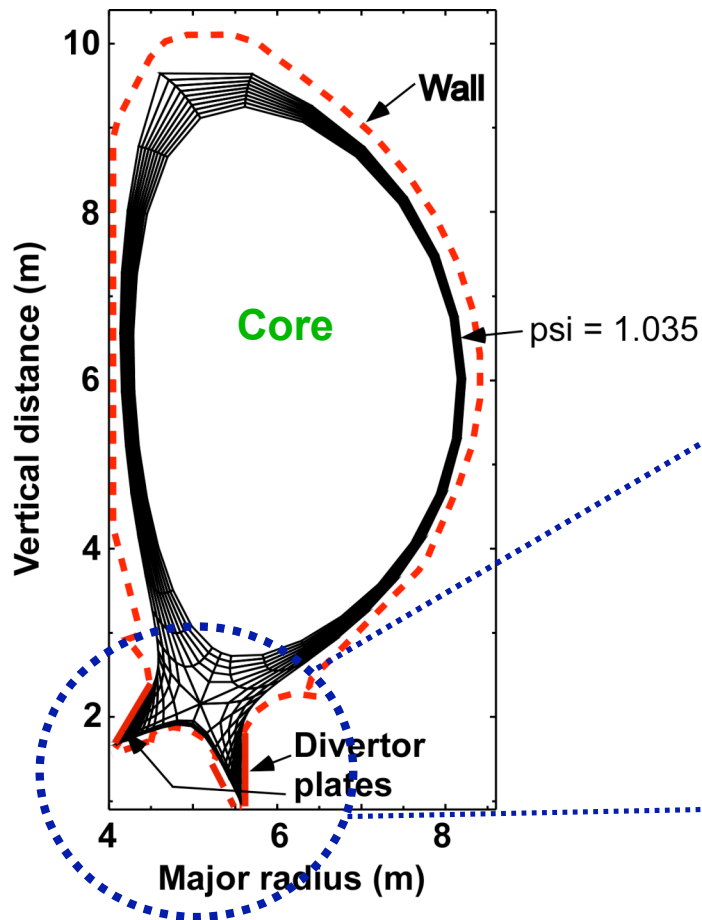


- Nearly vertical plates reduce heat flux & facilitate plasma detachment
- Carbon radiation helps reduce  $T_e$  near strike point to allow He pumping

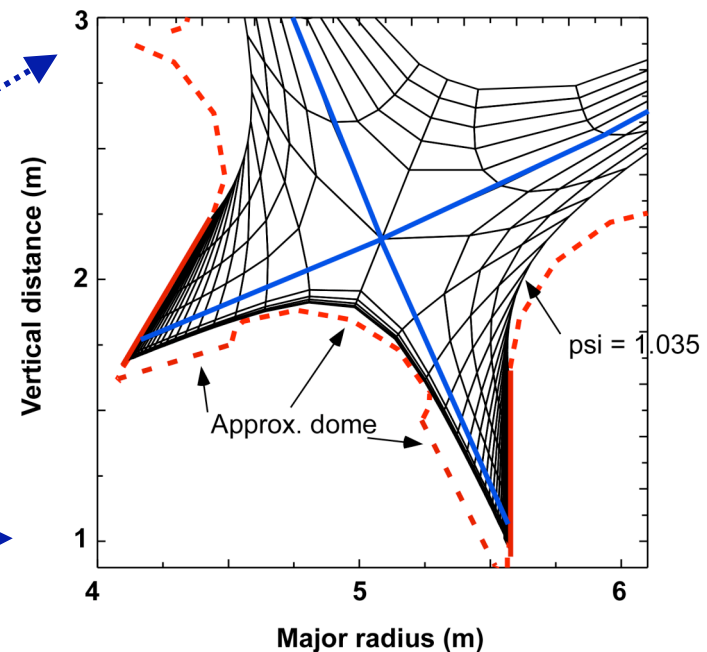
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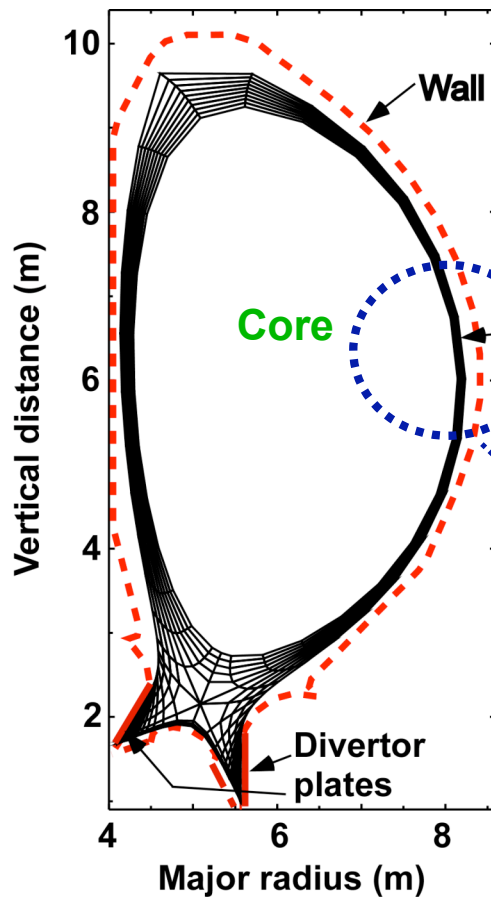
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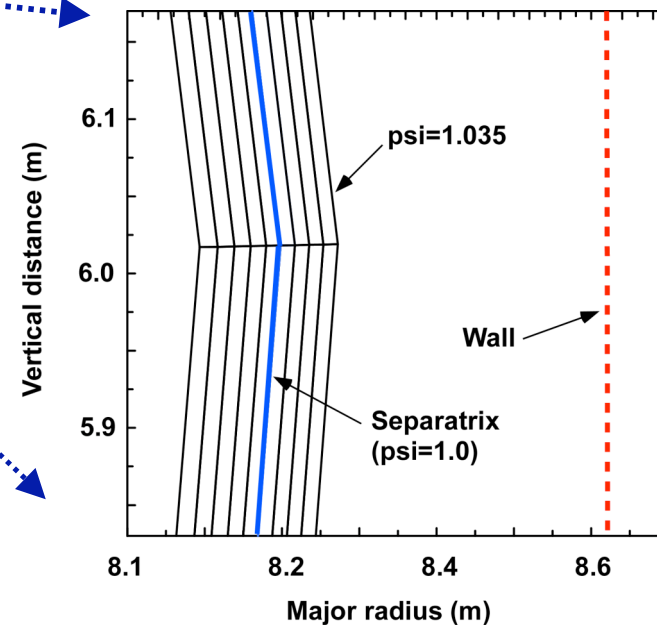
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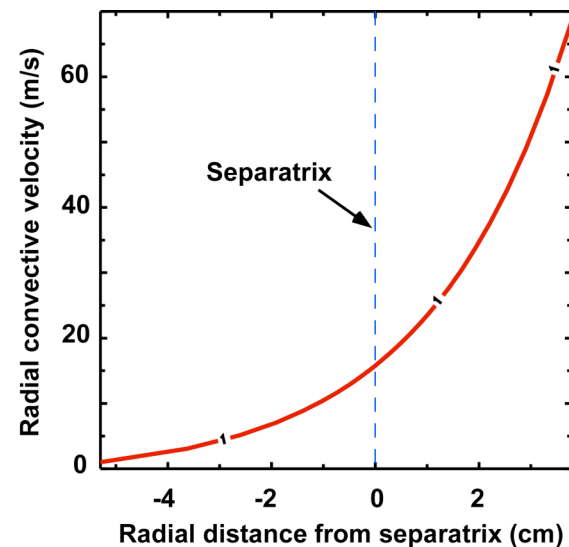
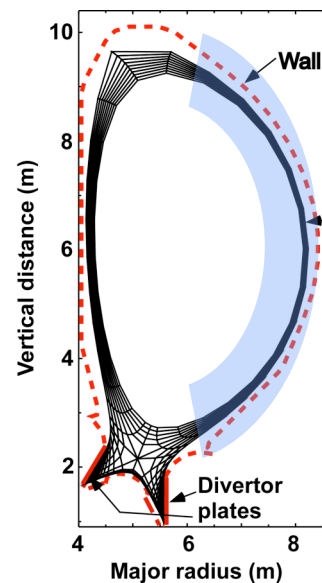


# Previous ITER divertor-plasma modeling assumed diffusive radial transport only; we add convection



- ITER assumes 100 MW power input to SOL
- Here carbon modeled as a 3% concentration
- Anomalous radial diffusion set at  $D = 0.3 \text{ m}^2/\text{s}$ ,  $\chi_{e,i} = 1 \text{ m}^2/\text{s}$
- We add a radial convection term on outboard side, as experiments and simulations imply

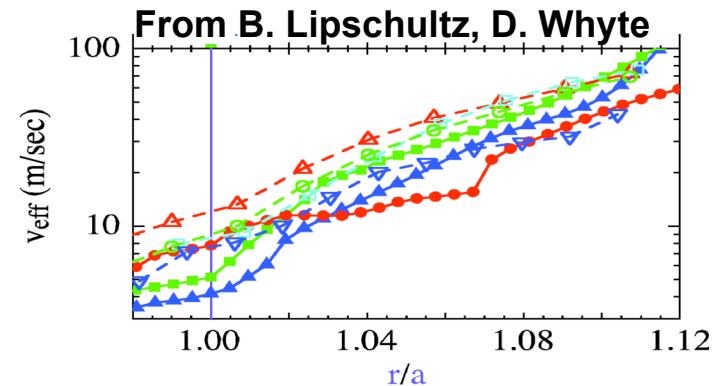
$$\text{radial particle-flux } \Gamma_n = -Ddn/dr + V_{\text{conv}}n$$



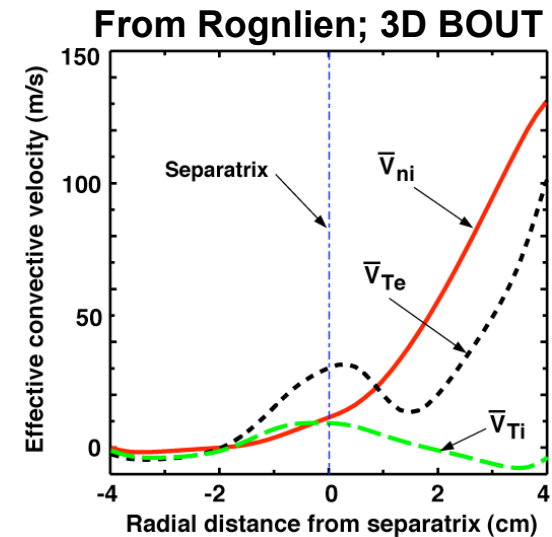
# Why? - there is experimental and theoretical evidence for strong outward convection in SOL



- Gas-puff imaging shows outward moving filaments (“blobs”) (Zweben et al.)
- Probes see outward moving perturbations (Boedo, Rudakov)
- Ionization balance from H-alpha (Lipschultz, Whyte)
- Theory and edge turbulence simulations show outward convection (Krasheninnikov, Pigarov et al.; Rognlien, Xu et al.)



DIII-D	$\bar{n}_e/n_{\text{Greenwald}}$	C-Mod	$\bar{n}_e/n_{\text{Greenwald}}$
—▽—	0.26	—▲—	0.16
—□—	0.38	—■—	0.23
—○—	0.45	—●—	0.36
—△—	0.58		



# But we don't know the scaling of $V_{\text{conv}}$ with size, edge temperature, strong H-mode flow shear ...

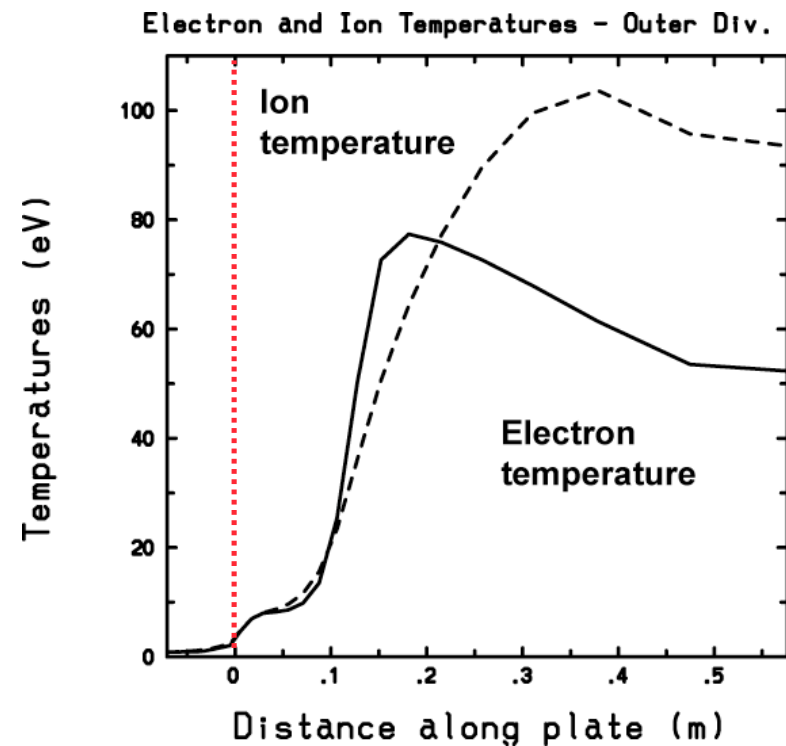
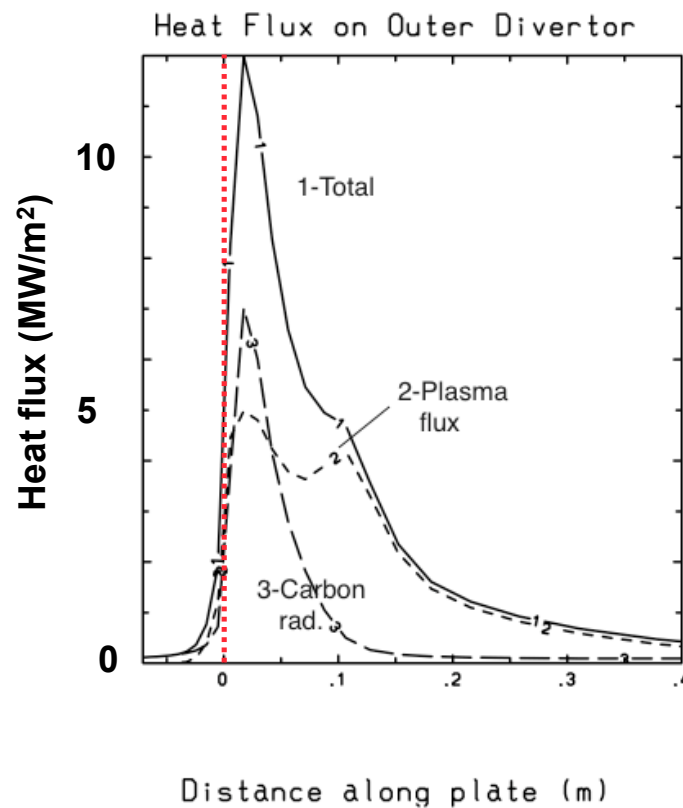


- Consequently, we can parameterize the effect by considering a range of  $V_{\text{conv}}$  cases, from a maximum of 100 m/s to 0 m/s.
- Divertor conditions near separatrix are relatively insensitive
- Midplane radial particle and energy fluxes are sensitive

# With no convection, we obtain conditions typical of past ITER divertor-plasma modeling

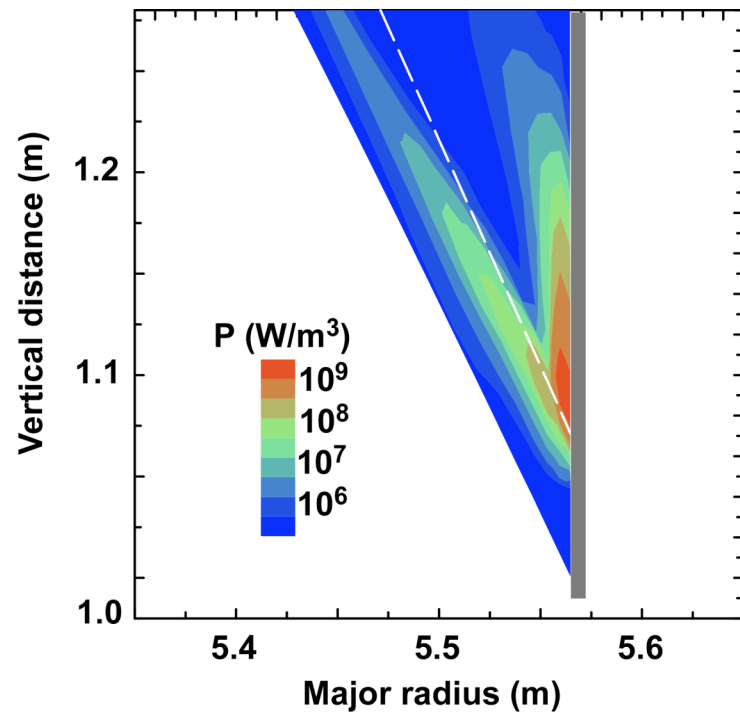
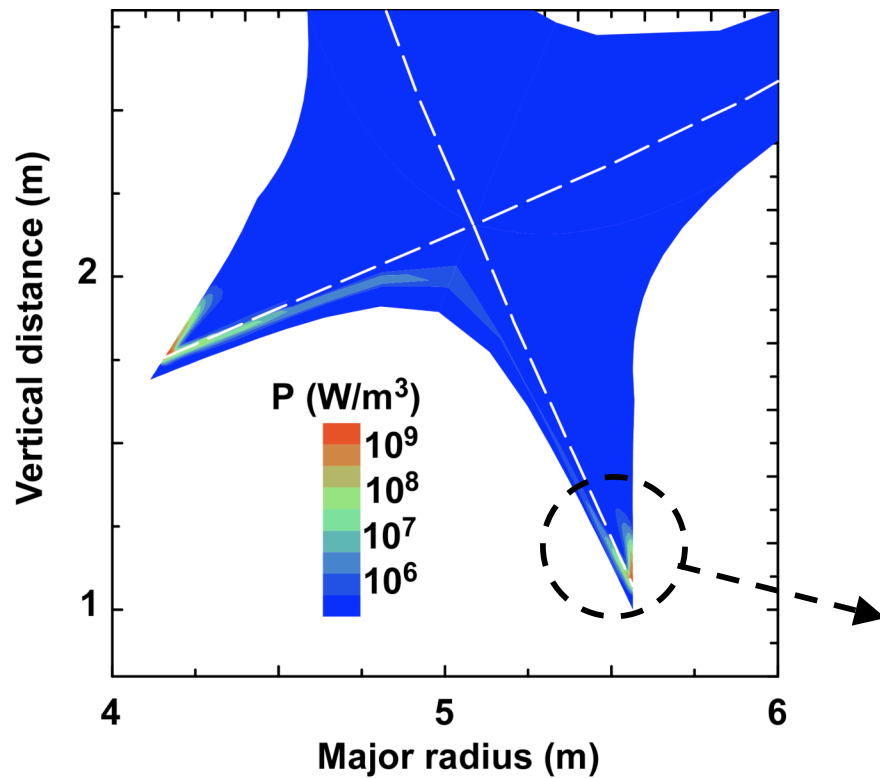


- Anomalous radial diffusion set at  $D = 0.3 \text{ m}^2/\text{s}$ ,  $\chi_{e,i} = 1 \text{ m}^2/\text{s}$
- Peak heat-flux on outer divertor =  $12 \text{ MW}/\text{m}^2$
- ~50% of the power is radiated by carbon





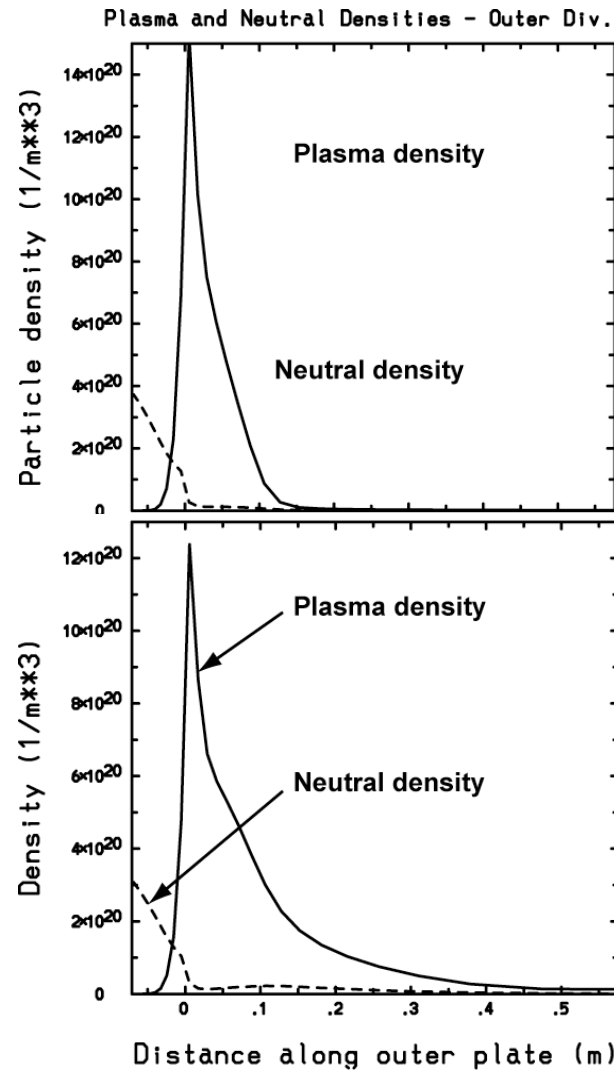
# Carbon radiation is localized near the divertor plates; neon would be more diffuse



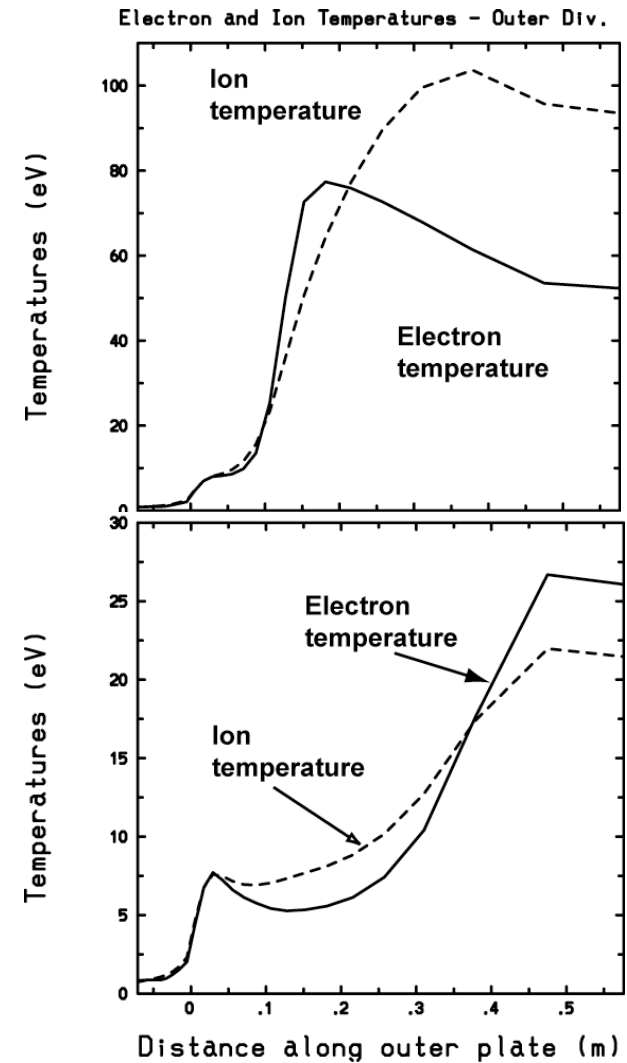
# Convection applied to ITER shows some radial broadening of profiles at the divertor



Without convection



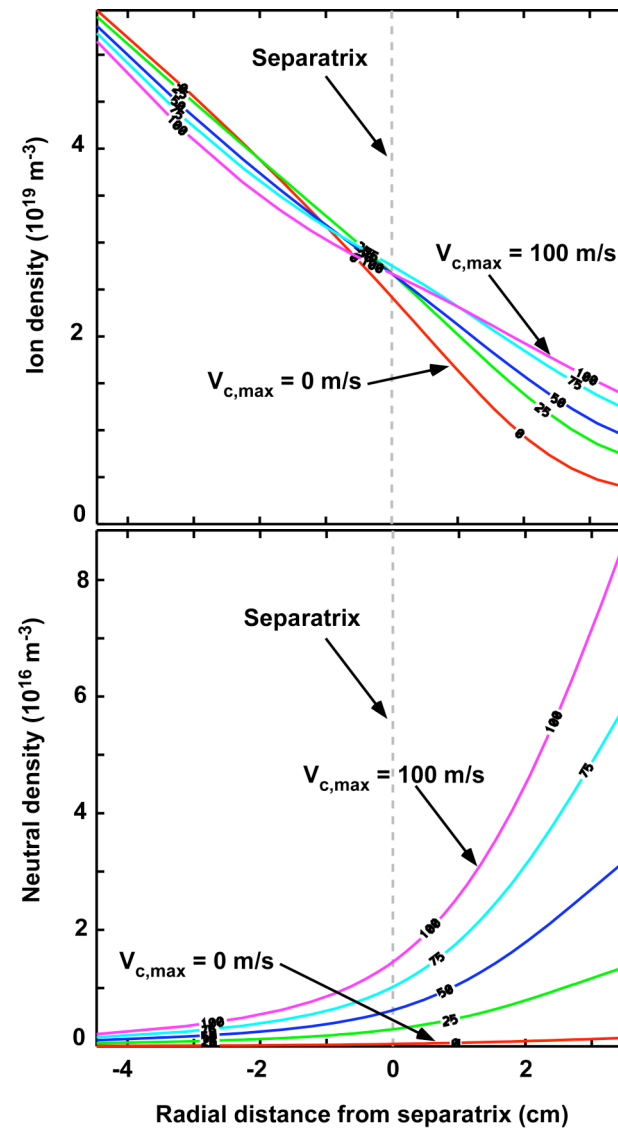
With convection



# Outer midplane density profiles near the wall change substantially with convection



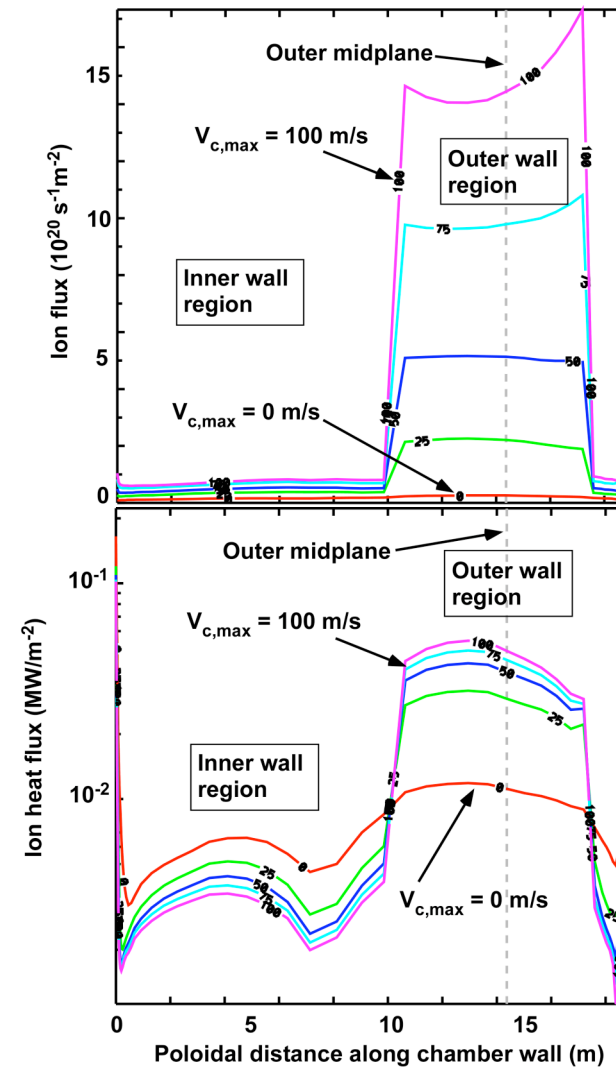
- Plasma density increases near the wall
- Neutral density increases at a faster rate owing to recycling
- Higher neutral density inside the separatrix  $\Rightarrow$  substantial charge-exchange loss and sputtering (Kotschenreuther, Rognlien, Valanju Fusion Eng. Design, '04)



# Plasma fluxes to the wall increase more than local density owing to ionization of recycled gas



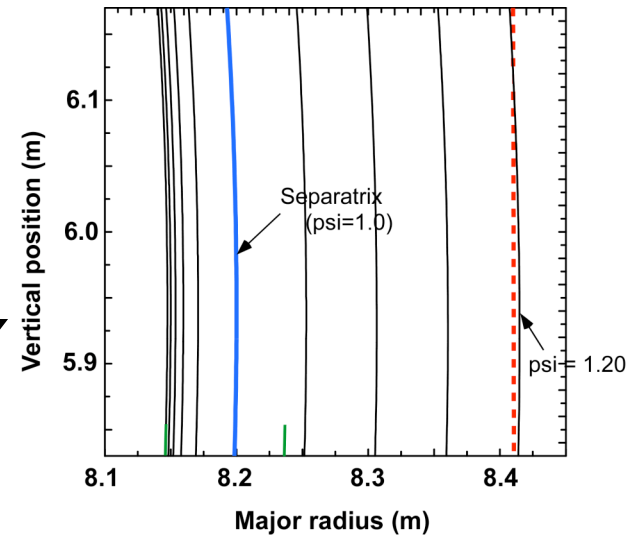
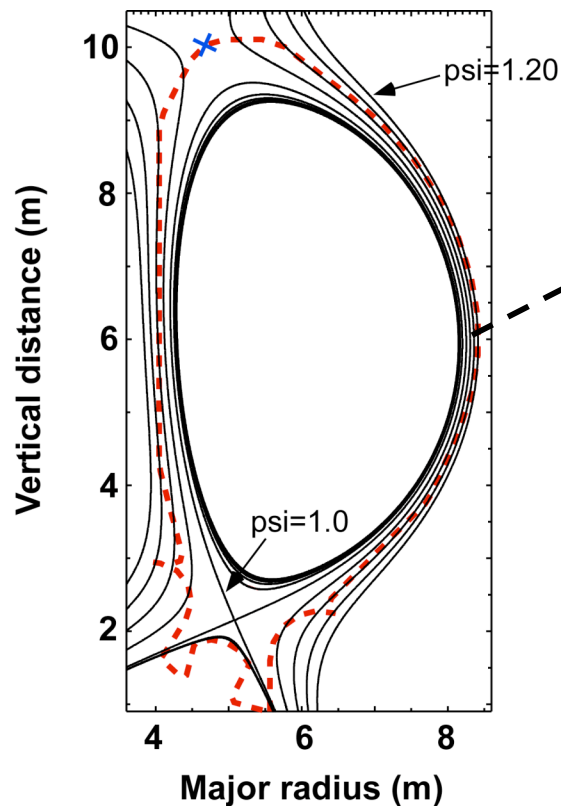
- Since  $n_i$  and  $V_{\text{conv}}$  increase, the  $nV$  flux is much larger
- Ionized neutrals contribute the flux
- Ion temperature decreases some owing to cold ionization source; ion energy flux slower
- Hot cx-neutrals, sheath drop to be added to energy flux



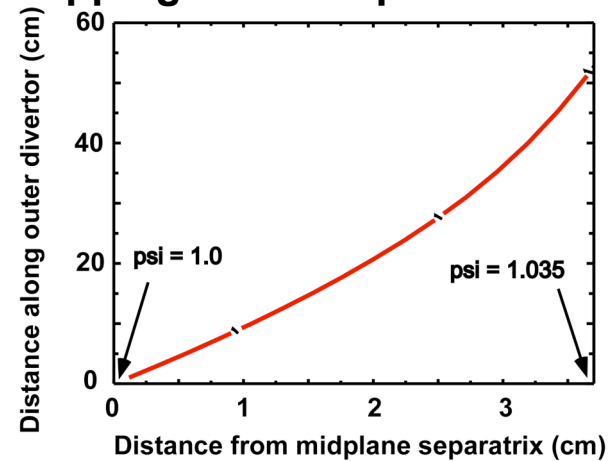
# Extending flux surfaces beyond $\Psi_n = 1.035$ into the “gap” region shows an upper X-point



Mapping along flux surfaces gives plate (and wall) intersections



Mapping from midplane to divertor



# Enhanced plasma fluxes can substantially increase ITER Beryllium wall sputtering



- These hydrogen plasma fluxes supplied to J. Brooks for WBC simulations of wall sputtering
- The “gap” plasma is an extrapolation from UEDGE; impacts ionization of Be
- Later: Be sources can be used within UEDGE to determine competition between transport along and across flux surfaces; requires understanding Be cross-field transport
- Iterate on hydrogen plasma if Be sputtering changes main edge plasma

# Summary and plans



- **Even moderate radial convection produces substantial plasma fluxes to the wall for ITER**
- **Sputtering of material (Be) from plasma and cx-neutrals can be large**
- **Gap plasma (for  $\Psi_n > 1.035$ ) needs a better model; UEDGE can perform unbalanced double-null simulations that will help**
- **Scaling of convective transport for hydrogen and beryllium needs improvement**