

# M3D-C<sup>1</sup> Simulations of the Plasma Response to Magnetic Perturbations in the NSTX-U Snowflake Divertor

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in collaboration with

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H. Frerichs<sup>4</sup>, O. Schmitz<sup>4</sup>, V.A. Soukhanovskii<sup>5</sup> and I. Waters<sup>4</sup>

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18<sup>th</sup> International Spherical Torus Workshop

Princeton University, 3-6 November, 2015



# The development of new divertor configurations is crucial on the road to a fusion reactor

- **Steady-state power handling in DEMO and future fusion reactors will only be possible with plasmas operated with high core radiation fraction**
  - About 90% of the heating power has to be radiated [M. Kotschenreuther, *Phys. Plasmas* (2007)]
  - ELMs will not be tolerated at all [H. Zohm, *Nucl. Fusion* (2013)]
- **Alternative solutions have to be researched to mitigate the risk that highly radiating regimes may not be extrapolated towards DEMO**
  - The snowflake (SF) is one of several alternative divertor configurations [D.D. Ryutov, *Phys. Plasmas* (2007)]
- **Solution for ELMs come in the form of applied 3D magnetic perturbations** [A. Loarte, *Nucl. Fusion* (2014)]
  - The effect of 3D magnetic perturbations in the SF configuration has to be investigated

Improved physics understanding & modeling of 3D fields in the SF divertor are needed in order to extrapolate towards larger devices

# Snowflake configuration proposed as a possible solution to reduce target power loads

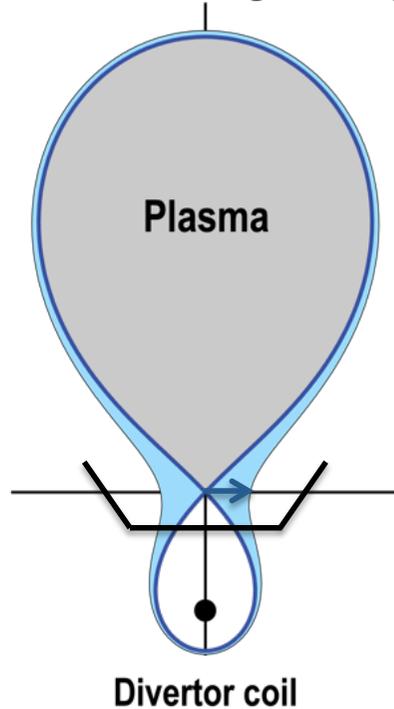
- Snowflake  $\equiv$  second order null point [D.D. Ryutov, *Phys. Plasmas* (2007)]

$$\mathbf{B}_p = 0$$

versus

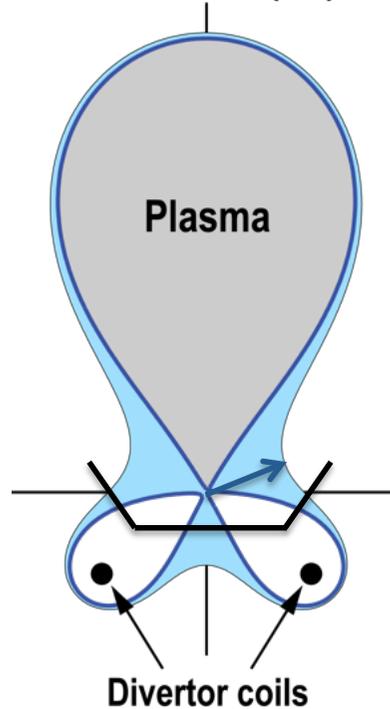
$$\mathbf{B}_p = 0 \text{ and } \nabla \mathbf{B}_p = 0$$

Conventional Single Null (SN)



$$|\mathbf{B}_P^{\text{SN}}| \propto \rho_{\text{npt}}$$

Snowflake (SF)

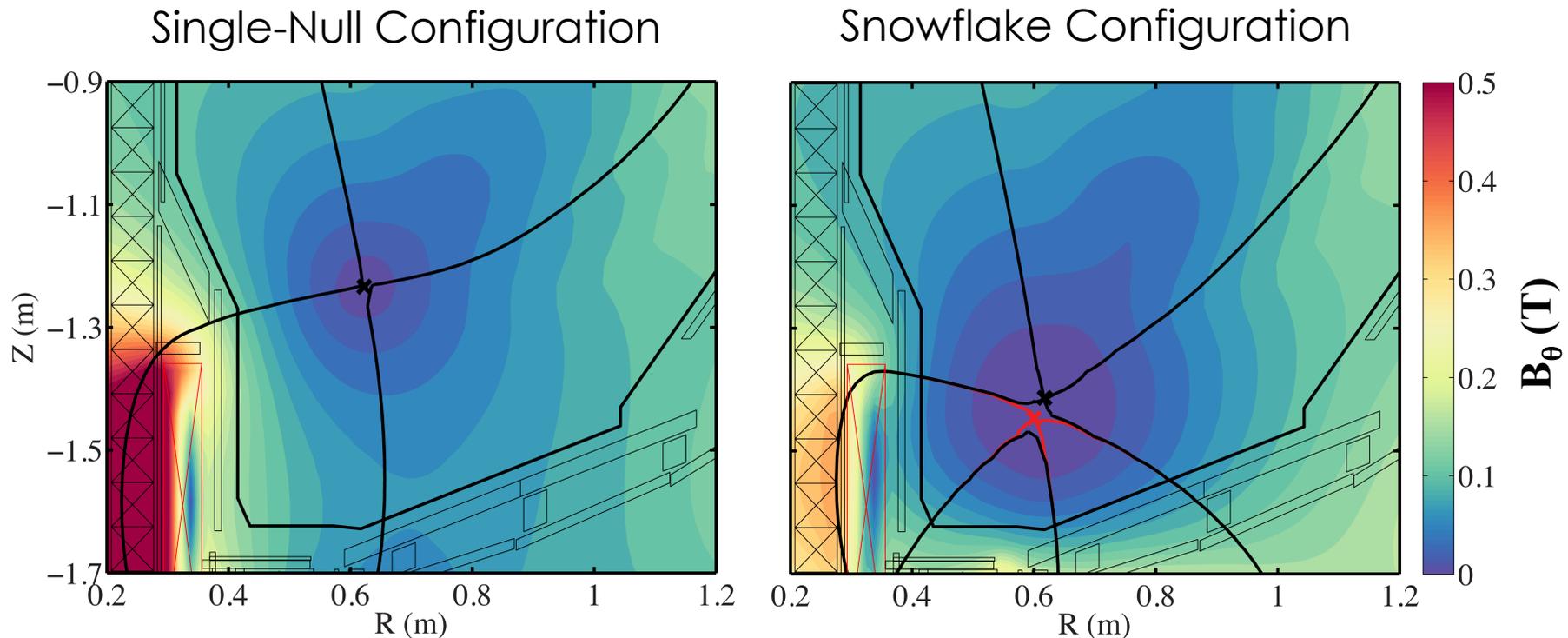


$$|\mathbf{B}_P^{\text{SF}}| \propto \rho_{\text{npt}}^2$$

- Two additional divertor legs
- Lower poloidal field near the null point
  - Larger flux expansion
  - Larger divertor volume
  - Longer connection length

# The snowflake configuration is more sensitive to magnetic perturbations than a single-null configuration

- The effect of magnetic perturbations in the plasma is expected to be magnified in the SF configuration due to its lower  $B_\theta$  near the null-point



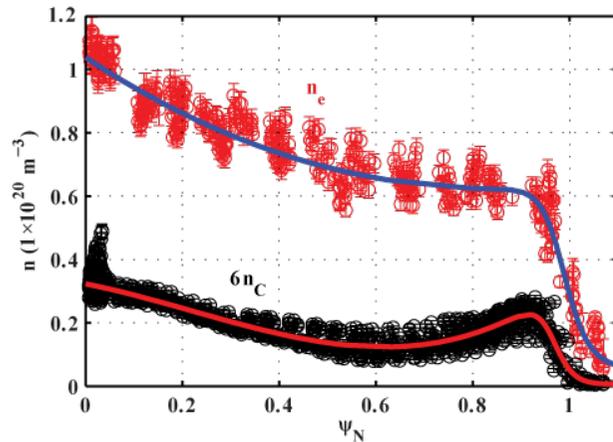
The SF configuration provides an excellent environment to study the plasma response to magnetic perturbations

# NSTX discharge provides the equilibrium profiles for the ISOLVER calculations of the NSTX-U SF divertor

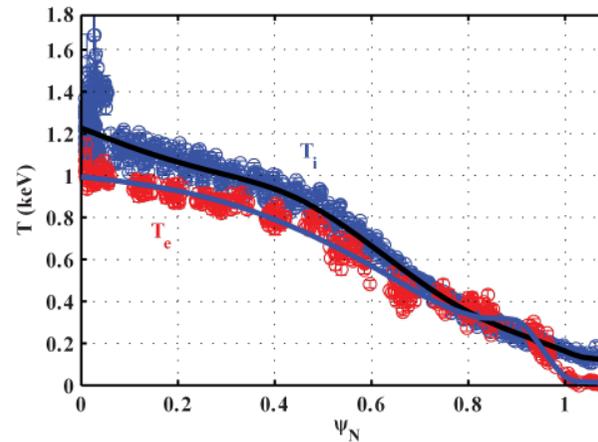
- Reference Discharge (#132543 @ 700 ms)

- $I_p = 1.0$  MA
- $B_T = -0.44$  T
- $P_{\text{NBI}} = 6.0$  MW
- $\kappa = 2.1$
- $\delta_{\text{top}} = 0.37$
- $\delta_{\text{bot}} = 0.71$

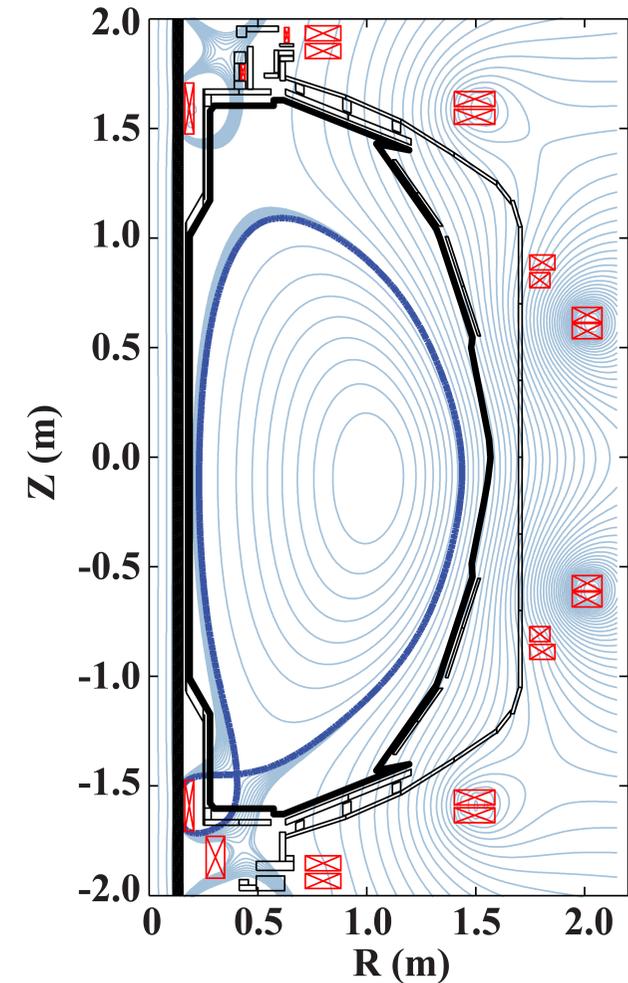
Plasma density



Plasma Temperature

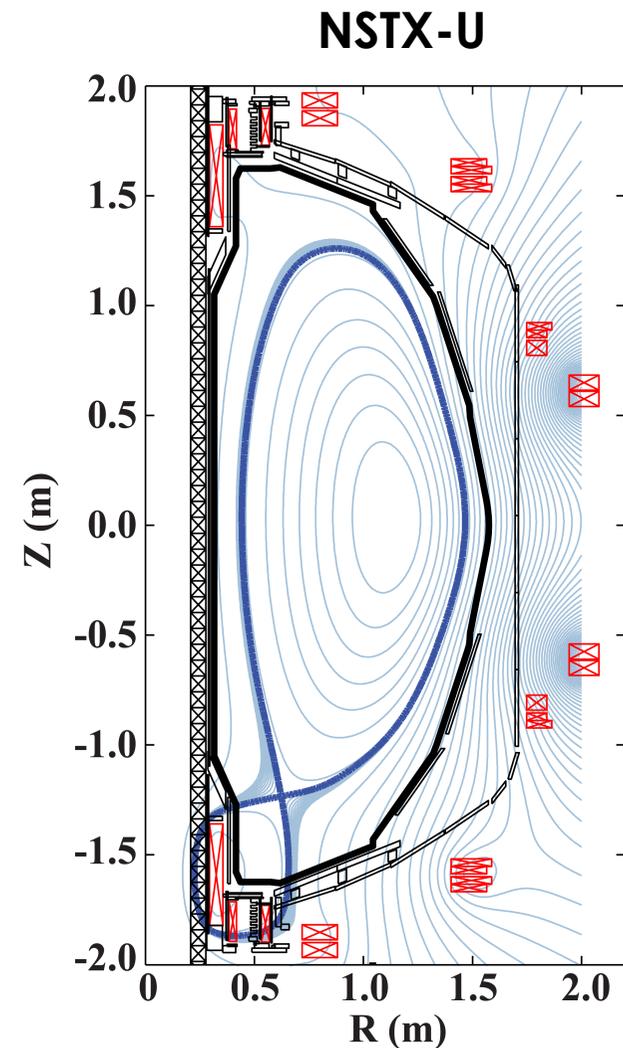
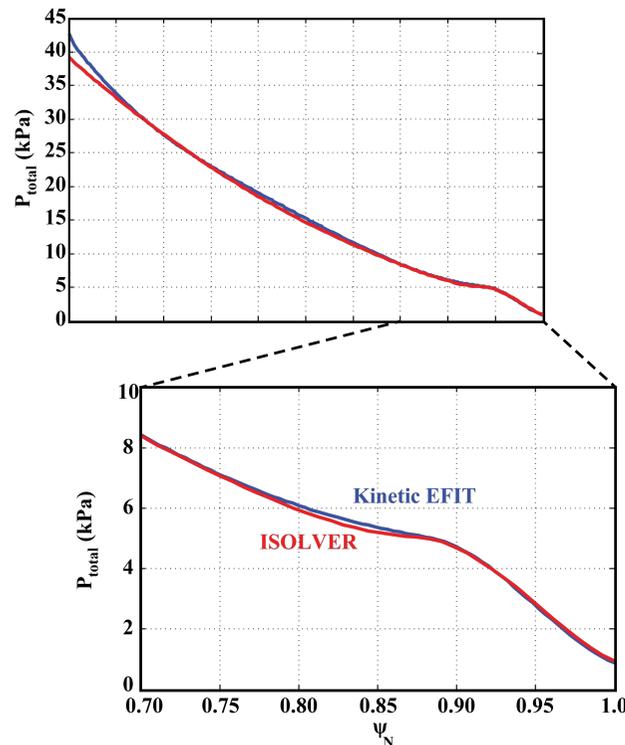


NSTX



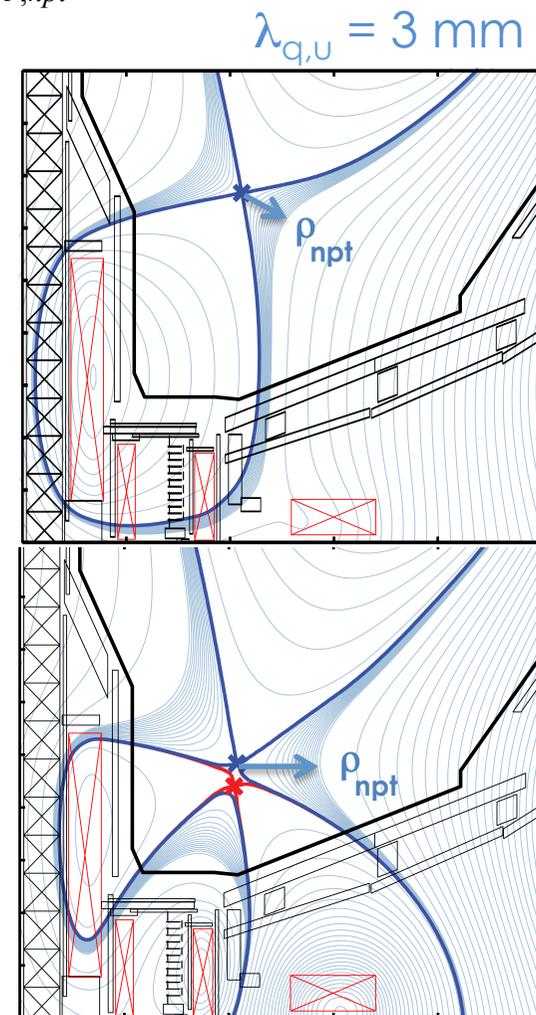
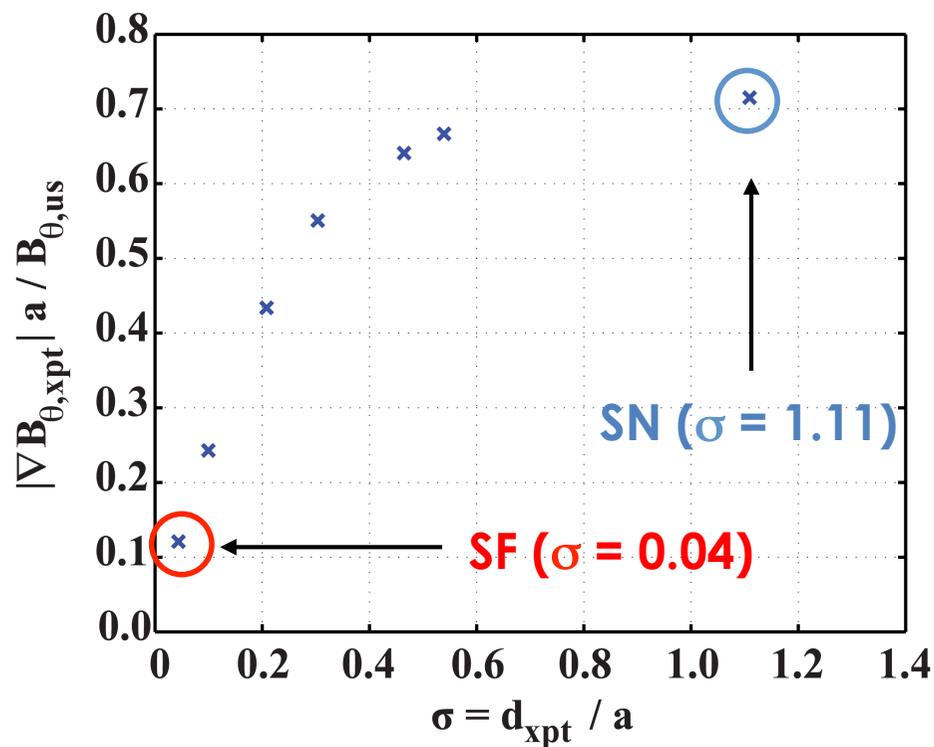
# ISOLVER calculations of the NSTX-U SF divertor assume approximately the same total plasma pressure profile

- SF configurations generated by ISOLVER have approximately the same  $P'$ ,  $FF'$  and total plasma pressure of the reference NSTX discharge
  - Total pressure profile does not depend on divertor configuration [V.A. Soukhanovskii, *Phys. Plasmas* (2012)]



# Configuration is varied from a SN reference to a SF

- An exact SF configuration ( $\sigma = 0$ ) features  $\vec{\nabla} B_{\theta, npt} = 0$ 
  - $\vec{\nabla} B_{\theta, npt}$  is a measure of the “proximity” of a divertor configuration from an exact SF



# Estimate the Plasma Response to Externally Applied Non-Axisymmetric Magnetic Fields Using Modelling

- The M3D-C<sup>1</sup> code is a two-fluid, resistive MHD code<sup>1</sup>

$$\frac{\partial n}{\partial t} + \nabla \cdot (n\mathbf{u}) = 0$$

- The M3D-C<sup>1</sup> computational domain includes the confined plasma, the separatrix and the open field-line region

$$n \left( \frac{\partial \mathbf{u}}{\partial t} + \mathbf{u} \cdot \nabla \mathbf{u} \right) = \mathbf{J} \times \mathbf{B} - \nabla p - \nabla \cdot \Pi$$

$$\frac{\partial p}{\partial t} + \mathbf{u} \cdot \nabla p = -\Gamma p \nabla \cdot \mathbf{u} - \frac{d_i}{n} \mathbf{J} \cdot \left( \Gamma p_e \frac{\nabla n}{n} - \nabla p_e \right) - (\Gamma - 1) \nabla \cdot \mathbf{q}$$

- Unstructured mesh allows increased spatial resolution near rational surfaces and x-point

$$\frac{\partial \mathbf{B}}{\partial t} = -\nabla \times \mathbf{E}$$

$$\mathbf{E} = -\mathbf{u} \times \mathbf{B} + \eta \mathbf{J} + \frac{d_i}{n} (\mathbf{J} \times \mathbf{B} - \nabla p_e)$$

- Two-fluid effects governed by ion inertial length,  $d_i$

$$\Pi = -\mu \left[ \nabla \mathbf{u} + (\nabla \mathbf{u})^T \right]$$

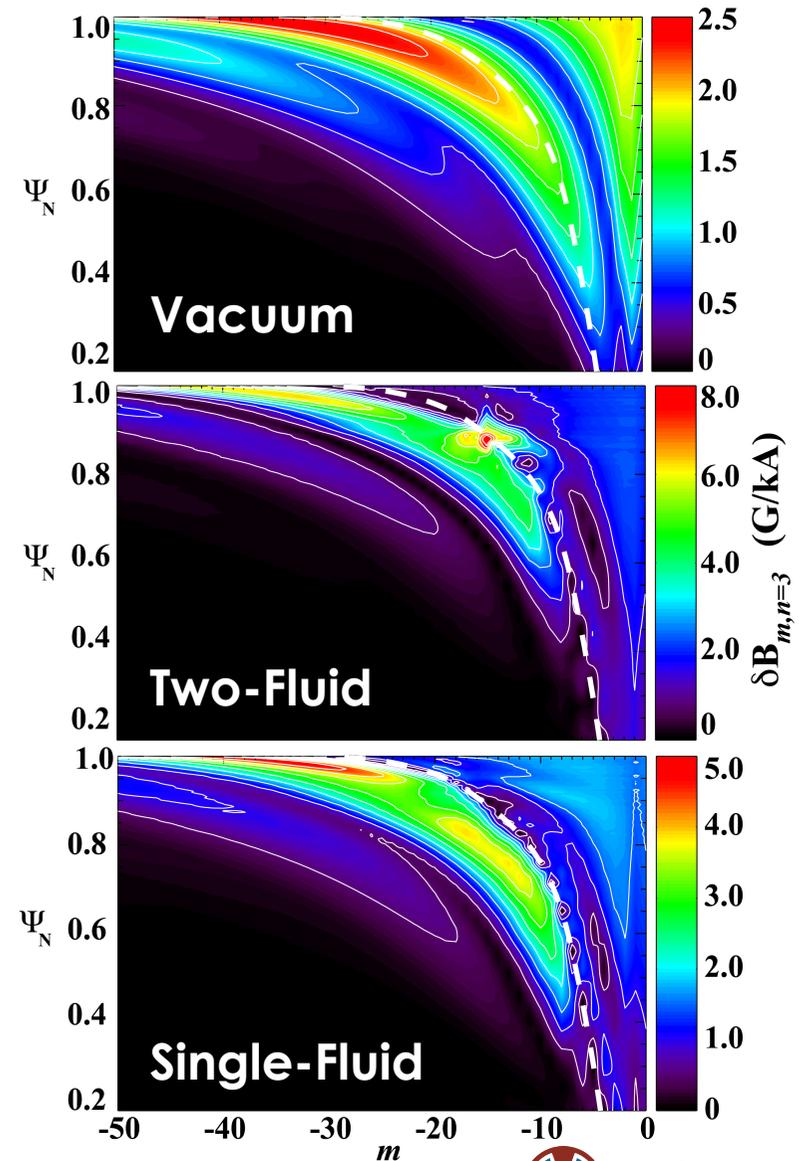
- Electron and ion fluids decouple at finite  $d_i$

$$\mathbf{q} = -\kappa \nabla \left( \frac{p}{n} \right) - \kappa_{\parallel} \mathbf{b} \mathbf{b} \cdot \nabla \left( \frac{p_e}{n} \right)$$

$$\mathbf{J} = \nabla \times \mathbf{B}$$

# Two-fluid effects significantly enhance resonant field components in the plasma edge of the SN configuration

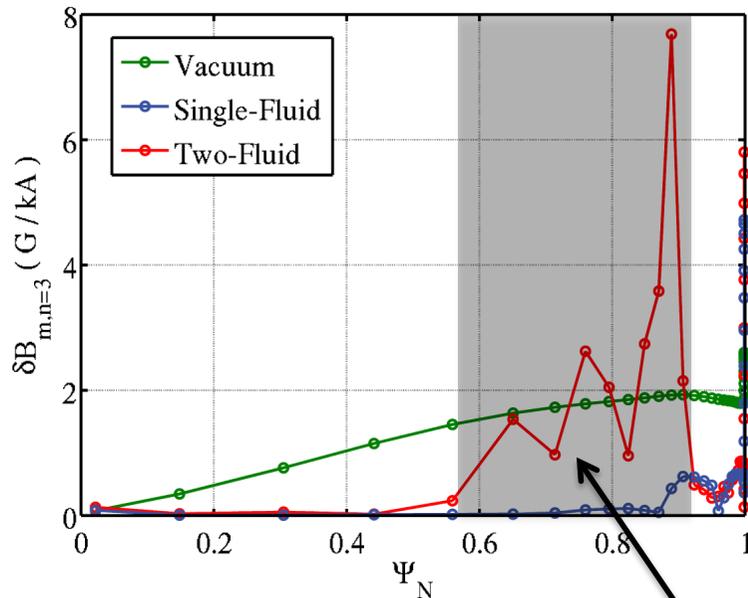
- **Vacuum field reveals broad Fourier spectrum of poloidal harmonics in the plasma edge**
  - Spectrum depends on the configuration of perturbation coil currents ( $n = 1, 2$ , etc.)
- **Two-fluid effects amplify resonant components in the plasma edge**
  - Indicative of stable tearing modes near marginal stability in the unperturbed case
- **Single-fluid calculations show strong screening of perturbation fields**



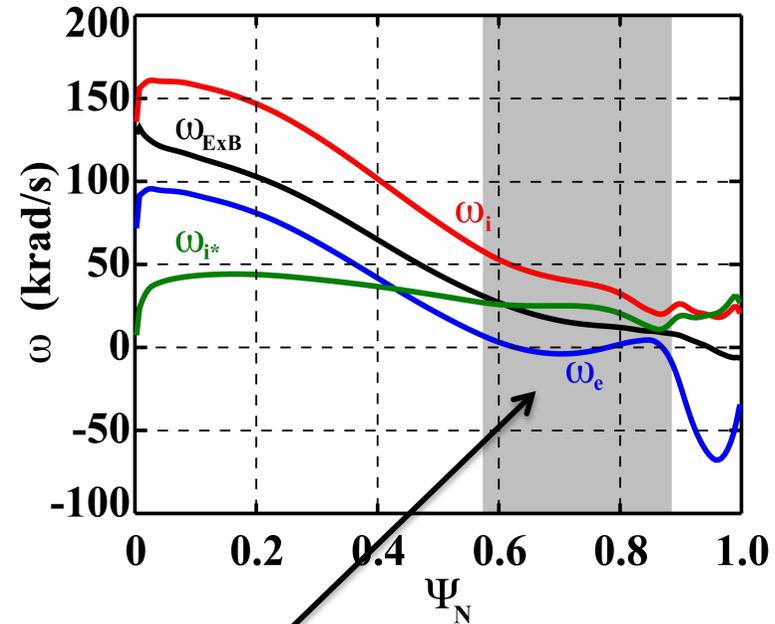
# Enhancement of resonant field components is caused by low electron fluid rotation in the plasma edge

- Region of enhancement of resonant components coincides with region of low electron fluid rotation [N. Ferraro, *Phys. Plasmas* (2012)]

## Resonant Magnetic Perturbation



## Plasma Rotation Profiles

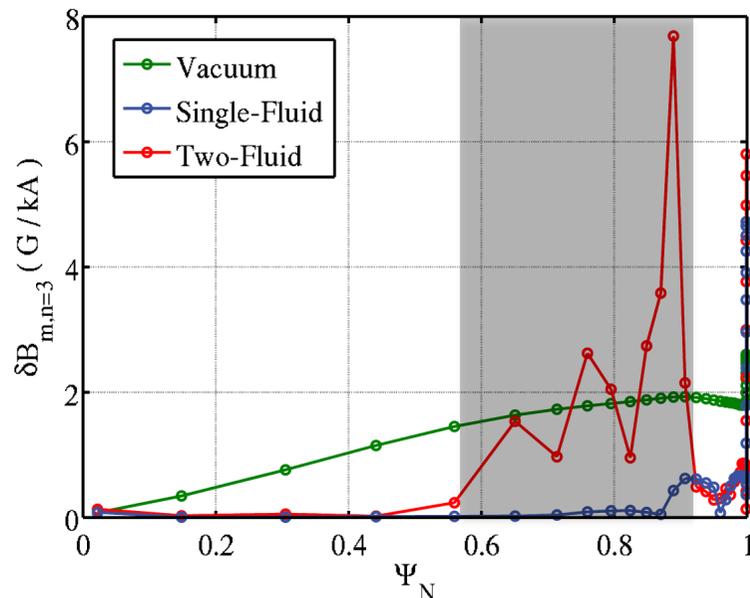


Low electron fluid rotation

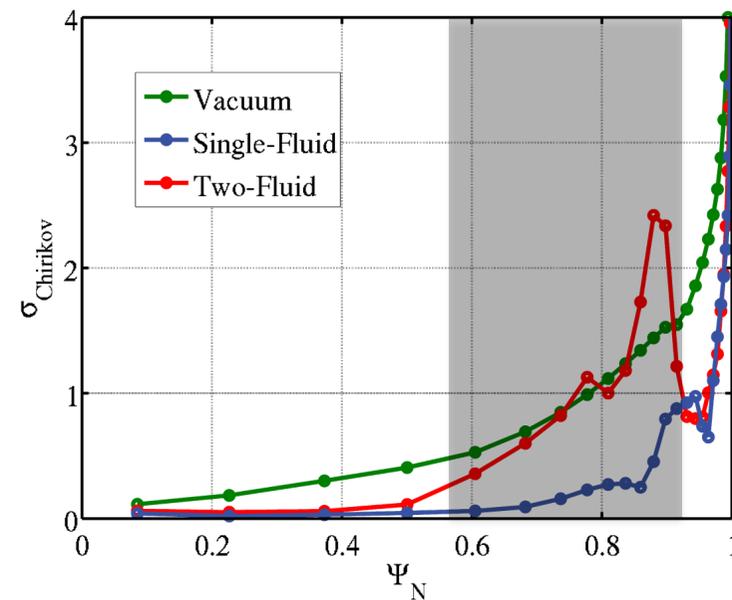
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## Resonant Magnetic Perturbation



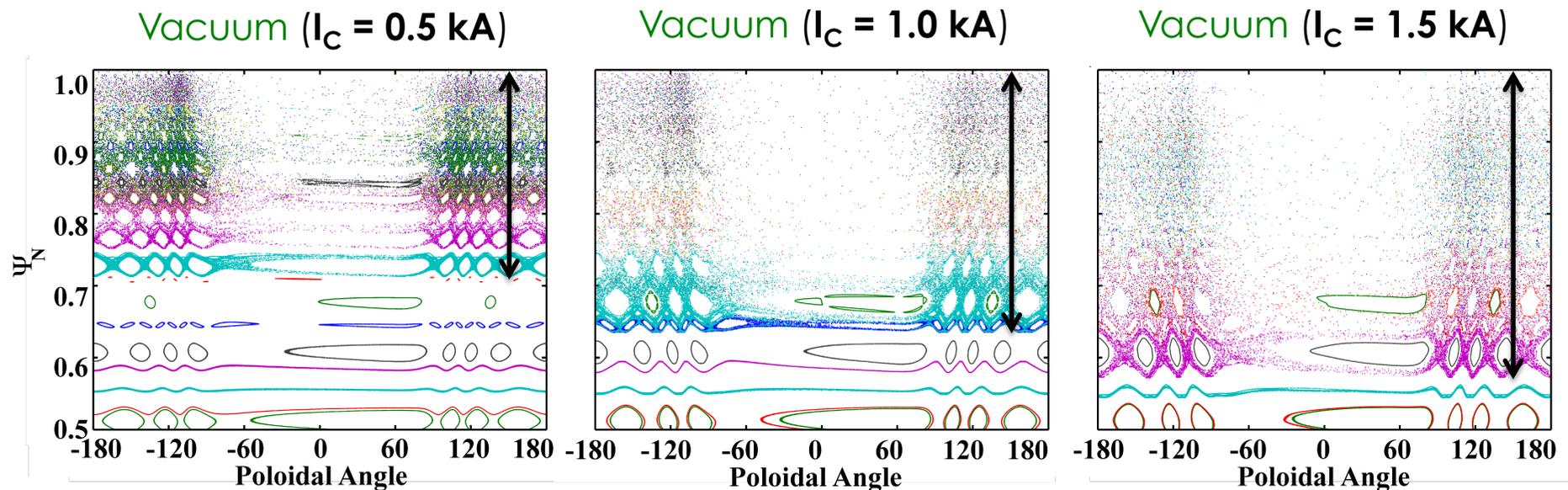
## Chirikov Parameter



- **Enhanced resonant fields indicate the formation of magnetic islands**
  - Two-fluid calculations predict stochastic layer in the plasma edge as large as in the vacuum field approach

# M3D-C<sup>1</sup> calculations predict strong stochastization of plasma edge with increasing perturbation coil current

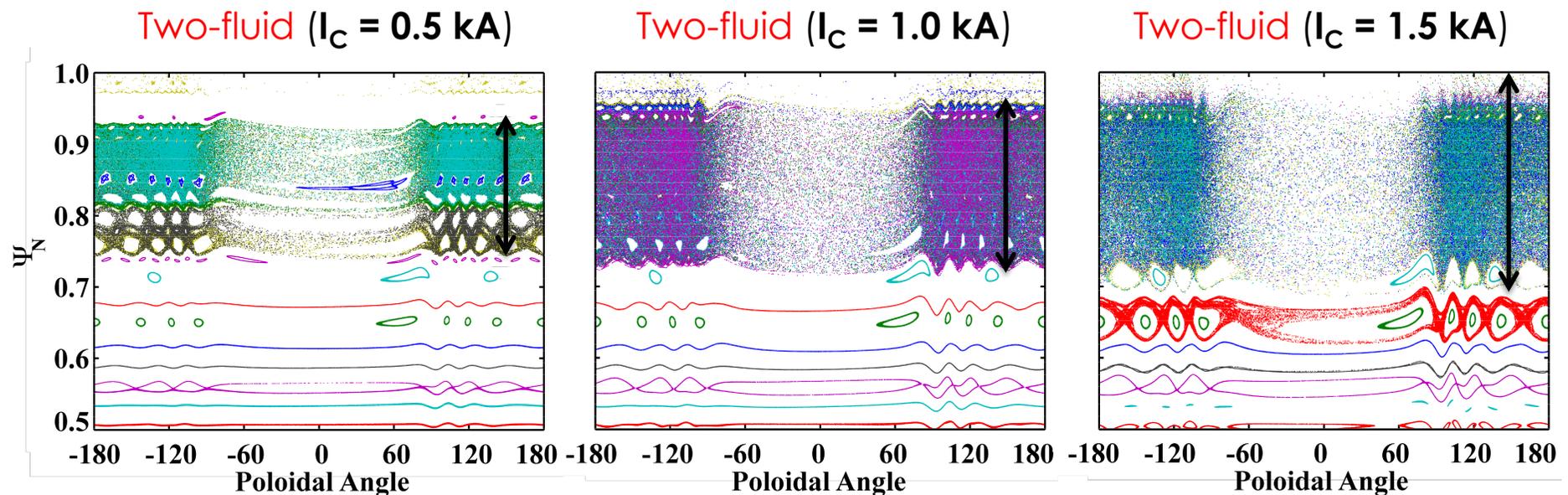
- **Vacuum**, two-fluid and single-fluid calculations predict the formation of magnetic islands in the plasma edge
  - For sufficiently large coil currents, islands overlap leading to stochastic layer in the edge



- Flux surfaces in the plasma edge are all broken in the **vacuum** calculations

# M3D-C<sup>1</sup> calculations predict strong stochasticization of plasma edge with increasing perturbation coil current

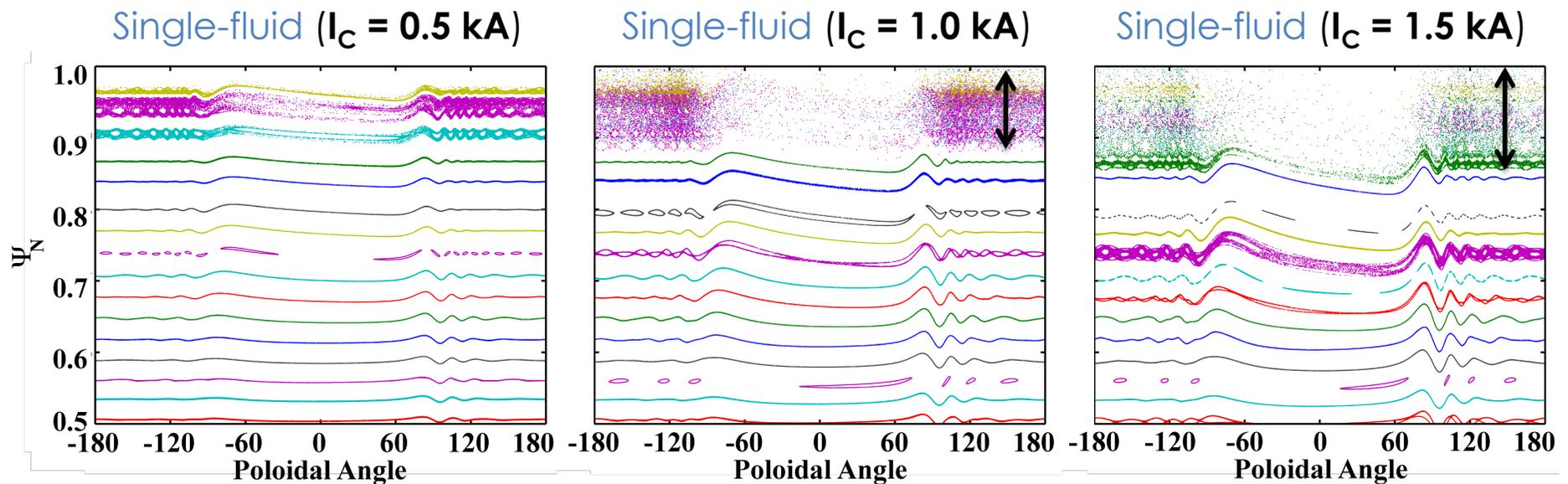
- Vacuum, **two-fluid** and single-fluid calculations predict the formation of magnetic islands in the plasma edge
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- Stochastic edge does not increase significantly with perturbation coil current in the **two-fluid** approach

# M3D-C<sup>1</sup> calculations predict strong stochasticization of plasma edge with increasing perturbation coil current

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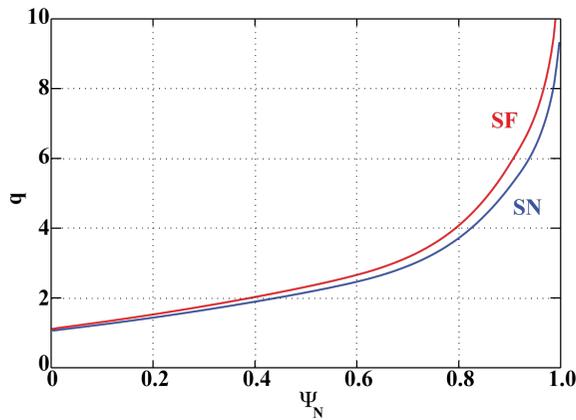


- **Single-fluid** calculations predict a thinner stochastic edge than vacuum and two-fluid calculations

# Two-fluid effects significantly enhance resonant field components in the plasma edge of the SF configuration

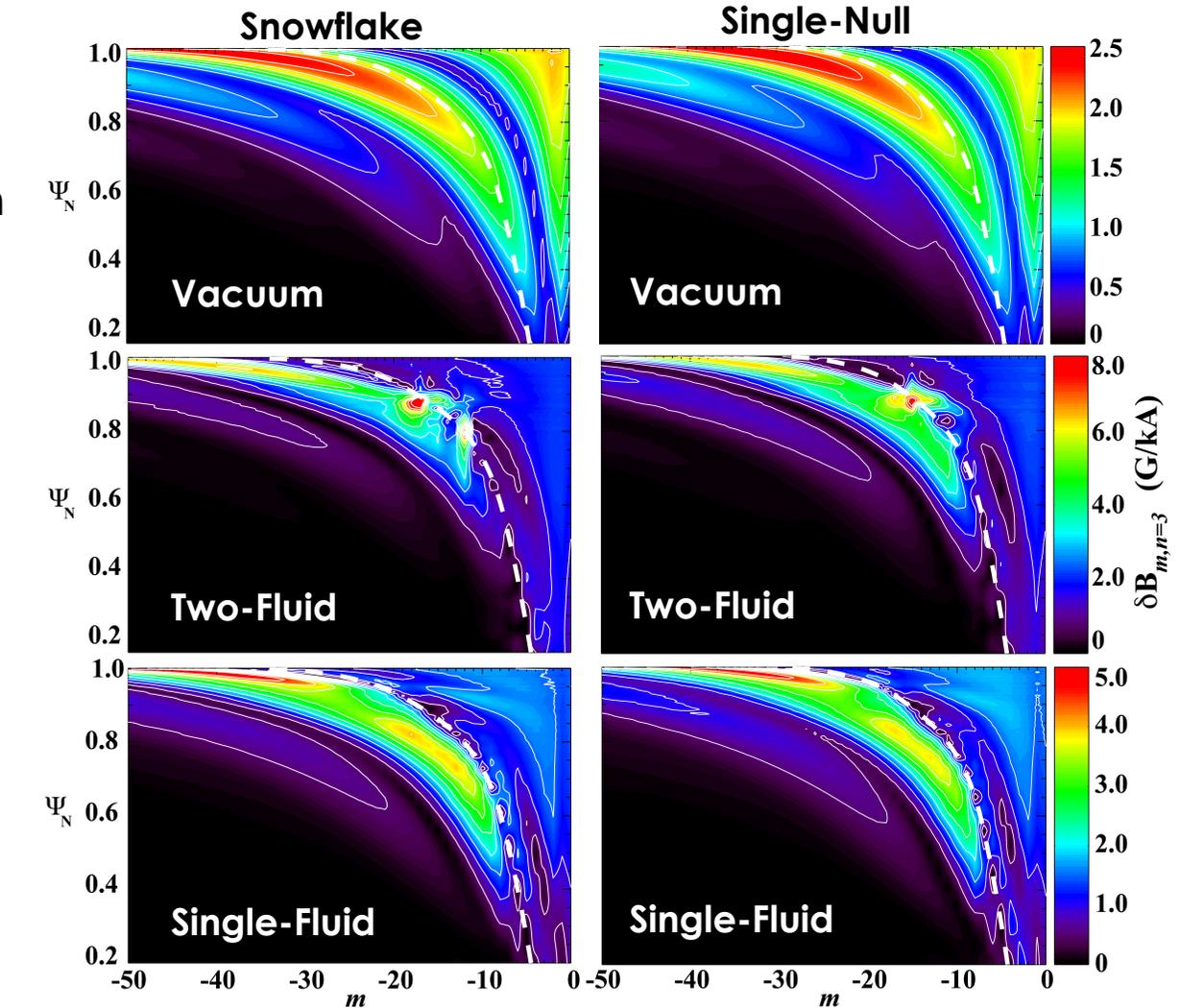
- Plasma response in a SF is not significantly different than in a SN

- Differences come from slightly different  $q$ -profiles



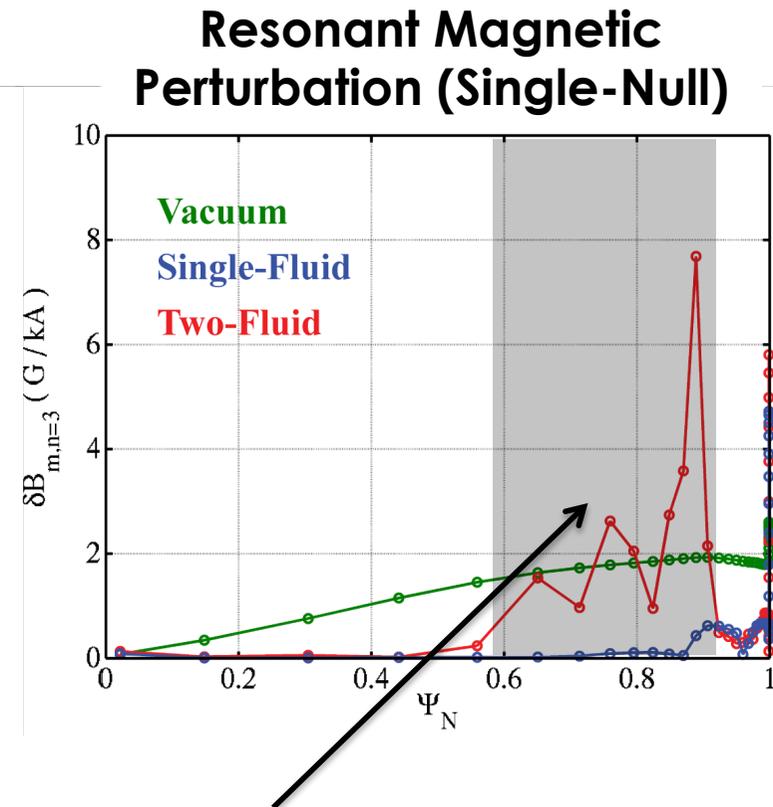
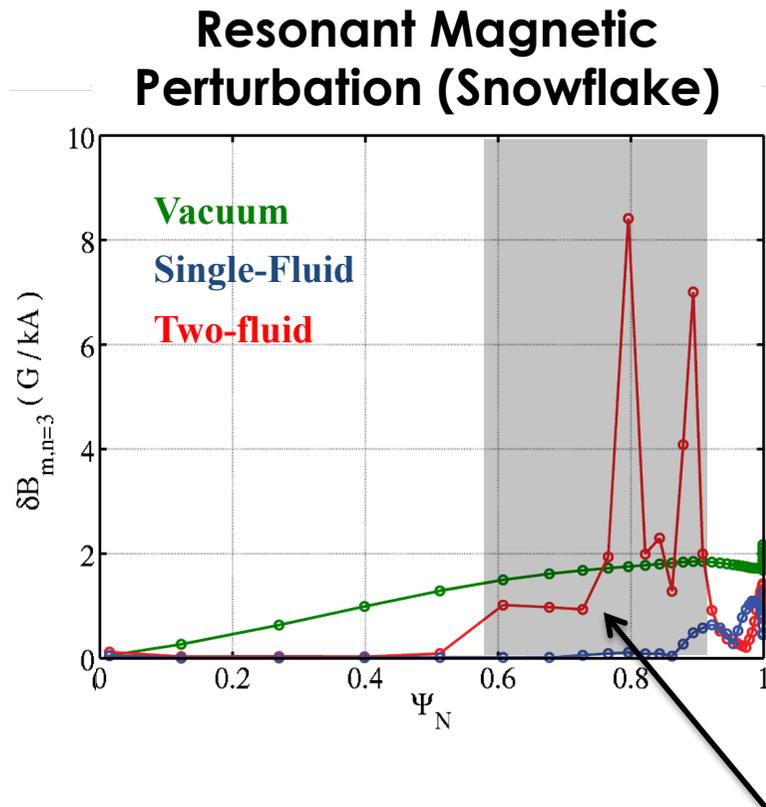
- Differences in  $q$ -profile come from change in poloidal current

$$J_\theta = \frac{dF}{d\psi} B_\theta$$



# Enhancement of resonant field components is caused by low electron fluid rotation in the plasma edge

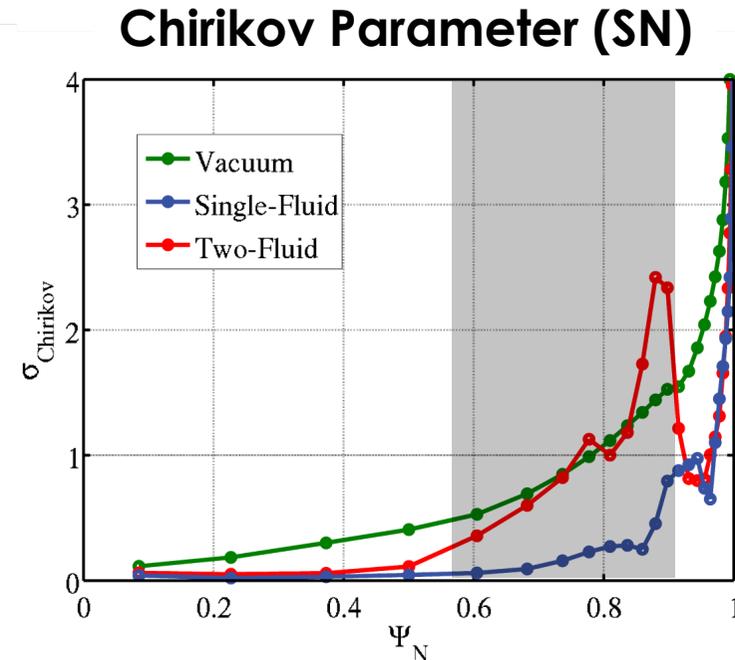
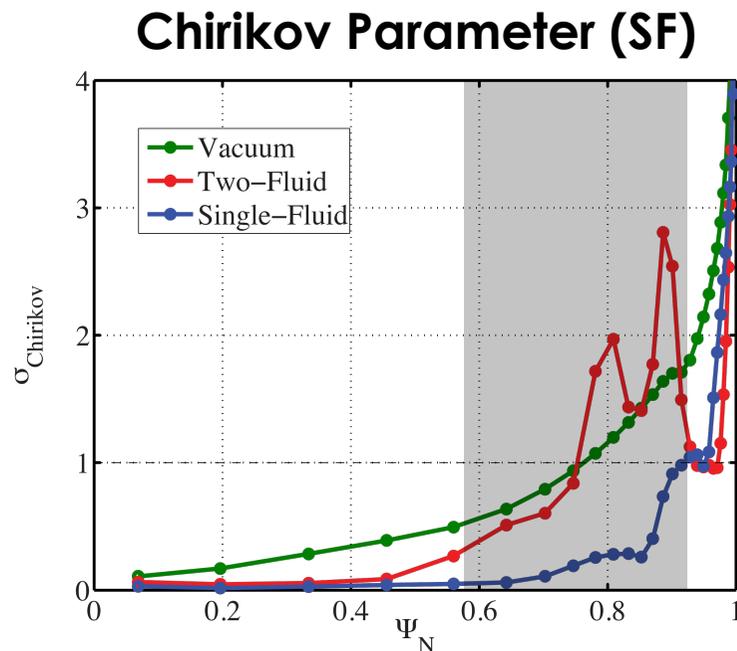
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Low electron fluid rotation

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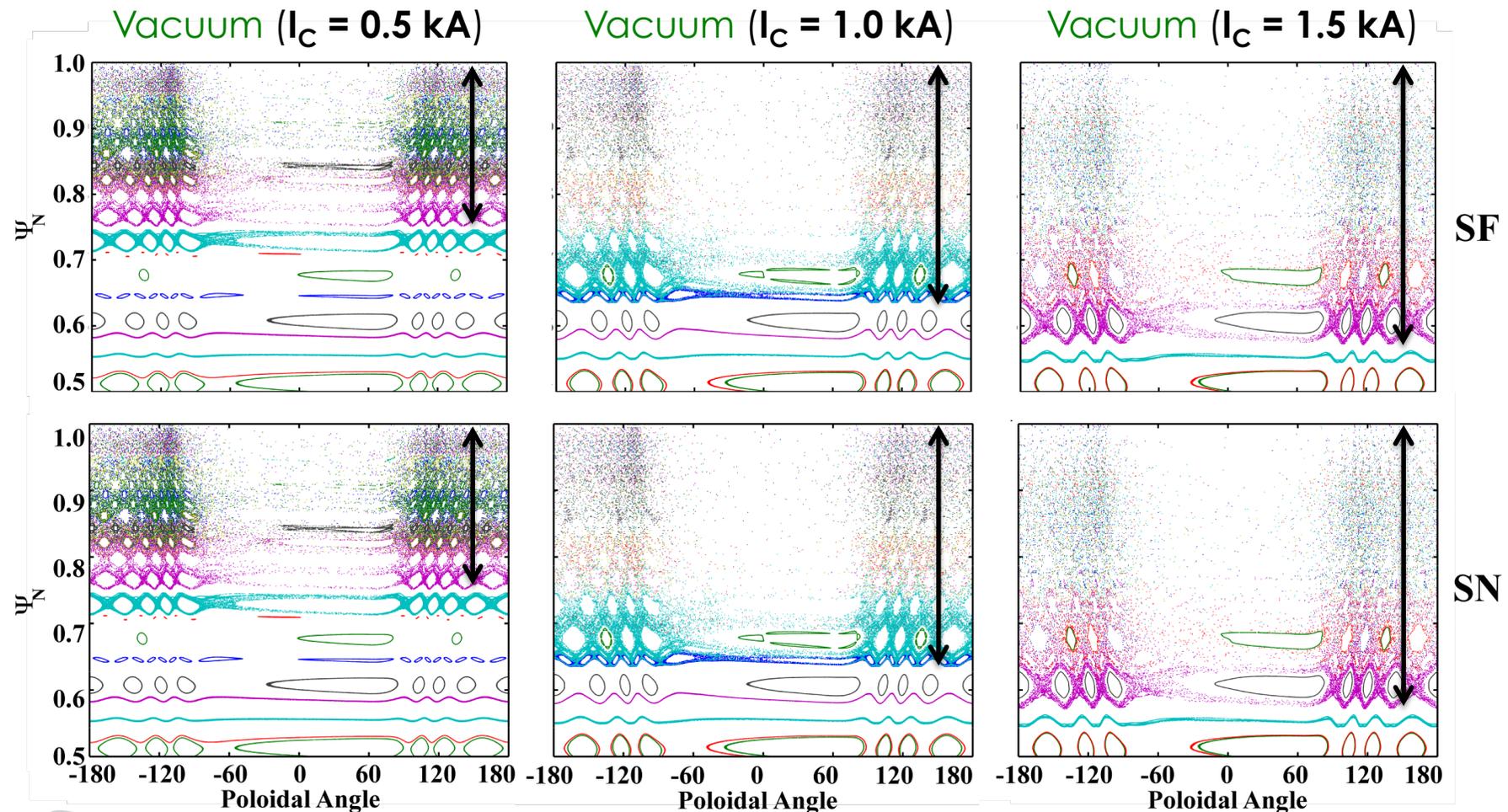
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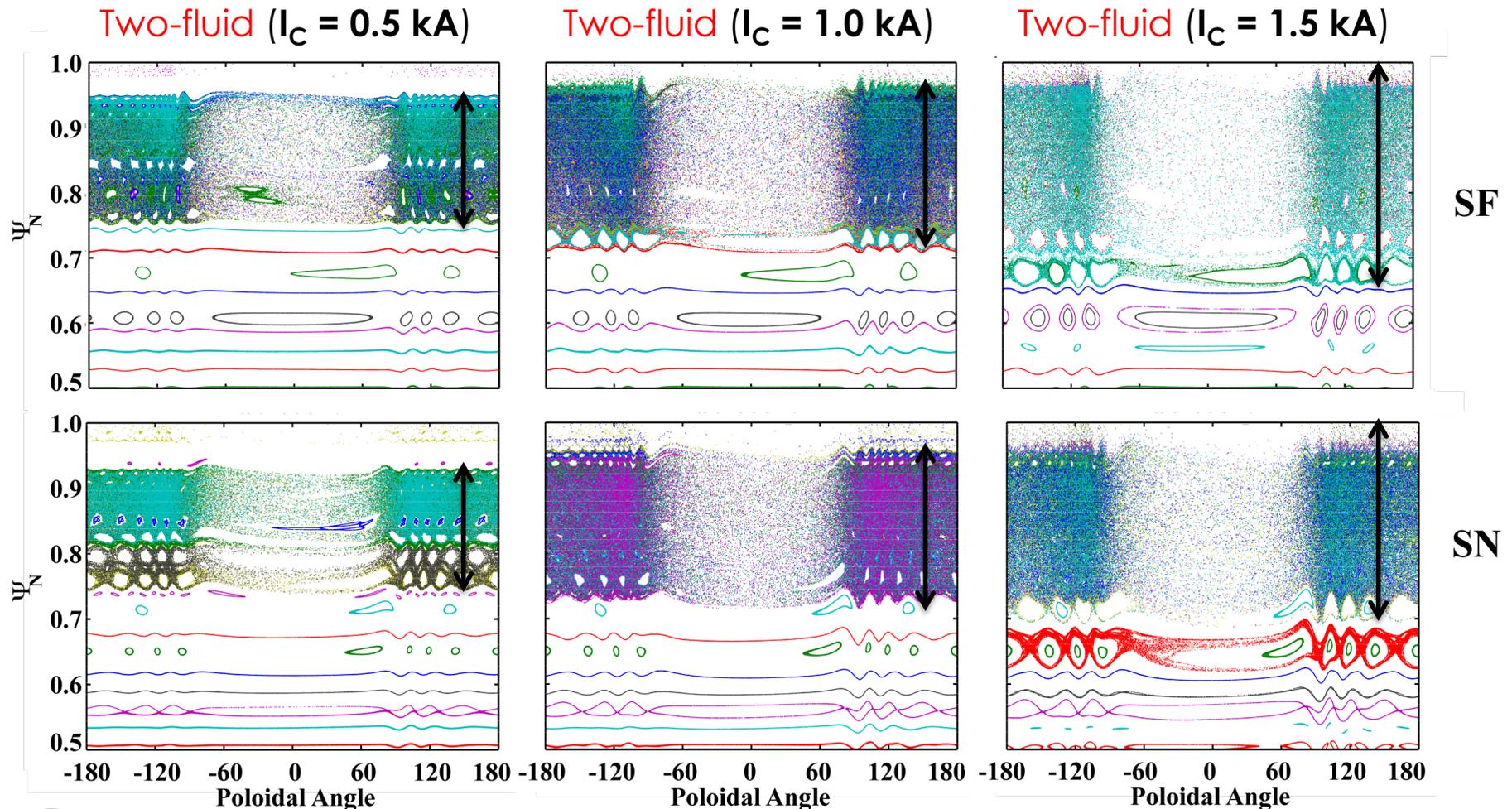
# Calculations show no difference between edge stochastization in SN and SF configurations

- As in a SN, **vacuum**, two-fluid and single-fluid calculations predict an increasing of the edge stochastization with  $I_C$  in a SF configuration



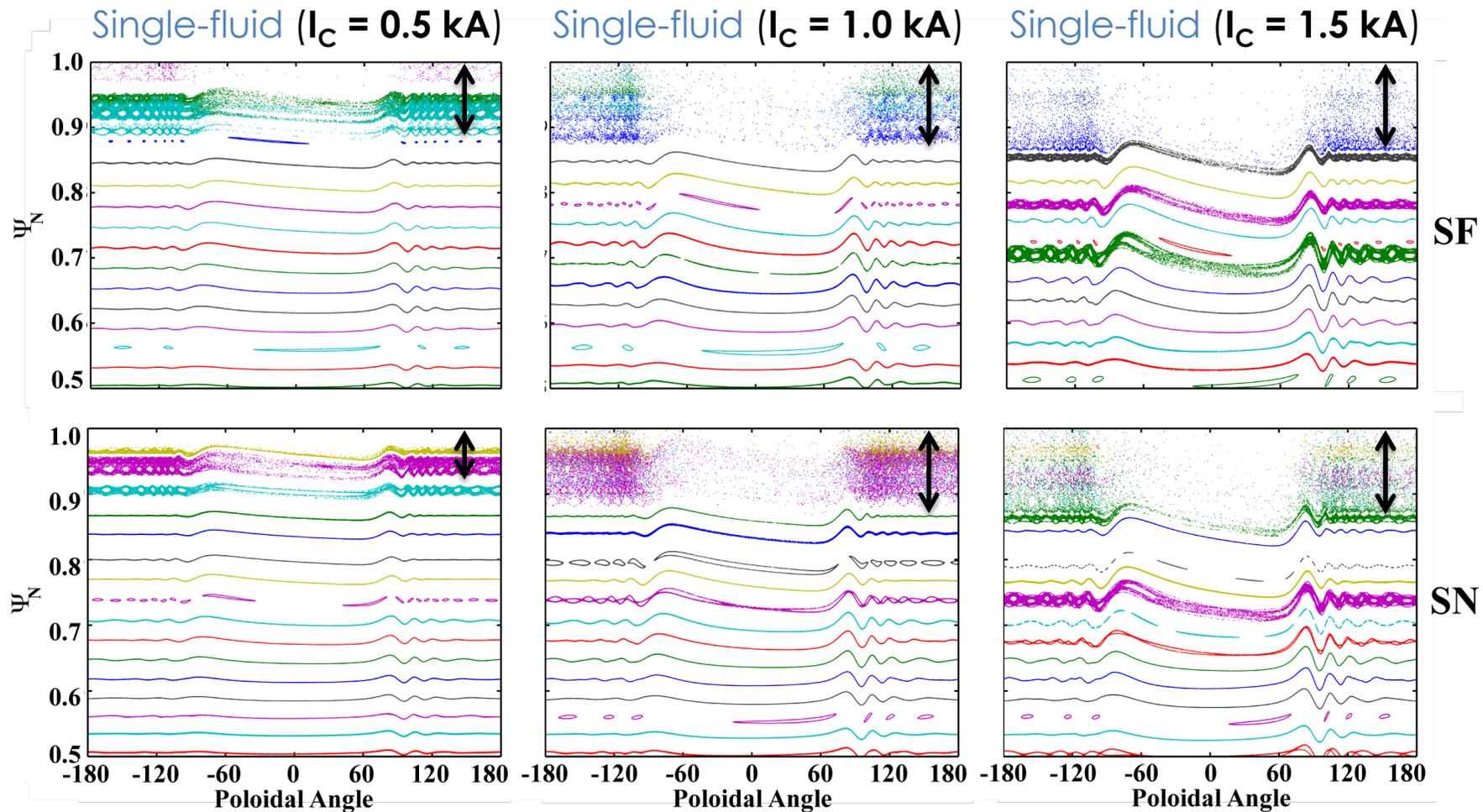
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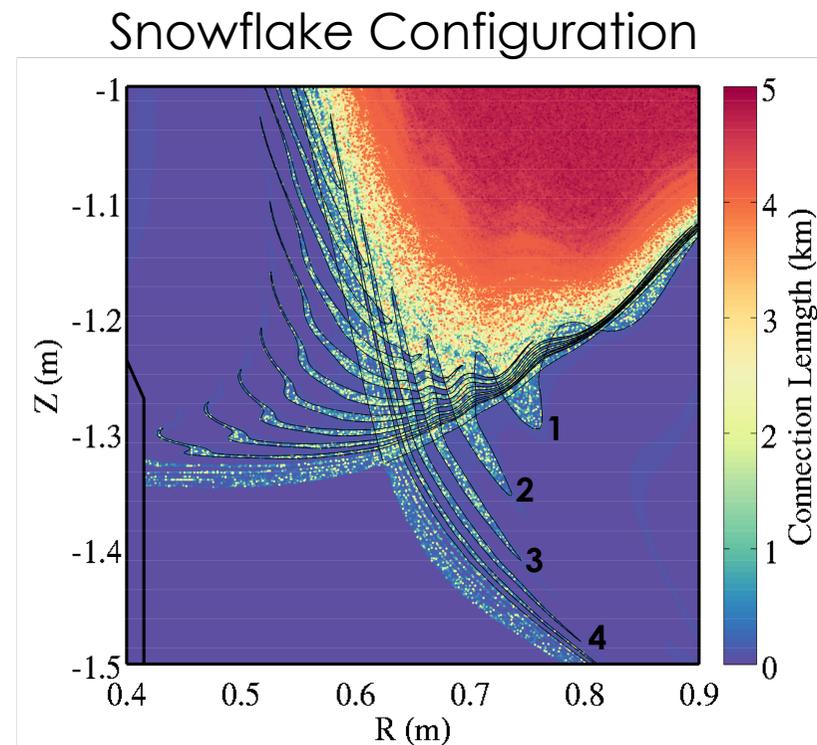
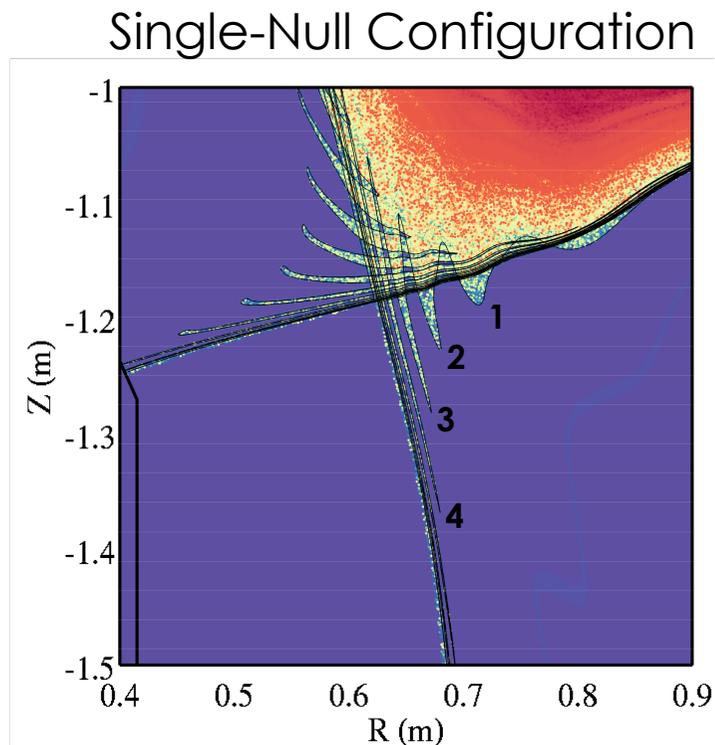
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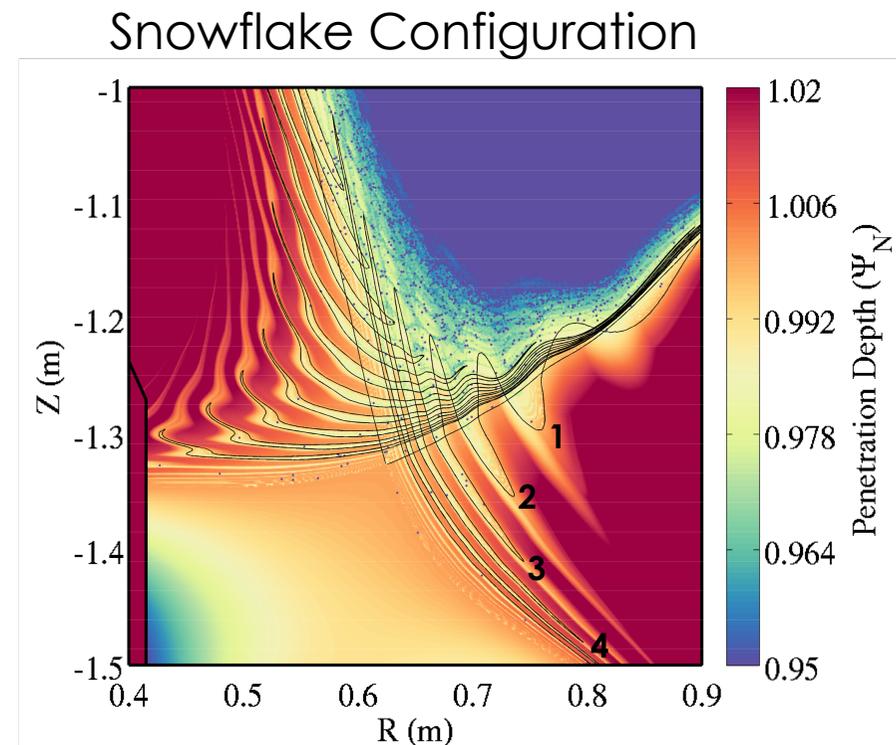
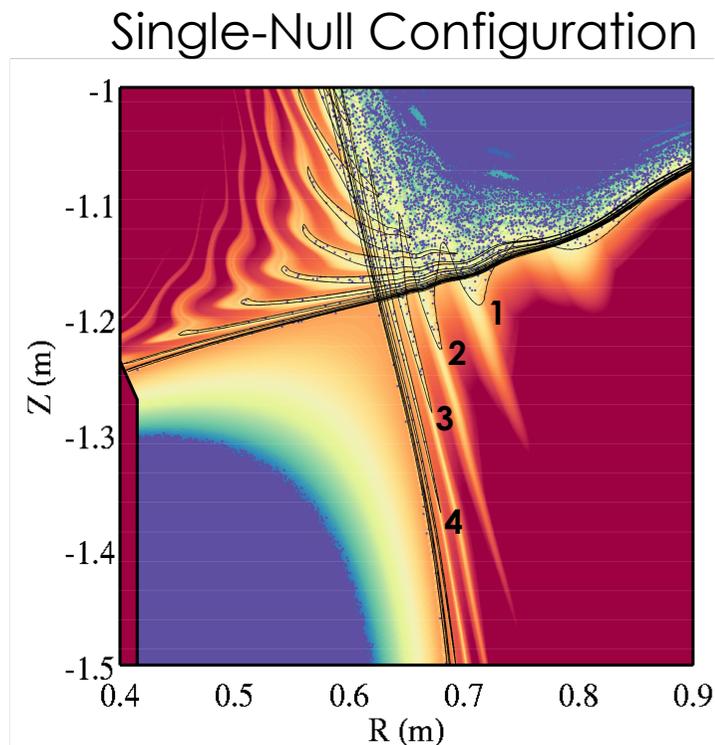
# Lower poloidal field in the null-point region of the SF configuration leads to the formation of longer lobes

- The SF configuration magnifies the effect of magnetic perturbations
  - More striations in the divertor may lead to lower peak heat fluxes



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Magnetic field lines in the null-point region of the SF divertor remain close to the edge

# Impurities can be used as a tool to manipulate the contribution of two-fluid effects to the plasma response

- **Two-fluid effects governed by ion inertial length,  $d_i$** 
  - Ions may decouple from electrons within  $d_i$

- **Ion inertial length depends on effective ion charge,  $Z_{\text{eff}}$**

$$d_i \equiv \frac{c}{\omega_{pi}} = \frac{c}{Z_{\text{eff}}} \sqrt{\frac{M_i}{4\pi n_0 e^2}}$$

- **Two-fluid effects are more significant in plasmas with low values of  $Z_{\text{eff}}$**

$$\frac{\partial n}{\partial t} + \nabla \cdot (n\mathbf{u}) = 0$$

$$n \left( \frac{\partial \mathbf{u}}{\partial t} + \mathbf{u} \cdot \nabla \mathbf{u} \right) = \mathbf{J} \times \mathbf{B} - \nabla p - \nabla \cdot \Pi$$

$$\frac{\partial p}{\partial t} + \mathbf{u} \cdot \nabla p = -\Gamma p \nabla \cdot \mathbf{u} - \frac{d_i}{n} \mathbf{J} \cdot \left( \Gamma p_e \frac{\nabla n}{n} - \nabla p_e \right) - (\Gamma - 1) \nabla \cdot \mathbf{q}$$

$$\frac{\partial \mathbf{B}}{\partial t} = -\nabla \times \mathbf{E}$$

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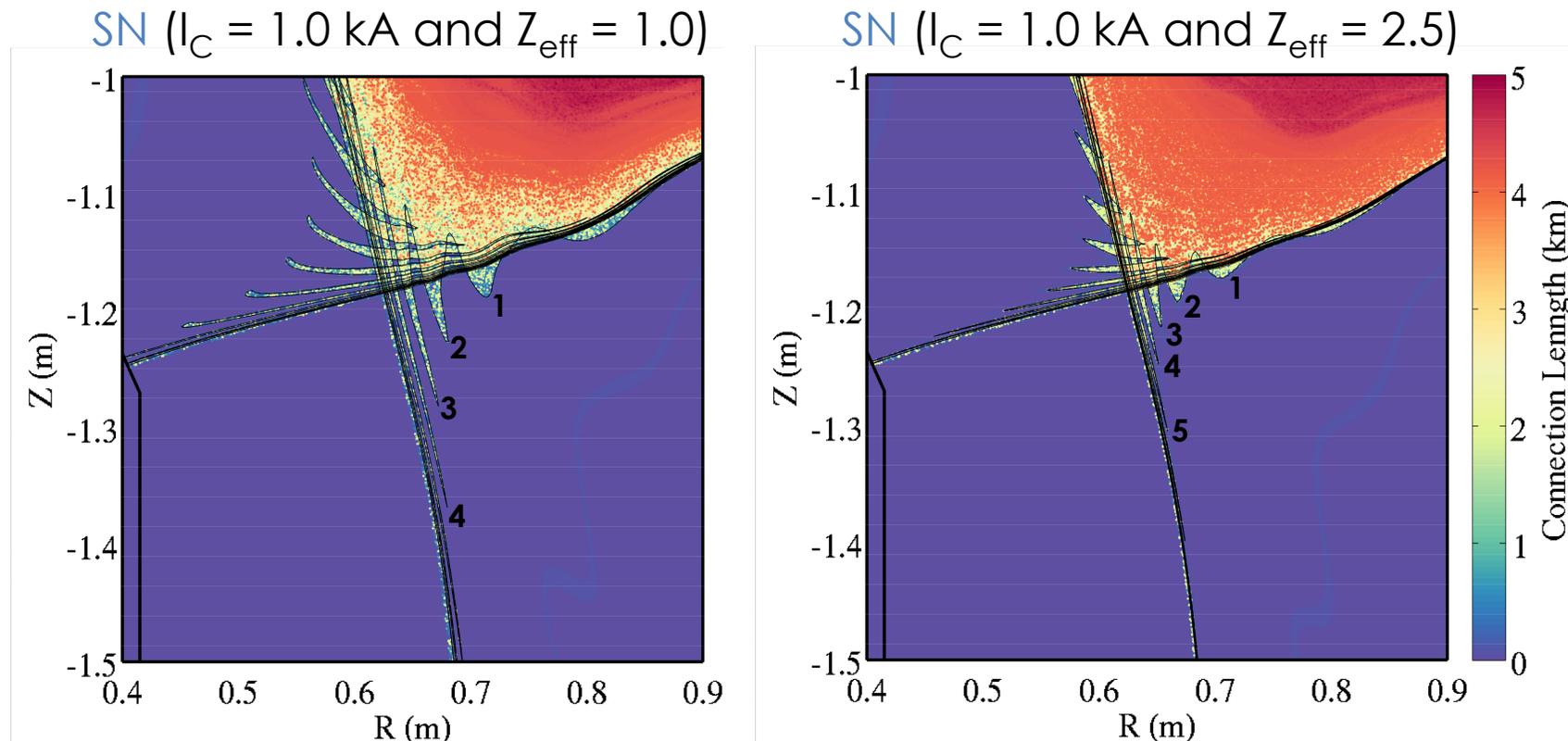
$$\Pi = -\mu \left[ \nabla \mathbf{u} + (\nabla \mathbf{u})^T \right]$$

$$\mathbf{q} = -\kappa \nabla \left( \frac{p}{n} \right) - \kappa_{\parallel} \mathbf{b} \mathbf{b} \cdot \nabla \left( \frac{p_e}{n} \right)$$

$$\mathbf{J} = \nabla \times \mathbf{B}$$

# M3D-C<sup>1</sup> calculations predict shorter lobes in plasmas with higher values of ion effective charge

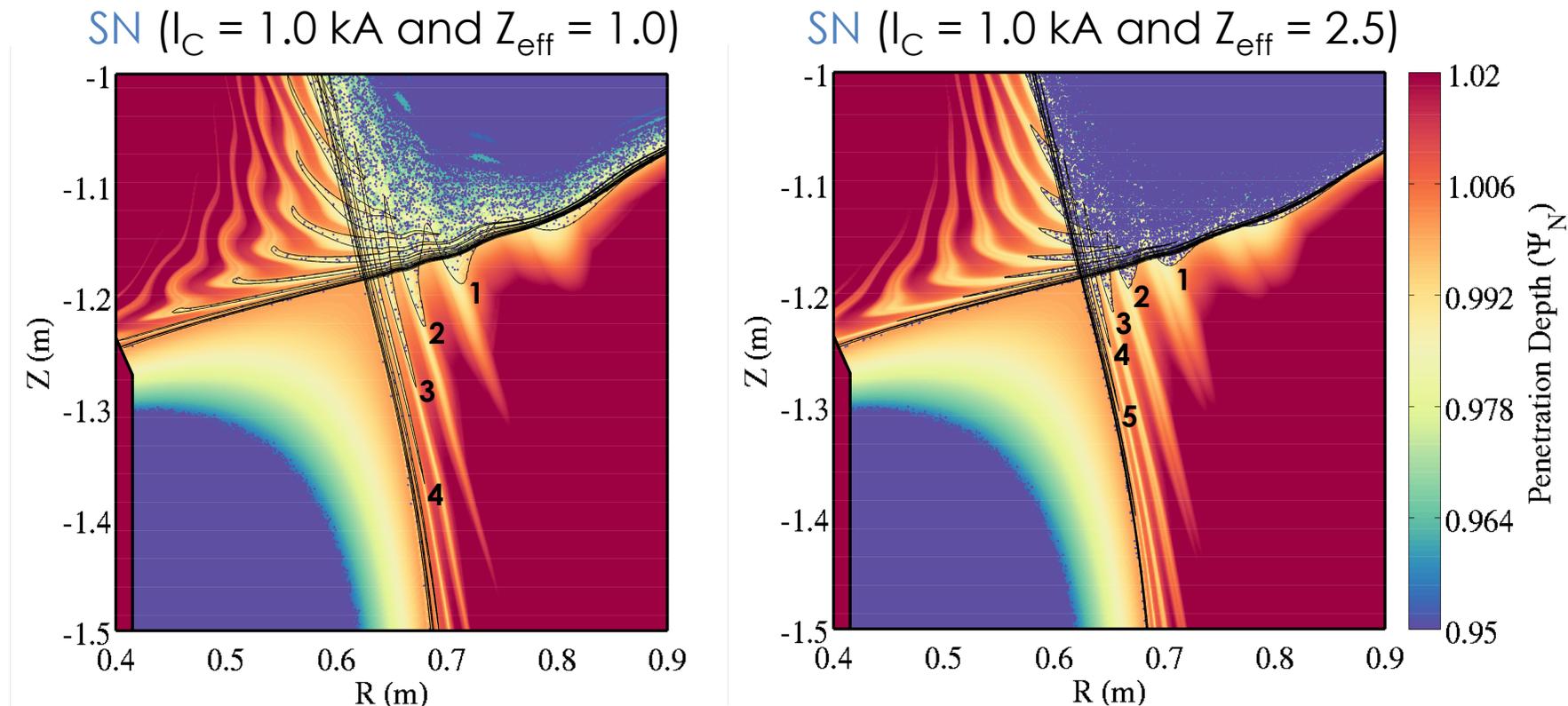
- Impurities tend to reduce two-fluid effects in both SN and SF configurations
  - Plasma respond as a single-fluid in plasmas with high  $Z_{\text{eff}}$



Magnetic field lines in lobes of higher  $Z_{\text{eff}}$  plasmas have longer  $L_C$

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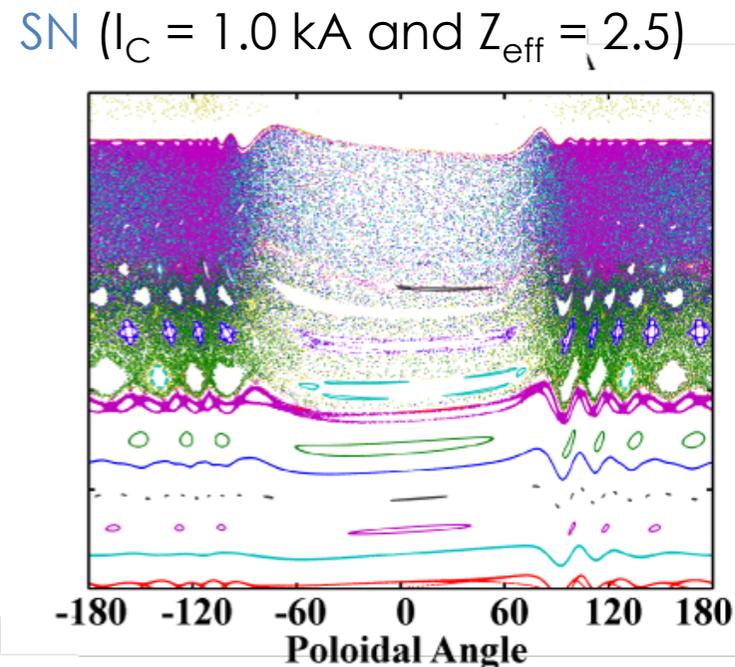
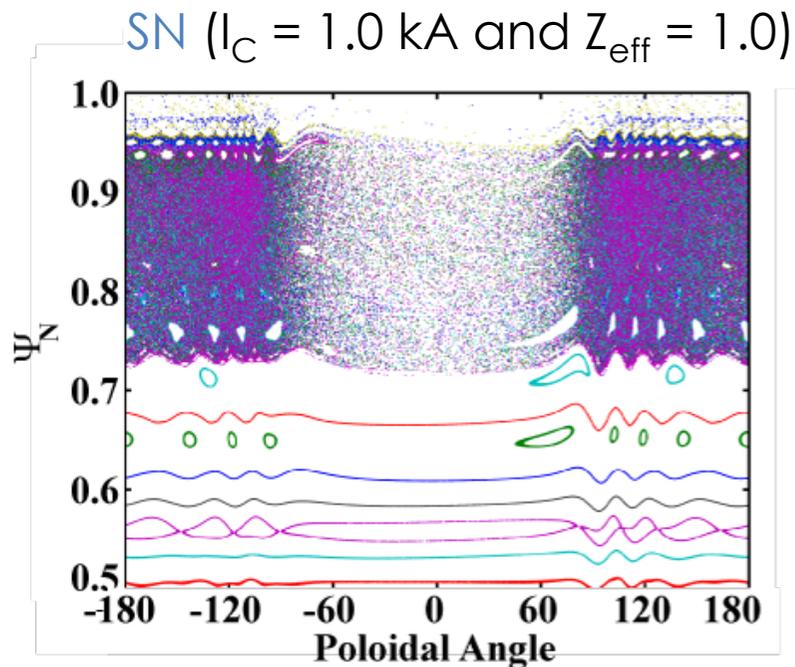
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Magnetic field lines in lobes of higher  $Z_{\text{eff}}$  plasmas goes deeper into the plasma

# Plasma edge stochasticity increases in plasmas with higher values of $Z_{\text{eff}}$

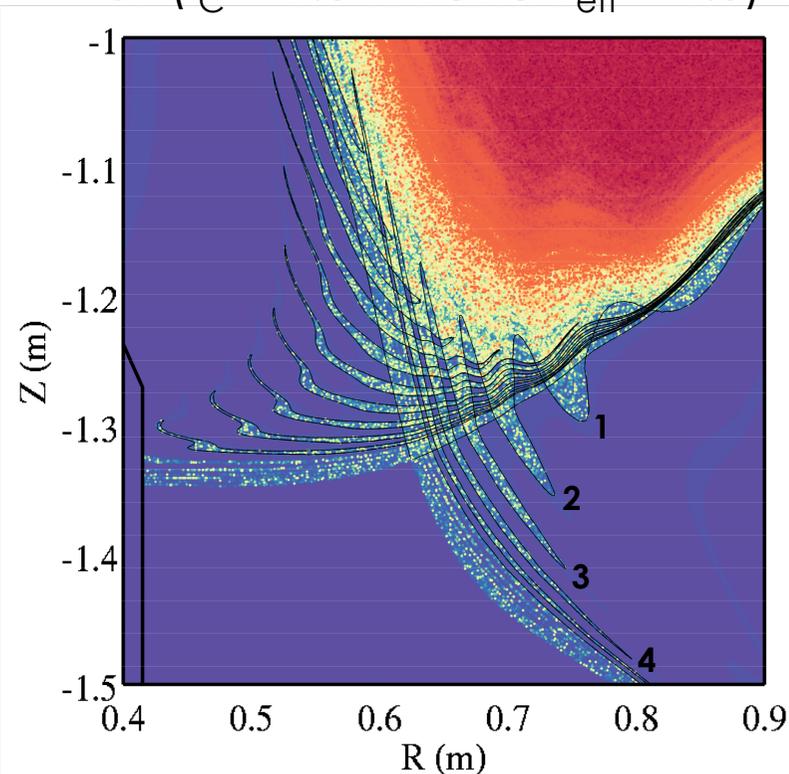
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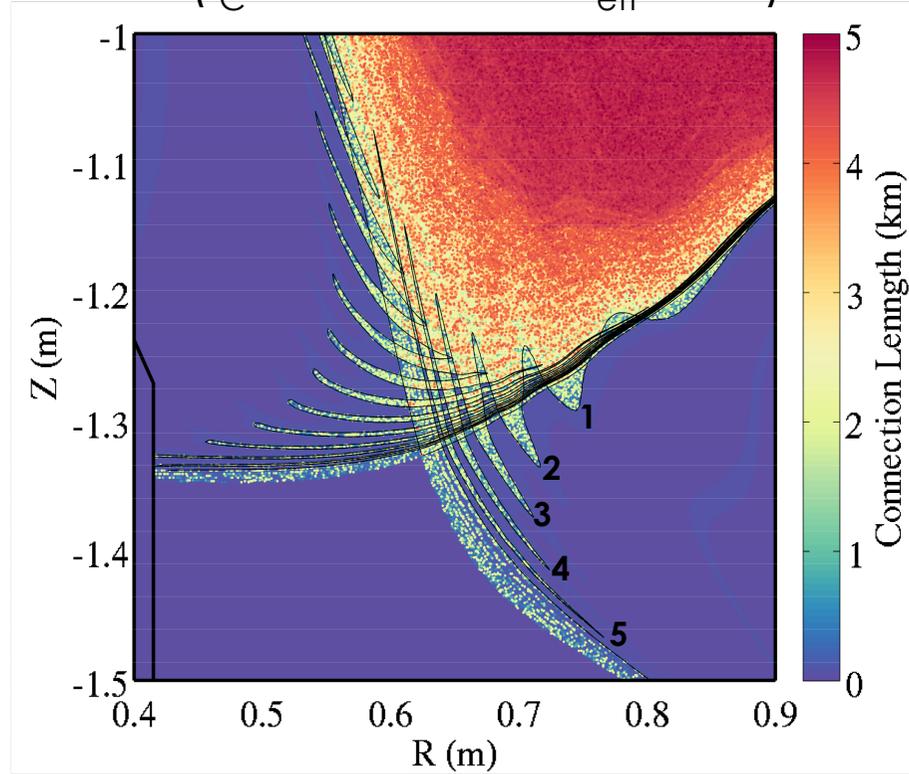
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SF ( $I_C = 1.0$  kA and  $Z_{\text{eff}} = 1.0$ )



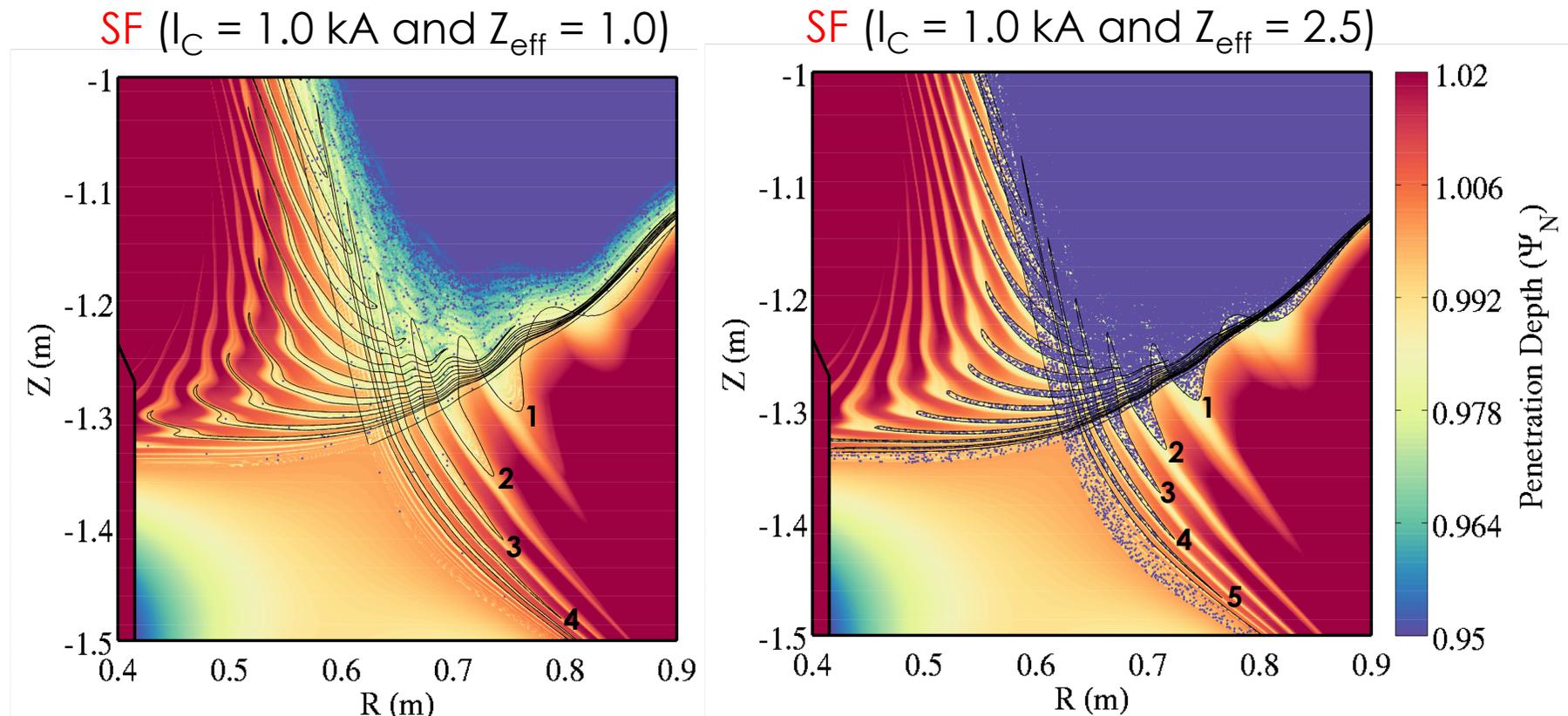
SF ( $I_C = 1.0$  kA and  $Z_{\text{eff}} = 2.5$ )



Magnetic field lines in lobes of higher  $Z_{\text{eff}}$  plasmas have longer  $L_C$

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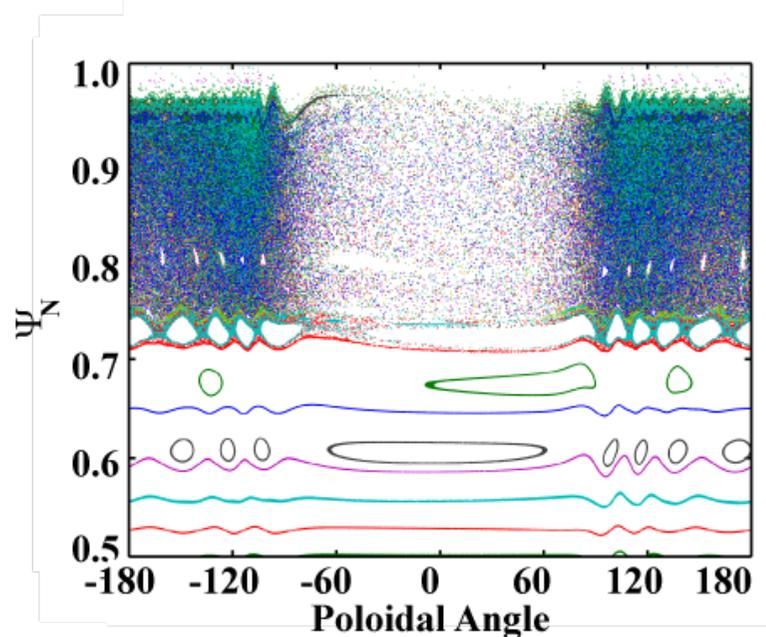


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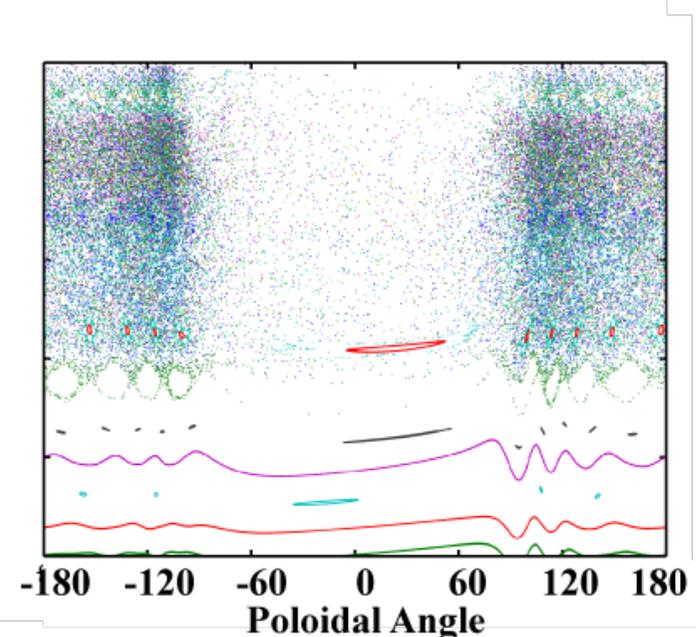
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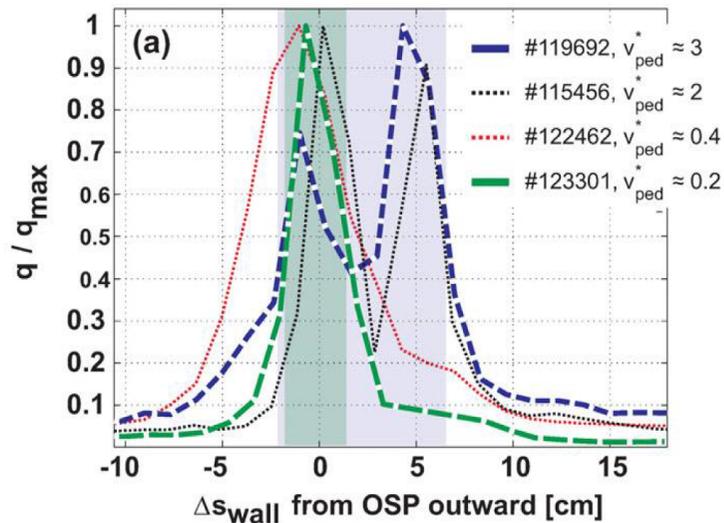


SF ( $I_C = 1.0$  kA and  $Z_{\text{eff}} = 2.5$ )



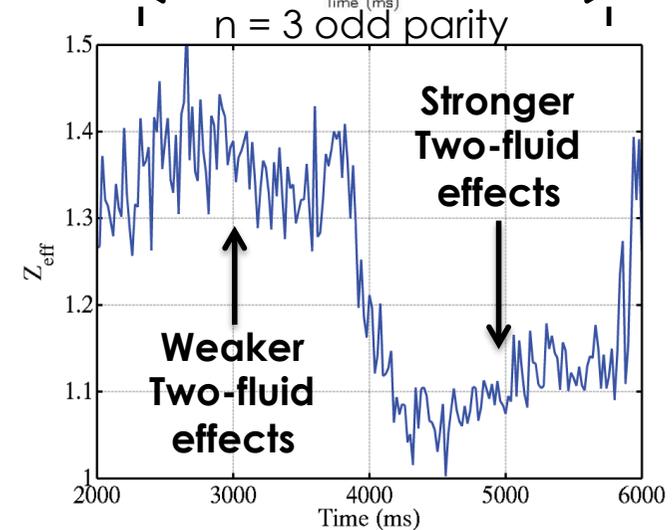
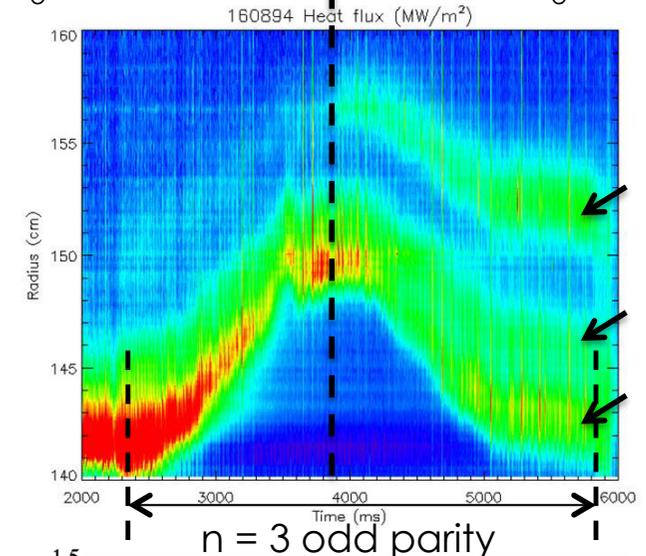
# The impact of impurities on the importance of two-fluid effects may be related to transport effects in high $\nu_e^*$ plasmas

- Heat flux splitting is visible only for  $\nu_e^* > 0.5$ 
  - Particle flux splitting occurs at lower values of  $\nu_e^*$



[M.W. Jakubowski, *Nucl. Fusion* (2009)]

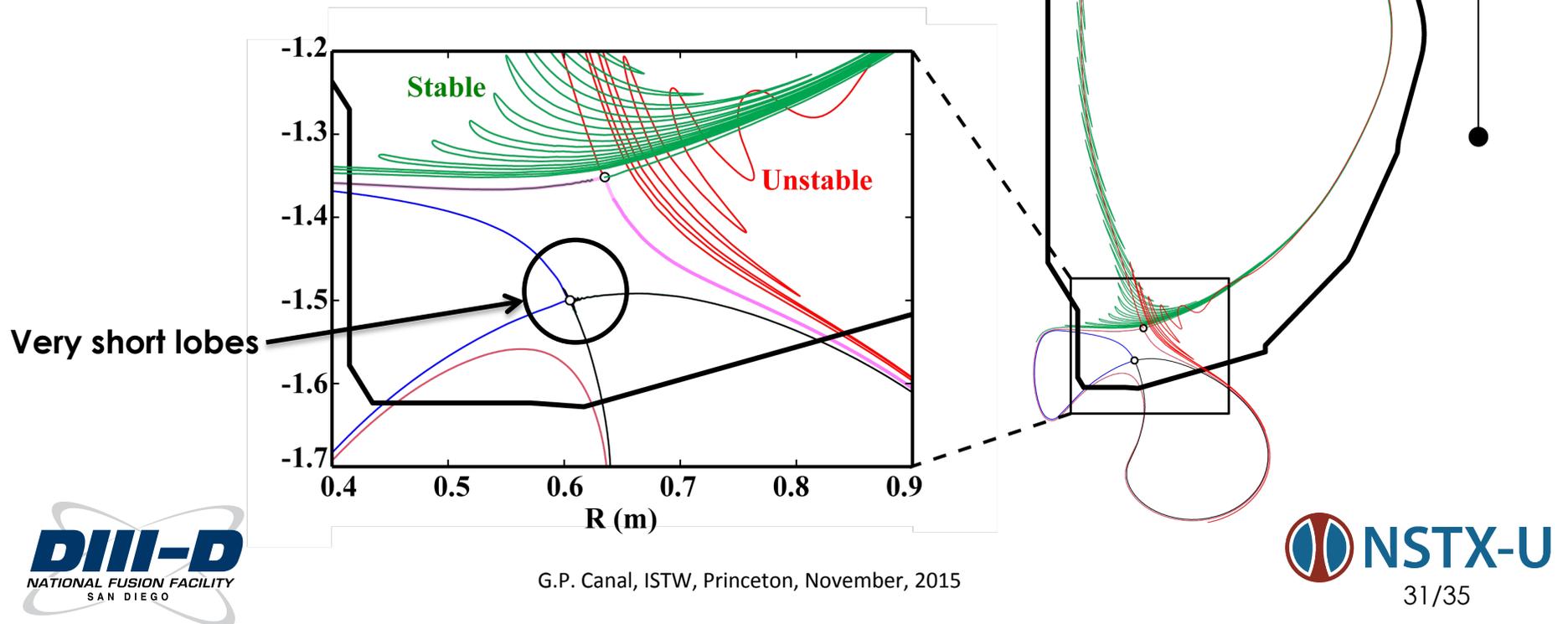
Low density  $\nu_e^* \sim 0.4$  ← → High density  $\nu_e^* \sim 1.2$



# Effect of 3D magnetic perturbations on secondary manifolds is negligible

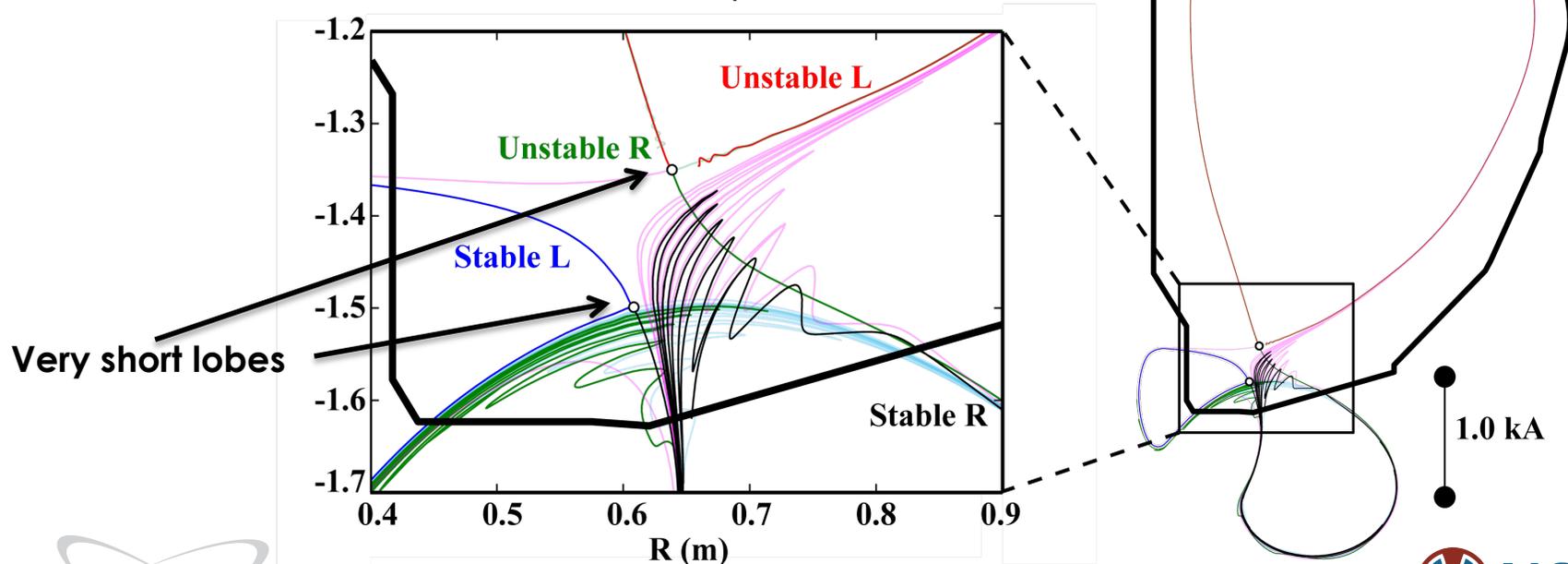
- Vacuum approach calculations show that C-coil currents have no significant effect on secondary manifolds

- Magnetic field lines in the private flux region are too far from the C-coil



# Secondary manifolds become apparent when perturbation coil is placed close to secondary x-point

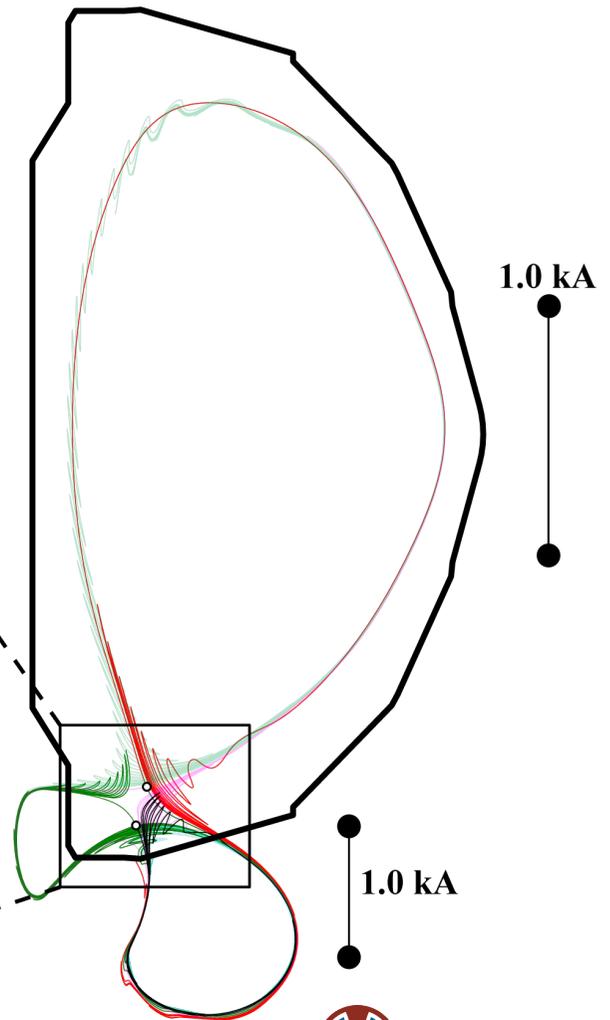
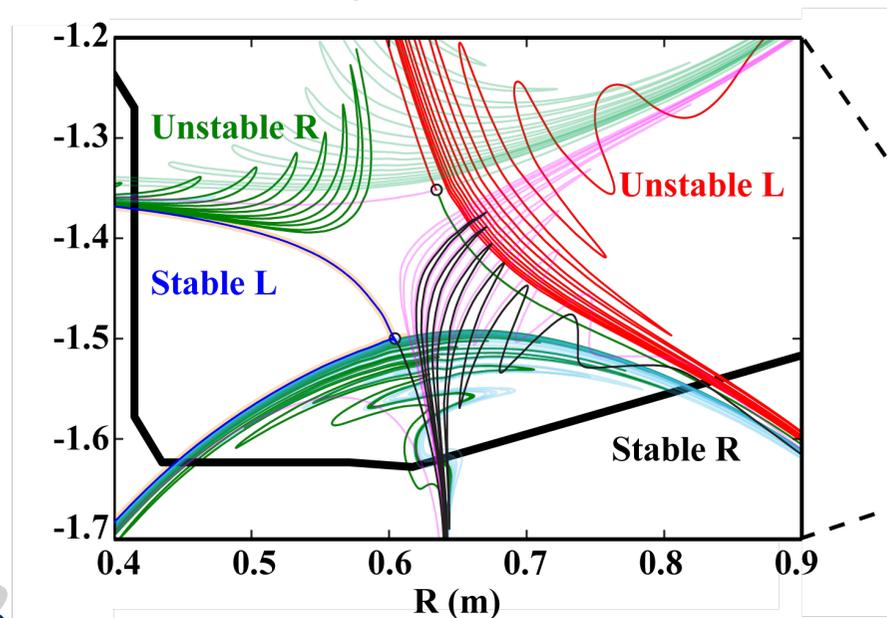
- Vacuum approach calculations show that only field lines passing close to the perturbation coil are affected
  - Manifolds are affected by radial (non-tangential) perturbed field
- Primary manifolds and left hand secondary manifolds are too far from the perturbation coil



# Primary and secondary manifolds are visible when both perturbation coils are used

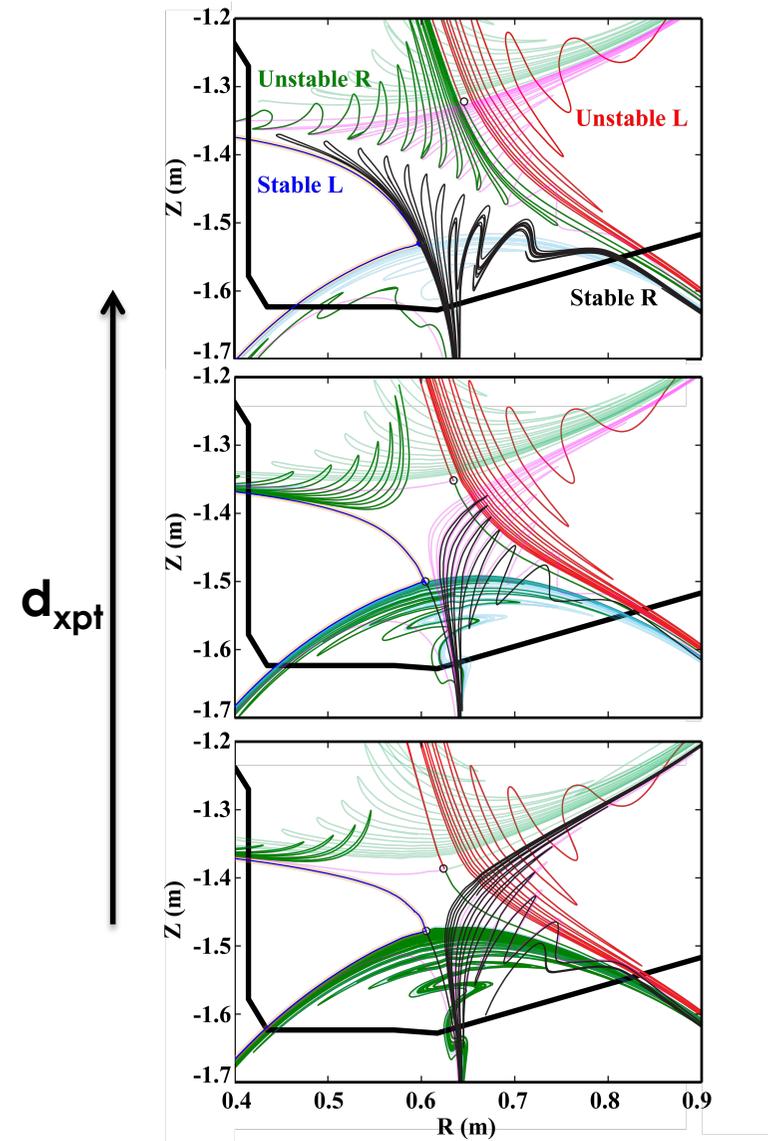
- Calculations show that, for a sufficiently close perturbation coil, both primary and secondary manifolds can be manipulated

➤ Left hand secondary manifolds are still too far from the perturbation coil



# Primary and secondary manifolds interact at sufficiently short distance between x-points

- **Vacuum approach calculations show that primary and secondary manifolds may interact at**
  - sufficiently close perturbation coils
  - sufficiently large perturbation coil currents
  - small distance between x-points
- **Interaction between manifolds may**
  - affect the edge plasma transport
  - improve the power repartition between plasma legs (reduction of peak heat flux)
  - increase divertor volume (radiated power fraction and easier access to detachment)



## Summary: Improved physics understanding & modeling of 3D fields in the SF divertor are needed to extrapolate towards larger devices

- **M3D-C<sup>1</sup> calculations predict a strong stochastization of the plasma edge with increasing perturbation coil current in both SN and SF configurations**
  - Plasma lobes in the SF are longer than in the SN configuration
- **Impurities can be used as a tool to manipulate the contribution of two-fluid effects to the plasma response**
  - Plasmas with higher  $Z_{\text{eff}}$  have shorter lobes and more stochastic edge
- **Interaction between primary and secondary manifolds may have a significant impact on**
  - plasma edge transport
  - improve power repartition between plasma legs
  - increase divertor volume