

Status of the MAST Upgrade

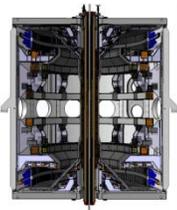
Hendrik Meyer

MAST-U physics case leader
for the

MAST-U Physics and Engineering Team

*EURATOM/CCFE Fusion Association,
Culham Science Centre, Abingdon,
Oxon, OX14 3DB, UK*

MAST



**MAST
Upgrade**

JET



ITER

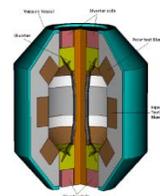


DEMO



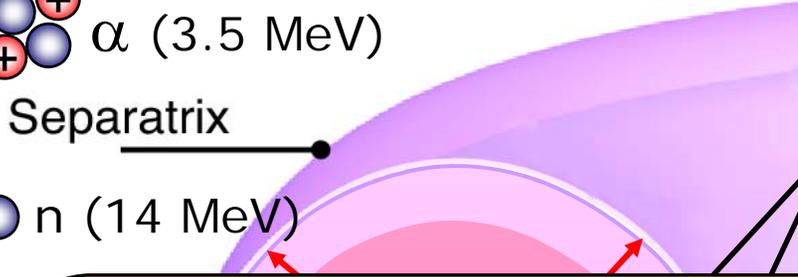
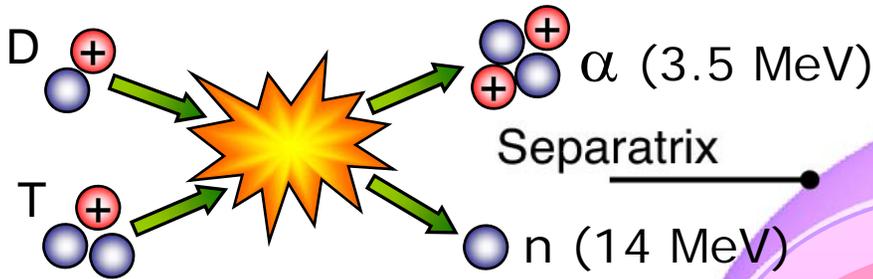
**Power
Plant**

CTF



A spherical tokamak (ST) could provide a cost effective Component Test Facility (ST-CTF)

Main issues for fusion tokamaks



Current Drive

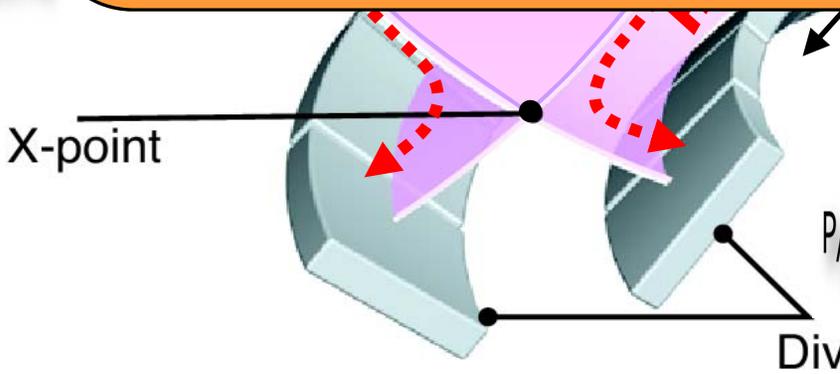
Fast ion physics

A 2 GW fusion power plant needs to exhaust roughly ~0.4 GW and drive of the order of 15 MA plasma current

Next Step << Power Plant
400s days - weeks

Now << Next Step << Power Plant
New physics - burning plasma

Heat loads



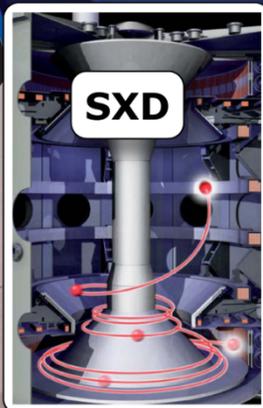
Now << Next Step << Power Plant
P/R ~ 10 MW/m 20 MW/m <100 MW/m

MAST Upgrade Stage 1

New center column
1.6 Wb solenoid flux
3.2 MA rod current
high field side shaping coils

Jackable beam box
2.5 MW off-axis

Double beam box
2.5 MW off-axis
2.5 MW on-axis



New upper and lower divertor
closed, pumped – unique SXD capability

MAST Upgrade

~~Stage 1~~

Core Scope

New center column

1.6 Wb solenoid flux
3.2 MA rod current
high field side shaping coils

Jackable beam
2.5 MW off-axis

**Need to deliver working tokamak within fixed budget and
and
Considerable uncertainties
prior to contract placement.**

Single beam box
5 MW on-axis

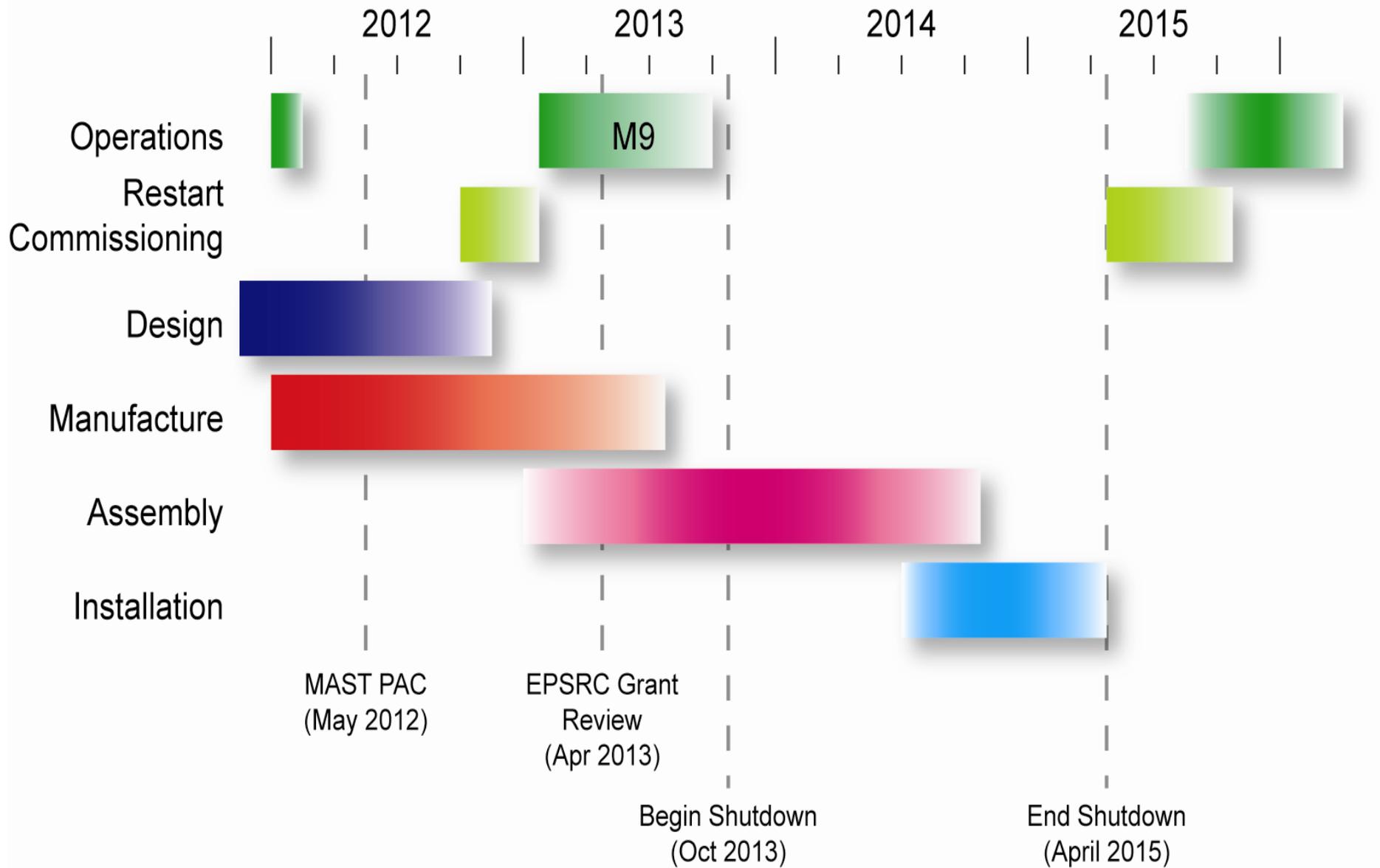
SXD

New upper and lower divertor
closed, ~~pumped~~ – unique SXD capability

Pump installed but cryoplant is dependent on funding

- 8 new 4 quadrant divertor power supplies.
 - Low level of ripple required for Super-X.
 - Contract now placed within estimated budget.
- New toroidal field power supply.
 - Contract now placed within estimated budget.
- 14 new in vessel coils.
 - Contract now placed within estimated budget.
- 3 Cyanate ester airside coils.
 - New solenoid – long conductor!
- Currently assessing which parts of Stage 1 can be re-introduced.
 - Cryo-plant and/or Double Beam box.

MAST-U simplified Time line



From the plasma core...

Enabling current drive physics

Through the plasma edge...

Enabling divertor physics

To what lies beyond the plasma

The engineering!

From the plasma core...

Enabling current drive physics

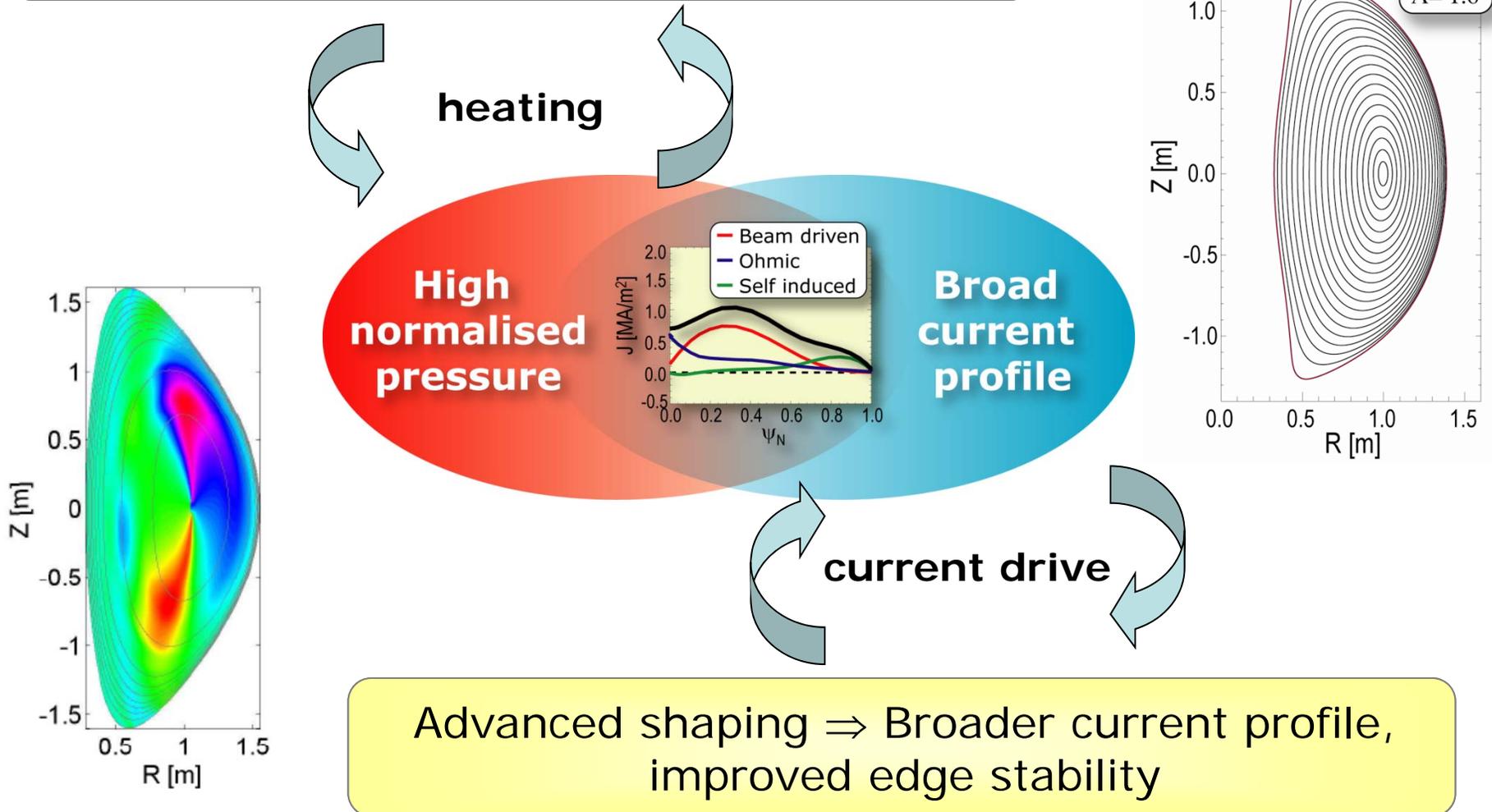
Through the plasma edge...

Enabling divertor physics

To what lies beyond the plasma

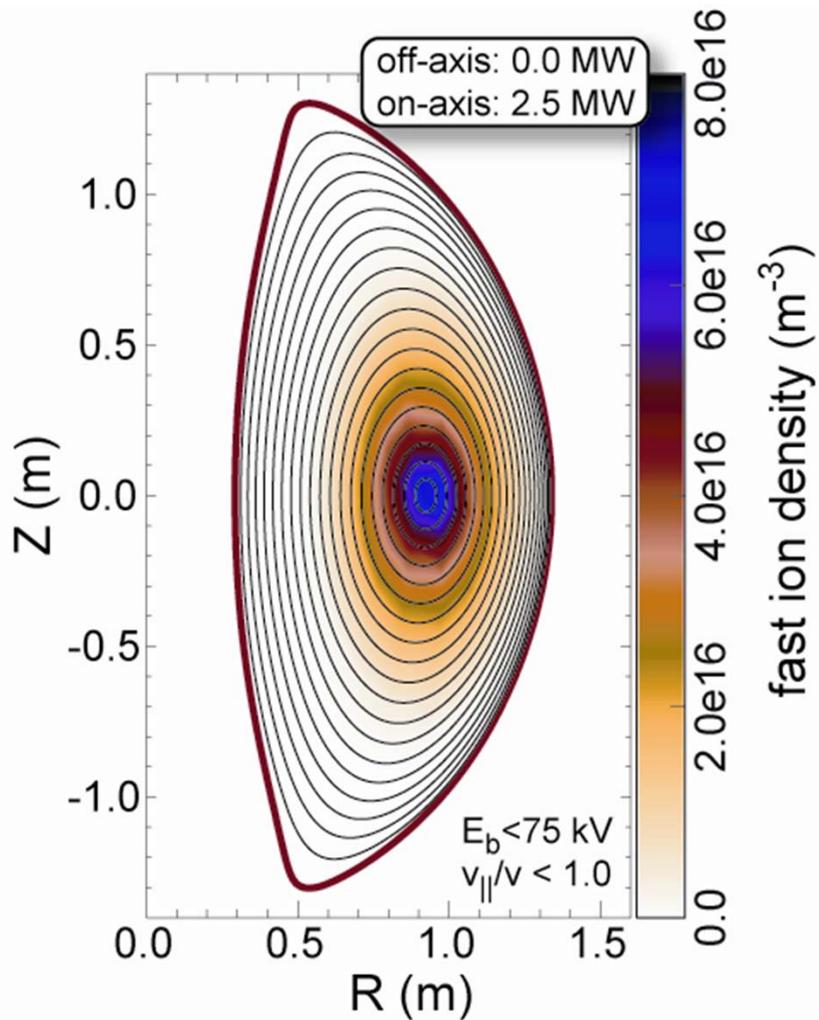
The engineering!

Advanced MHD stability \Rightarrow Higher pressure



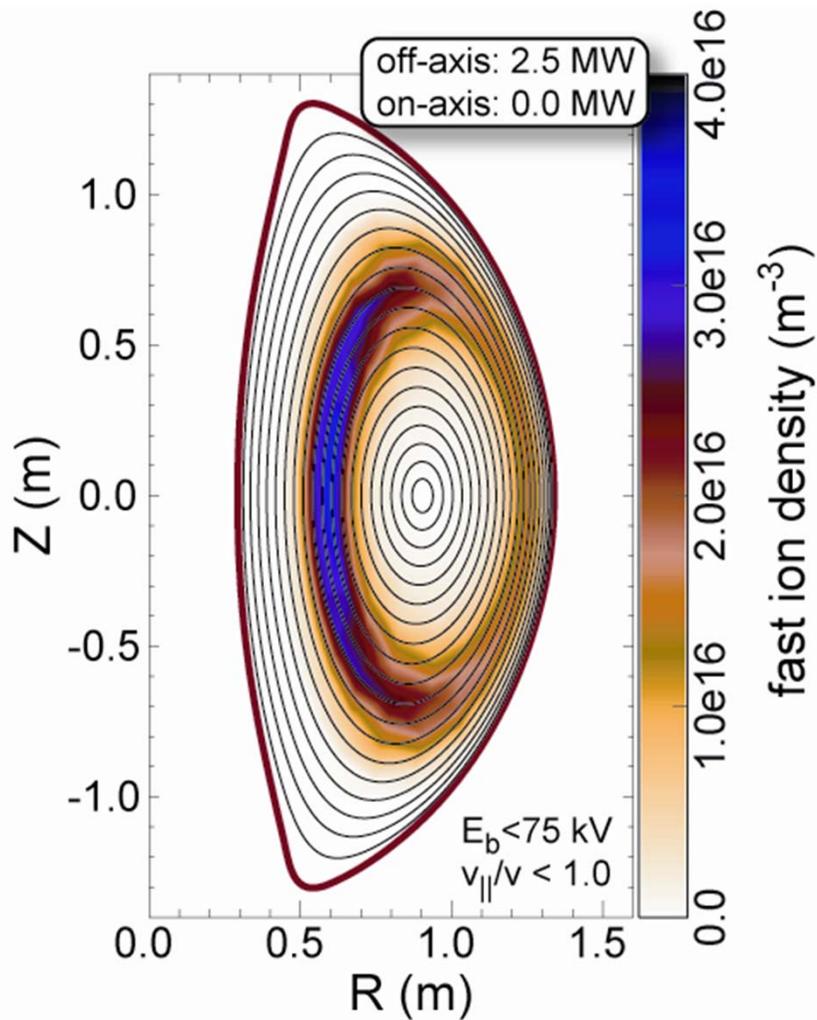
Advanced profile control

- On-axis \Rightarrow peaked.



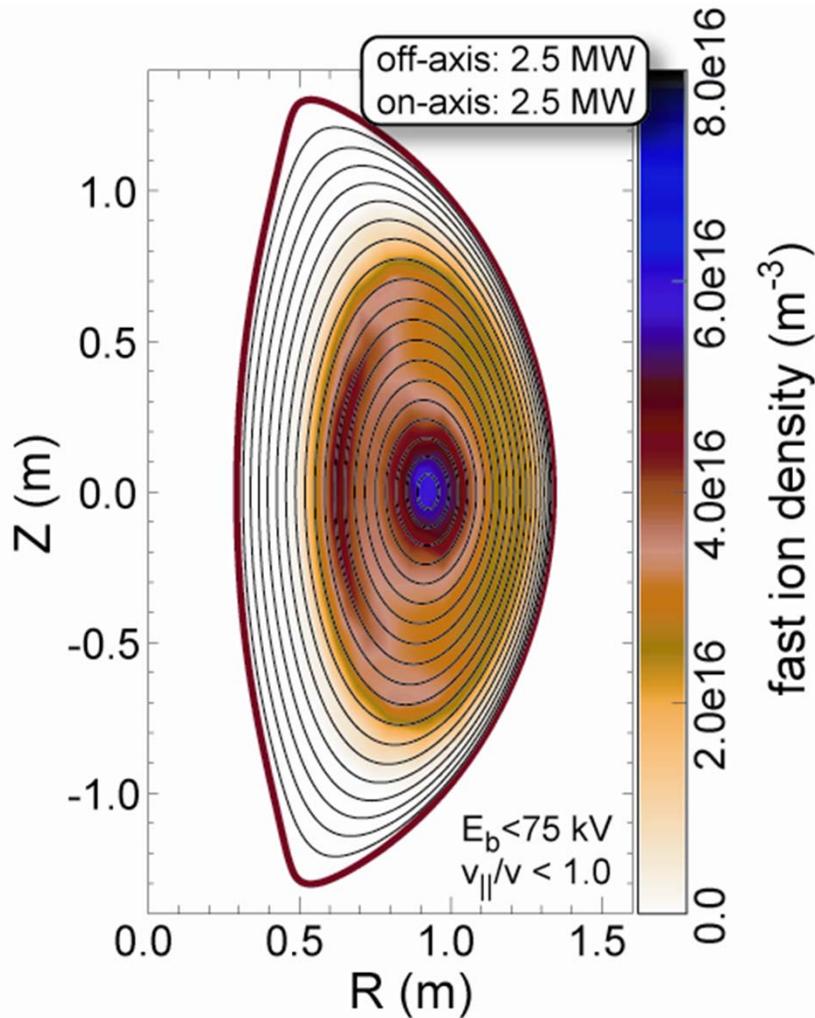
Advanced profile control

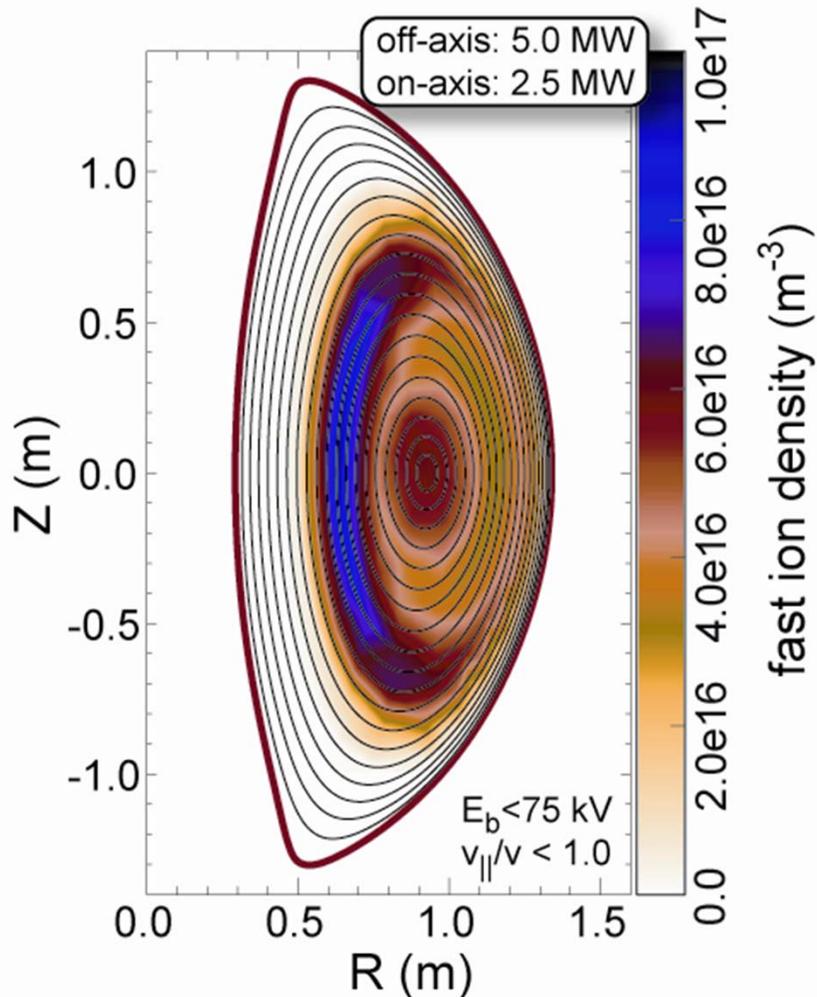
- On-axis \Rightarrow peaked.
- Off-axis \Rightarrow hollow.



Advanced profile control

- On-axis \Rightarrow peaked.
- Off-axis \Rightarrow hollow.
- On- and off-axis \Rightarrow broad.





Advanced profile control

- On-axis \Rightarrow peaked.
- Off-axis \Rightarrow hollow.
- On- and off-axis \Rightarrow broad.

MAST-U physics studies

- About 1 MA of non-inductive current drive \Rightarrow long pulse length.
 - Needs all 3 beams!
- High fast-ion pressure (60% of total pressure)

- Fast ions have only few collisions.

- Should be localised to sources.

- Experiments suggest source profile broadens.

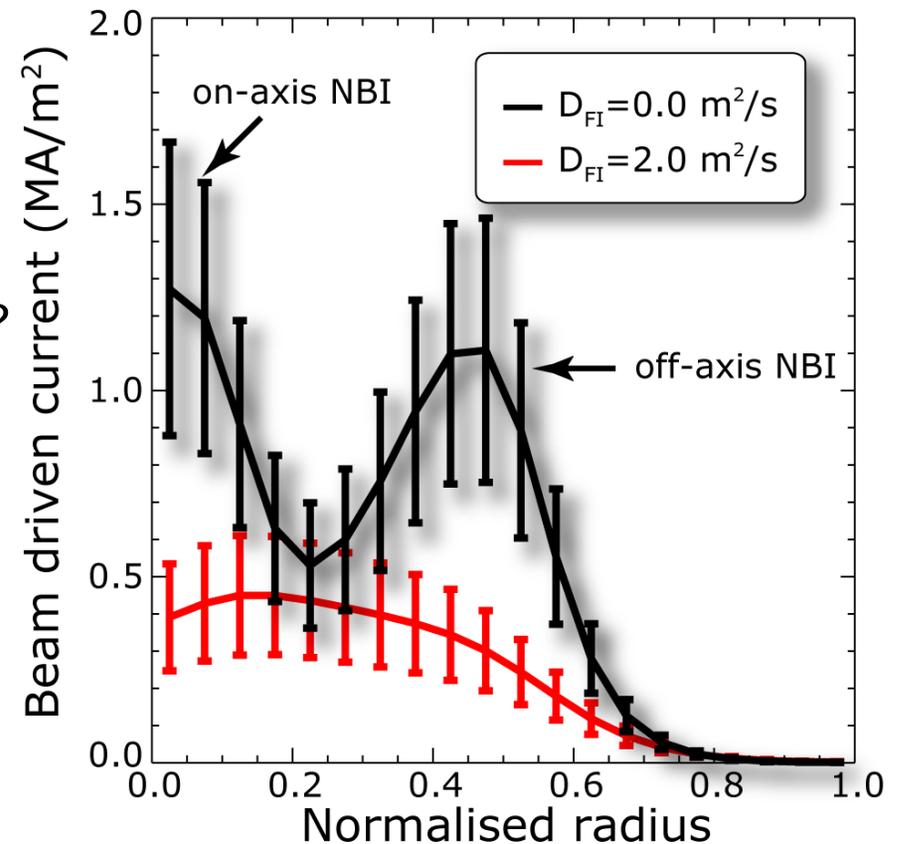
- Instabilities? – yes, localised?
 - Turbulence? – yes, how?

- Modelled as “anomalous” diffusion $D_{FI}(r, E, \dots)$

- Crude, ad hoc.
 - Wrong for instabilities.

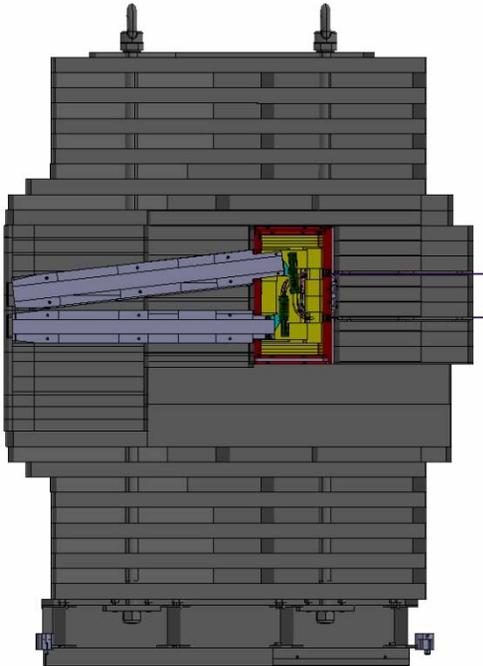
- Need Integrated modelling

- Current profile affects stability/turbulence.
 - stability/turbulence changes current drive.

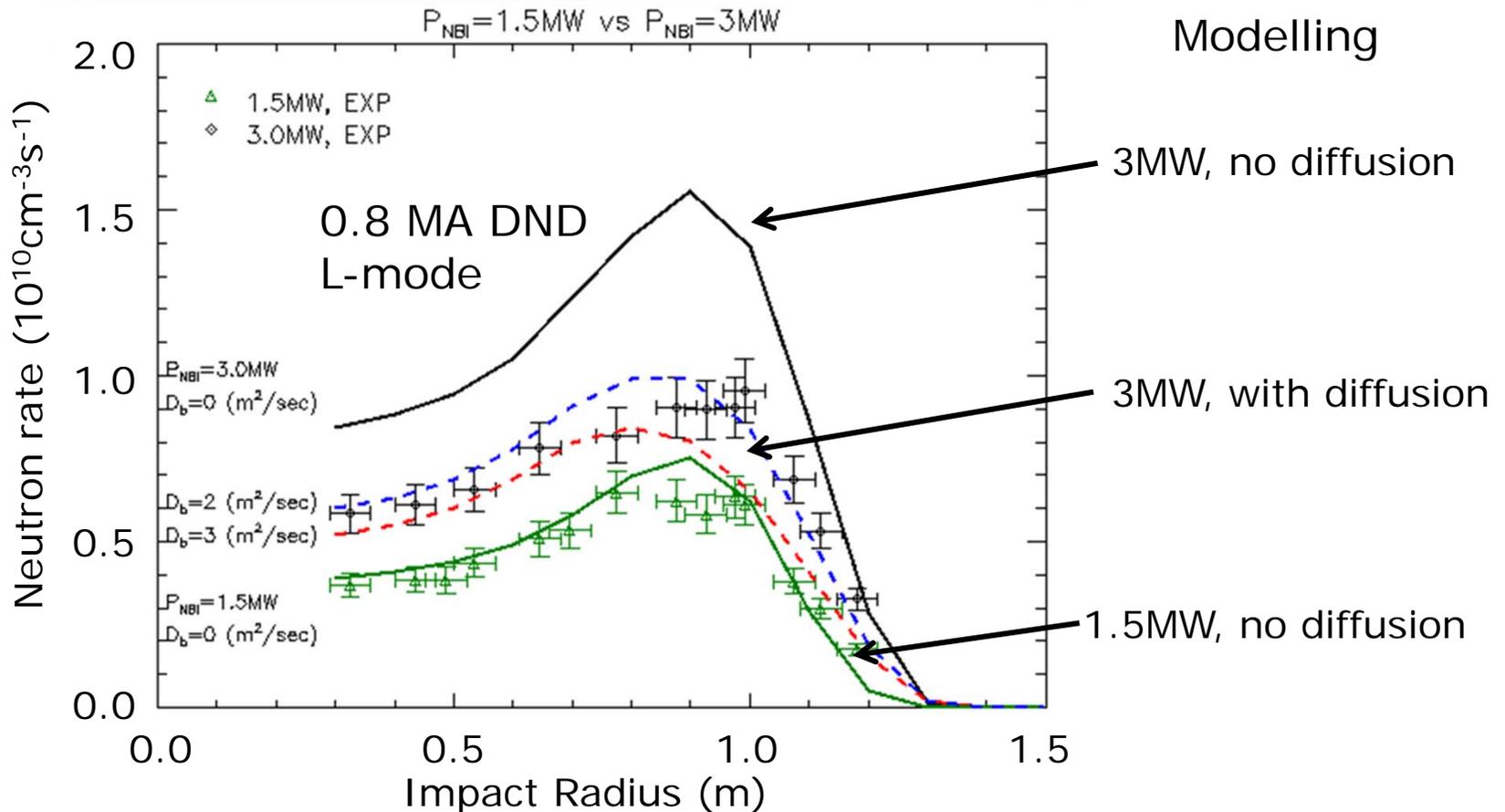




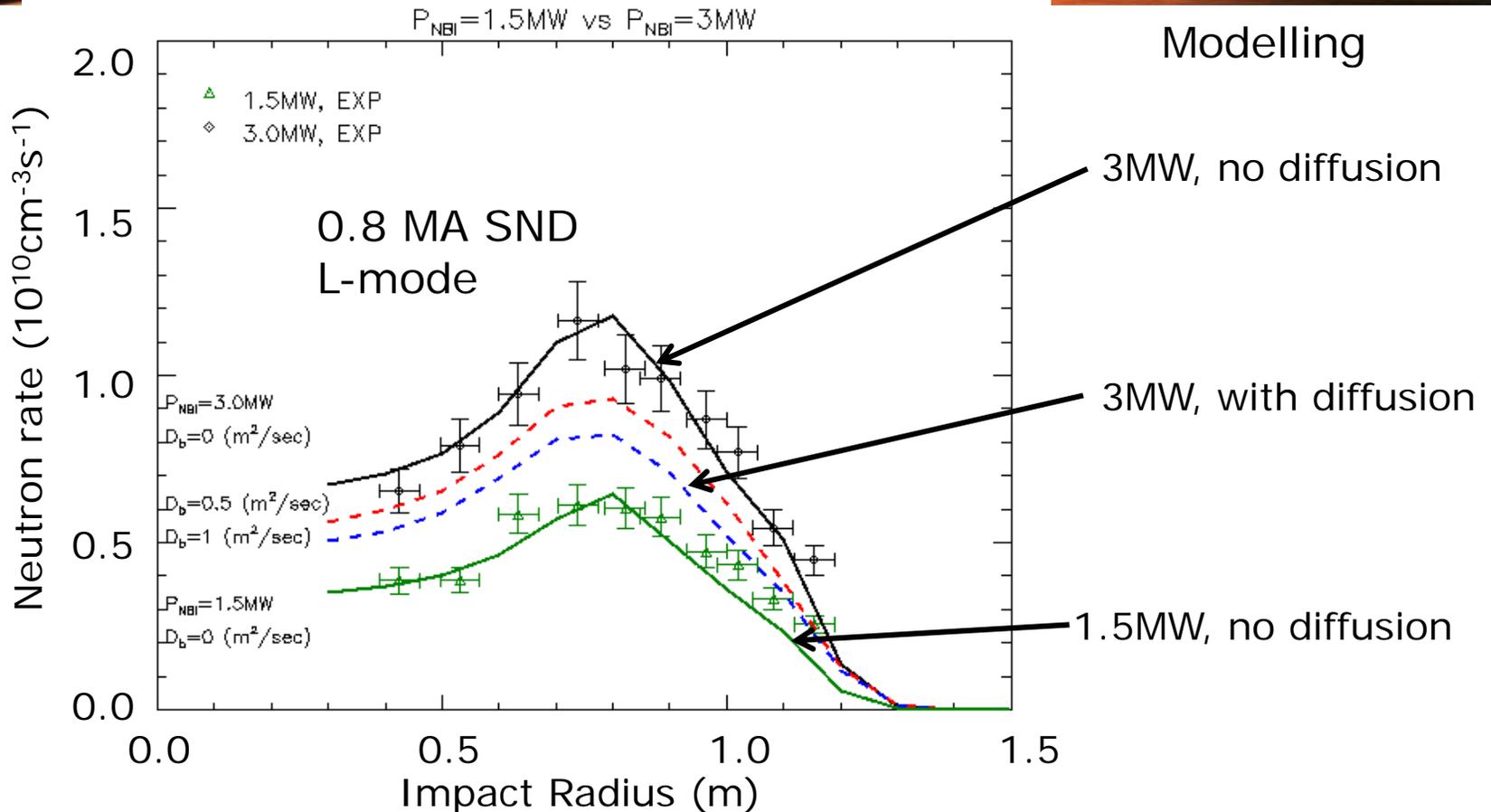
UPPSALA
UNIVERSITET



- 4 channel neutron camera in collaboration with Uppsala University (Sweden).
- Can be scanned from shot to shot.



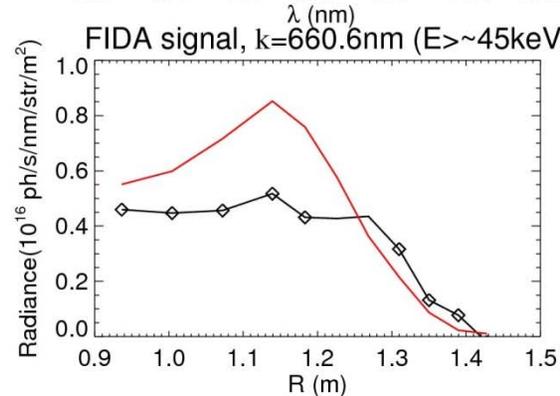
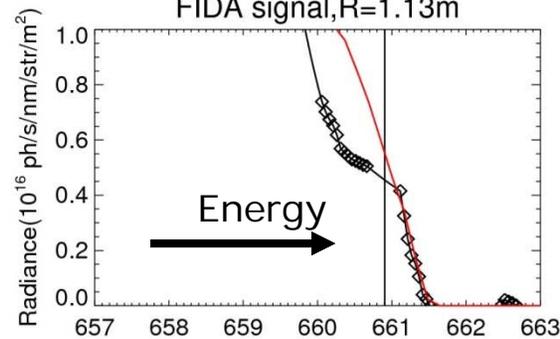
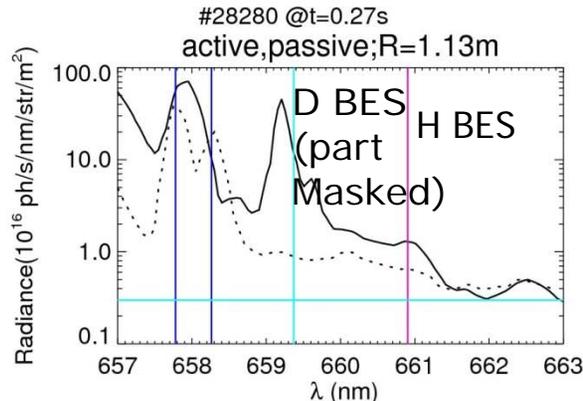
- High fast ion redistribution in DN MAST discharges with 2 beams ($I_p=0.8$ MW)
- With one beam no redistribution is needed.
- Off axis heating may be more beneficial.



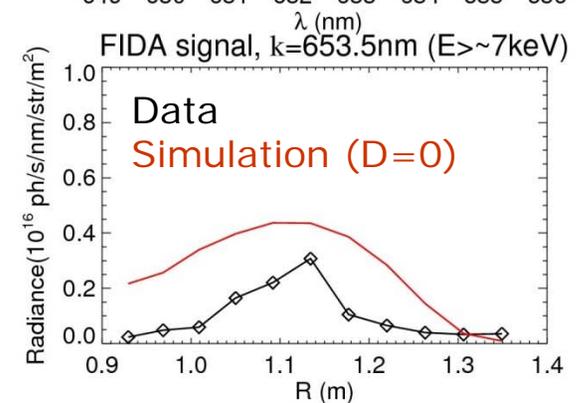
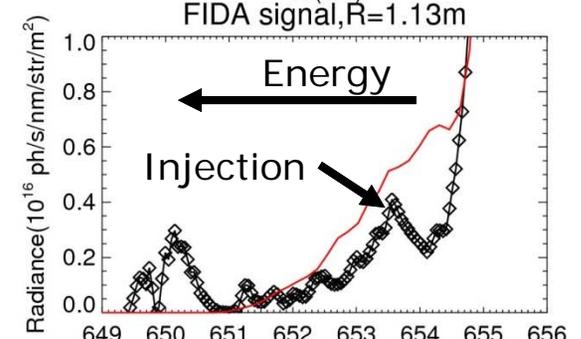
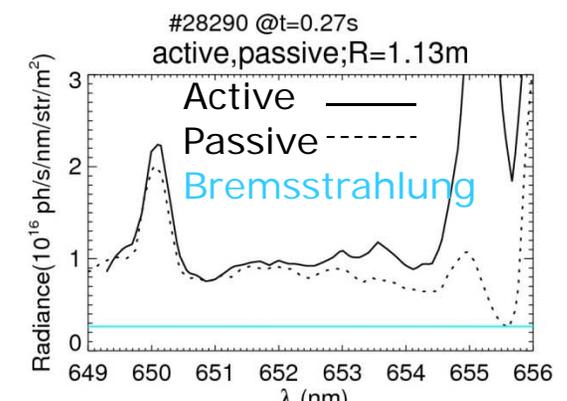
- Higher $q_{\min} > 1.3$ helps to avoid detrimental MHD.
- Neutron rate doubles with double the beam power.

- Fast ion transport with TRANSP/FIDASIM modelling.
- Example: 1 beam SND ($D_{FI}=0$, classical).
- Horizontal: below classical in core and above in edge.
- Vertical: Signal is much lower indicates loss of trapped particles.

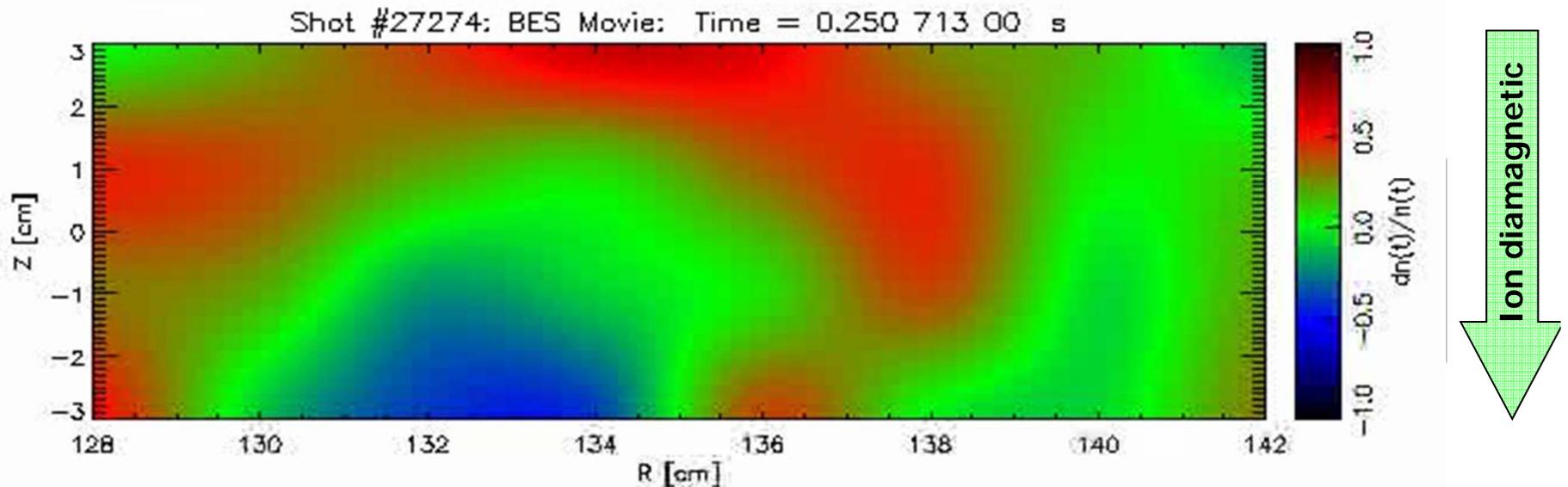
HORIZONTAL (PASSING)



VERTICAL (TRAPPED)

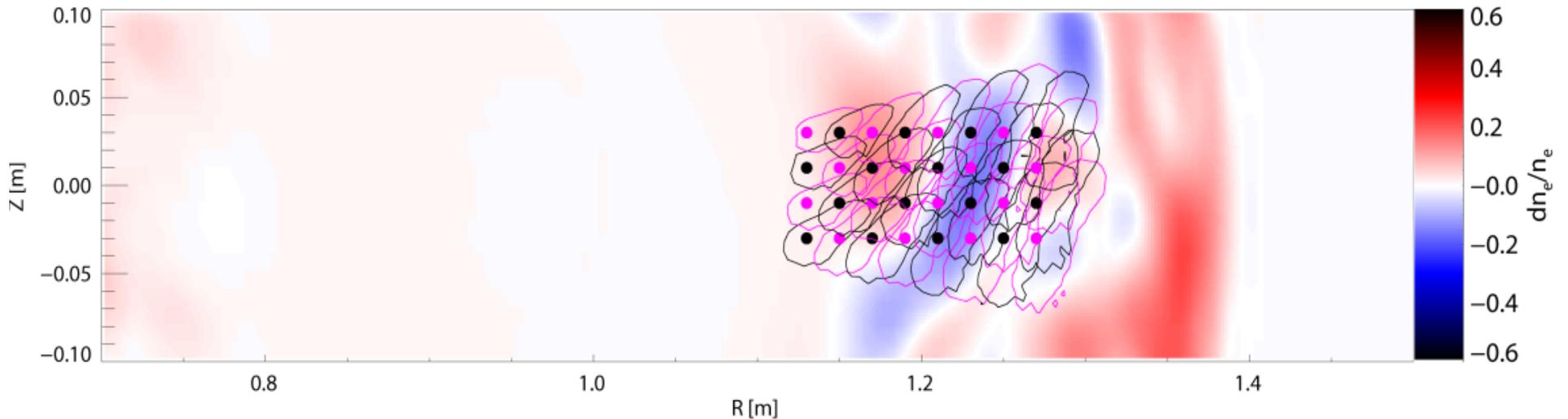


- Ion-scale density turbulence measured with 2D BES
- Allows study of interaction of flow-shear and anomalous, ion-scale turbulent transport
- Signal-to-noise sufficient to measure core turbulence



- Ion-scale density turbulence measured with 2D BES
- Allows study of interaction of flow-shear and anomalous, ion-scale turbulent transport
- Signal-to-noise sufficient to measure core turbulence
- Synthetic diagnostic to compare with GK modelling.

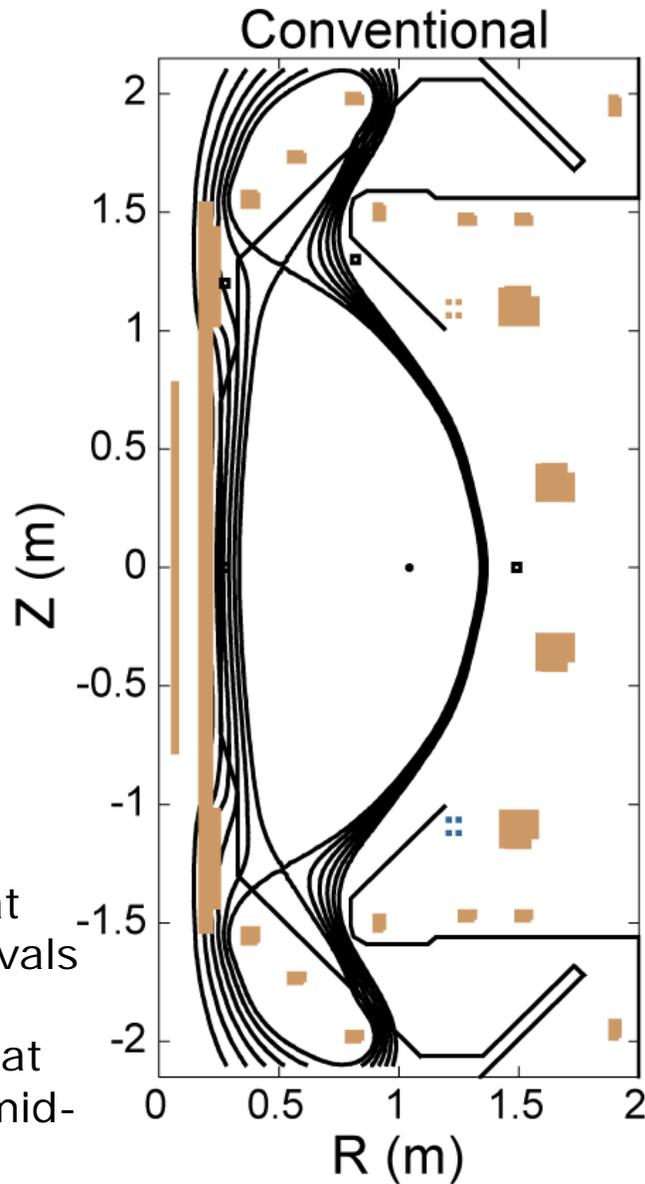
ORB5 density fluctuations, dn_e/n_e , orb5_22807_0.25_01



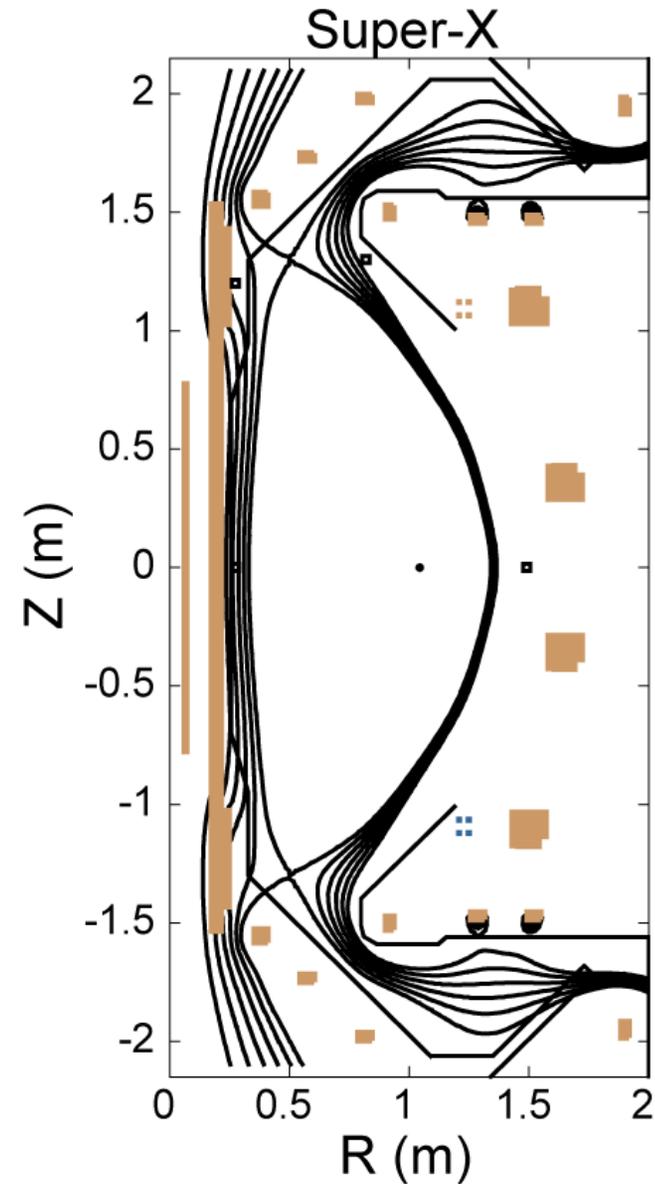
From the plasma core...
Enabling current drive physics

Through the plasma edge...
Enabling divertor physics

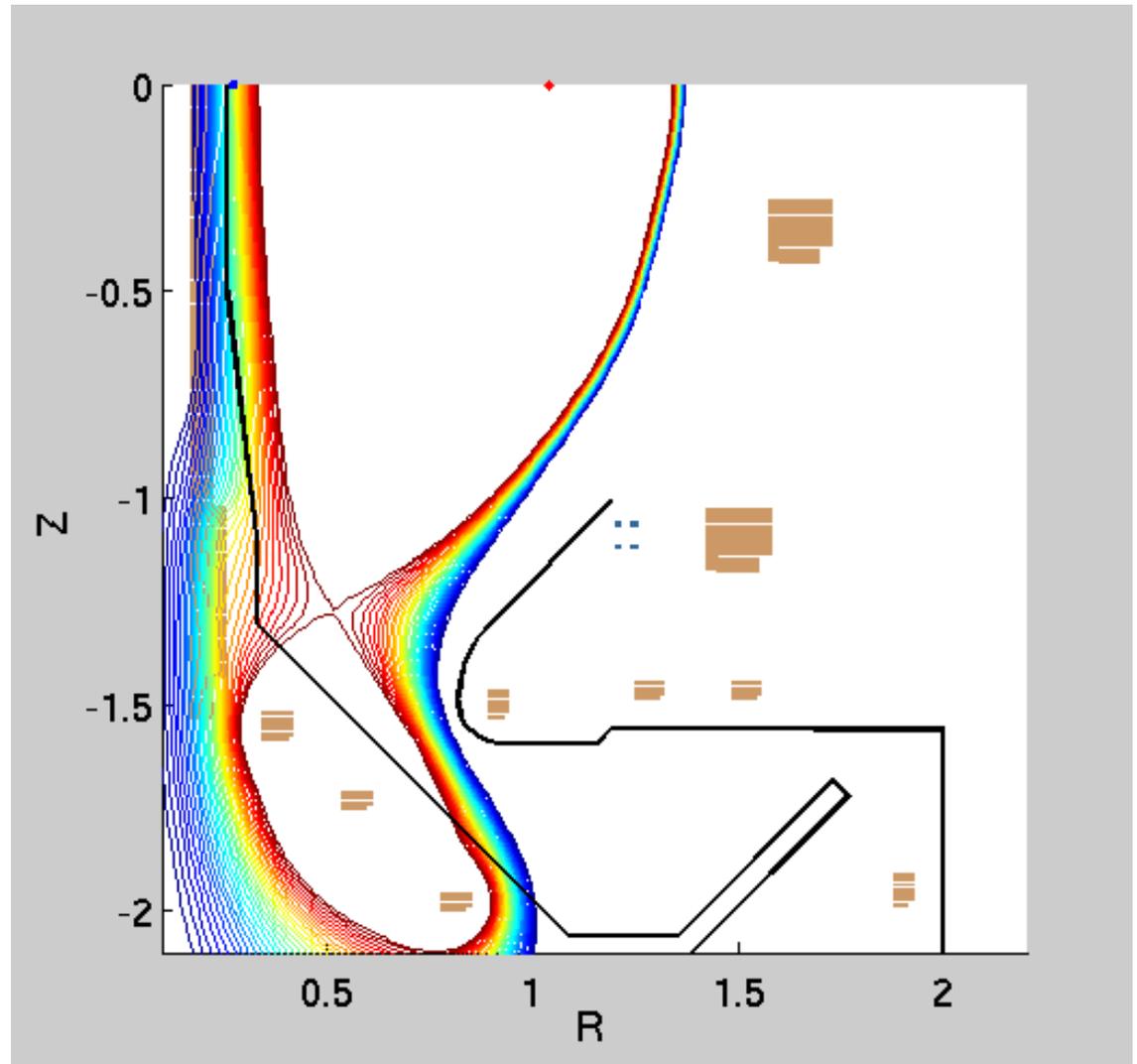
To what lies beyond the plasma
What is upgraded?

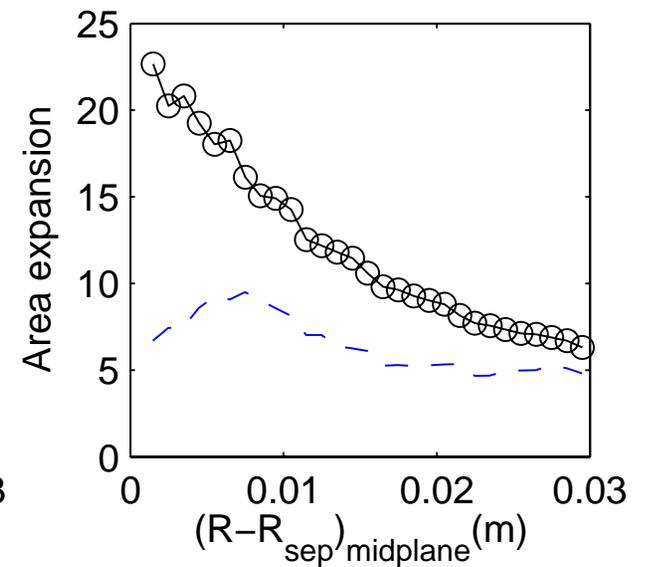
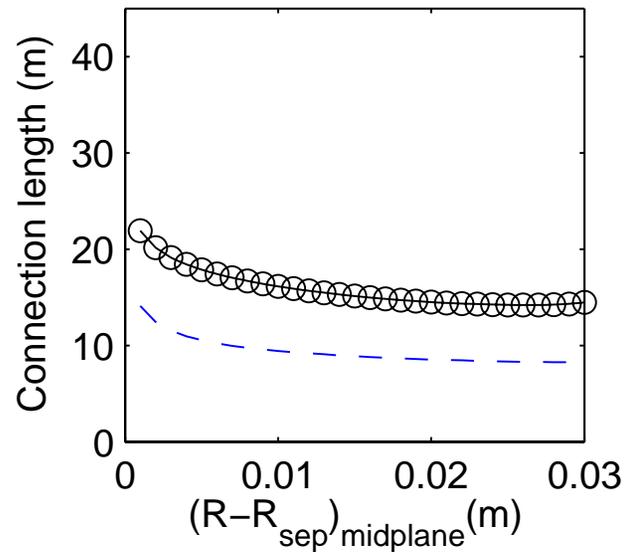
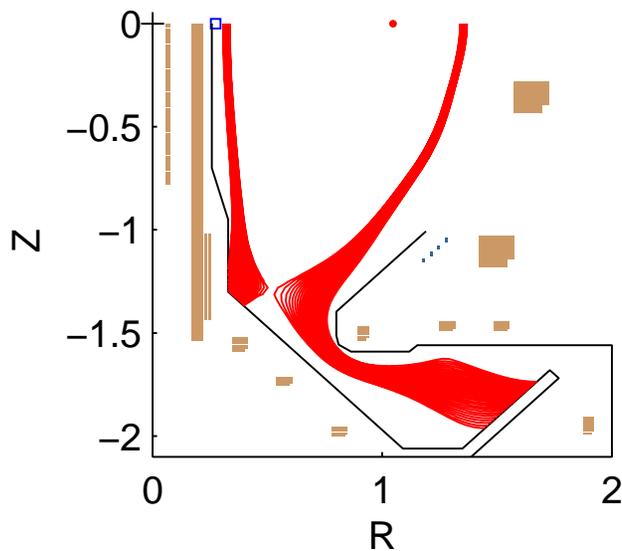
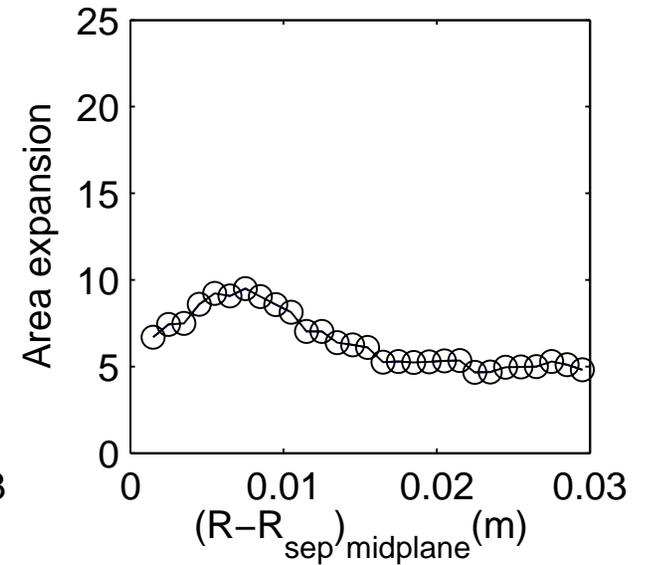
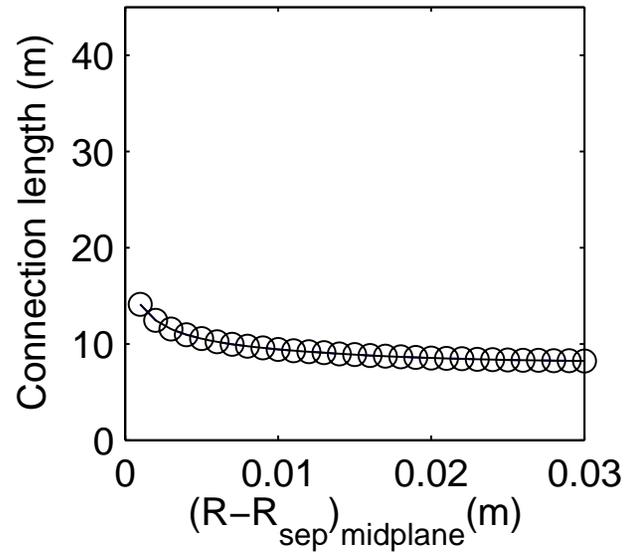
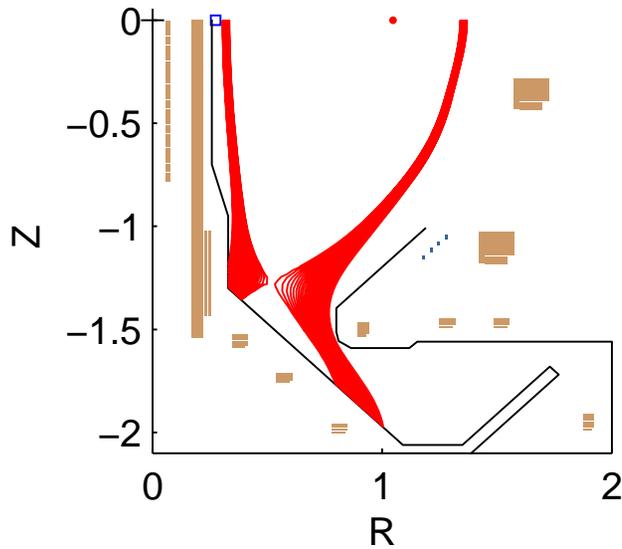


Contours at
5mm intervals
from
separatrix at
outboard mid-
plane

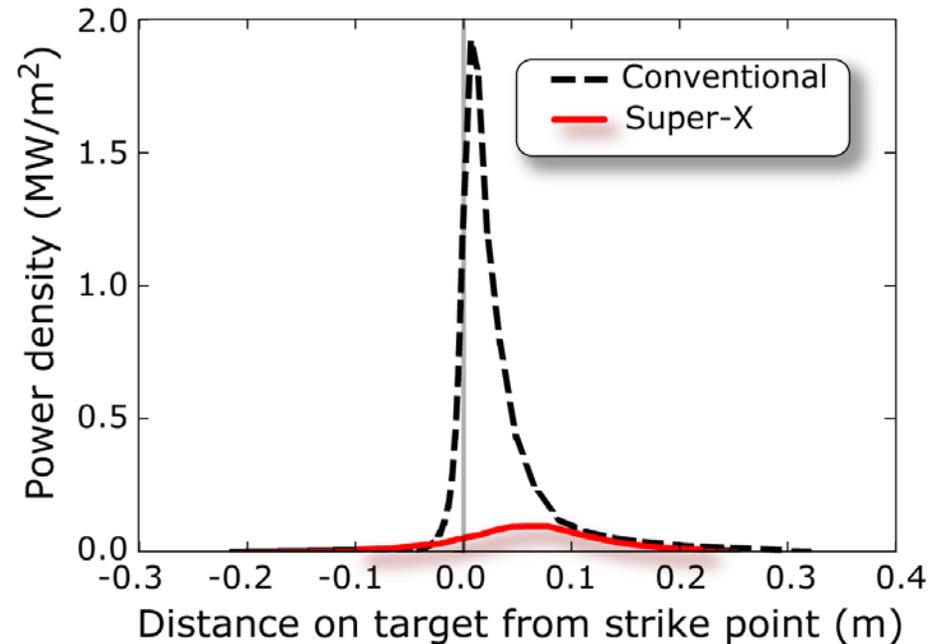
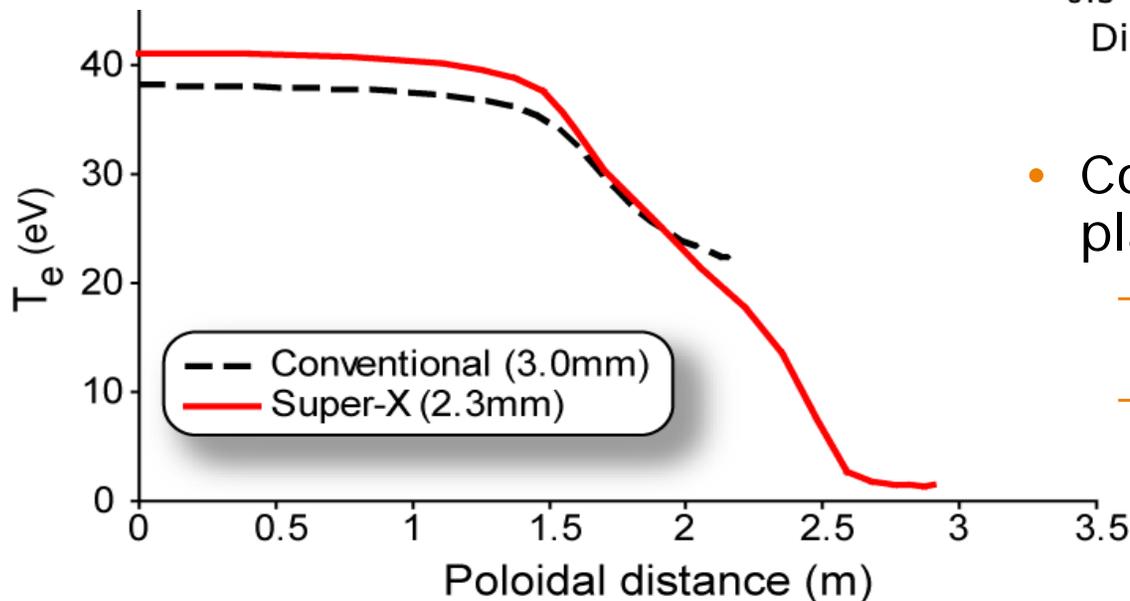


- Unique divertor concept \Leftrightarrow Super-X
- Large plasma volume in the divertor.
 - Neutral-plasma interactions.
 - Impurity radiation.
- Can the target heat load be reduced?
- Can the geometry be controlled?



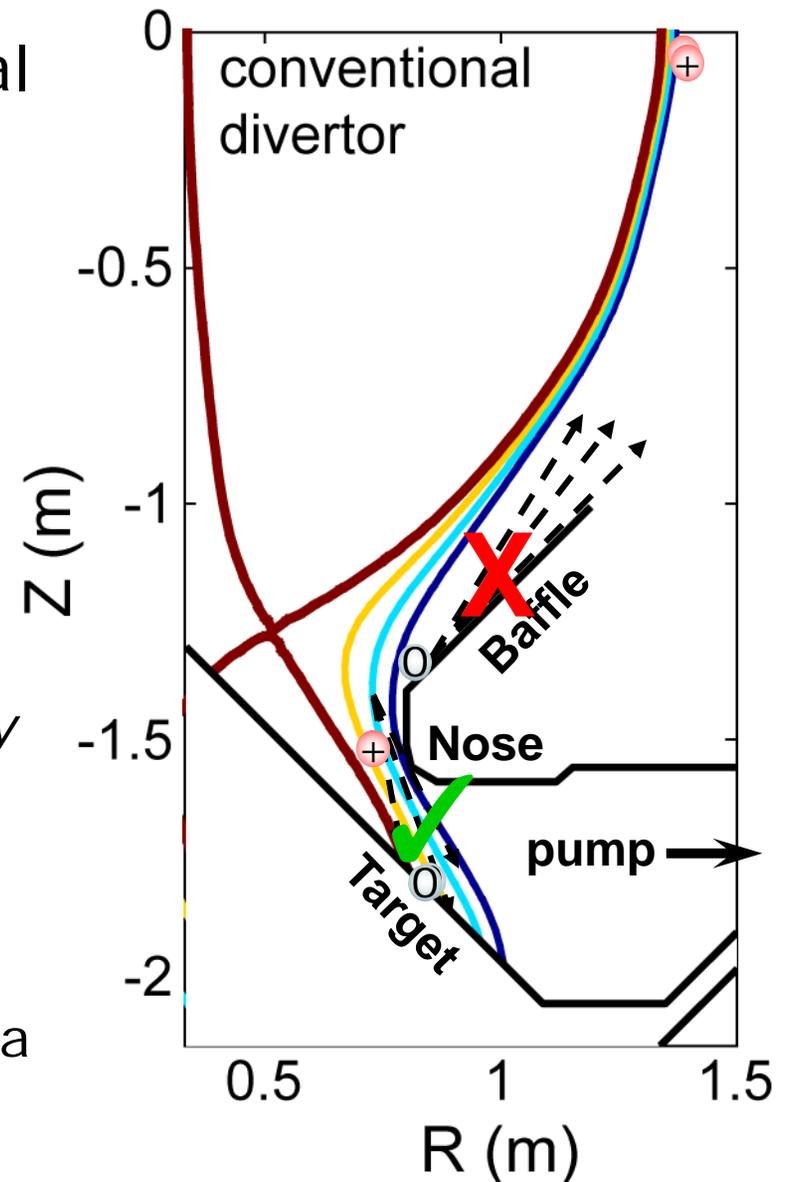


- Low collisionality (hot SOL)
 - ⇒ T const. along the field line
- High collisionality
 - ⇒ T drops towards the target.
- Increased connection length
 - ⇒ increased collisionality.
 - ⇒ increased temperature drop along field line

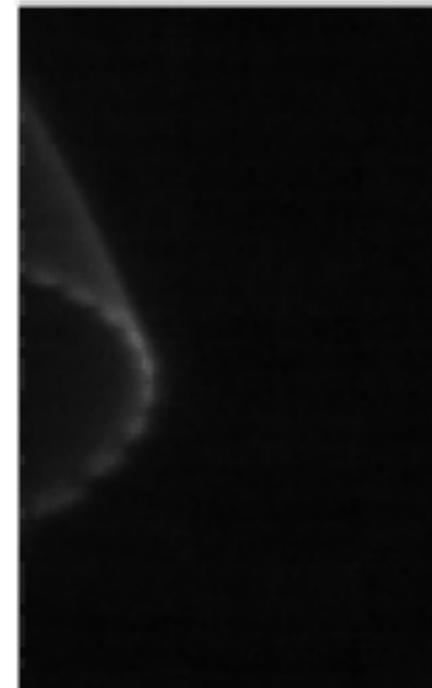
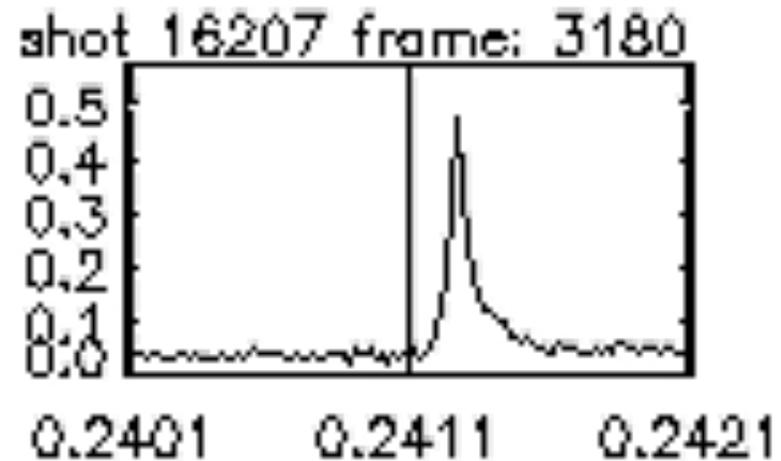
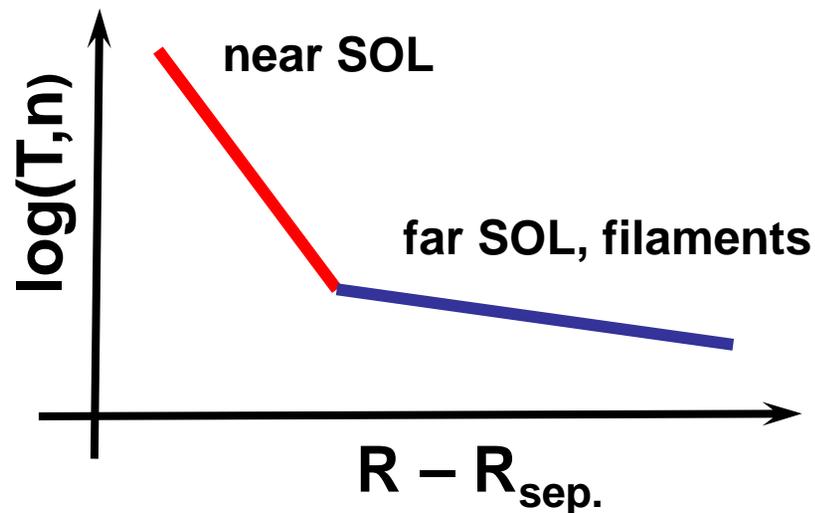


- Confirmed by 2D fluid edge plasma simulations:
 - Here, low power (1.8MW) into SOL, but ...
 - high power simulations show same trend.

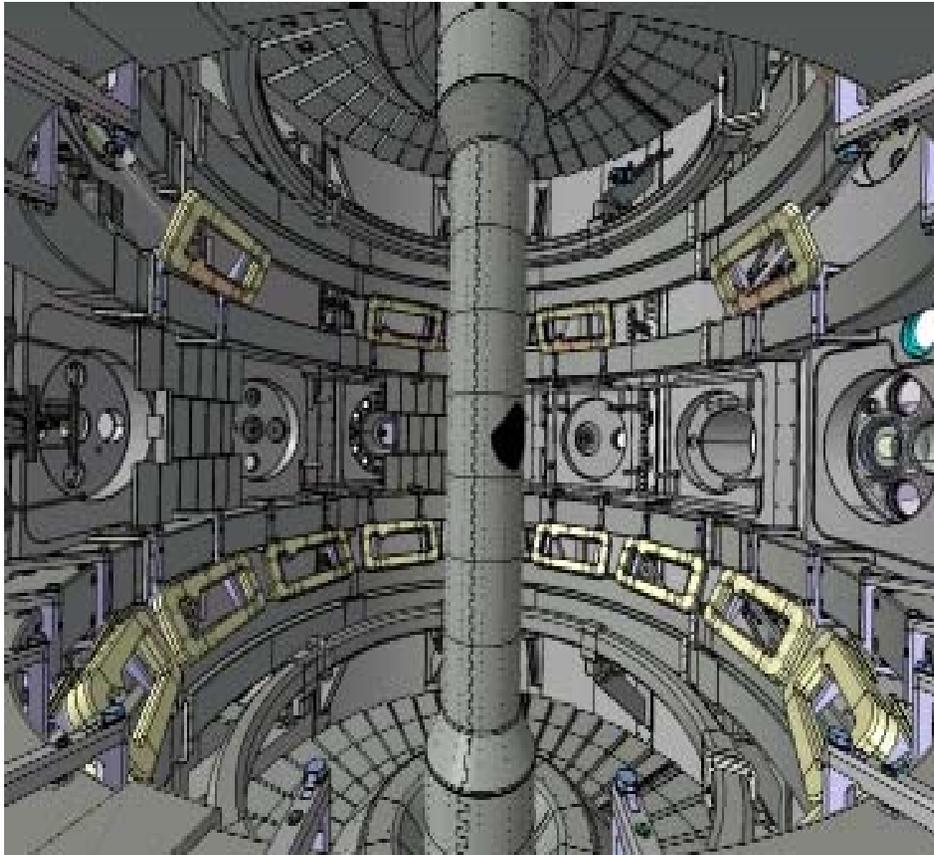
- Density control is fundamental to MAST-U.
- Seek high neutral density in the divertor \Leftrightarrow efficient pumping
 - \Rightarrow Require sufficient plasma in throat
 - *to prevent divertor neutrals leaving*
 - *for low main-chamber density*
- Seek minimal recycling from the main-chamber baffle surface
 - \Rightarrow require sufficiently diffuse plasma at edge of throat



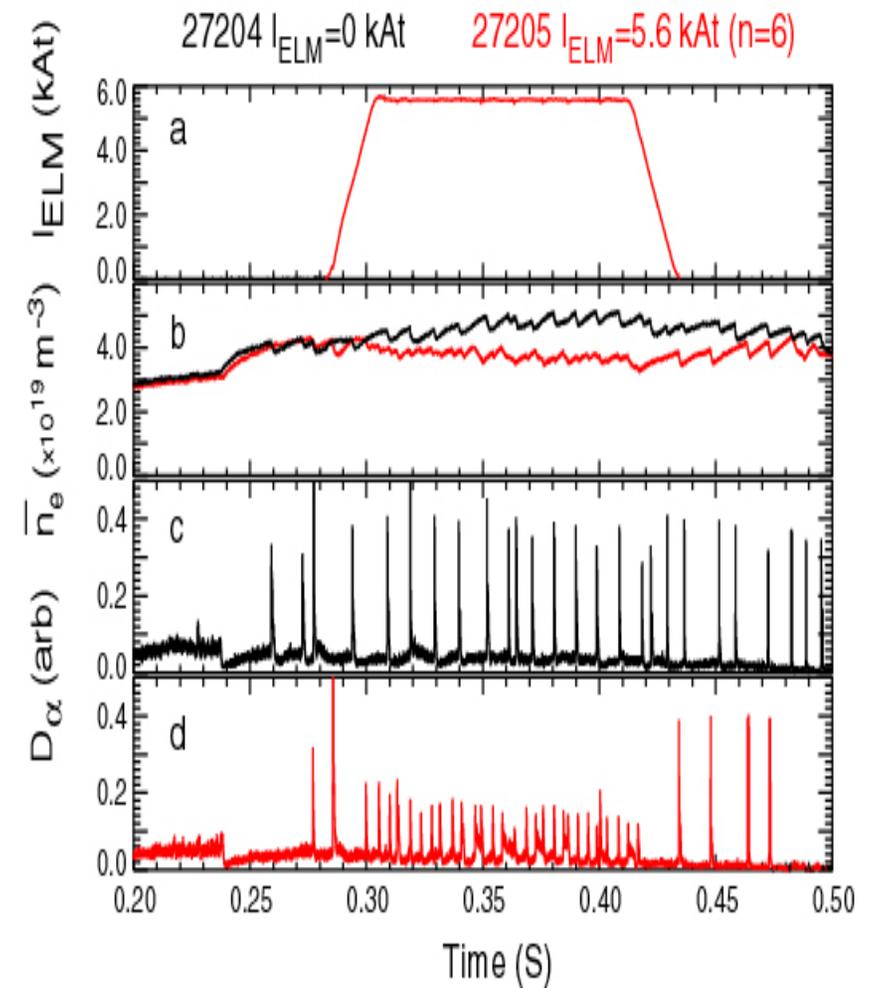
- Perpendicular SOL transport is not diffusive.
- Mean field approximation is not sufficient.
- Need a further understanding of the ELM and ELM avoidance.



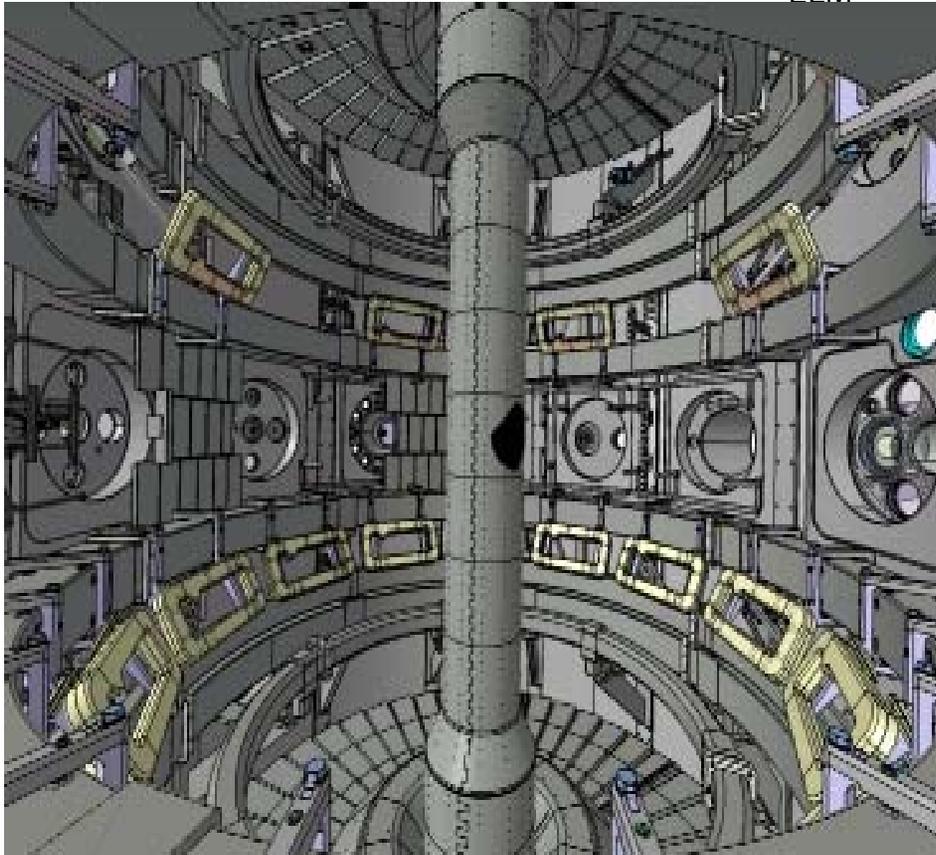
18 in vessel coils for ELM mitigation



Application of resonant magnetic perturbations in $n=6$

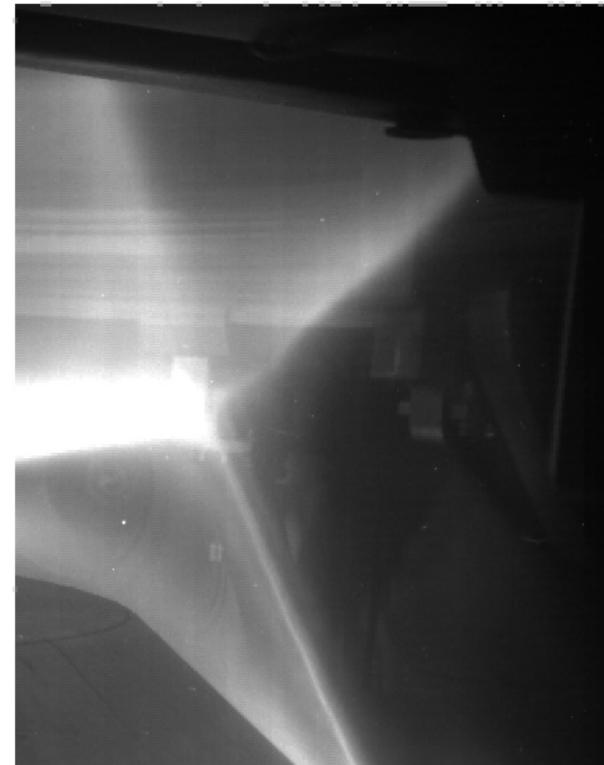
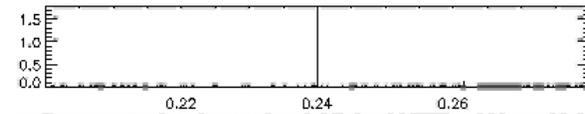


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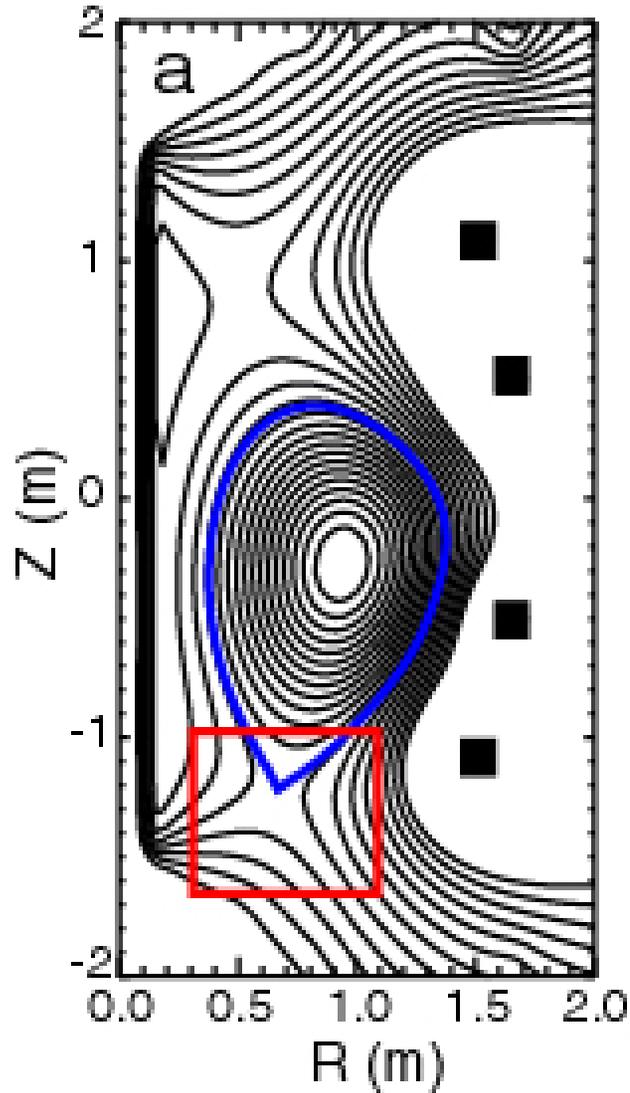


I_{ELM} (kA)

Application of resonant magnetic perturbations in $n=6$

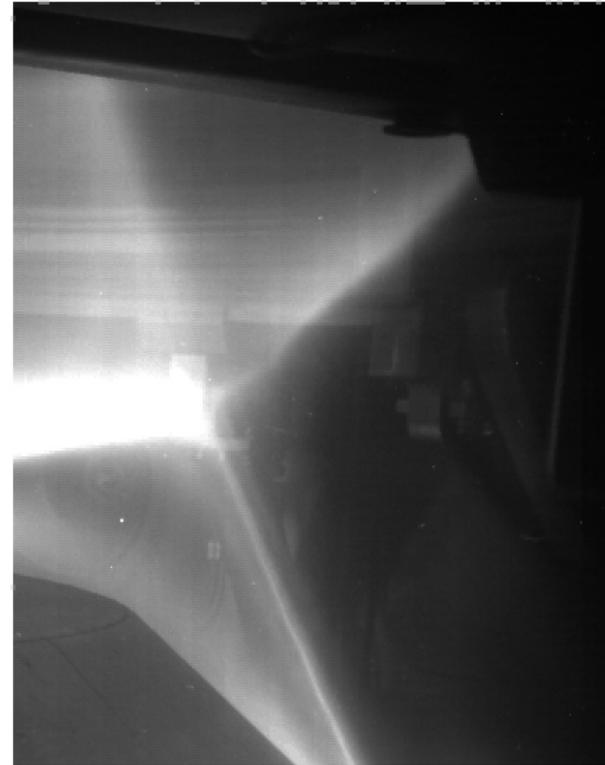
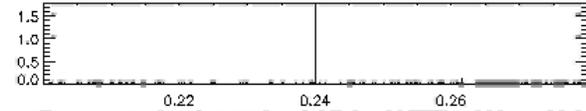


Lobe structures are observed when $I_{ELM} > I_{THR}$



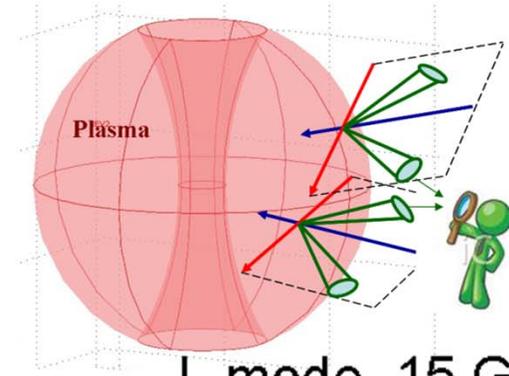
I_{ELM} (kA)

Application of resonant magnetic perturbations in $n=6$

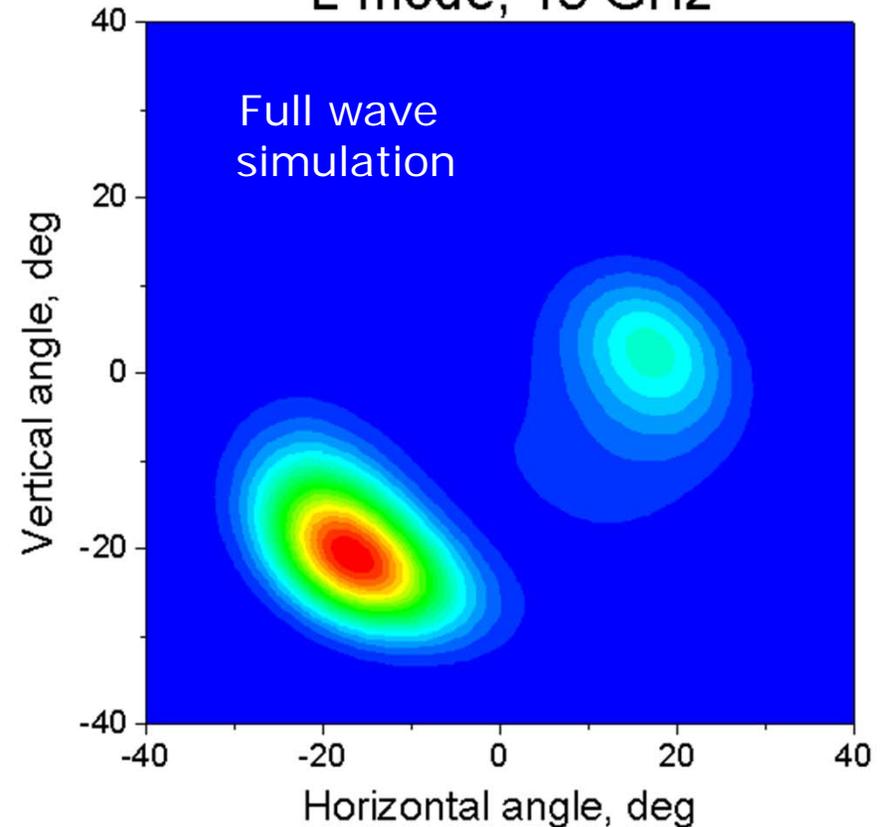


Lobe structures are observed when $I_{ELM} > I_{THR}$

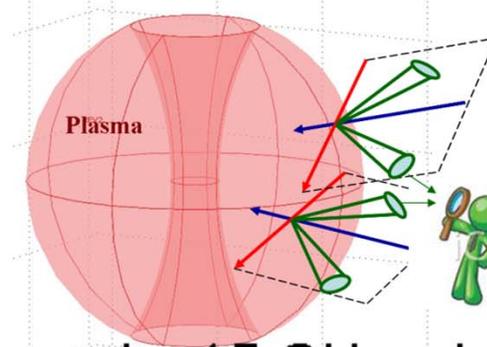
- Electron Bernstein Wave mode conversion mechanism contains information on ∇n_e and B .
 - Need to measure EBW emission window.
- The new Synthetic Aperture Microwave Imaging (SAMI) diagnostic uses a novel technique for producing images of thermal emission in the range 10 - 40 GHz.



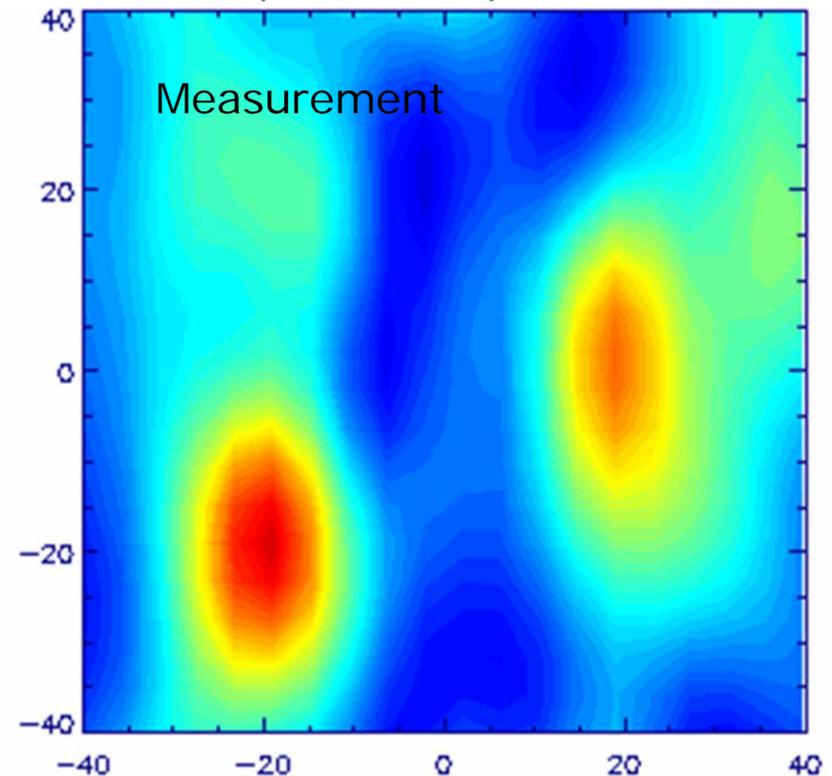
L-mode, 15 GHz



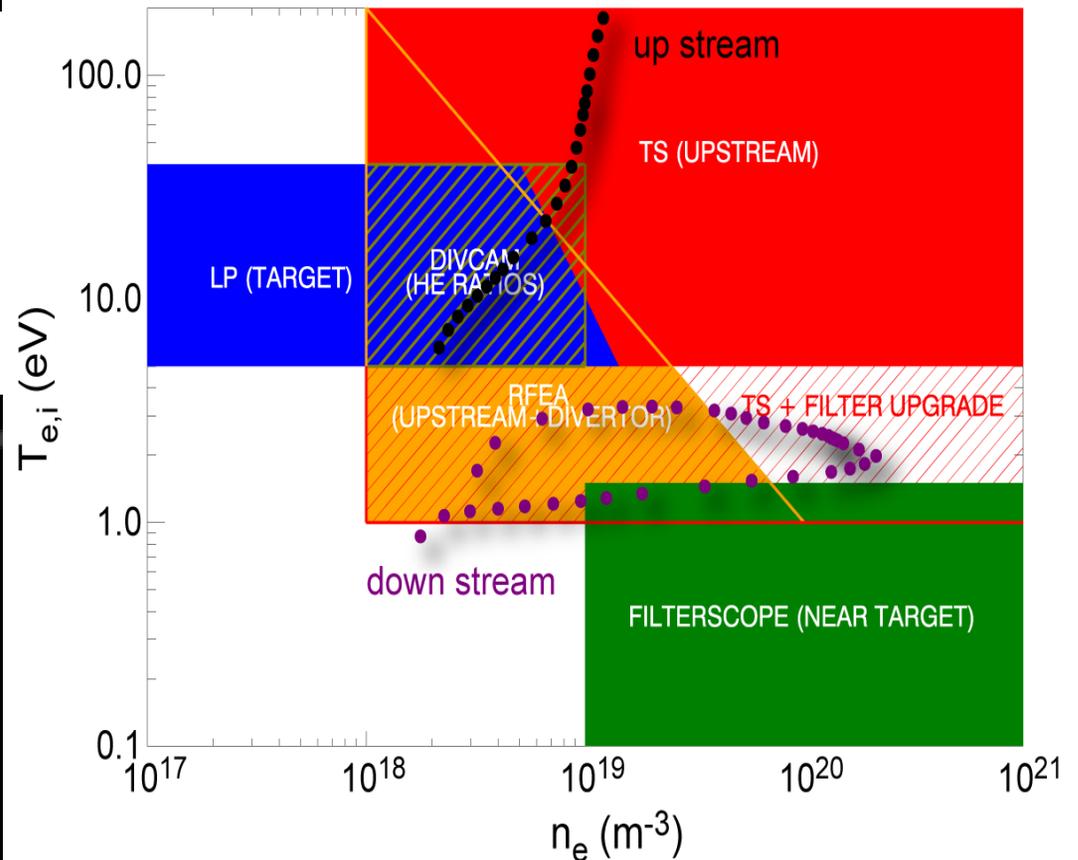
- Electron Bernstein Wave mode conversion mechanism contains information ∇n_e and B .
 - Need to measure EBW emission window.
- The new Synthetic Aperture Microwave Imaging (SAMI) diagnostic uses a novel technique for producing images of thermal emission in the range 10 - 40 GHz.
- First SAMI images from EBW emission obtained.
 - Technique works, but some differences between experiment and simulation.
- First steps to calculate $j(r)$.



L-mode, 15 GHz, shot #27022

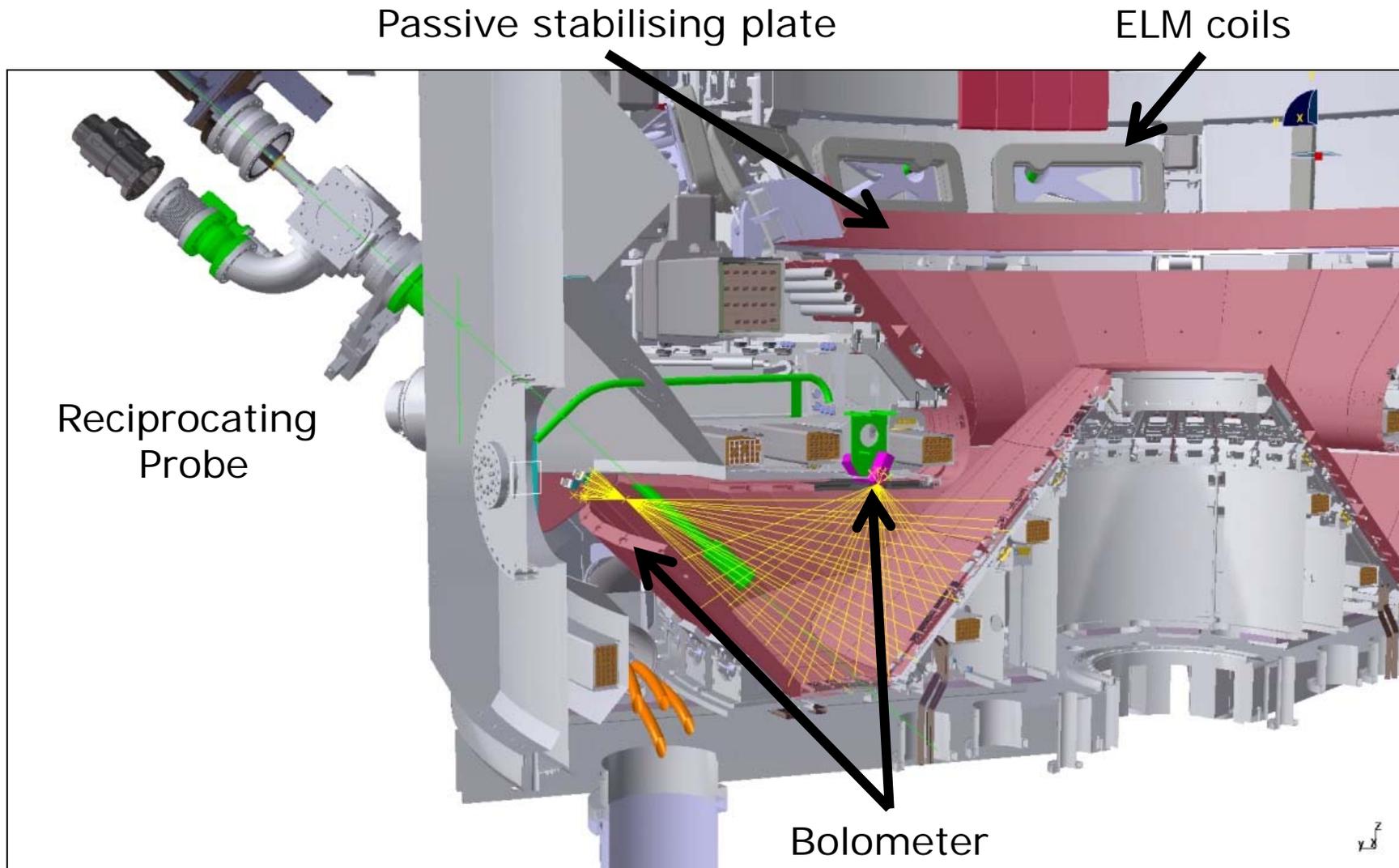


- Large range in n_e and T_e in the SOL.
- Low temperature plasma.
 - ⇒ Neutrals, impurities
 - ⇒ Line radiation, atomic data.



- The SOL is turbulent: $\delta n/n \sim O(1)$
- Many different diagnostics needed

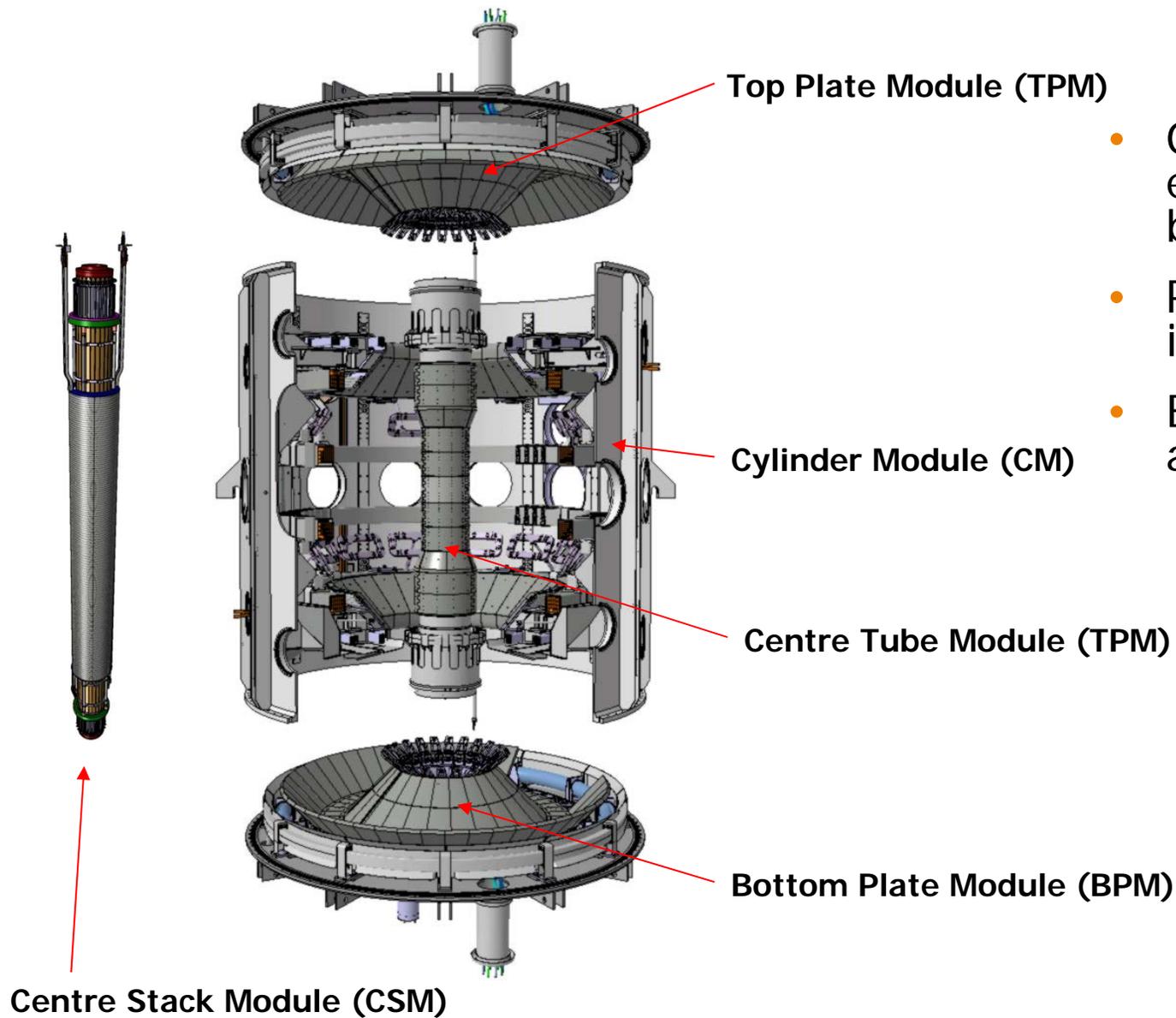
Divertor diagnostic challenge



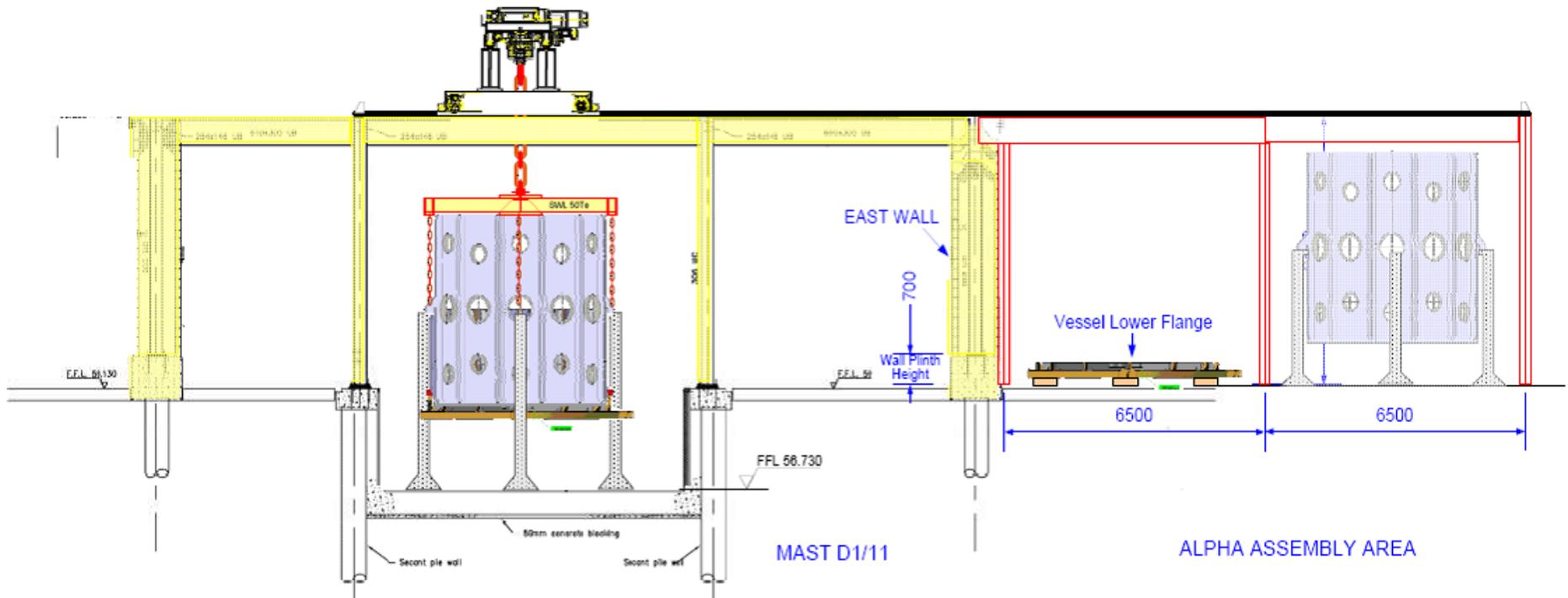
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Enabling current drive physics

Through the plasma edge...
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The engineering!

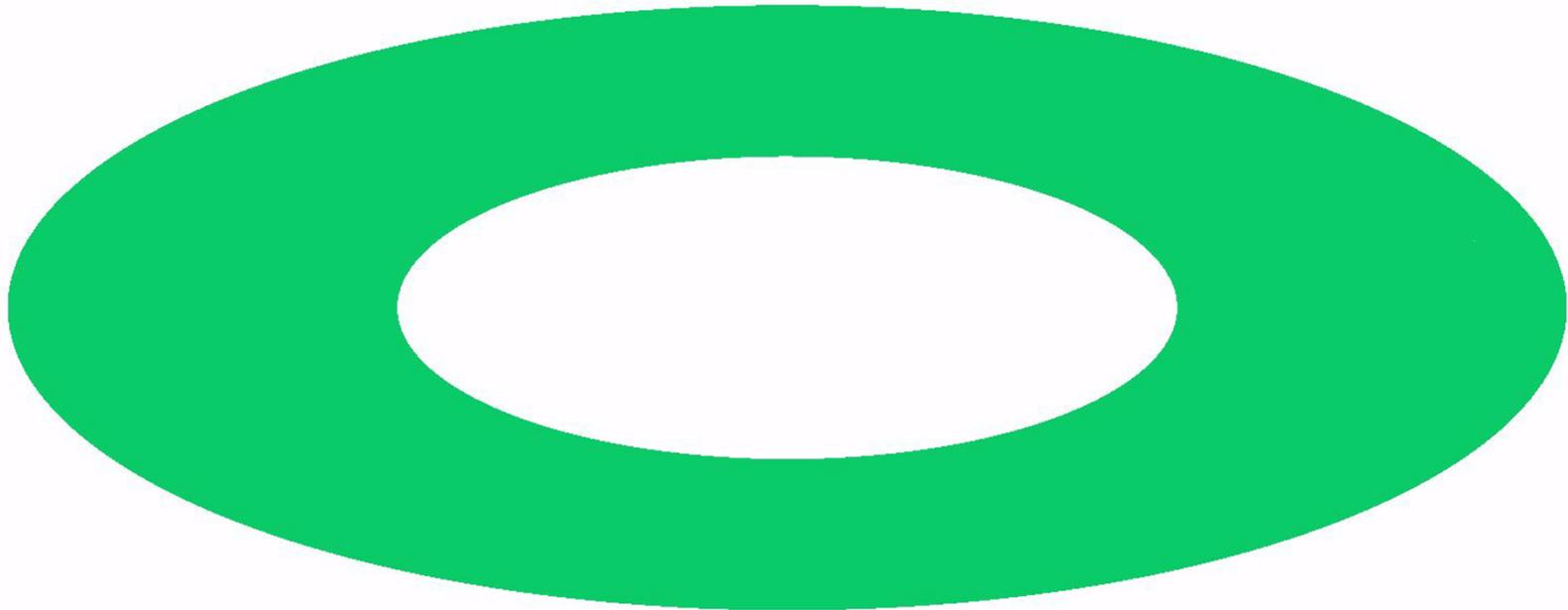


- Optimise engineering break.
- Progress work in parallel.
- Better alignment.



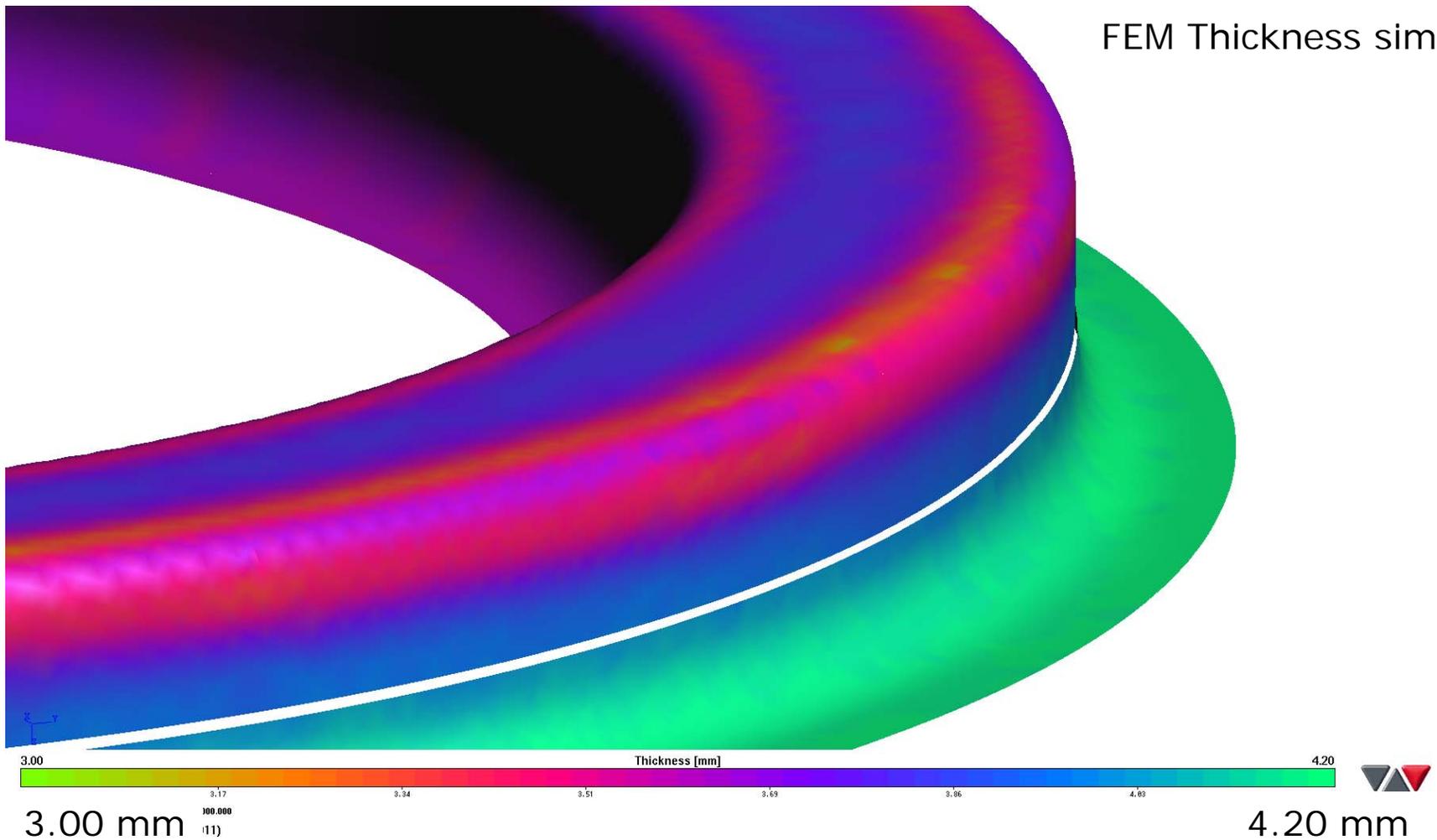
- Move MAST out of the block house (test cell).
- Parallel assembly outside the machine area.

FEM Strain simulation



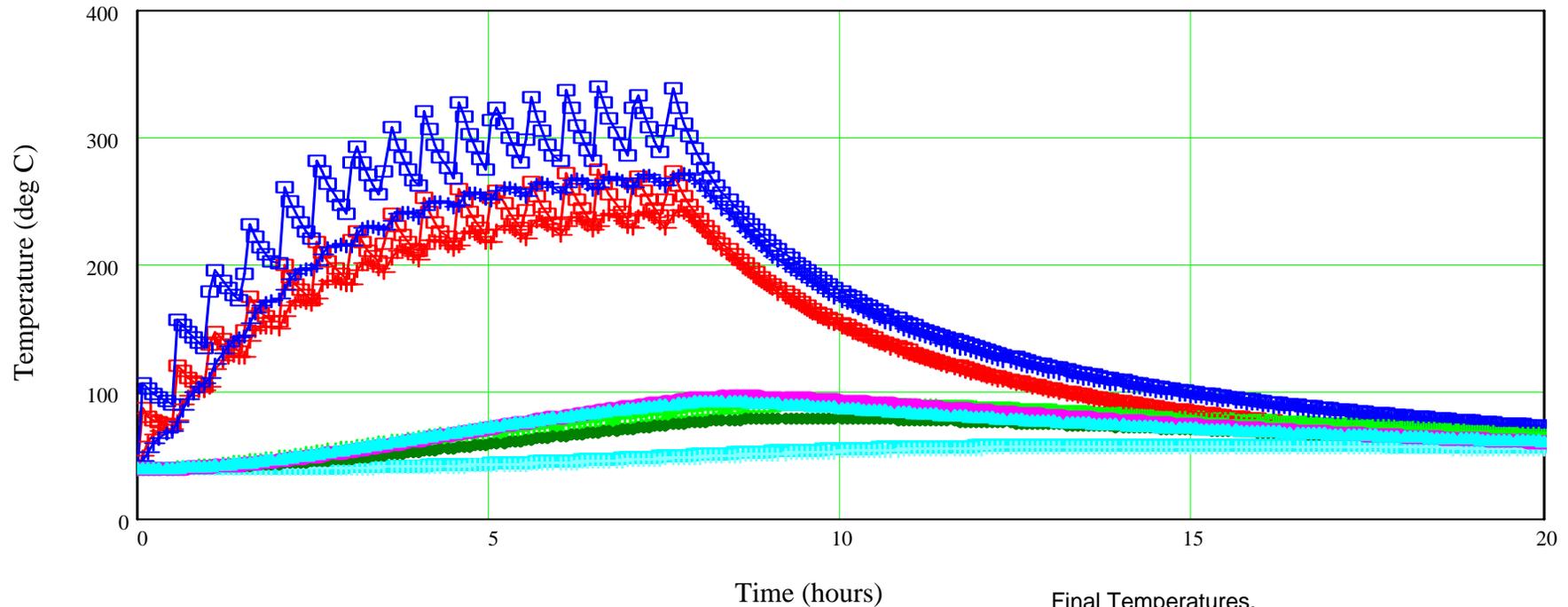
- D1 can (4mm SS 316L) most complicated.
- Assessment of prototype is positive.

FEM Thickness simulation



- D1 can (4mm SS 316L) most complicated.
- Assessment of prototype is positive.

MAST-U Divertor Cooling



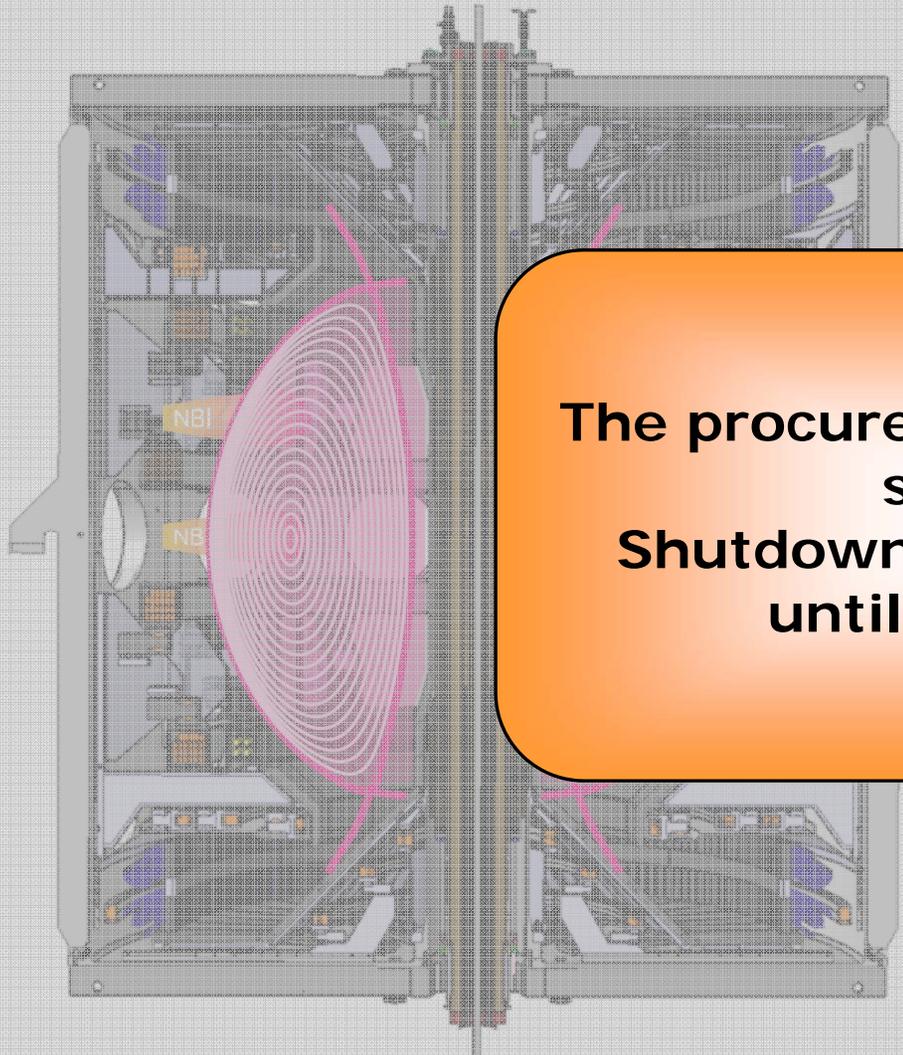
- DT1 Tiles
- DT2 Tiles
- DT3 Tiles
- DT4 Tiles
- B1 Back-Plate
- B2 Black-Plate
- B3 Back-Plate
- B4 Back-Plate
- Strike Plate
- Nose
- Baffle

Heating Fractions

- $\lambda_{DT1} = 0.2$
- $\lambda_{DT2} = 0.8$
- $\lambda_{DT3} = 0.0$
- $\lambda_S = 0.0$
- $\lambda_B = 0.0$

Final Temperatures.

- $T1t_{n_{int}} = 61.3 \text{ C}$ DT1
- $T2t_{n_{int}} = 73.4 \text{ C}$ DT2
- $T3t_{n_{int}} = 67.9 \text{ C}$ DT3
- $T4t_{n_{int}} = 56.4 \text{ C}$ DT4
- $\lambda_{bo} = 0.0$
- $\lambda_{op} = 1.0$



Super-X divertor
⇒ Finding solutions for the exhaust problem

The procurement process has started!
Shutdown from Oct. 2013 until April 2015

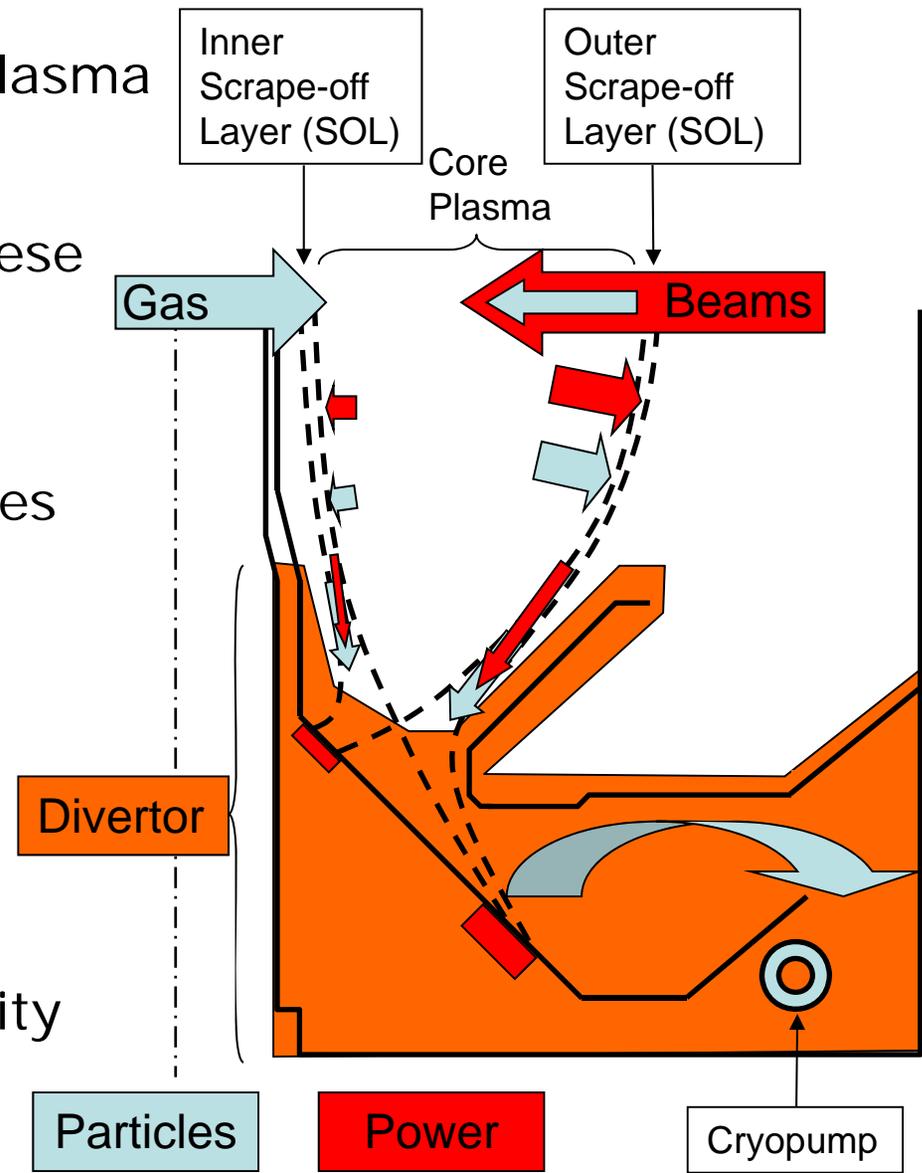
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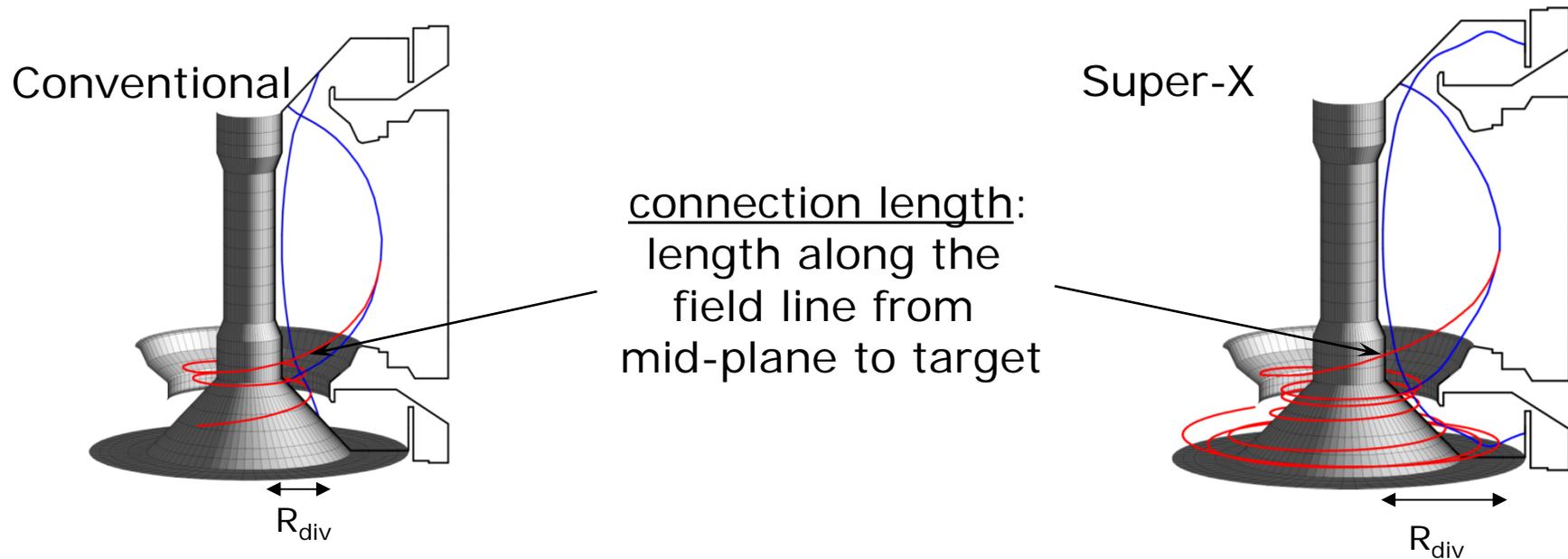
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Advanced current profiles
⇒ Advanced stability for better confinement

A Divertor (Simplistically...)

- Put heat + particles into core plasma
- Transport processes remove these
- Divertor handles “exhaust”:
 - Controls surface temperatures
 - Controls impurities
 - Removes particles
- Benefits:
 - Improved performance
 - Cleaner experiments
 - Control of core plasma density





- *Basic concept:* Increase radius of strike point (R_{div}).
 - + Larger $R_{div} \Rightarrow$ larger wetted area.
 - + Lower toroidal field \Rightarrow lower parallel heat flux.
- Increased connection length using low poloidal field in divertor chamber.

- + Increased volume for plasma interactions.

Super-X concept:
Valanju et al. Phys. Plasmas
16, 056110 (2009)