

National Spherical Torus Experiment (NSTX)

The title 'NSTX Mission, Design & Results' is written in a large, blue, serif font. It is centered over a background image of the NSTX tokamak's internal structure, which is a complex arrangement of yellow and orange metal components. The text is slightly shadowed to stand out against the background.

NSTX Mission, Design & Results

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University of Washington

Columbia University

General Atomics

Outline

- Mission of NSTX
- Engineering Overview
- Research Plan
- Accomplishments

NSTX Mission

- NSTX is an alternate concept *Proof of Principle* experiment whose mission is to demonstrate the Physics and Technology of the *Spherical Torus* (ST) plasma

USDOE Fusion Roadmap

Concept Exploration



Proof of Principle



Proof of Performance



Energy Technology



DEMO

Physics Mission: Demonstrate ST Plasma

- **Efficient magnetic confinement**
 - High beta (e.g. $\beta = 2\mu_0\langle p \rangle / B_0^2$ 25 - 40%)
 - Reduced toroidal field requirement for confinement
- **Natural elongation**
 - Reduced poloidal field requirement for shape control
 - Flux expansion in divertor regions, reduced power density on walls
- **Enhanced MHD stability**
 - Reduced turbulence, improved confinement and transport
- **High pressure driven (bootstrap) current f_{BS} 50 - 90%**
 - Offsets the fact that an ST reactor cannot rely on inductive current drive using a central solenoid
- **Reduced disruption severity**

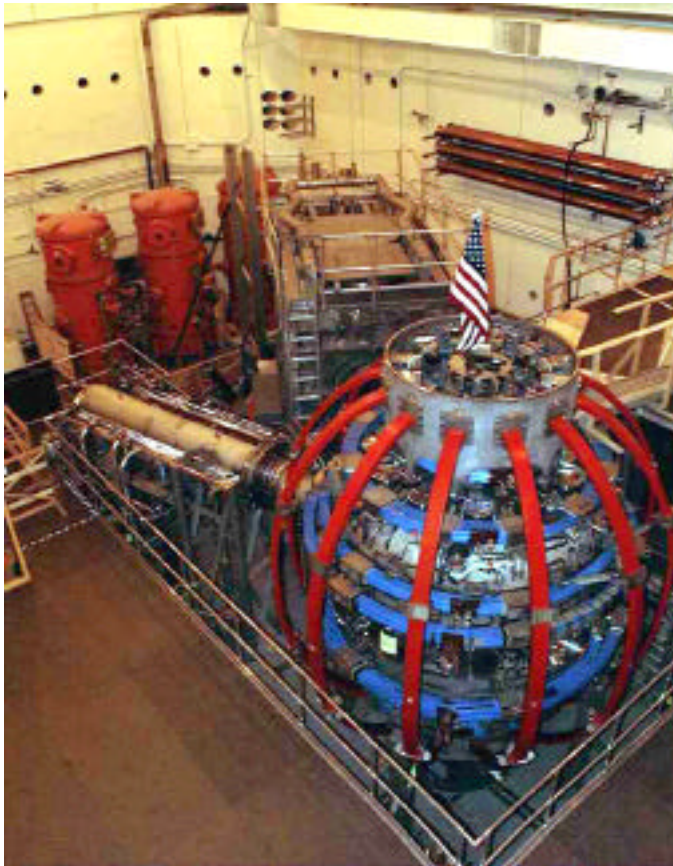
Technology Mission

- **Demonstrate Non-Inductive Heating & Current Drive**
 - High Harmonic Fast Wave (HHFW) Heating & Current Drive
 - Coaxial Helicity Injection (CHI)
- **Construct an ST machine**
 - Low aspect ratio, high performance Center Stack
 - Special provision for CHI

Strategy

- **Begin with thin, low aspect ratio OH Solenoid**
 - Initial experimentation using double swing inductive drive (T 0.5s)
 - Single swing plasma with sustainment using HHFW & CHI (T->5 sec)

NSTX Facility and Engineering



NSTX at 1st Plasma

- Located in the Hot Cell adjacent to the TFTR Test Cell, making extensive use of existing facilities:

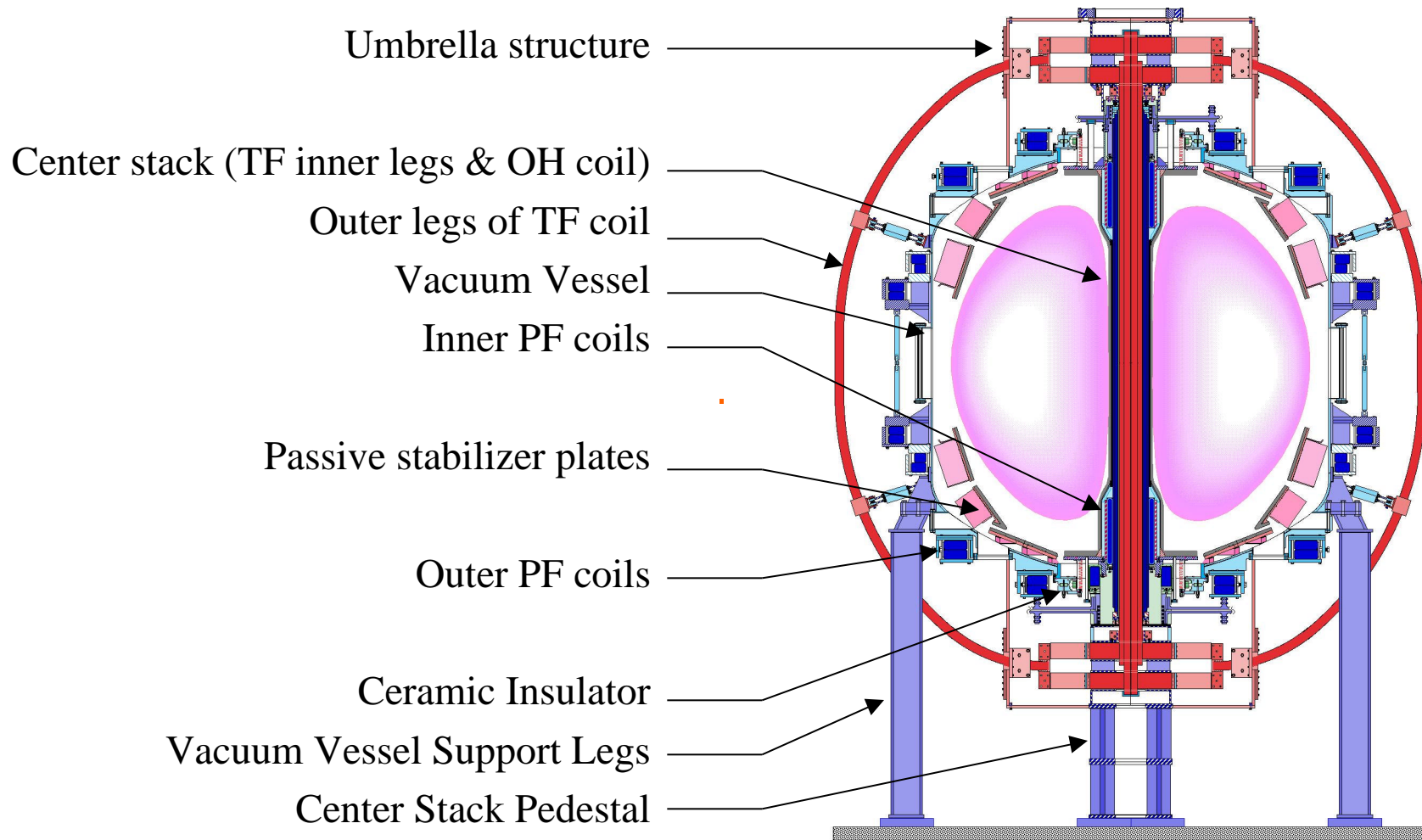
- AC power
- Magnet power supplies
- RF systems
- NBI systems
- Water systems
- Buildings and HVAC systems
- Many components from TFTR

- 1st Plasma in February 1999
- Total Project Cost \$23.6M
- Site Credits \$77M

NSTX Ratings and Parameters

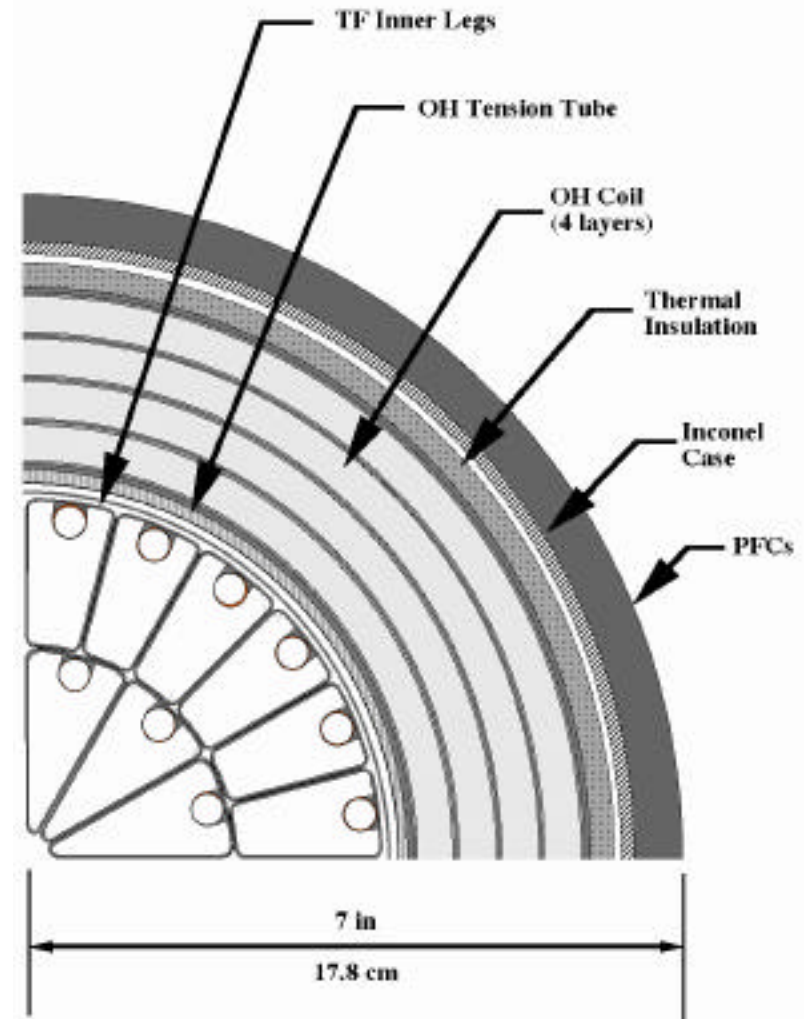
| | | |
|---------------------------|--------------------------------------|--------------------------------|
| Plasma | Major Radius (R_0) | 85.4 cm |
| | Aspect Ratio (R/a) | 1.26 |
| | Volume | 12m ³ |
| | Elongation | 1.6 2.2 |
| | Triangularity | 0.2 0.5 |
| | Current | 1.0 MA (1.5MA) |
| | Ramp Time | 0.2 - 0.4 sec |
| | Flat Top (Inductive) | 0.5 sec per 600 sec |
| | Flat Top (non-Inductive) | 5.0 sec per 300 sec |
| Toroidal Field | Field @ R_0 | 3.0/6.0 kG |
| Ohmic Heating | Flux (double swing) | 0.9 volt-sec |
| | Initiation Loop Voltage @ R_0 | 5.0 volt/turn |
| Heating/ Current Drive | High Harmonic Fast Wave (HHFW) RF | 6.0 MW, 30MHz, 5 sec |
| | Coaxial Helicity Injection (CHI) | 500kA via 50kA injection @ 1kV |
| | Neutral Beam Injection Upgrade (NBI) | 5.0 MW, 80kV, 5 sec |
| Pre-Ionization | Electron Cyclotron | 30kW, 18GHz, 0.1 sec |
| Bakeout | Bakeout Temperature | 350°C PFCs, 150°C VV |

NSTX Machine Cross Section

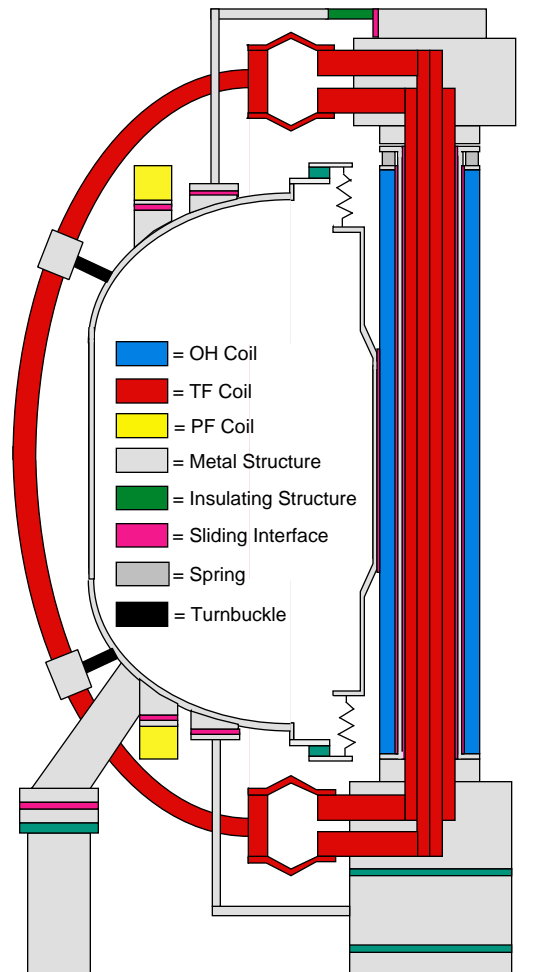


Center Stack Arrangement

- Compact nested TF Inner Legs
 - efficient use of space
 - ease of manufacture
- OH Tension cylinder
 - launching load reaction
 - ease of manufacture
- Four layer, 2-in-hand OH
 - high performance solenoid
 - rapid cool down time
- OH-CS Casing gap
 - R 10mm, 0.4" for thermal insulation, diagnostics & installation clearance
 - Microtherm insulation T 500C



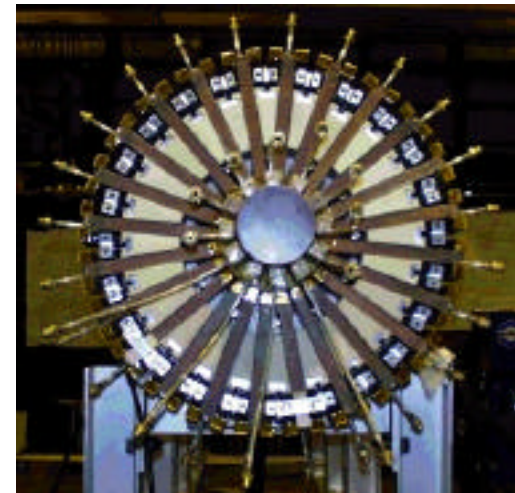
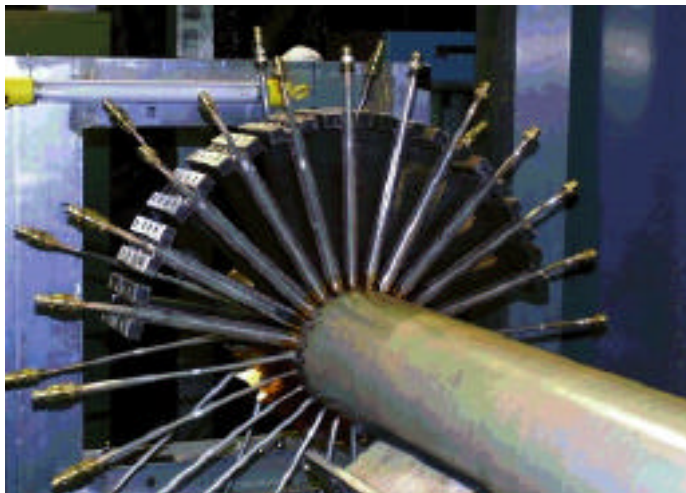
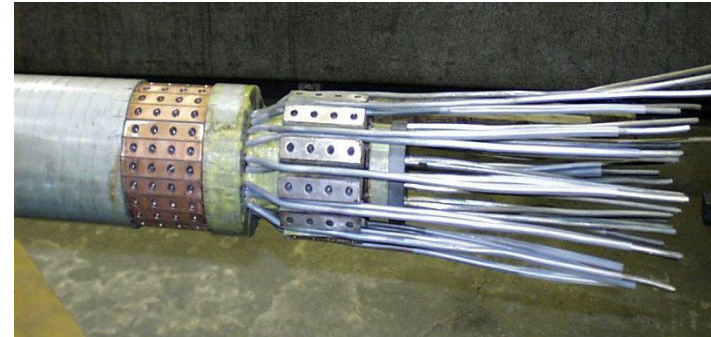
Mechanical Support Scheme



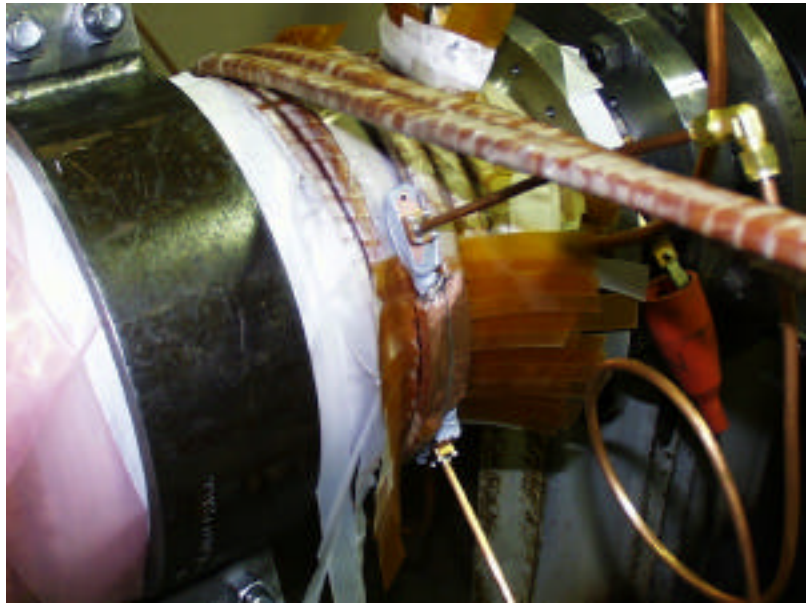
- Center Stack rests on pedestal on floor
- TF Inner Leg assembly thermal growth
 - slides inside OH tension tube
 - connects to outer legs via flex joint
 - connection to top umbrella via sliding spline joint
- TF Inner Leg assembly torsion
 - hub ass'ys transfer load via umbrella to outer VV
- TF Outer Leg dead weight and overturning moment
 - reacted to outer VV via turnbuckles
- OH Thermal Growth
 - slides over tension tube and inside CS casing
 - compresses washer stack at top
- Center Stack Thermal Growth absorbed by bellows
- VV Thermal Growth
 - sliding joints to legs, umbrellas, PF coils, and spline

TF Inner Leg Assembly

- 12 inner + 24 outer = 36 turns
- Each turn water cooled
- Radial flags secured by wedged hub assembly
- 72kA/turn, 1kV produces 6kG at R_0
- 11.8 MPa (1.7ksi) insulation shear

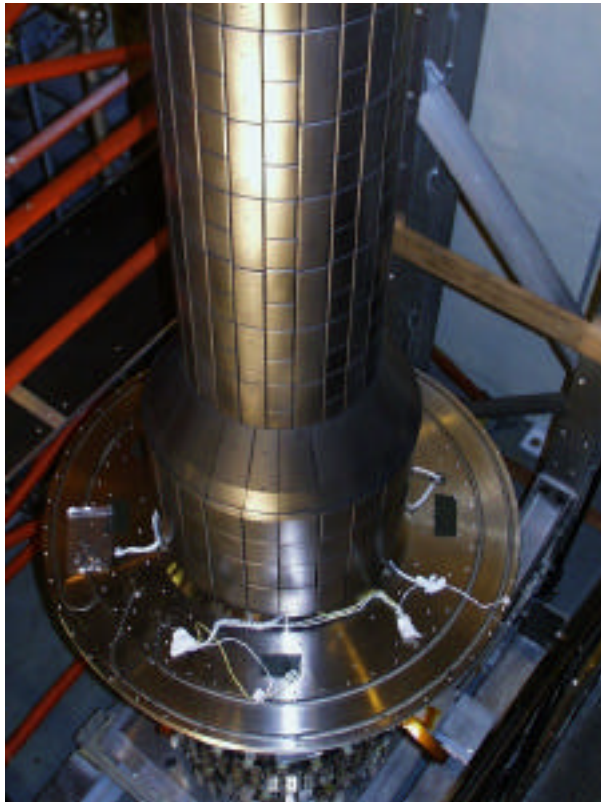


OH Solenoid



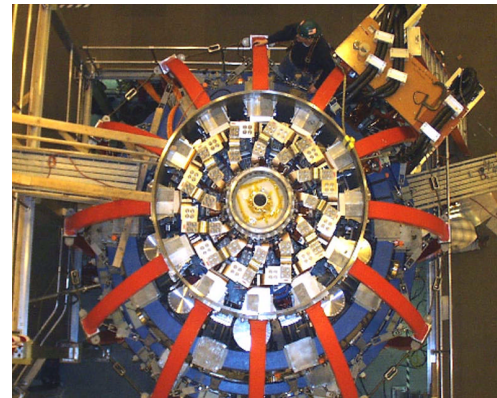
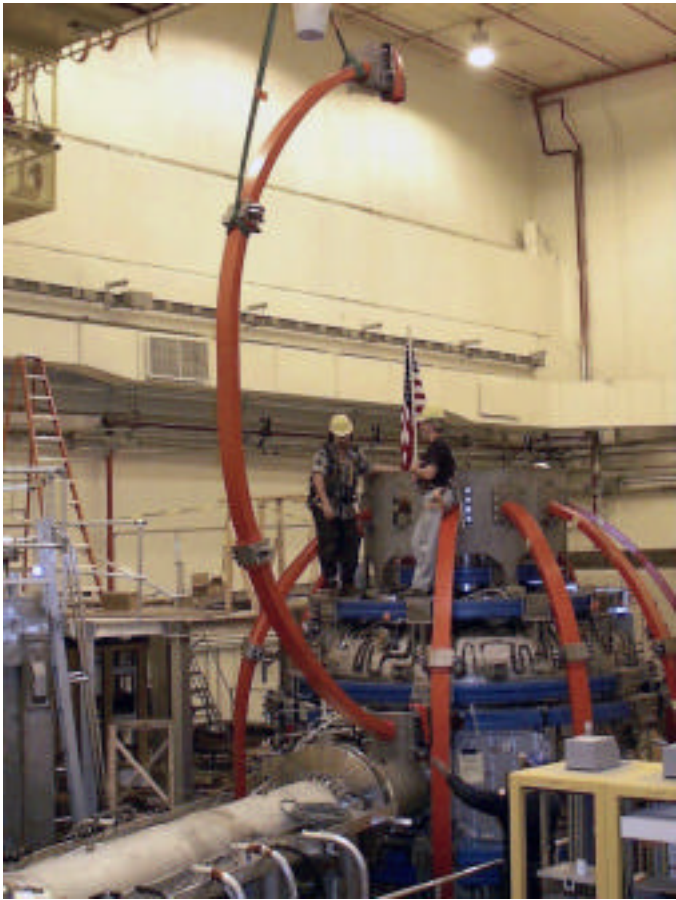
- approx. 1000 turns in 4 layers wound 2-in-hand
- 8 parallel cooling paths, 600 second cool down
- +/- 24kA/turn, 6kV produces 0.6 volt-sec flux swing
- Central B approx. 8T, 138 MPa (20ksi) Cu stress

Center Stack Assembly



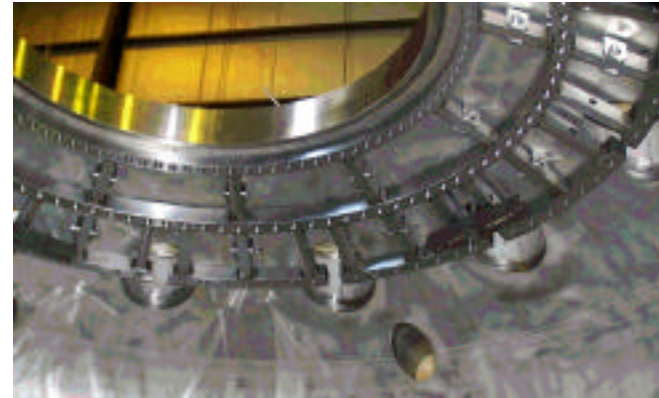
- Compact interlocking CFC tiles on Inner Wall ($R=14\text{mm}$, $0.55''$)
- Center Stack Assembly is completely removable

TF Outer Legs



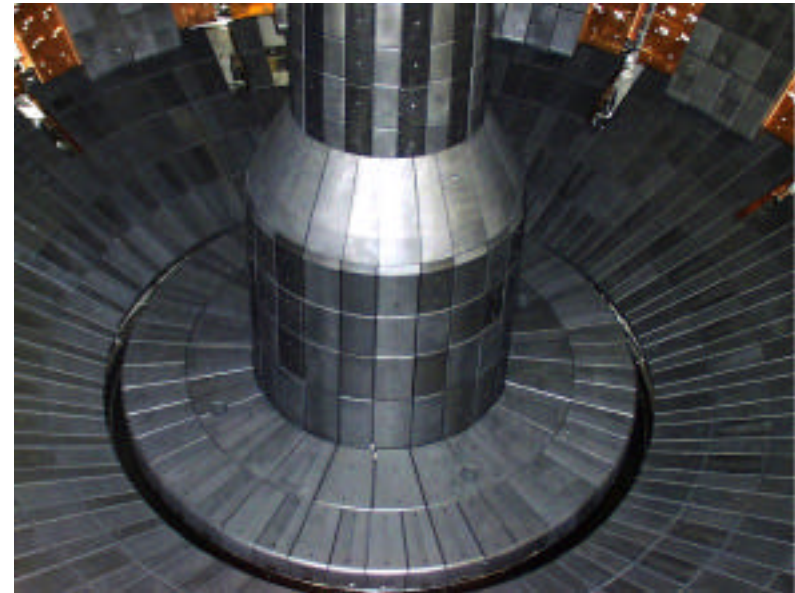
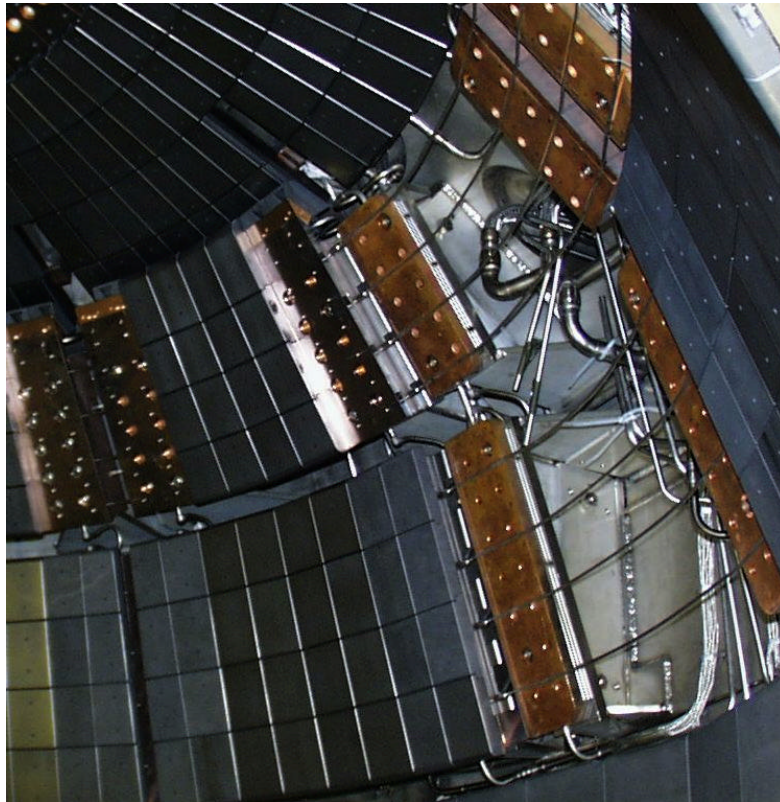
- Demountable bow shaped outer legs with turnbuckle supports

Vacuum Vessel



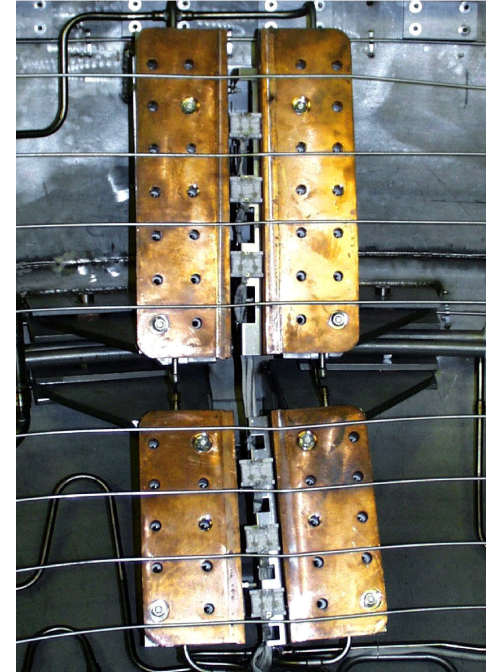
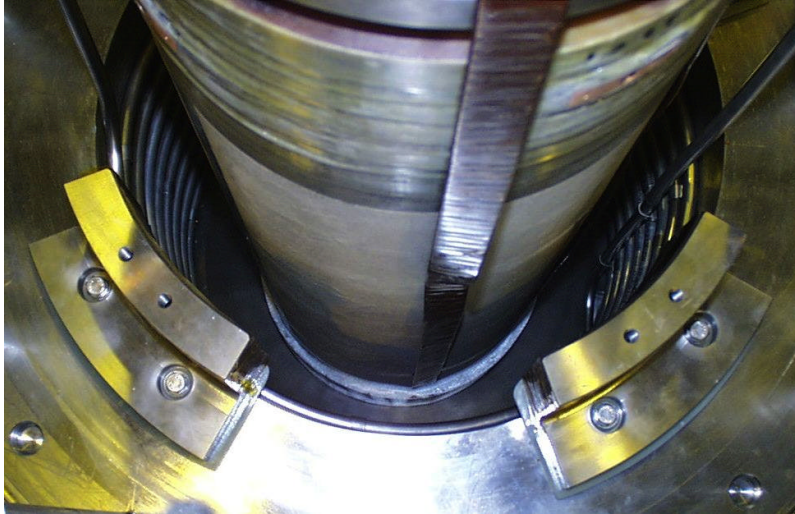
- Continuous 304 Stainless Steel VV (thickness = 16mm, 5/8")
- Sliding supports for outer PF coils and internal hardware

Internal Hardware & Plasma Facing Components



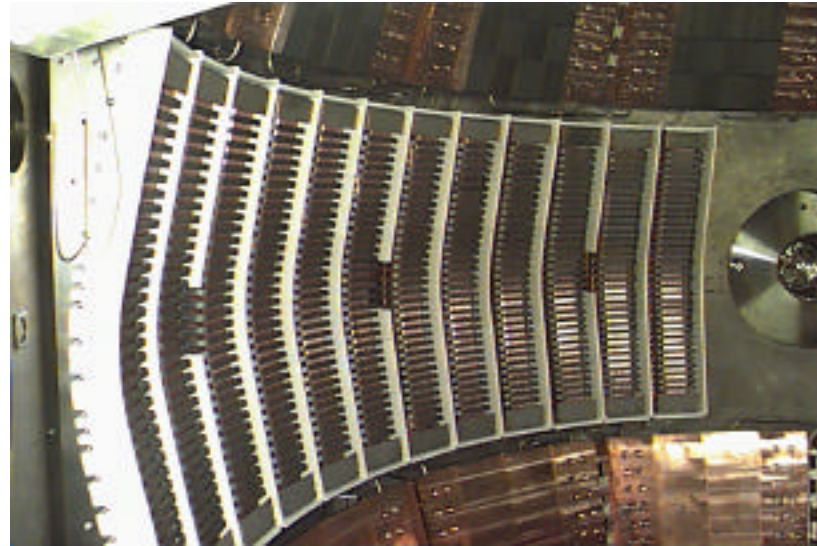
- CuCrZr passive stabilizer plates with heating/cooling system
- approx. 3000 tiles (design provided by ORNL)

Diagnostic Sensors



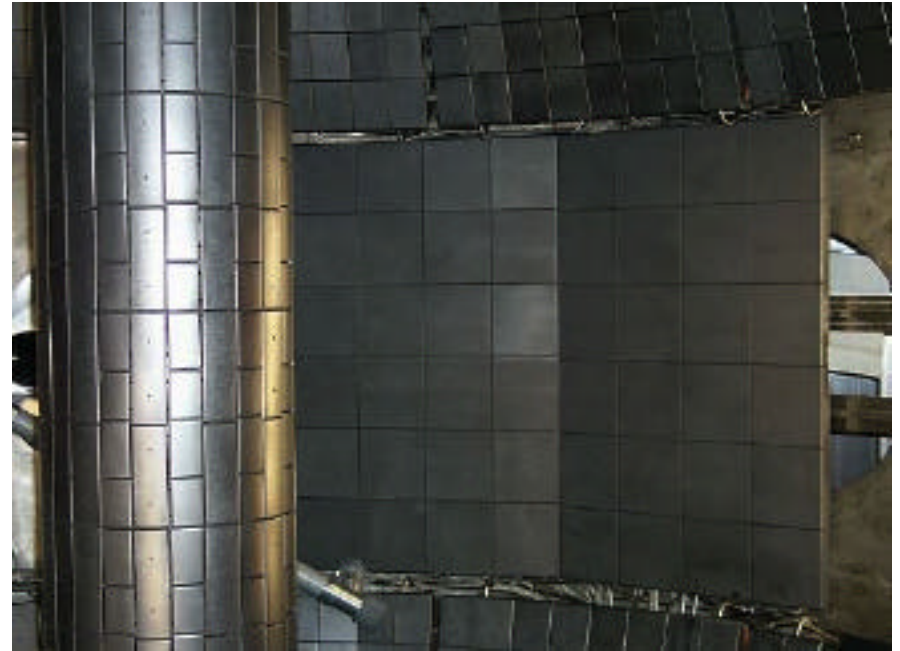
- Compact Ip Rogowski's in center stack ($R=3.5\text{mm}$, $0.135''$)
- High temp (up to 600C) in-vessel sensors
- 17 Rogowskis, 105 flux loops, 135 Mirnovs, 24 Langmuirs, 128 TCs

RF Systems



- High Harmonic Fast Wave (HHFW) RF
 - 6MW @ 30MHz, absorption mainly by electrons
 - Six PPPL sources, TFTR transmission lines, new 12 strap antenna
 - Tuning and matching network designed by ORNL
- Electron Cyclotron (EC) Pre-ionization system provided by ORNL

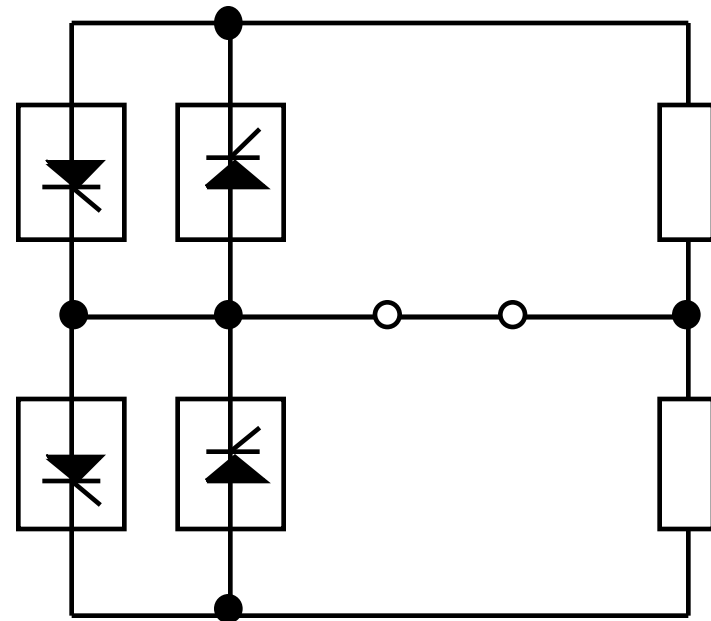
Neutral Beam Injection (NBI)



- One TFTR Beam Line @ 5MW, 80kV, 5 seconds

Magnet Power Supplies

- Extensive use of existing facilities
- One MG set to buffer load from grid
 - S_{max} 200MVA, W_{max} 170MJ
- Modular TFTR rectifier design
 - 74 six-pulse bridges available
 - Direct digital control from central computer
- Advanced 3-wire, antiparallel configuration
 - 4-quadrant operation
 - separate control of upper and lower PF coil pairs



Key Engineering Features

- Compact, removable Center Stack
 - Nested two-tier TF inner leg conductors
 - Four layer, two-in-hand high performance OH solenoid
 - Tight tolerances and precise assembly
 - Compact inner wall PFC tile design
 - Miniature diagnostic sensors
 - Microtherm insulation
- Unique support schemes and mechanical load paths
- Extensive use of TFTR facility and components

Research Plan

| | | | | | |
|--------|--|---|--|--|--|
| TOPICS | <ul style="list-style-type: none"> • Ohmic studies • Initial CHI • Initial HHFW | <ul style="list-style-type: none"> • Transport • Full HHFW • Macro-stability | <ul style="list-style-type: none"> • Full CHI • Plasma-wall • $-E$ integration | <ul style="list-style-type: none"> • Turbulence • Active Stabil. • Edge control | <ul style="list-style-type: none"> • Noninductive • Integration >> E |
| (FY99) | (FY00) (14 weeks) | (FY01-FY03) (40weeks) | | (FY04-FY06) (40weeks) | |

 *1st Plasma February 99*
 *1 MA*  *200kA CHI*  *NBI, 4MW HHFW*

Capabilities

Plasma Current
Pulse
HHFW Power
NBI Power
CHI Startup
Toroidal Beta
Bootstrap
Control
Measure

Inductive

- 0.5 MA
- 0.5 s
- 4 MW
- 0.2 MA
- current, R, shape
- $T_e(r)$, $n_e(r)$

Assisted Non-Inductive

- 1 MA
- 1 s
- ~ 6 MW
- 5 MW
- 0.5 MA
- 25%
- 40%
- heating, density
- $j(r)$, $T_i(r)$, flow, edge

Full Non-Inductive

- ~ 1 MA
- 5 s
- ~ 6 MW
- ~ 5 MW
- ~ 0.5 MA
- 40%
- 90%
- profiles, modes
- turbulence

NSTX 2001



Parameters

Baseline and (Achieved)

Elongation ≤ 2.2 (2.5)

Triangularity ≤ 0.6 (0.5)

Plasma Current

1 MA (1.07 MA)

Toroidal Field

0.3 to 0.6 T (0.45 T)

Heating and CD

5 MW NBI (3.2 MW)

6 MW HHFW (4.2 MW)

0.5 MA CHI (0.26 MA)

Pulse Length

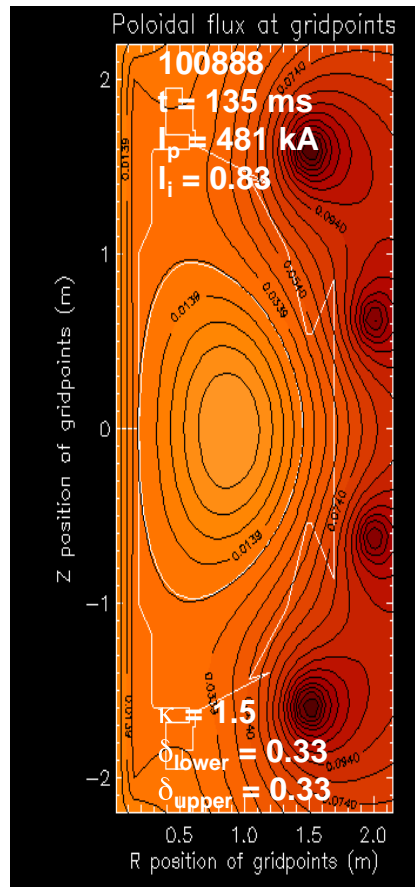
≤ 5 sec (0.5 sec)

Objectives of Initial Phase of Research Have Been Accomplished and Exceeded

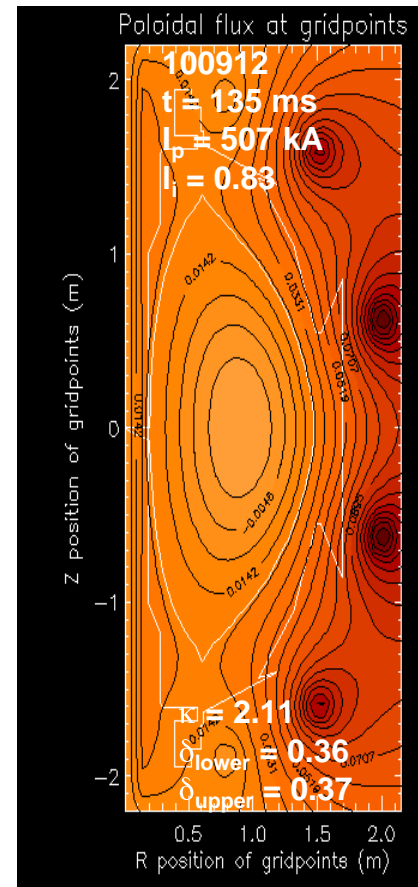
- Explored Inductive (OH) Operating Regime
 - Established target equilibria
 - Studied confinement trends, operating limits, and limiting mechanisms up to 1 MA
- Achieved $\langle \tau \rangle = 22\%$ with Neutral Beam Injection (NBI)
 - Confinement looks very good
- High Harmonic Fast Waves (HHFW) used for electron heating
 - Significant increase in $T_e(0)$
- Non-inductive startup using Coaxial Helicity Injection (CHI)
 - Significant toroidal currents generated
- H-mode has been observed

Variety of Ohmic Plasmas Have Been Produced

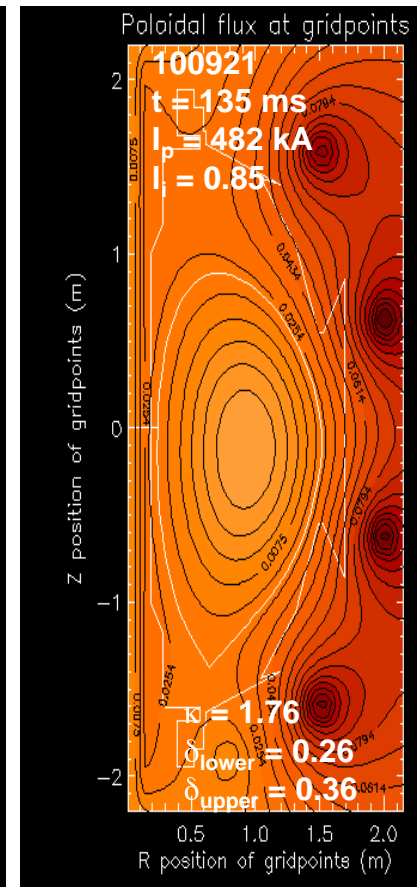
Wall Limited



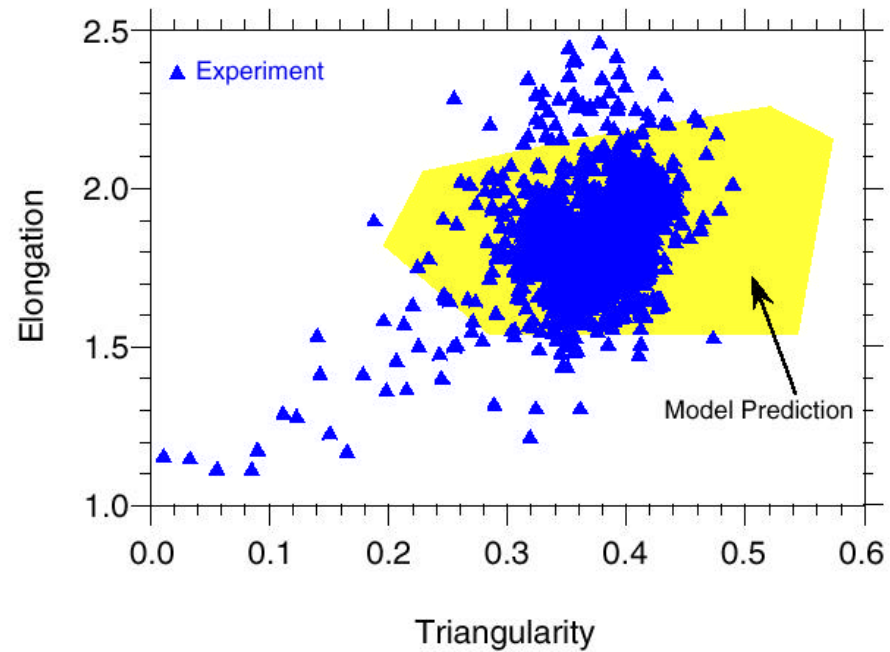
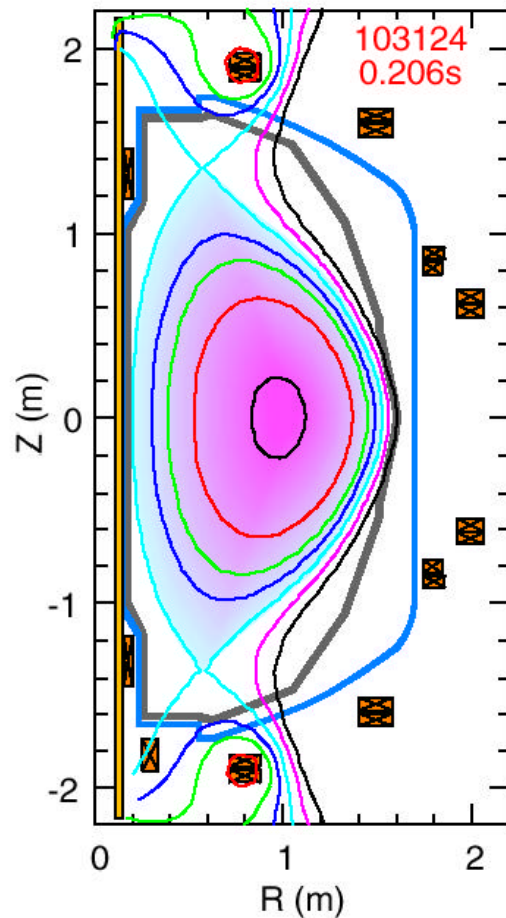
Double-Null



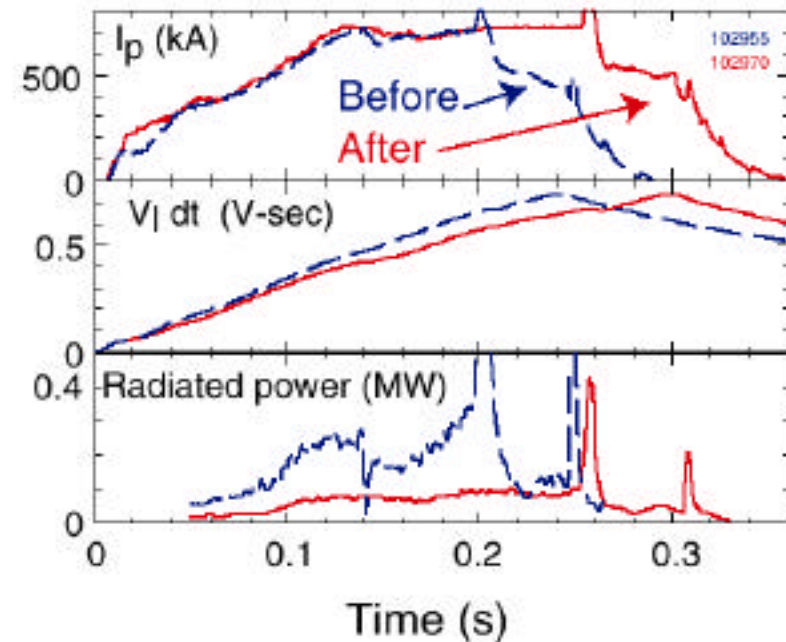
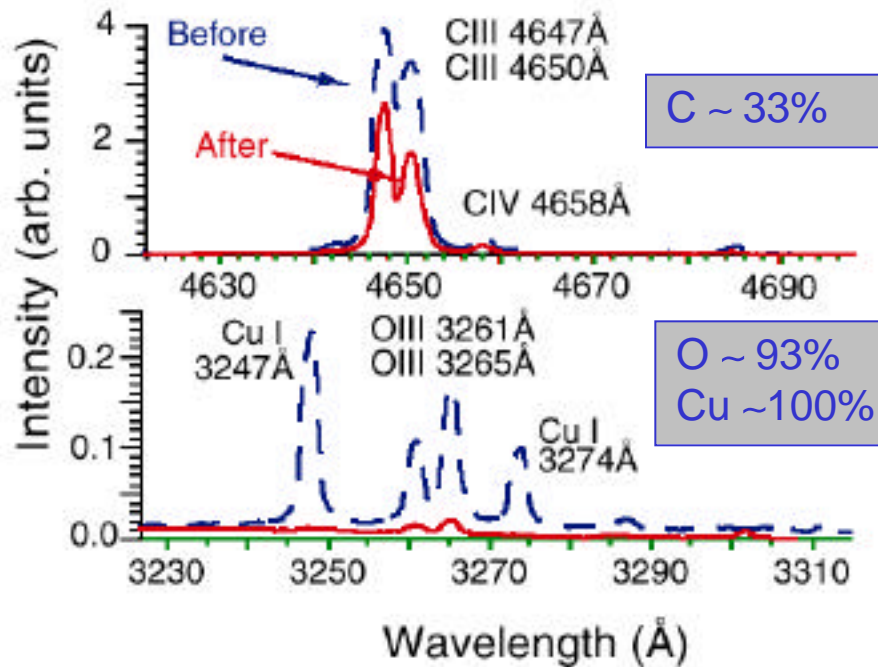
Single-Null



Range of Plasma Shapes Has Been Explored

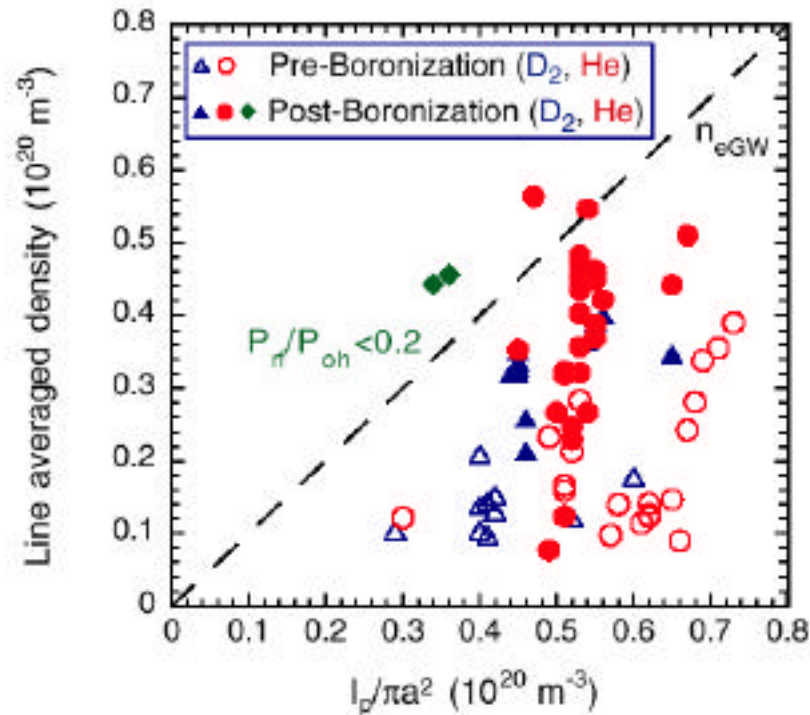


Boronization Used To Reduce Impurities and Flux Consumption

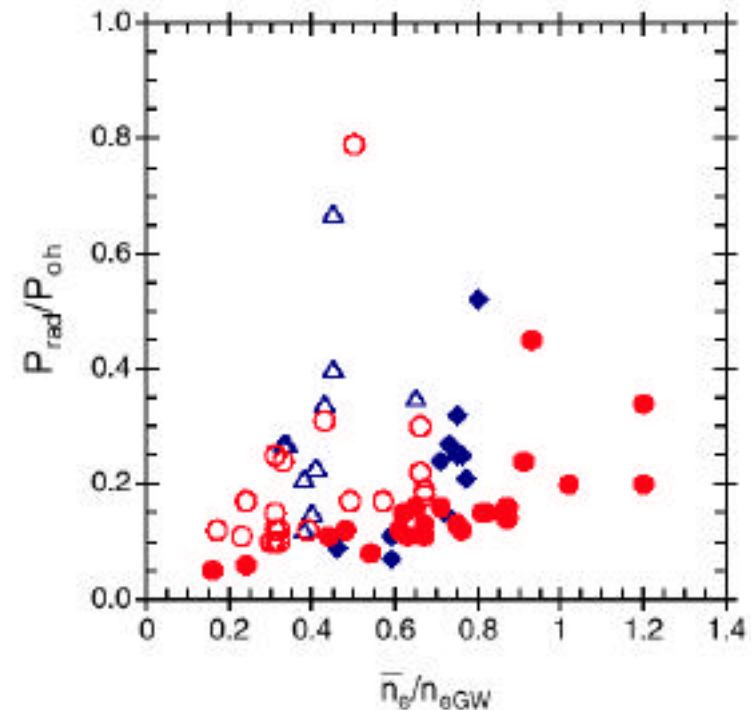


Achieved Densities Above Greenwald Limit

Boronization extends accessible n_e range



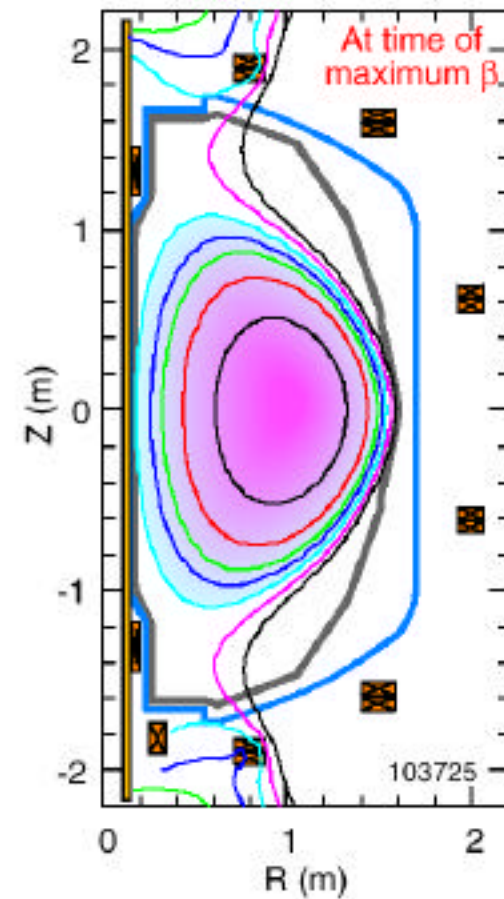
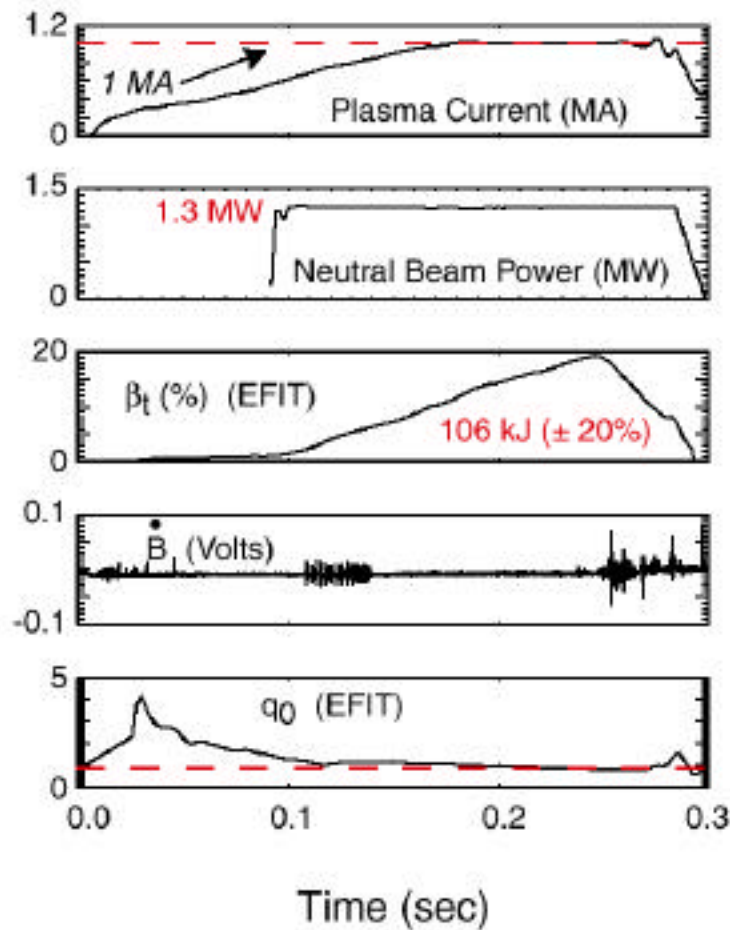
P_{rad}/P_{oh} increases, but still low (≤ 0.40) at high density



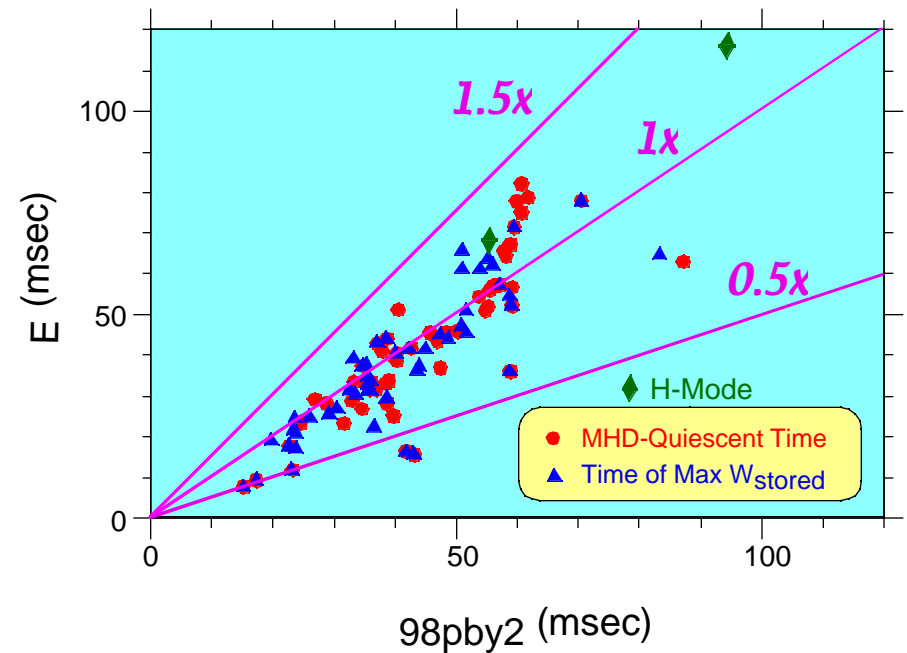
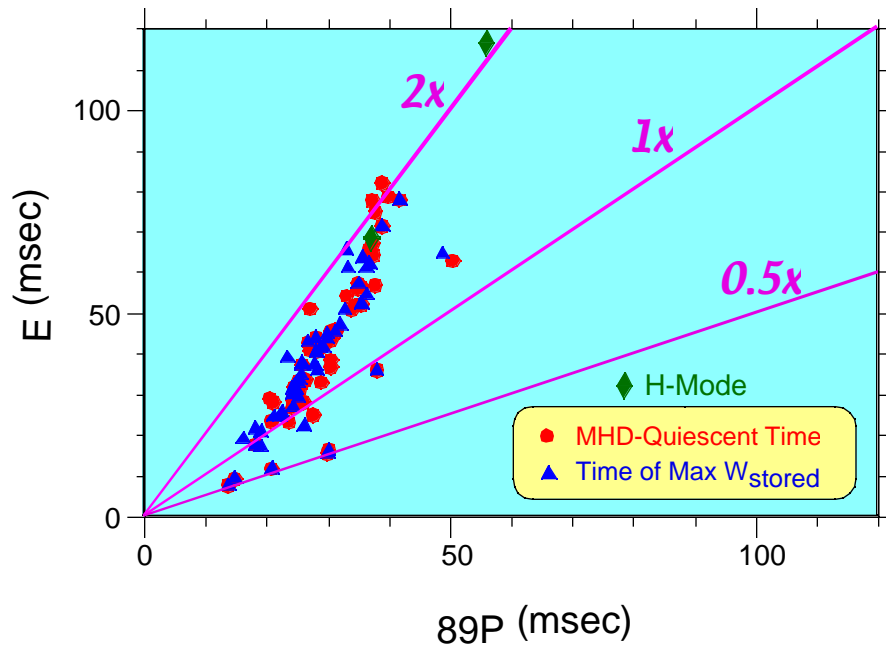
Density appears to be set by fueling limitations

High- β_t With Good Confinement Obtained

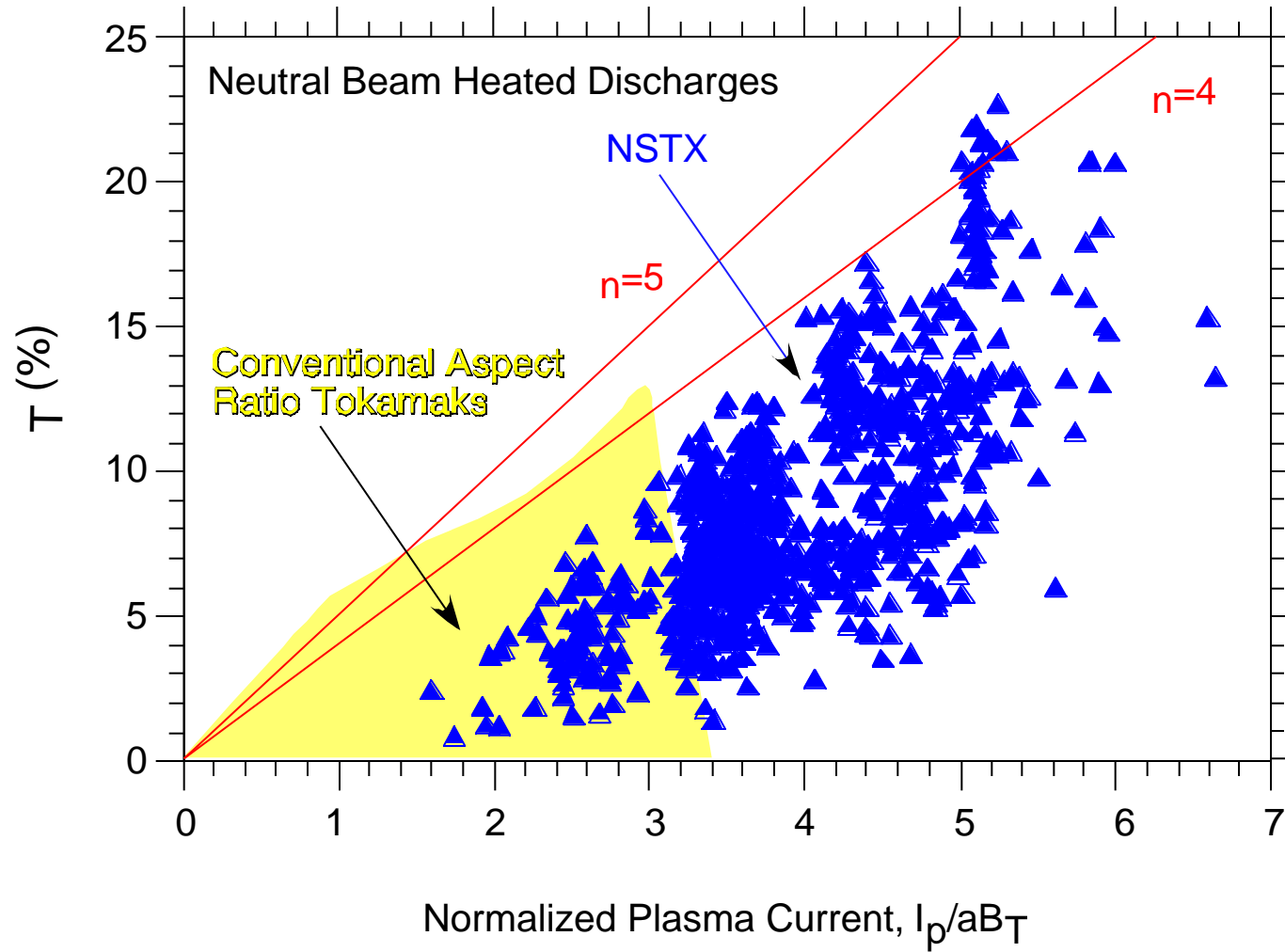
$$\beta_t = 2\mu_0 \langle p \rangle / B_0^2 = 19.7\%, \quad n = 3.9, \quad B_0 = 0.3 \text{ T}, \quad q = 7.5$$



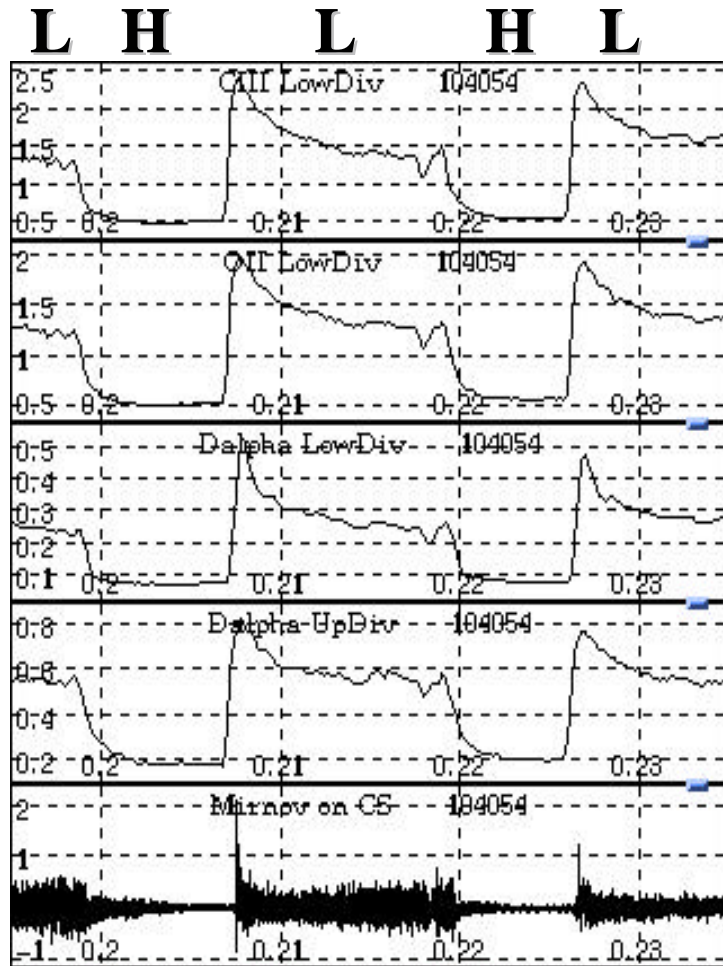
Energy Confinement Enhanced Over Both L- and H-mode Predictions



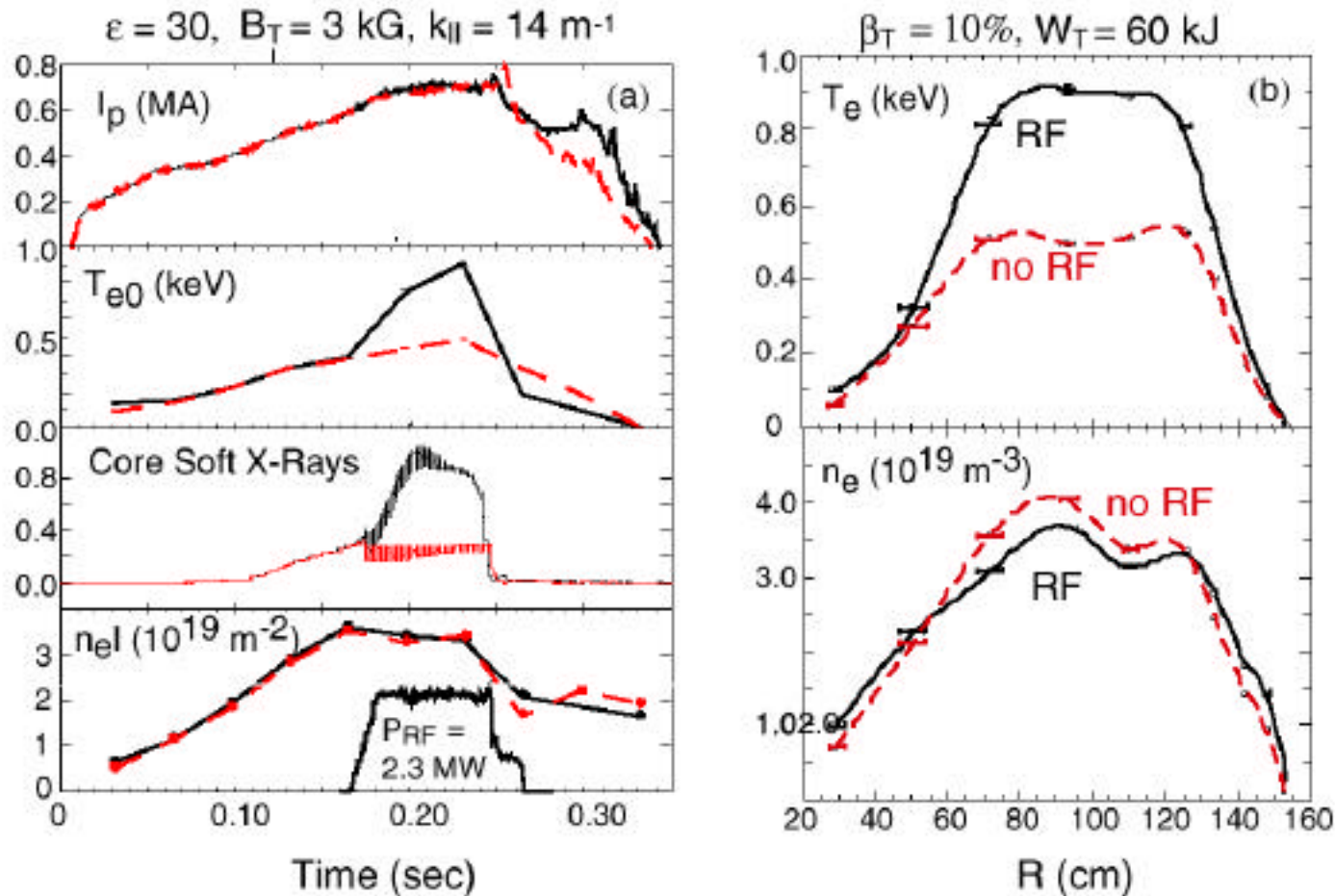
High- τ Achieved



Brief H-mode Observed in NSTX



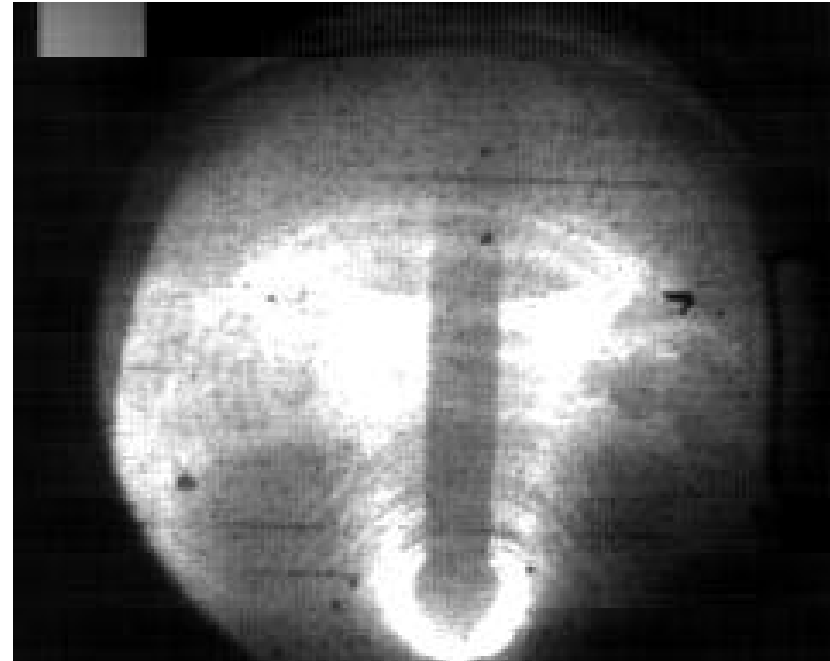
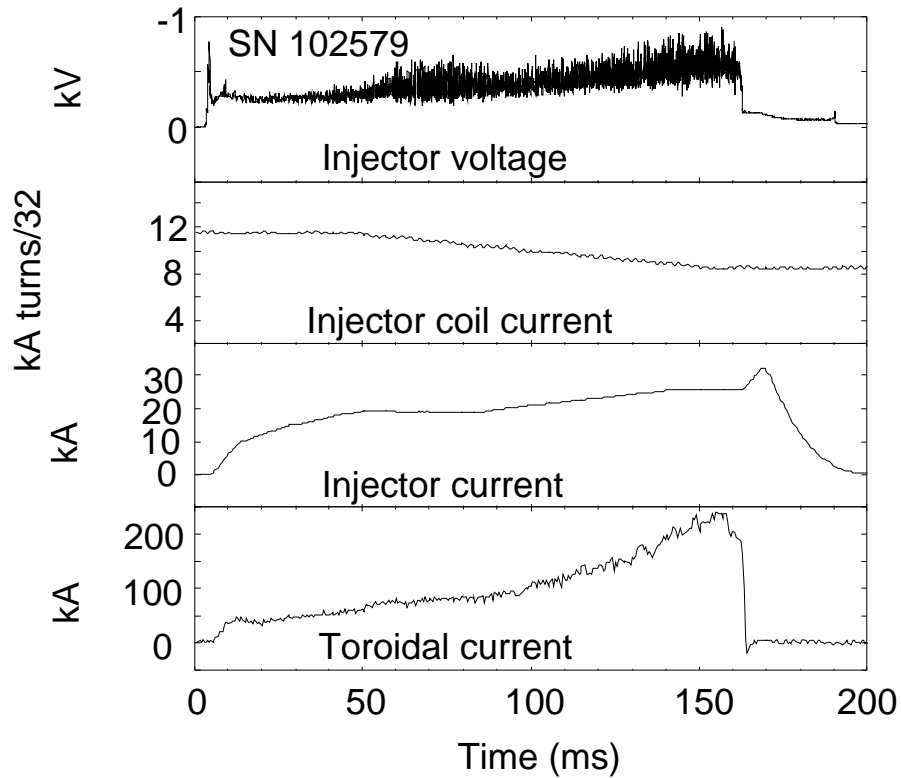
HHFW Electron Heating On- and Off-Axis



Coaxial Helicity Injection (CHI)

- Discharge initiated across inboard and outboard divertors in lower half plane
- $J \times B$ forces lead to toroidal current transported upwards from injector region
- Toroidal current 10 to 20 times injected current
- Reconnection leads to closed flux surfaces
- I_p 270kA has been produced with multiplication factor 10
- Issues
 - understanding reconnection physics, confirming flux closure
 - feedback control and absorber arc avoidance
 - impurity influx
 - eventual coupling with RF current drive

240kA CHI Discharge



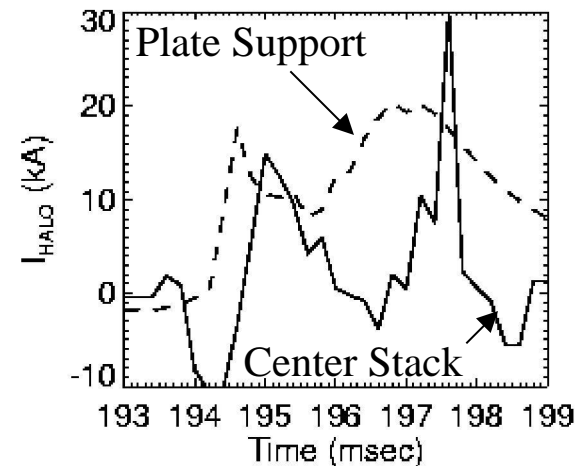
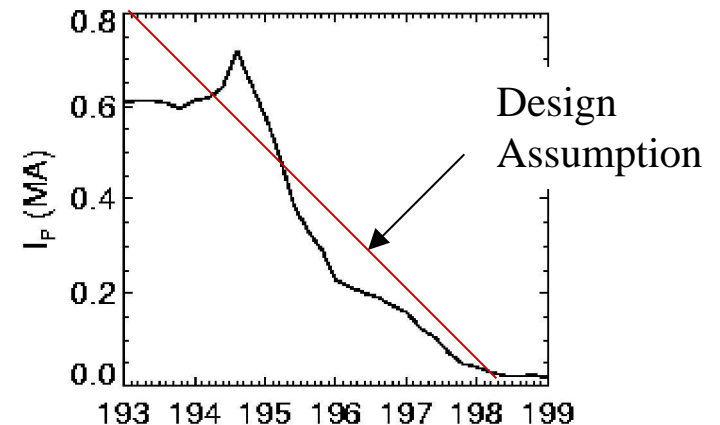
T = 150ms

Summary of Performance (best non-simultaneous)

| | |
|--|---|
| Plasma Current I_p | 1.0 MA |
| I_p Flat Top @ 1MA | 250 ms |
| dI_p/dt | ~ 5.5 MA/s |
| Ejima Coefficient = $V_{res} dt / \mu_0 R_0 I_p$ | ~ 0.35 |
| Stored Energy W | ~ 164 kJ |
| Confinement time τ_E | ~ 120 mS |
| $\tau_T = 2\mu_0 \langle p \rangle / B_0^2$ | $\sim 21\%$ |
| T_e | ~ 1.5 keV |
| T_i | ~ 2 keV |
| n_e | $1 \sim 6 \times 10^{-19} \text{ m}^{-3}$ |
| $n_e/n_{greenwald}$ | ~ 1 |
| P_{nbi} | 3.2MW |
| P_{hhfw} | 4.2MW |
| I_p CHI | 270kA |
| Z_{eff} | ~ 3 |

Plasma Disruptions

- Disruption / halo current $j \times B$ forces were a major design concern
- Vertical Displacement Events (VDEs) result in fast disruptions
 dI_p/dt 166 MA/sec as predicted
- Observed halo-currents are modest, 5 % of I_p (10% was design requirement)



Summary

- First phase operations have been highly successful
- RF and NBI heating experiments are underway
- β (21%) and confinement ($2 \tau_E^{89P}$, $1.6 \tau_E^{ELMy}$) very good
- Challenging current sustainment phase comes next